

Program EVALPLOT  
(Version 2018-1)

by

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(Present Contact Information)

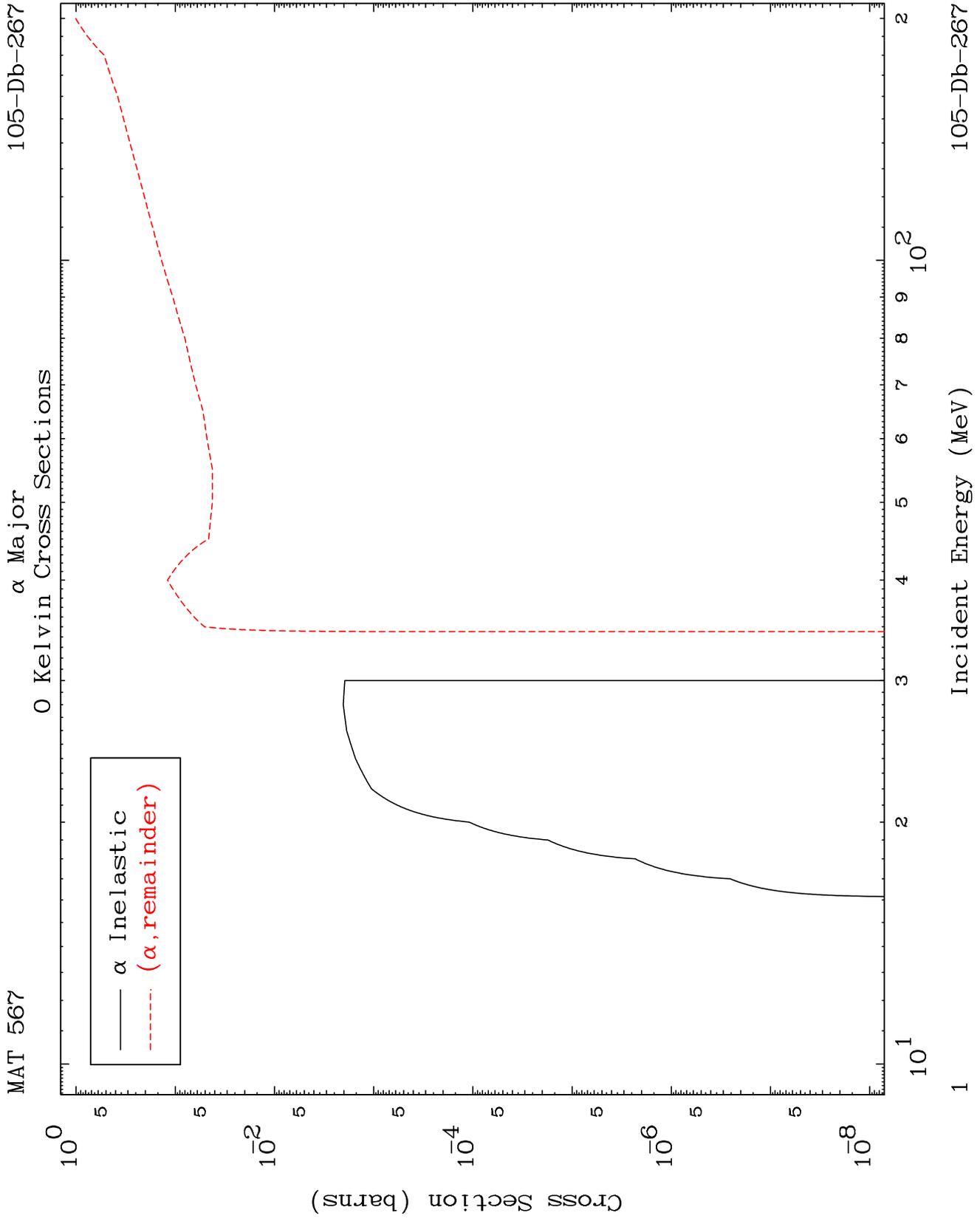
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E.Mail: [redcullen1@comcast.net](mailto:redcullen1@comcast.net)  
Web: [redcullen1.net/HOMEPAGE.NEW](http://redcullen1.net/HOMEPAGE.NEW)

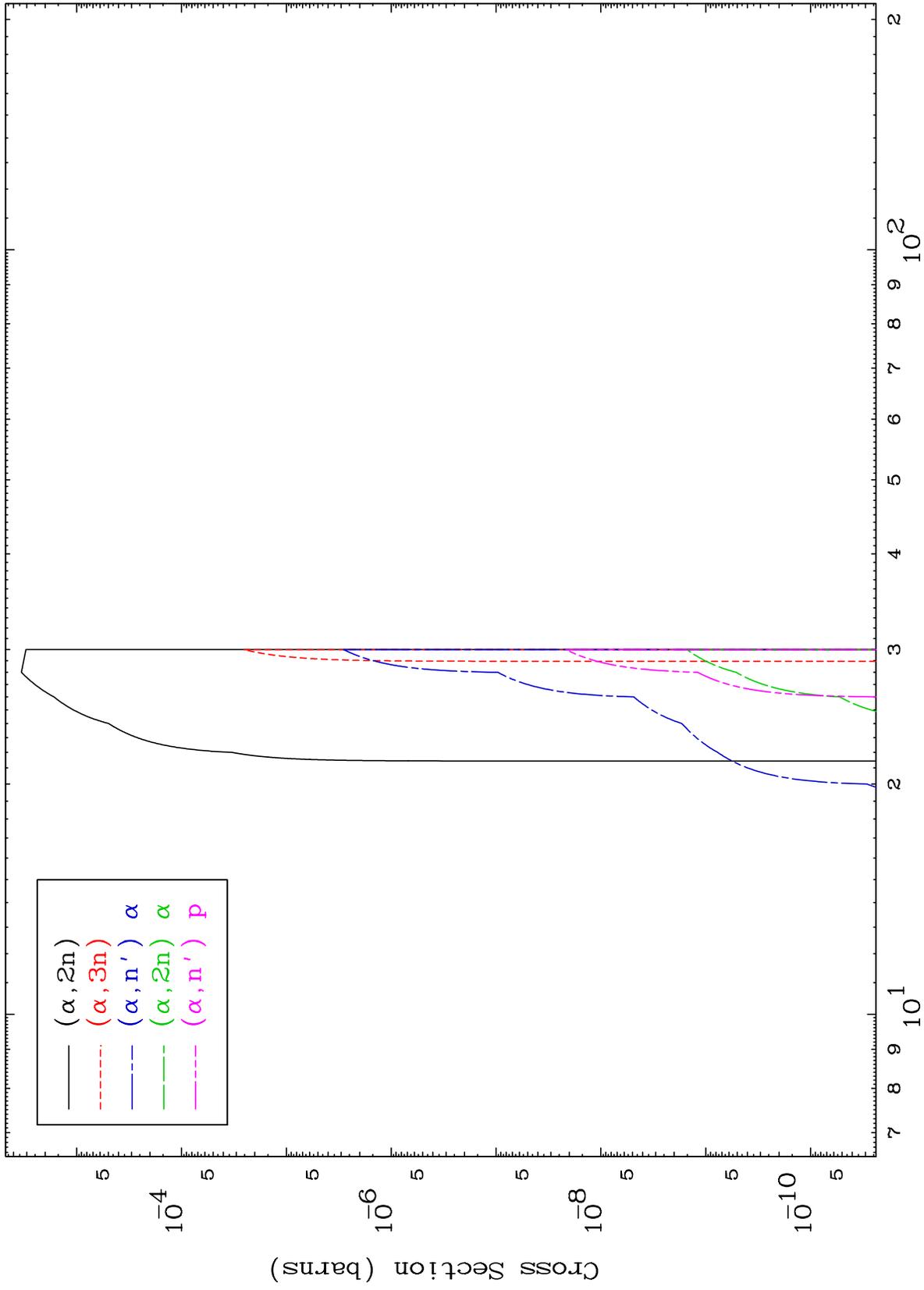
Press Mouse Button to Start



MAT 567

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

105-Db-267



2

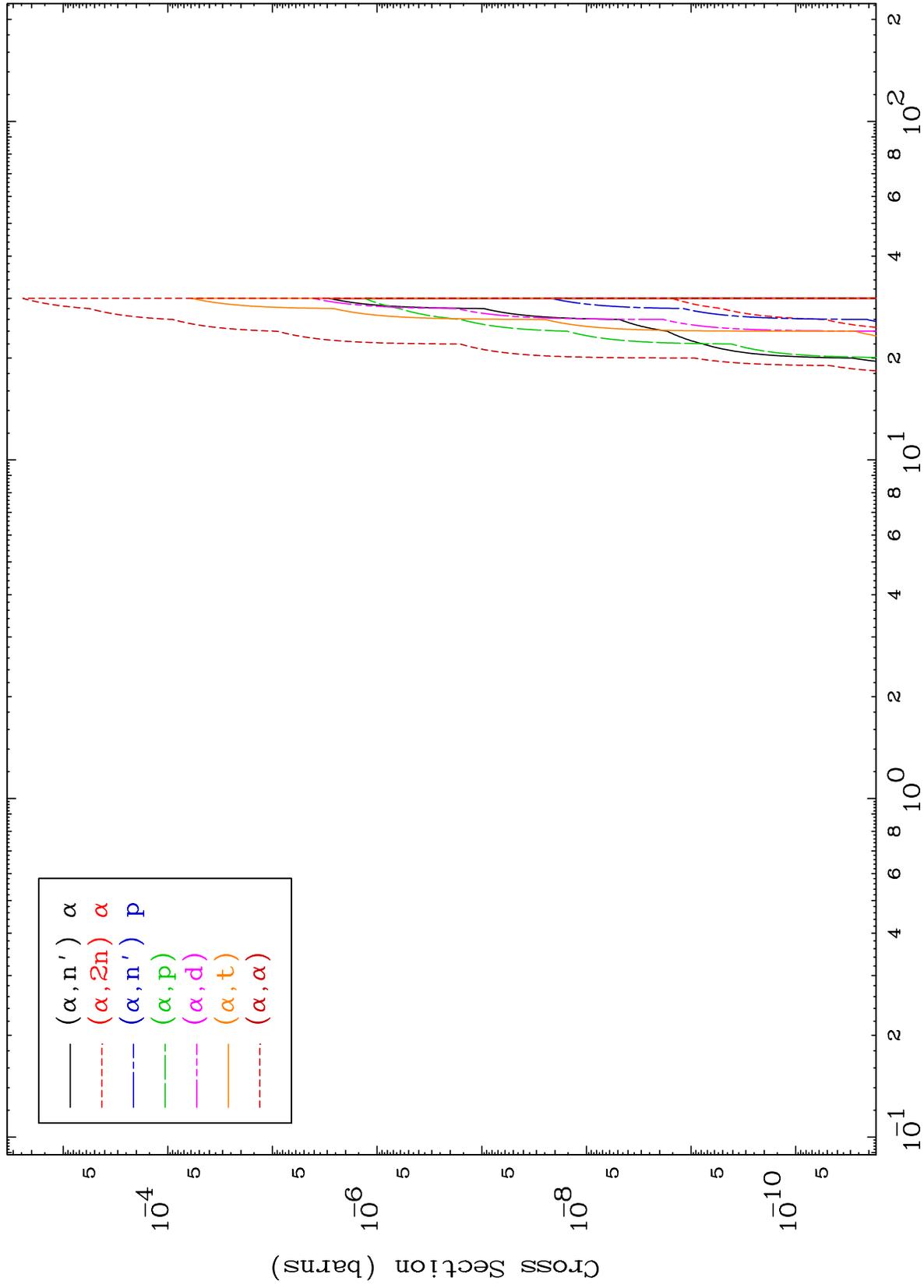
Incident Energy (MeV)

105-Db-267

MAT 567

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

105-Db-267



3

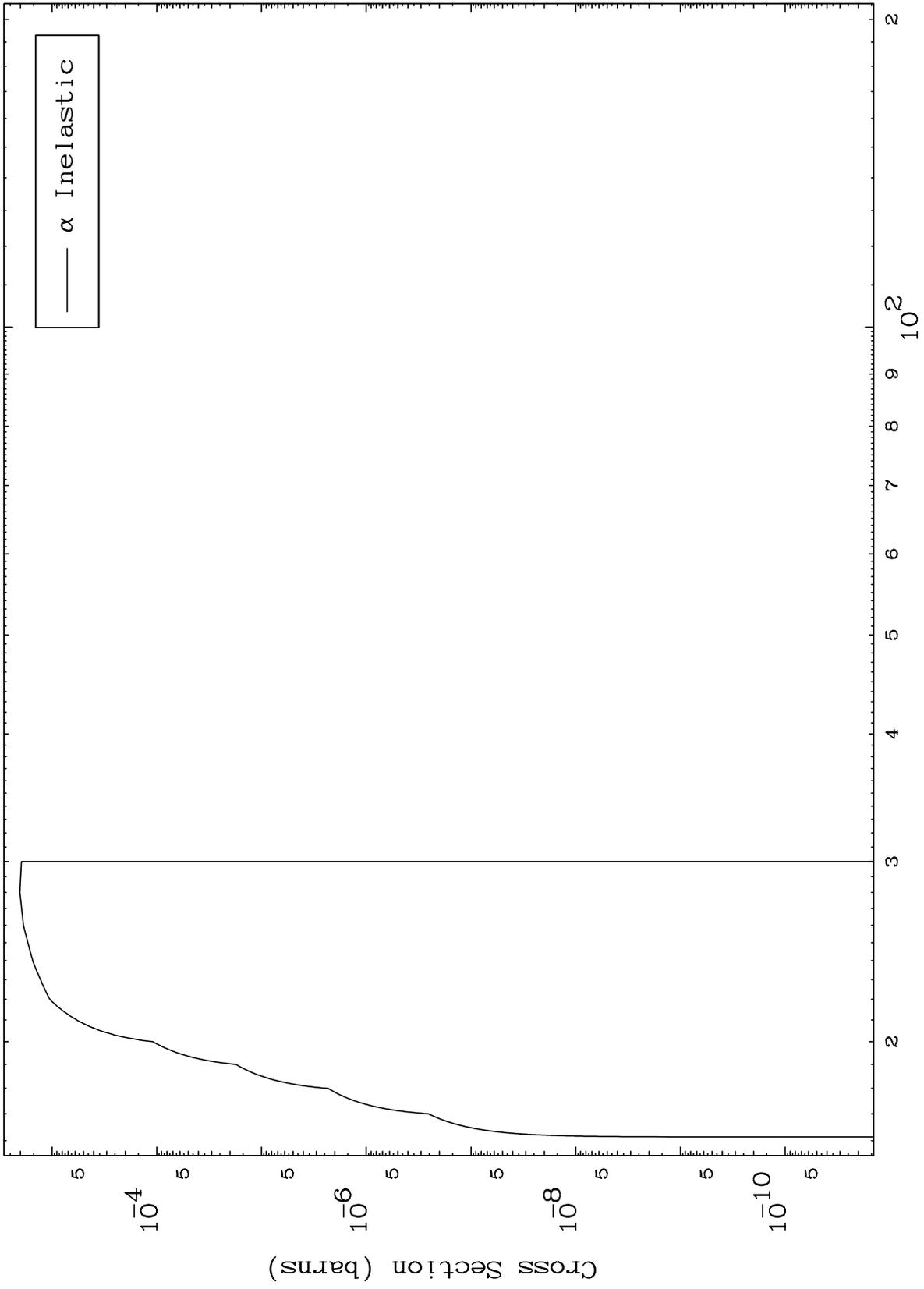
Incident Energy (MeV)

105-Db-267

MAT 567

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

105-Db-267



4

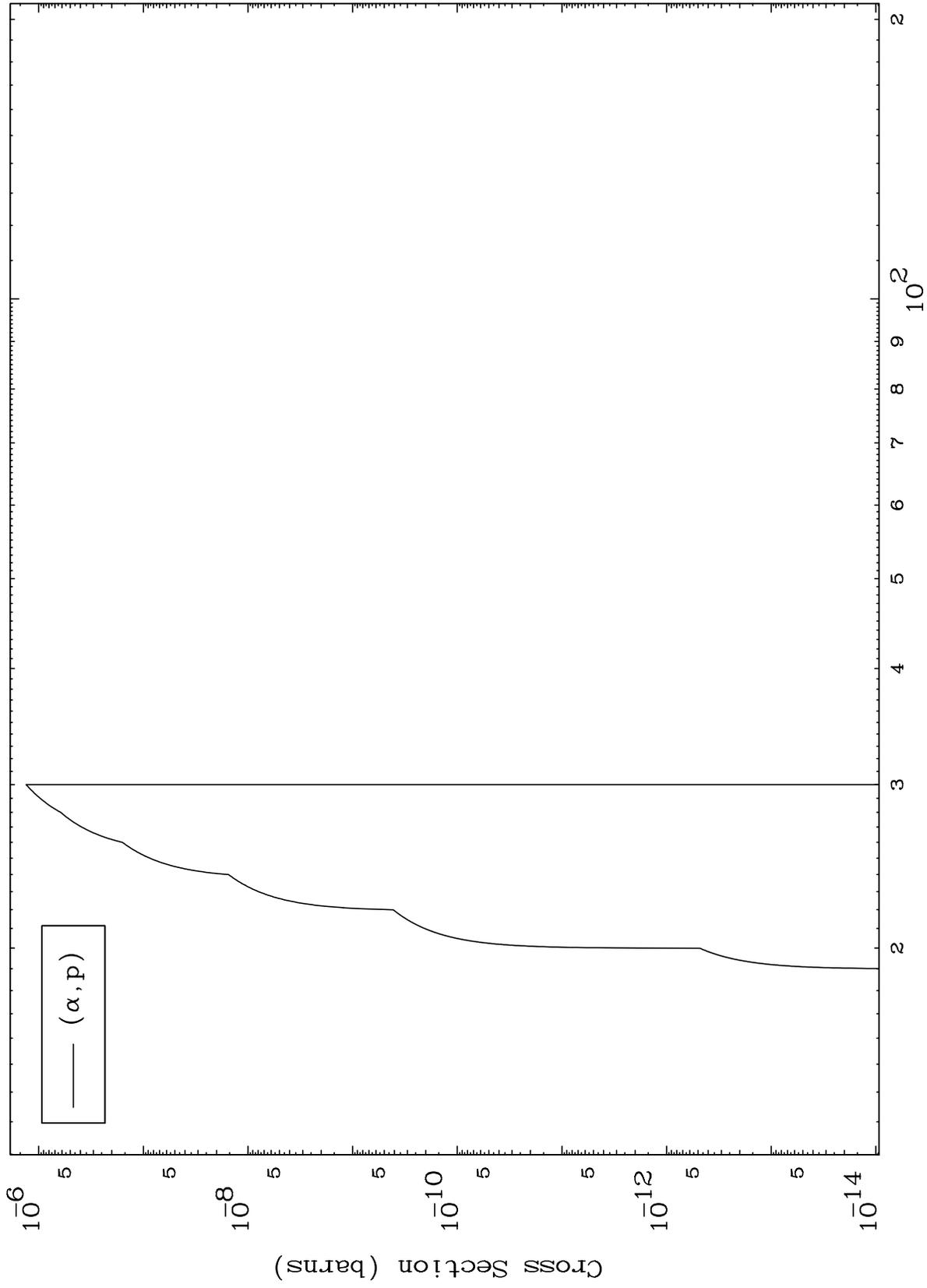
Incident Energy (MeV)

105-Db-267

MAT 567

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

105-Db-267



5

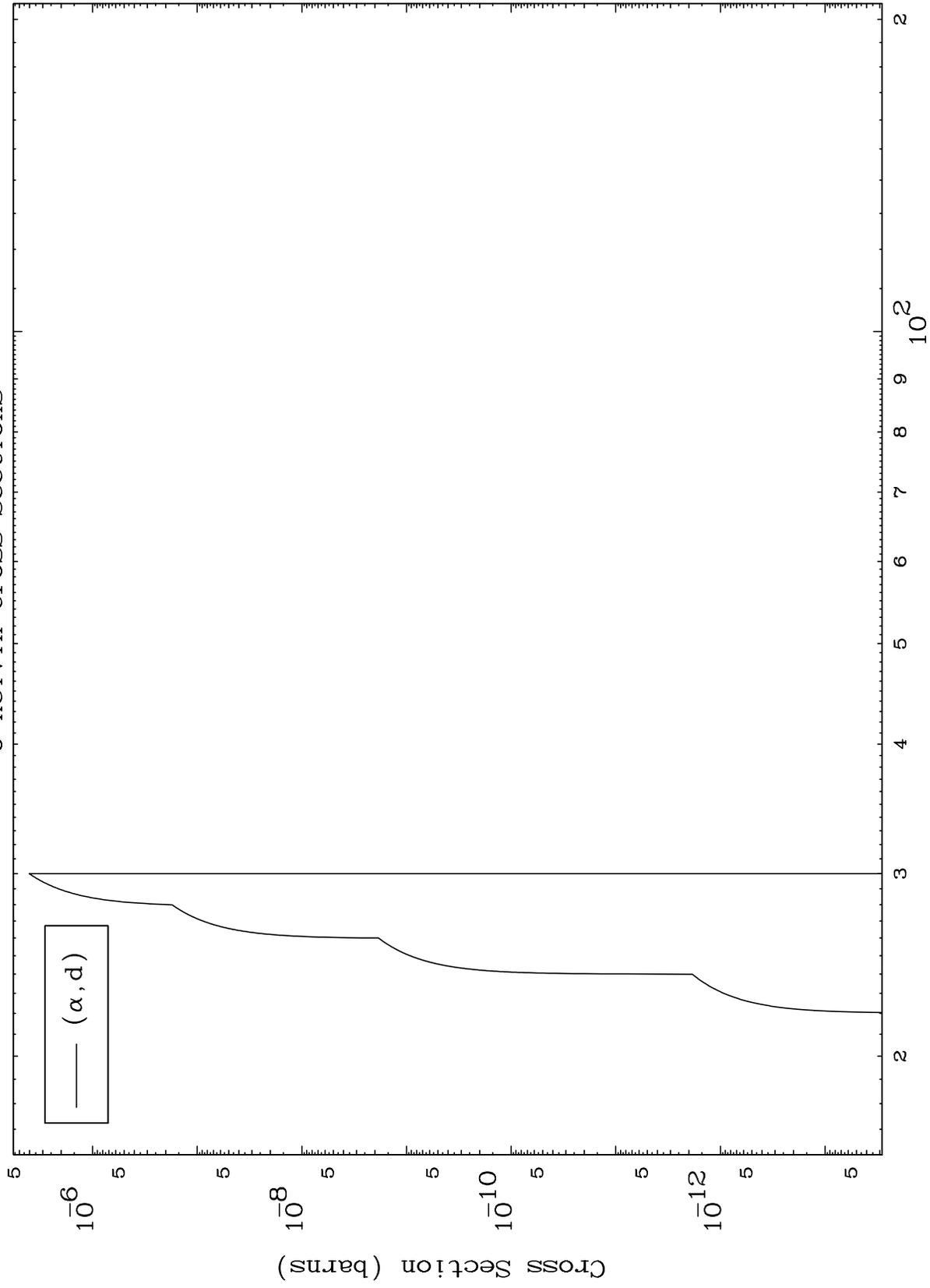
Incident Energy (MeV)

105-Db-267

MAT 567

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

105-Db-267



6

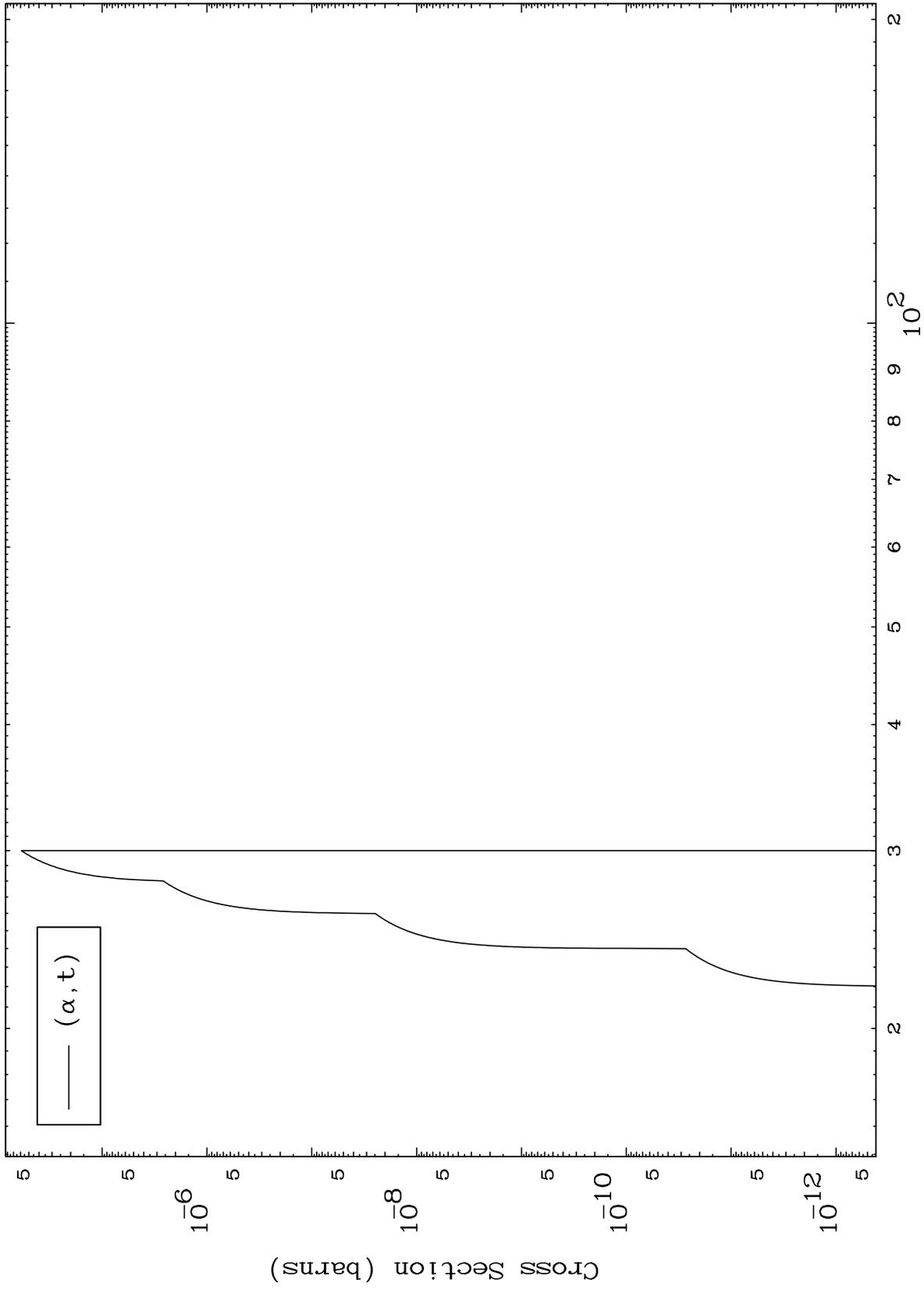
Incident Energy (MeV)

105-Db-267

MAT 567

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

105-Db-267



7

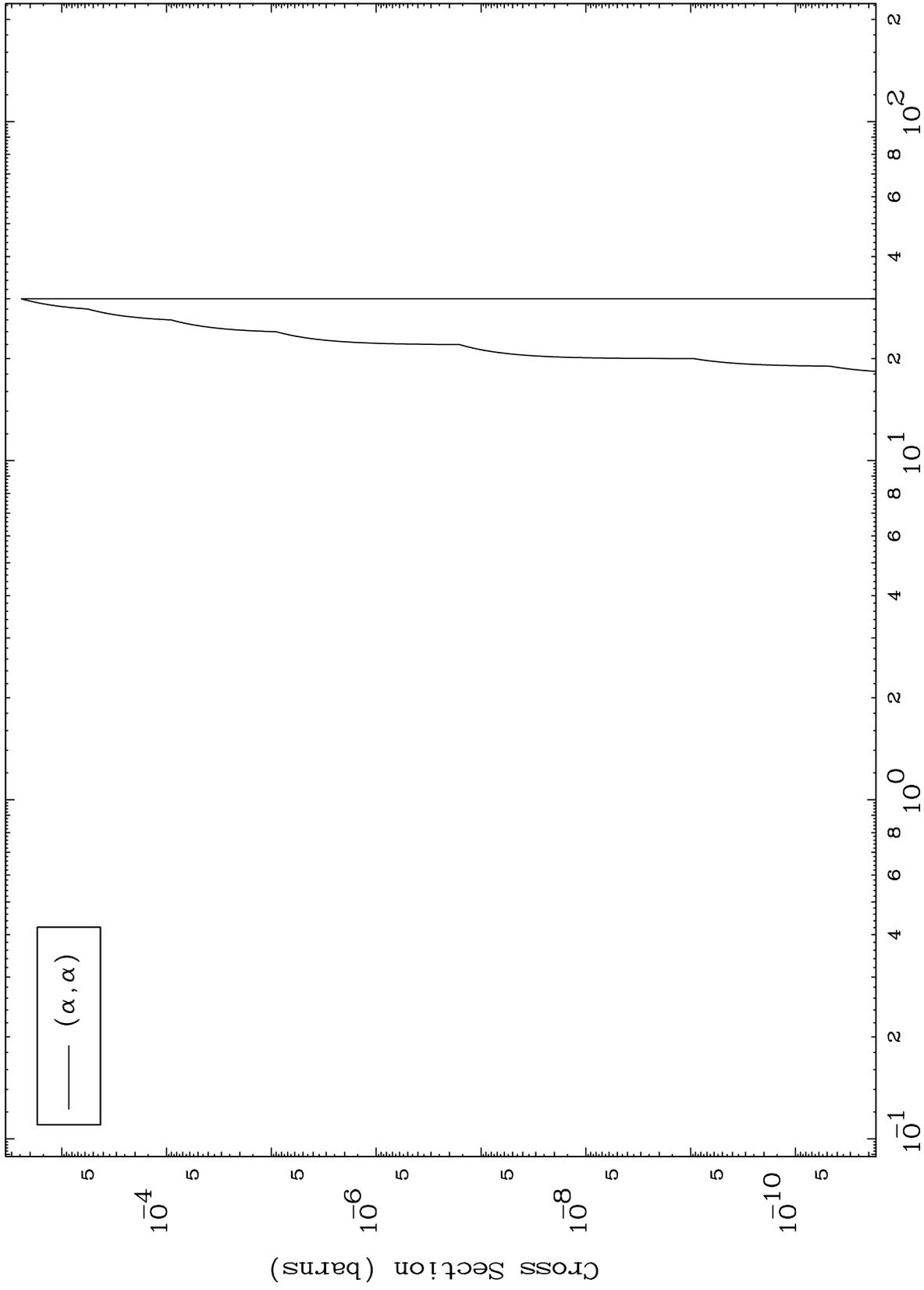
Incident Energy (MeV)

105-Db-267

MAT 567

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

105-Db-267



Incident Energy (MeV)

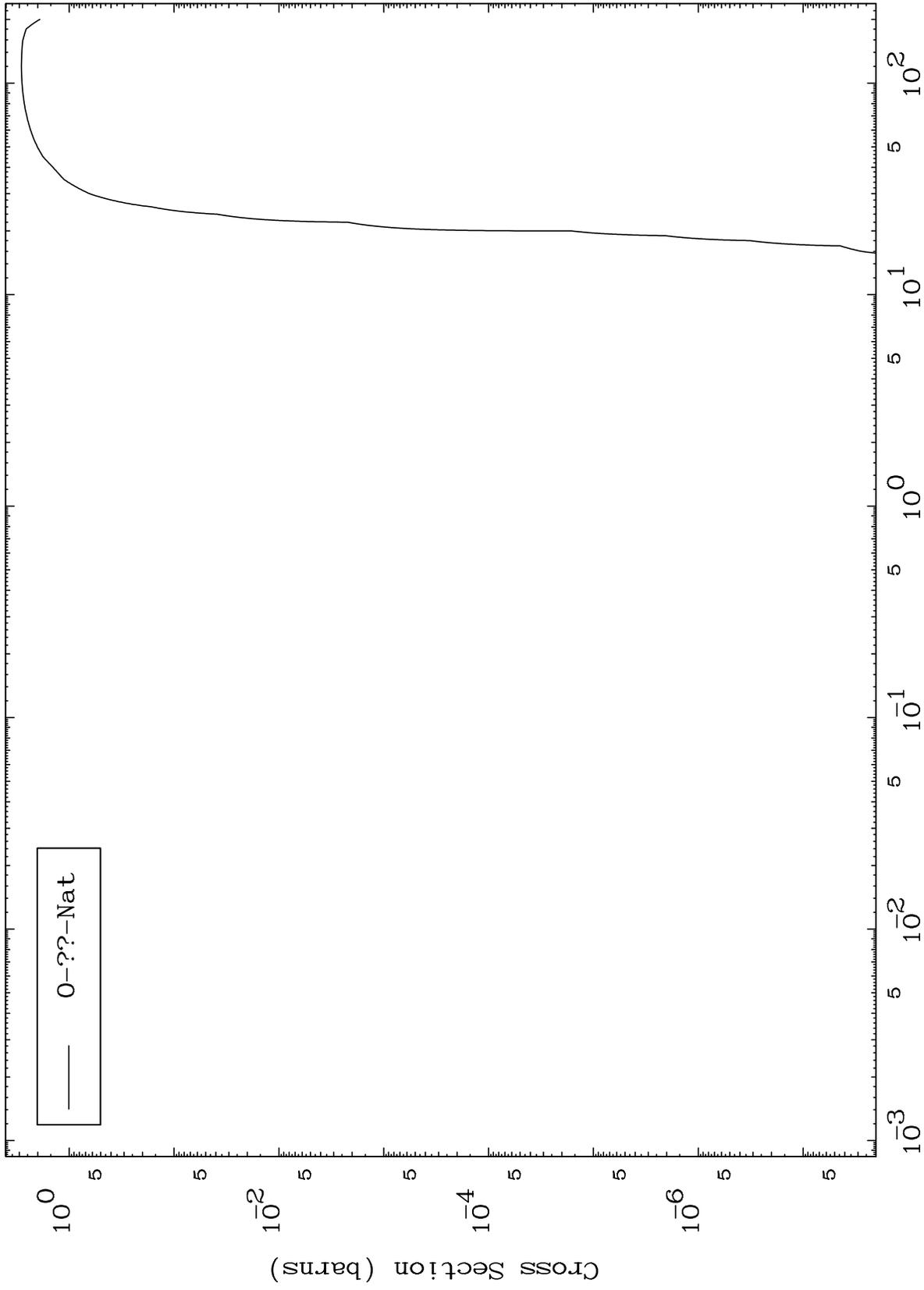
105-Db-267

8

MAT 567

$\alpha$  Fission  
Radionuclide Production Cross Section

105-Db-267



9

Incident Energy (MeV)

105-Db-267