

Program EVALPLOT
(Version 2018-1)

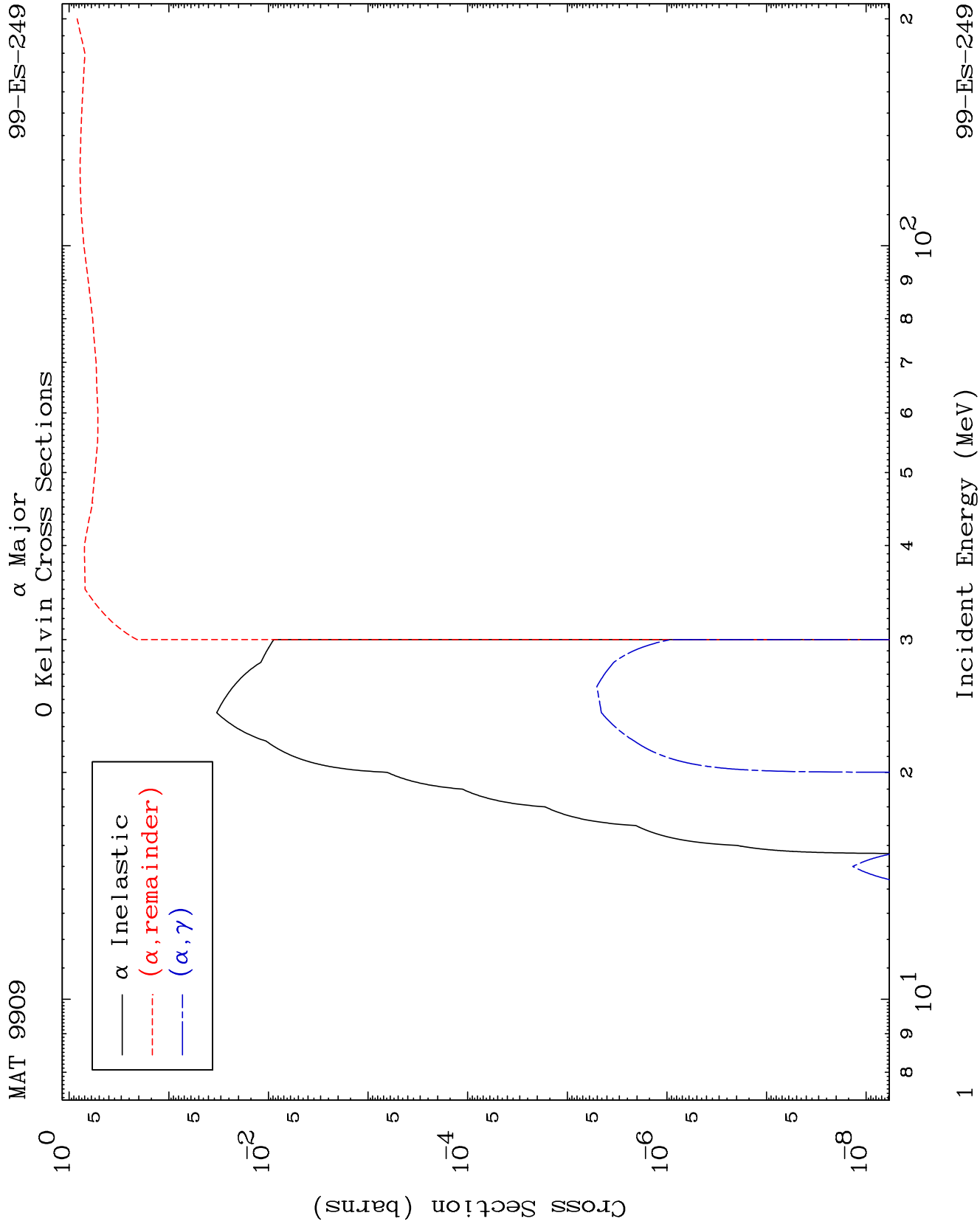
by

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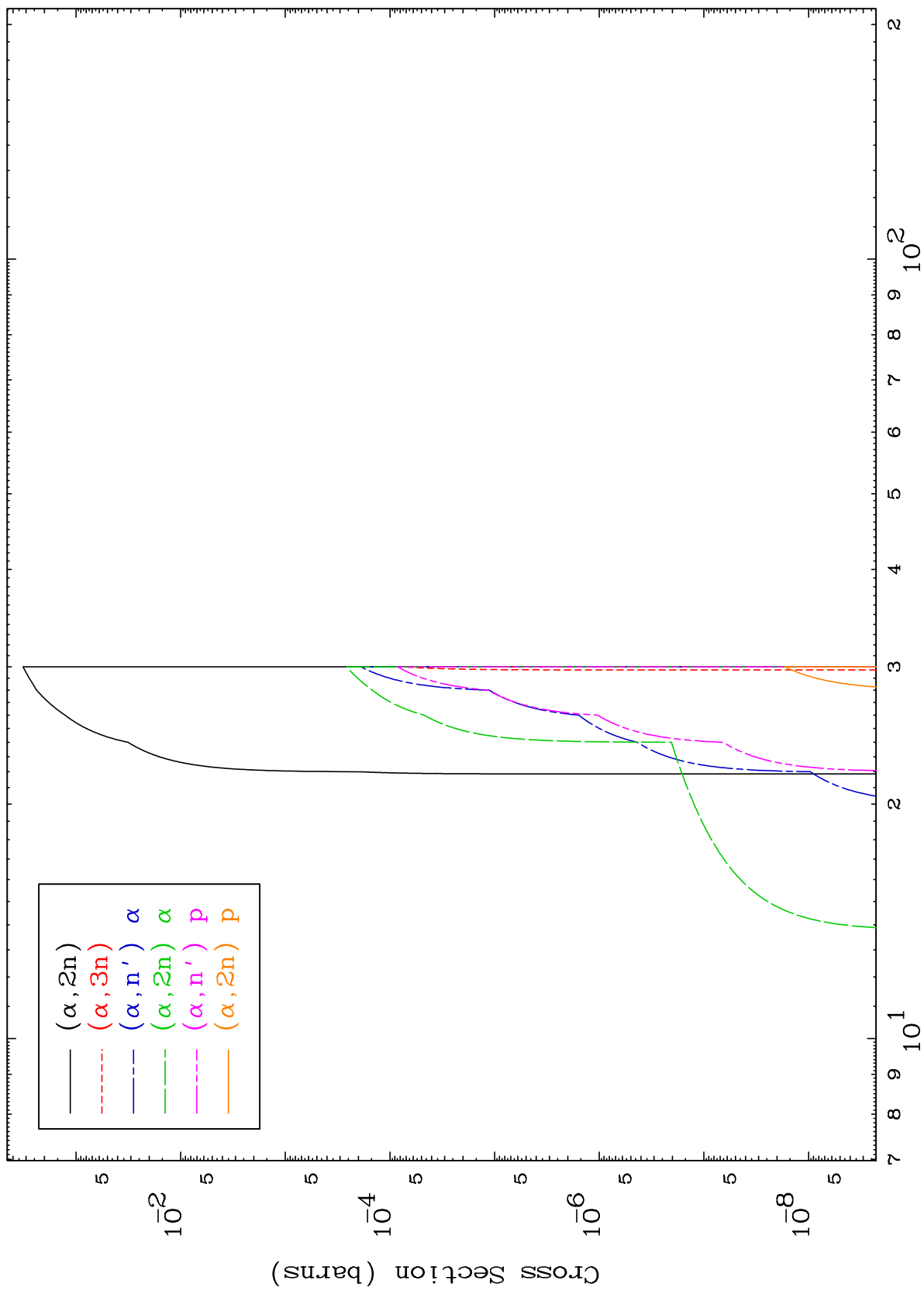
Press Mouse Button to Start



MAT 9909

α Neutron Production
0 Kelvin Cross Sections

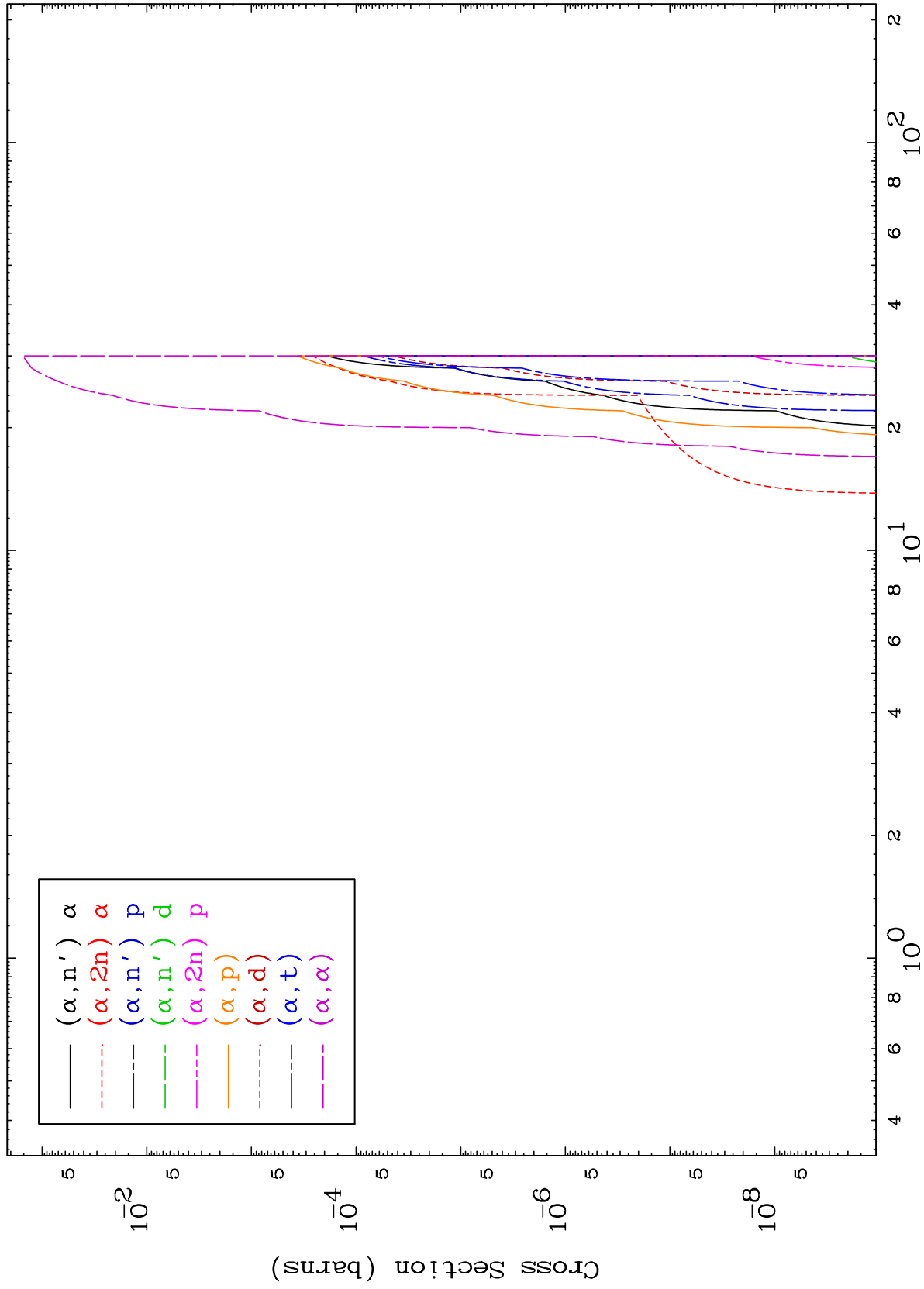
99-Es-249



2

Incident Energy (MeV)

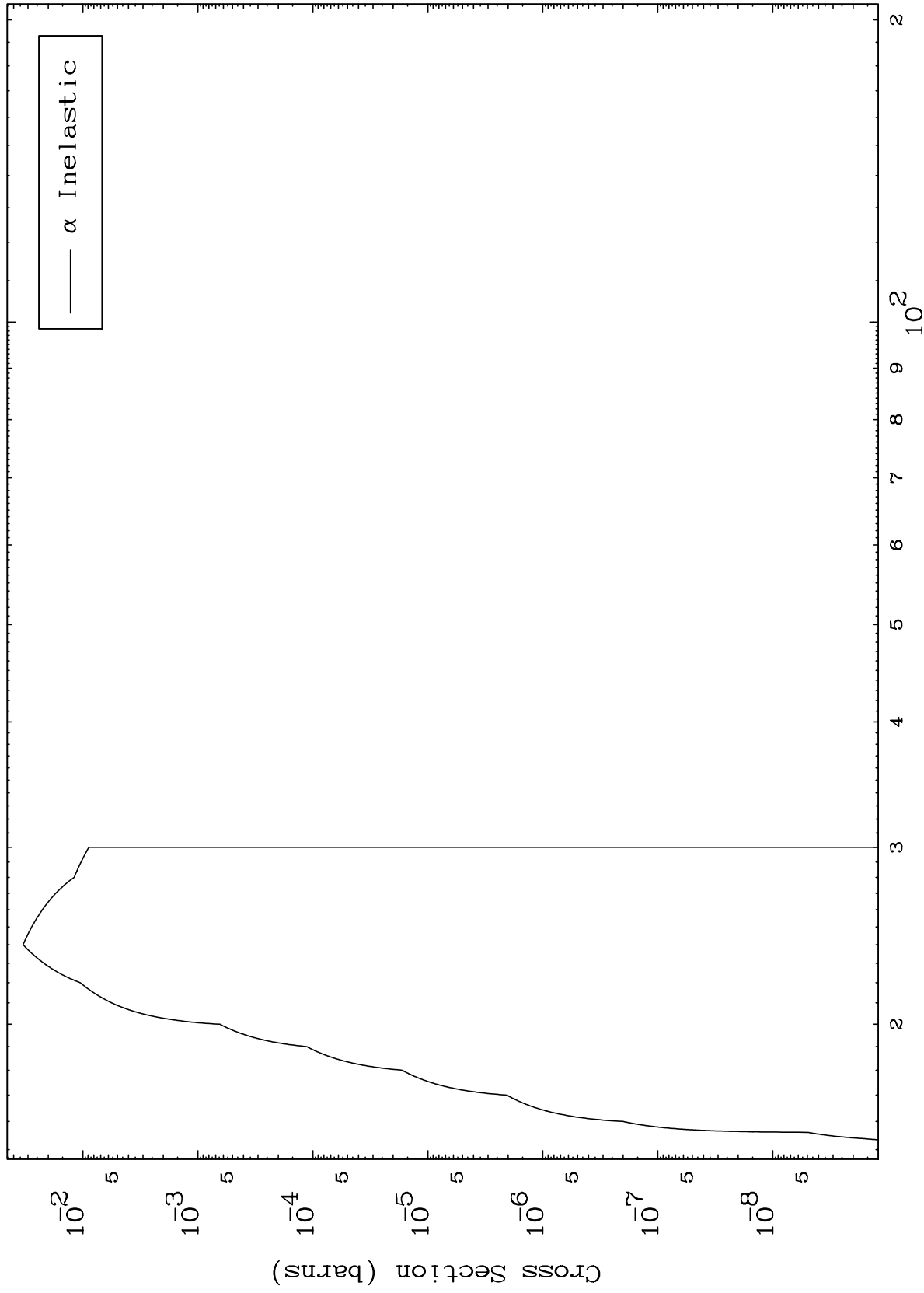
99-Es-249



MAT 9909

(α, n') Level
0 Kelvin Cross Sections

99-Es-249



4

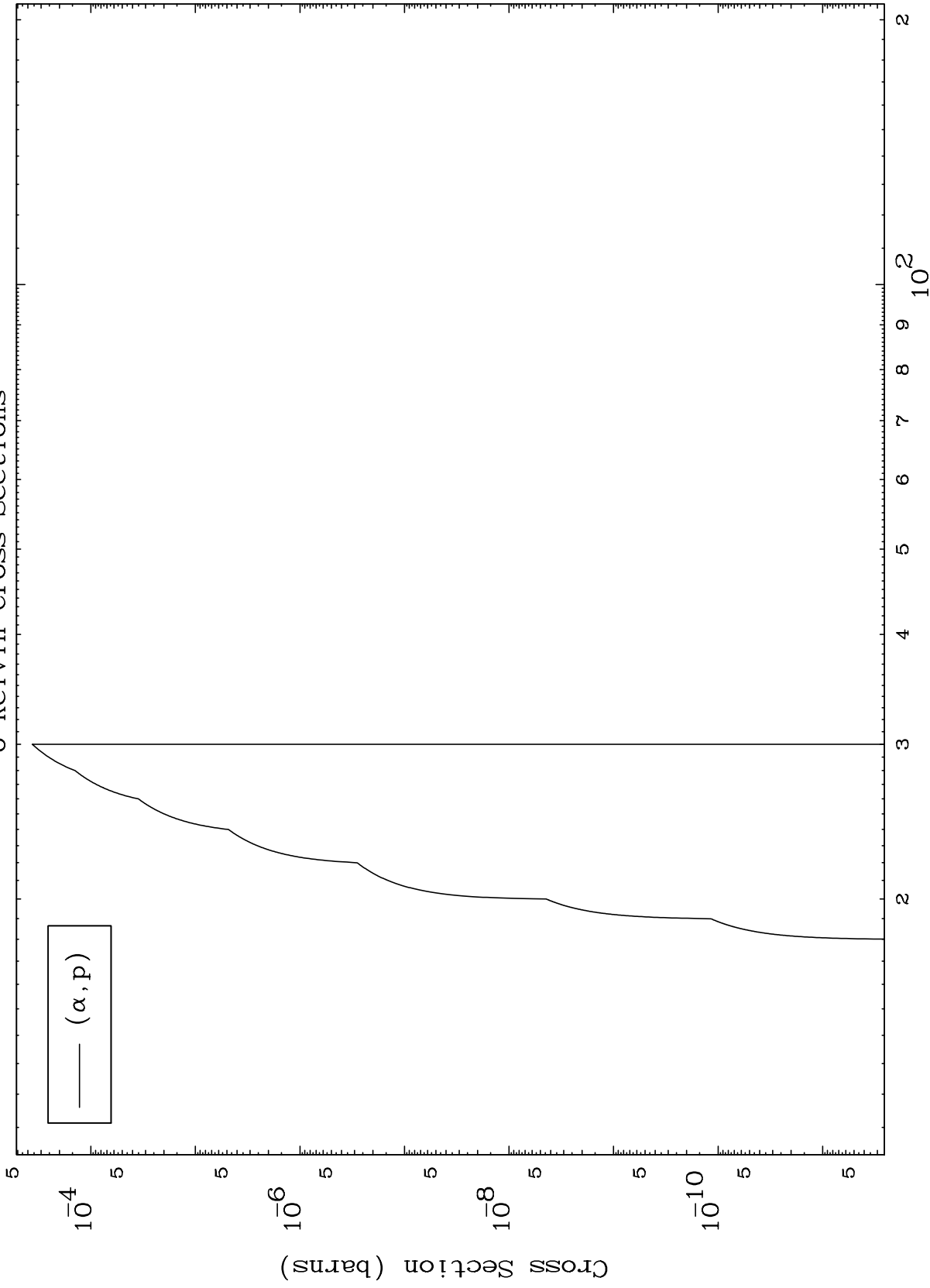
Incident Energy (MeV)

99-Es-249

MAT 9909

99-Es-249

(α ,p) Levels
0 Kelvin Cross Sections



5

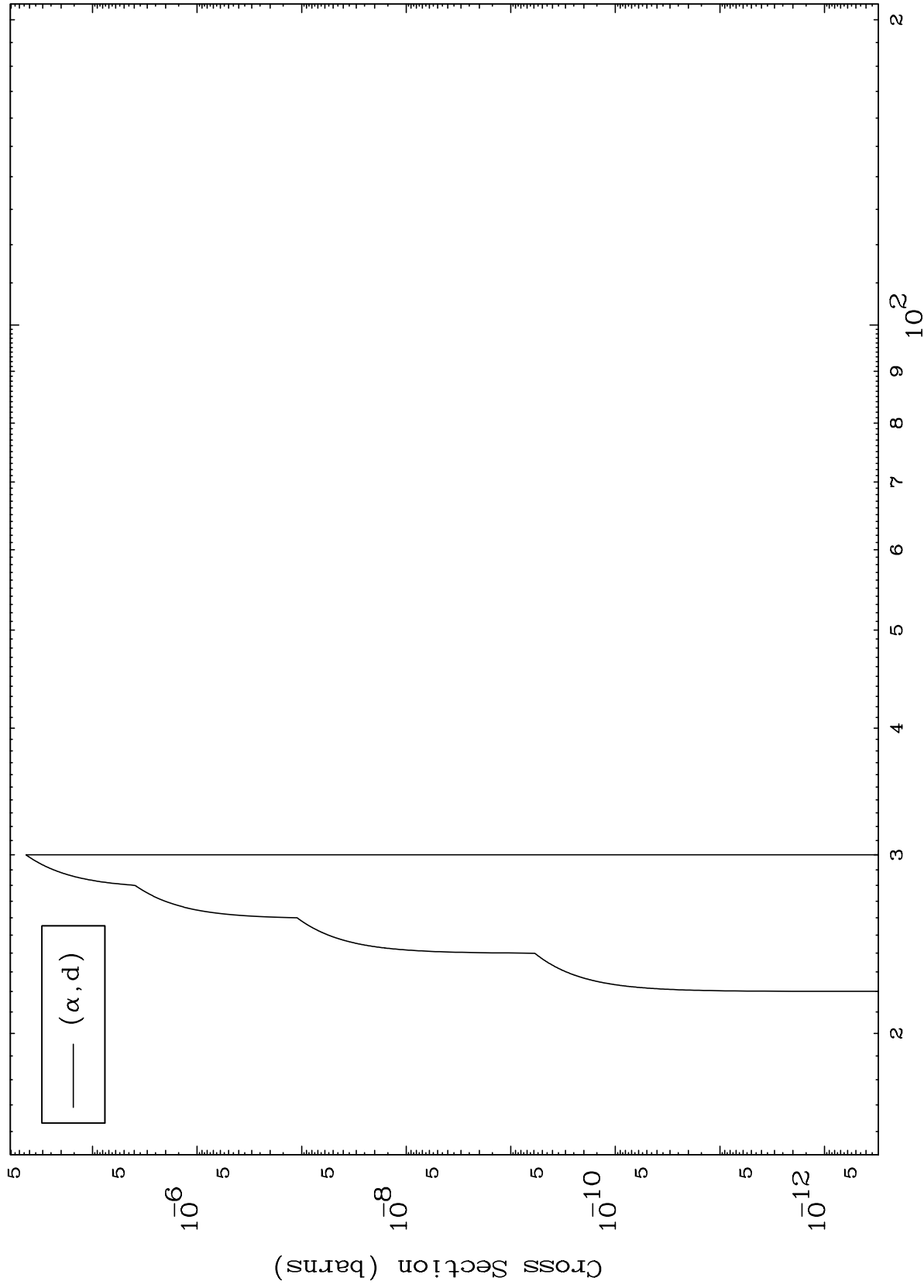
99-Es-249

Incident Energy (MeV)

MAT 9909

(α ,d) Levels
0 Kelvin Cross Sections

99-Es-249



6

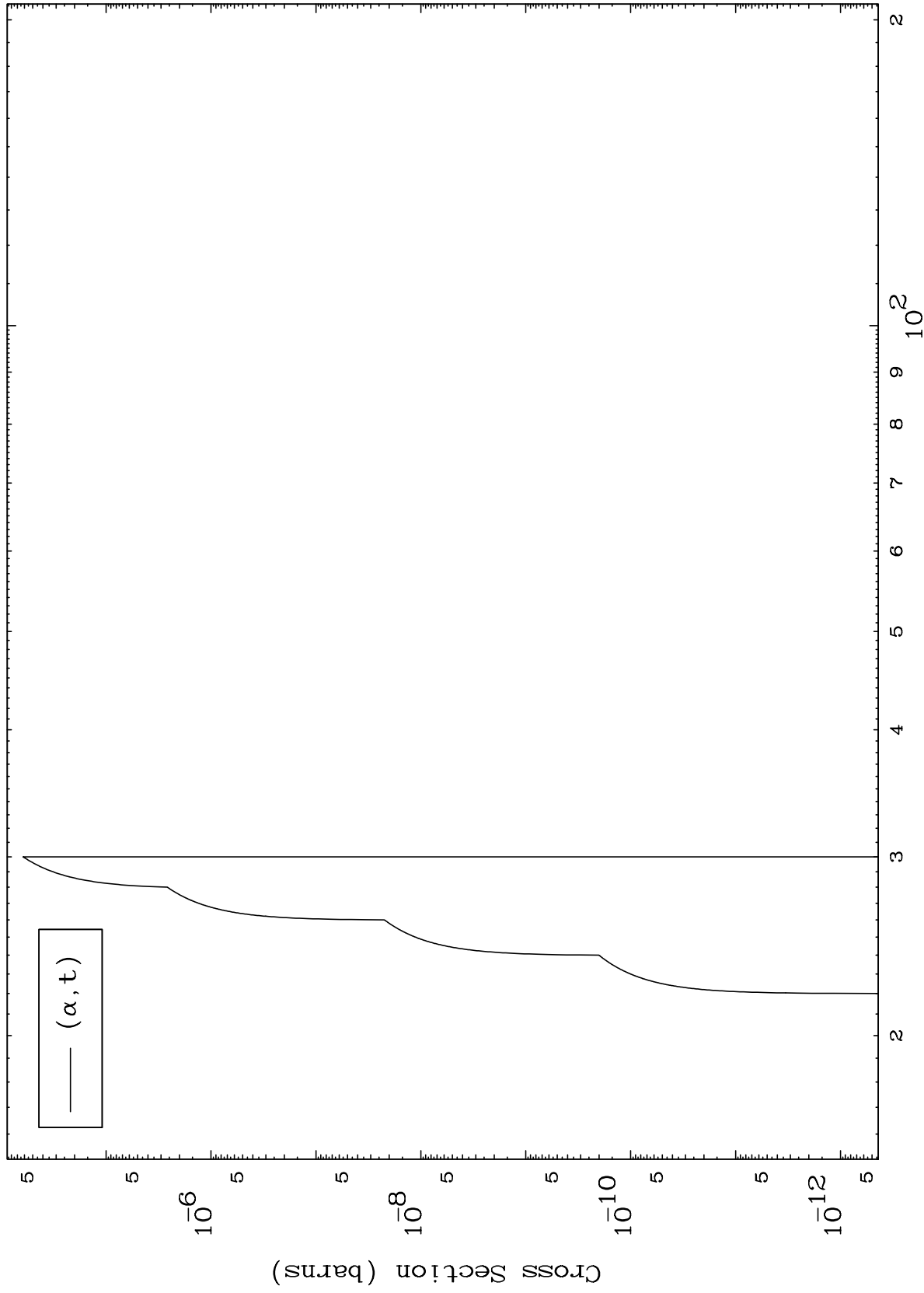
Incident Energy (MeV)

99-Es-249

MAT 9909

99-Es-249

(α, t) Levels
0 Kelvin Cross Sections



7

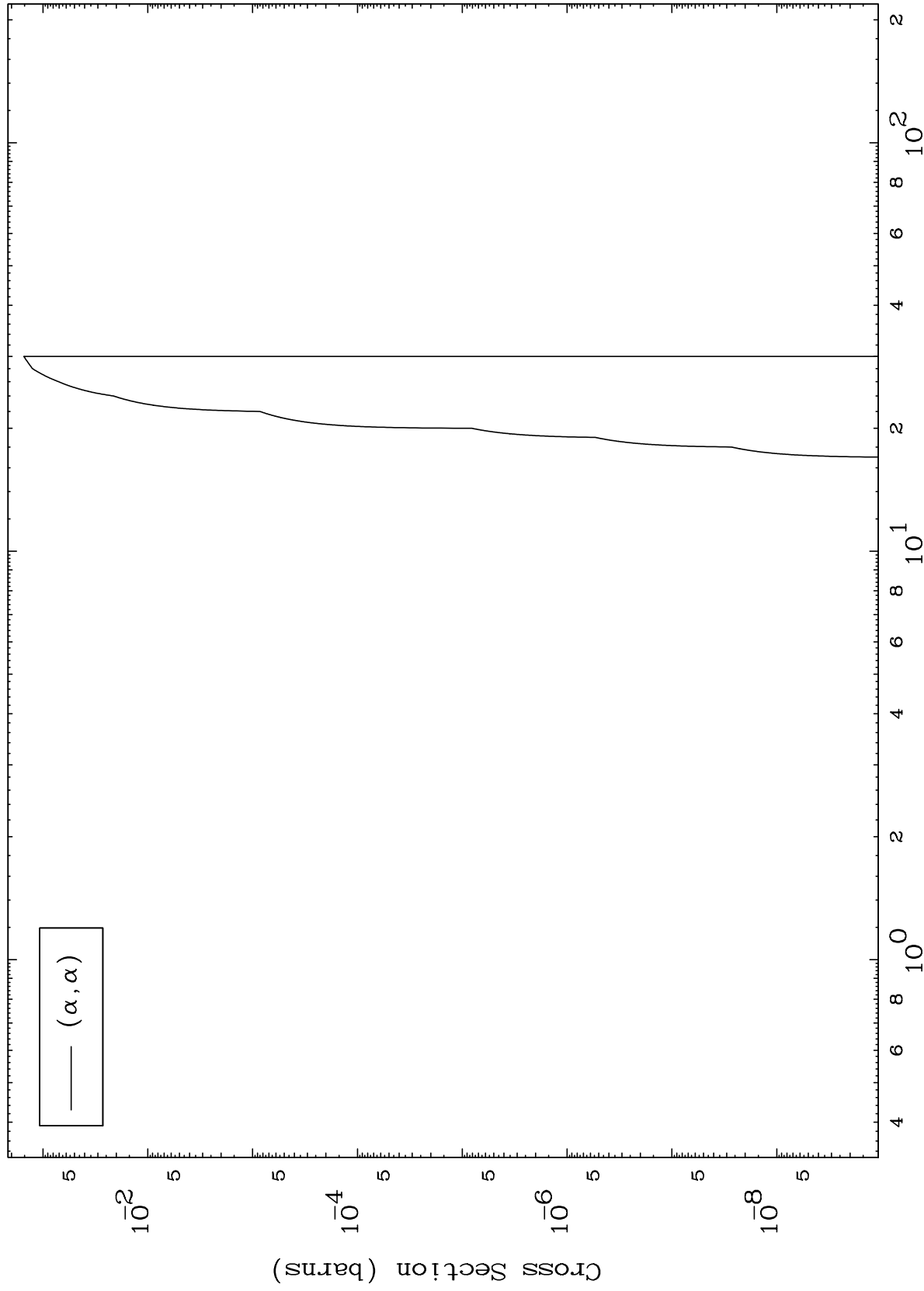
99-Es-249

Incident Energy (MeV)

MAT 9909

99-Es-249

(α, α) Levels
0 Kelvin Cross Sections



8

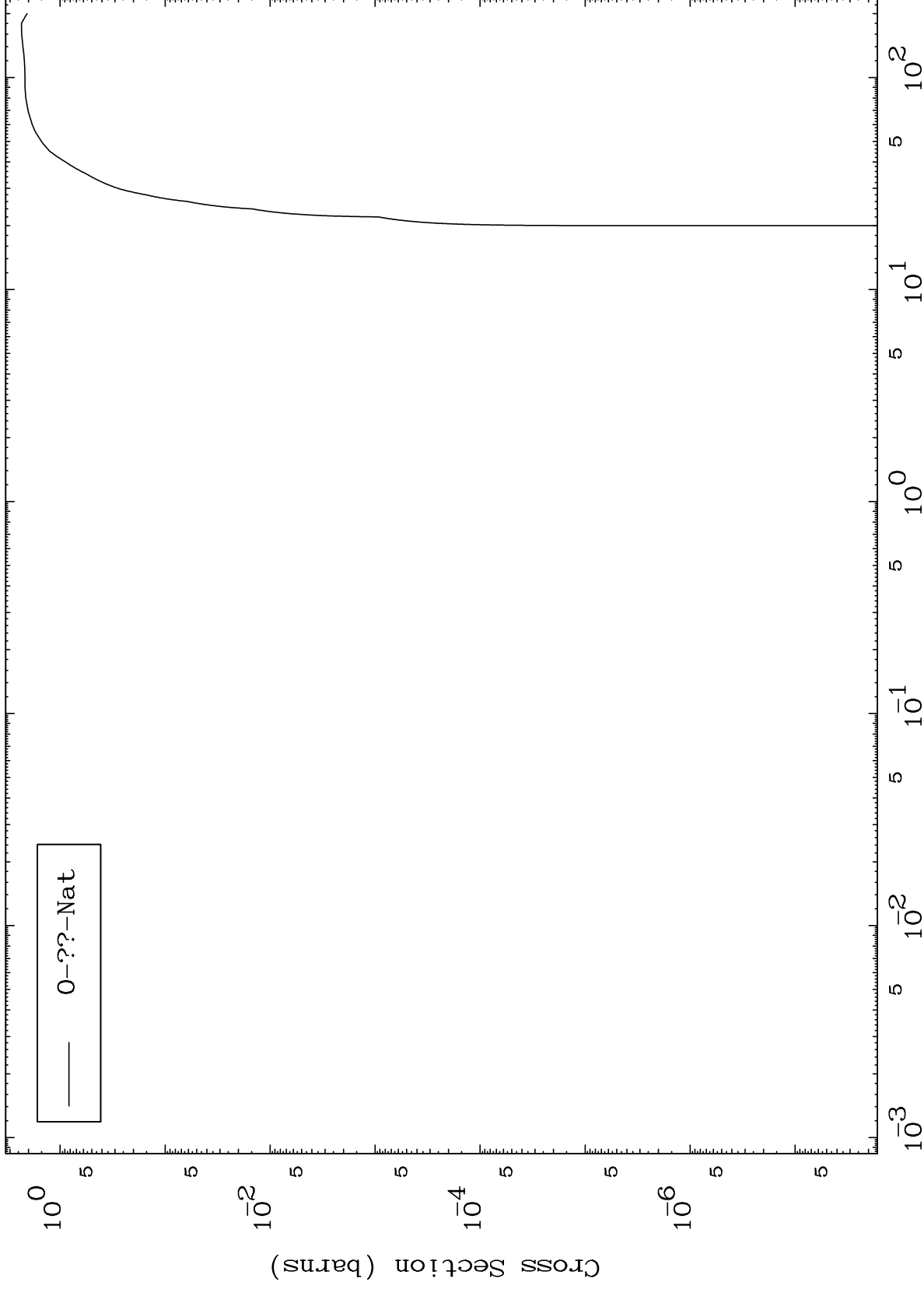
Incident Energy (MeV)

99-Es-249

MAT 9909

α Fission
Radionuclide Production Cross Section

99-Es-249



Incident Energy (MeV)

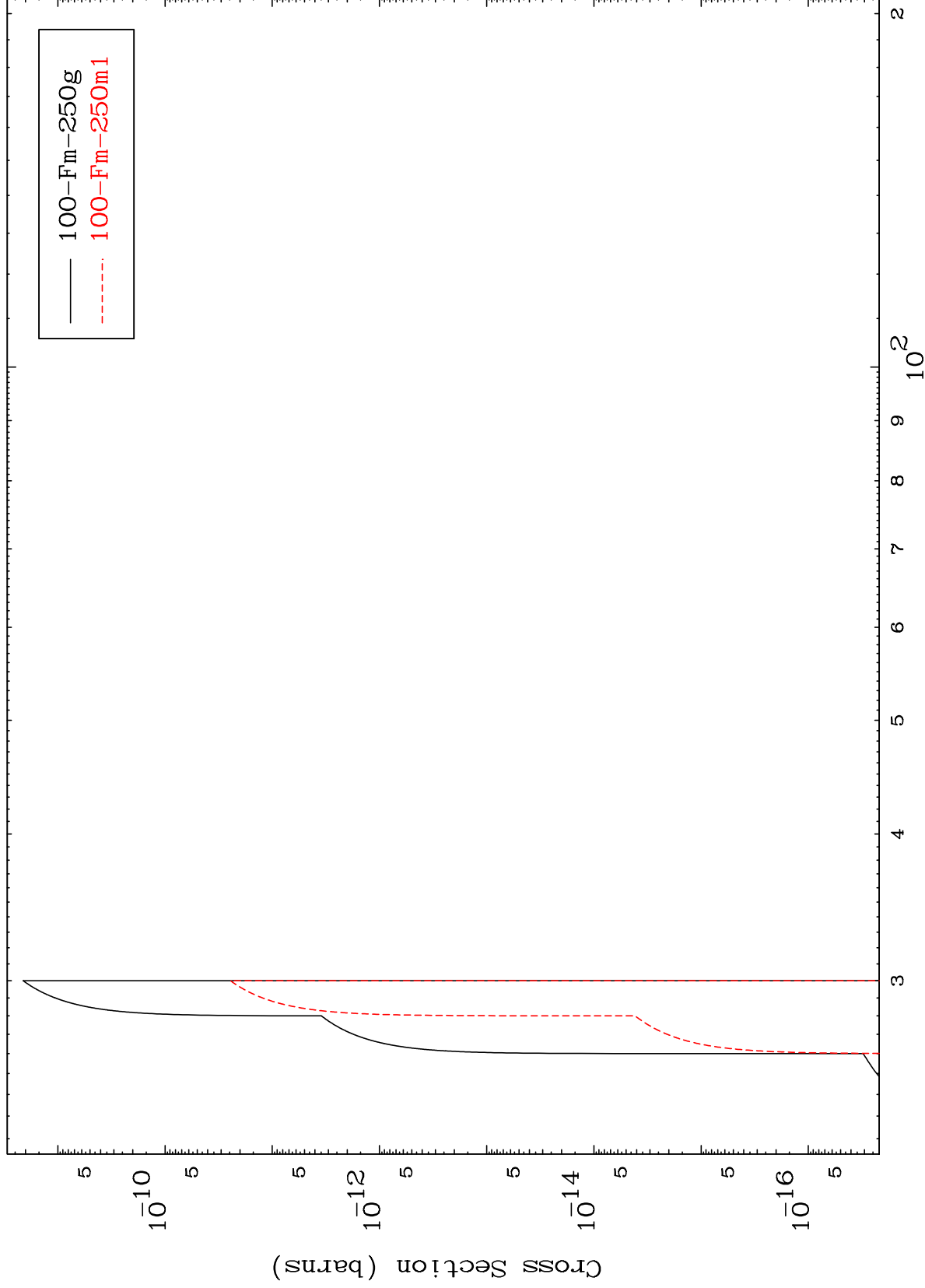
99-Es-249

MAT 9909

(α, n') d

99-Es-249

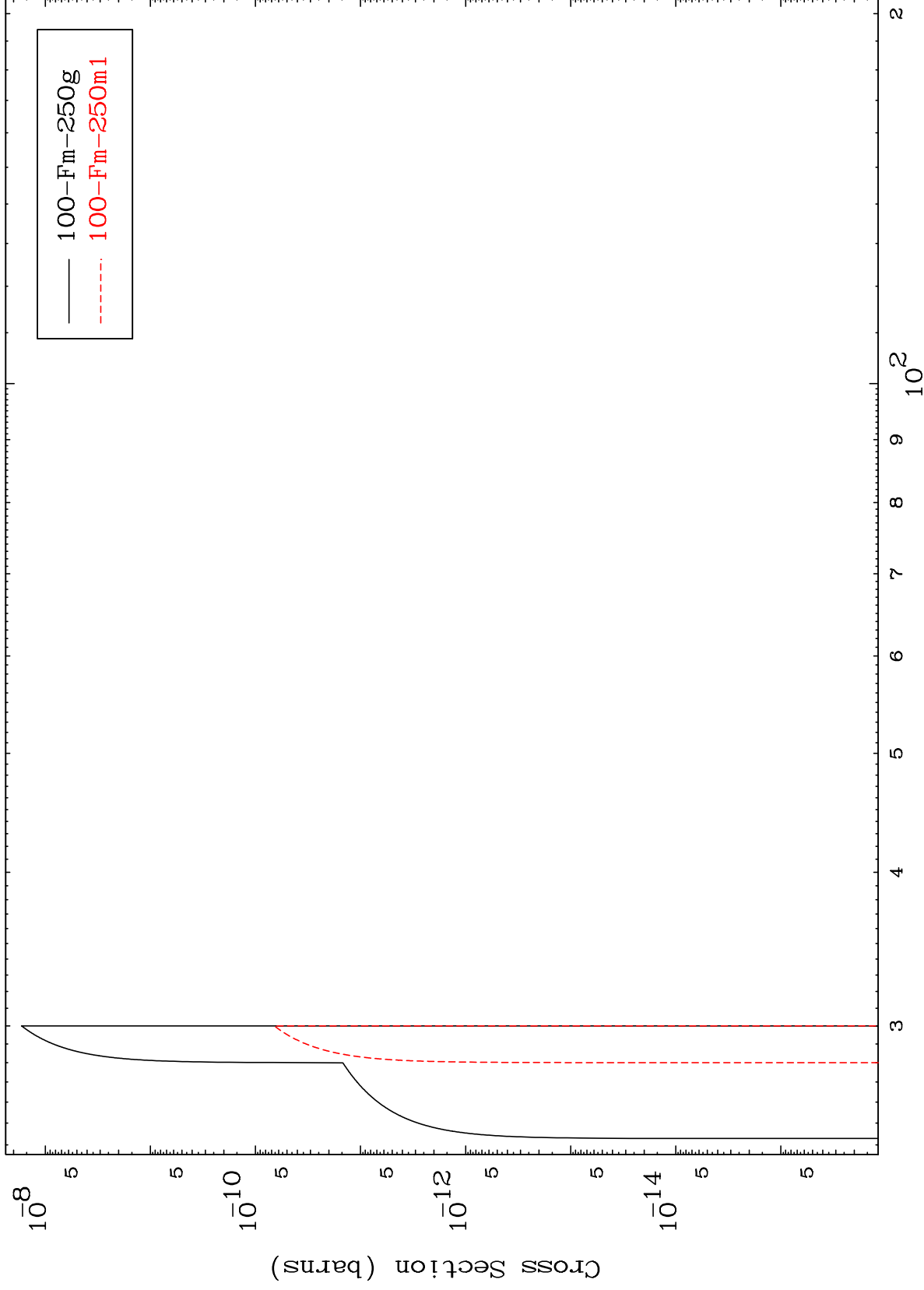
Radionuclide Production Cross Section



10

Incident Energy (MeV)

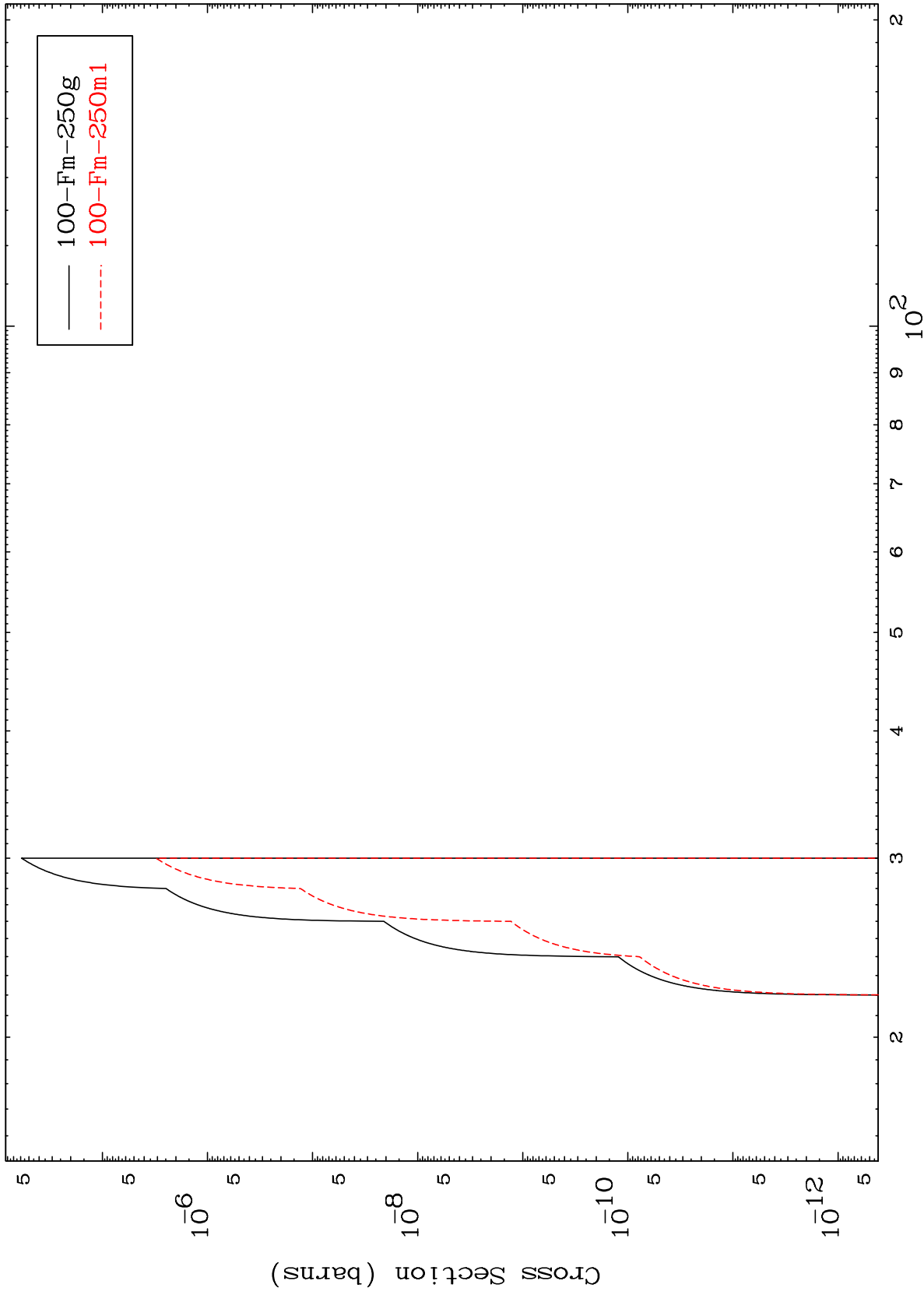
99-Es-249



MAT 9909

99-Es-249

(α, t)
Radionuclide Production Cross Section



12

Incident Energy (MeV)

99-Es-249