

Program EVALPLOT  
(Version 2018-1)

by

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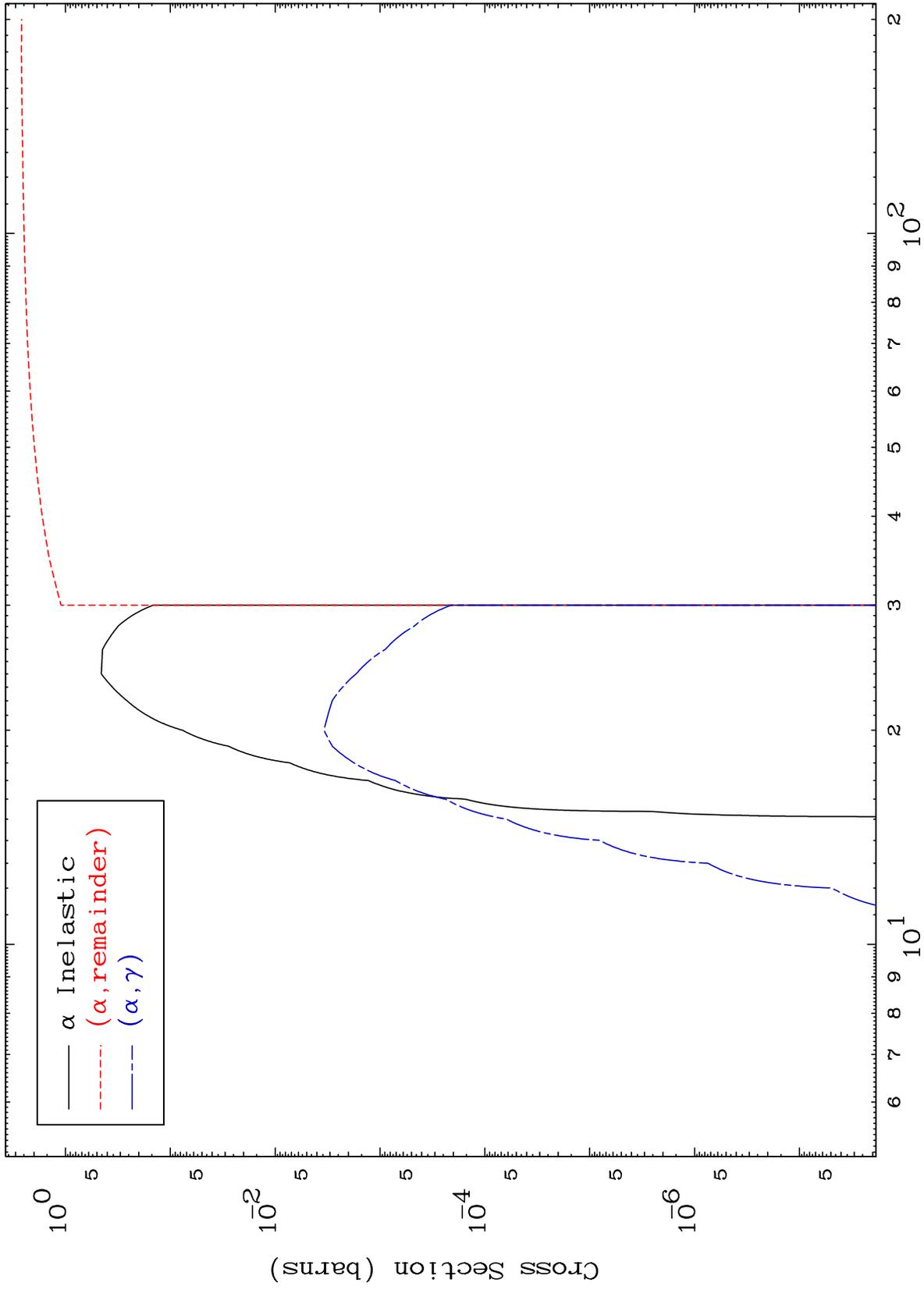
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 7695

0 Kelvin  $\alpha$  Major  
Cross Sections

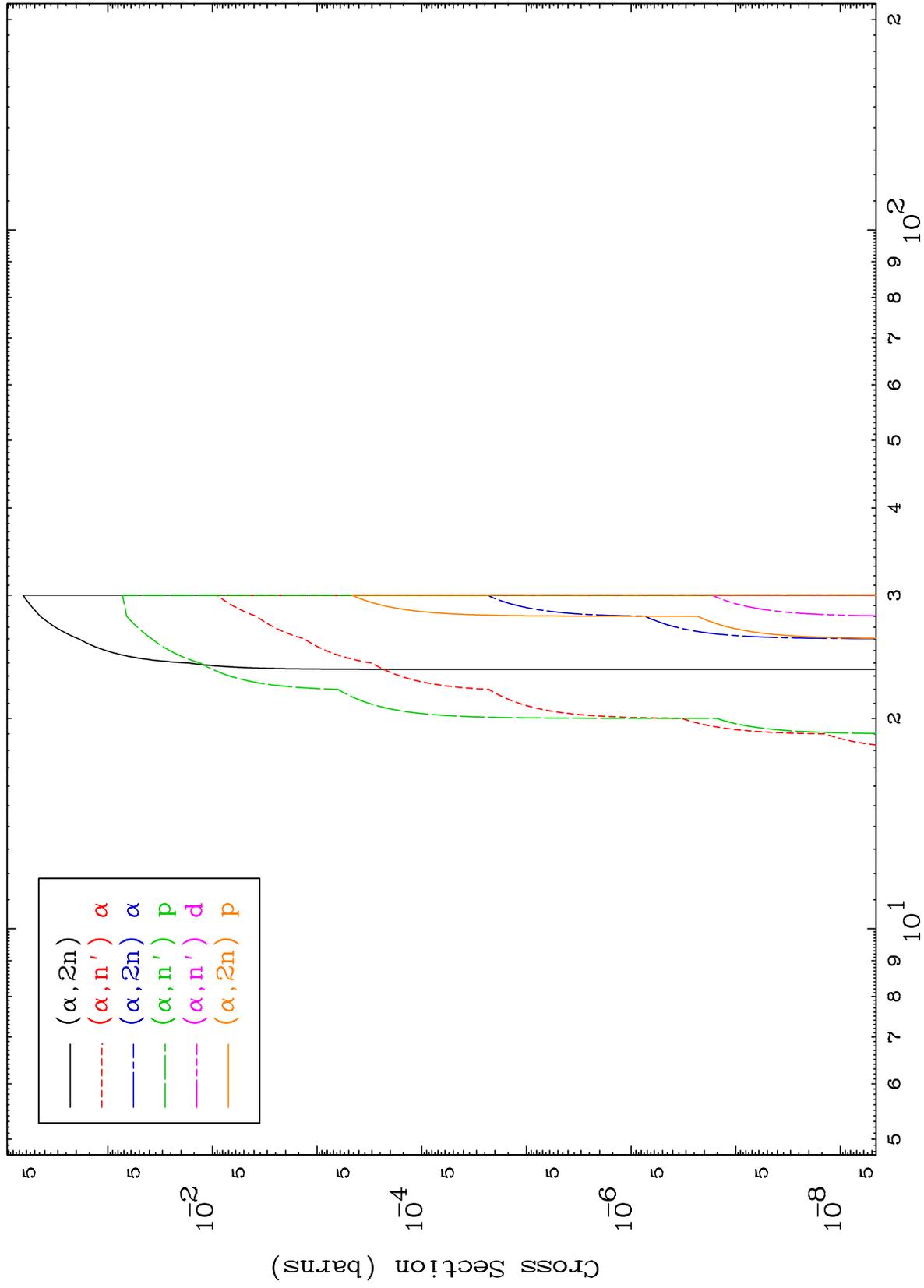
77-Ir-181



MAT 7695

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

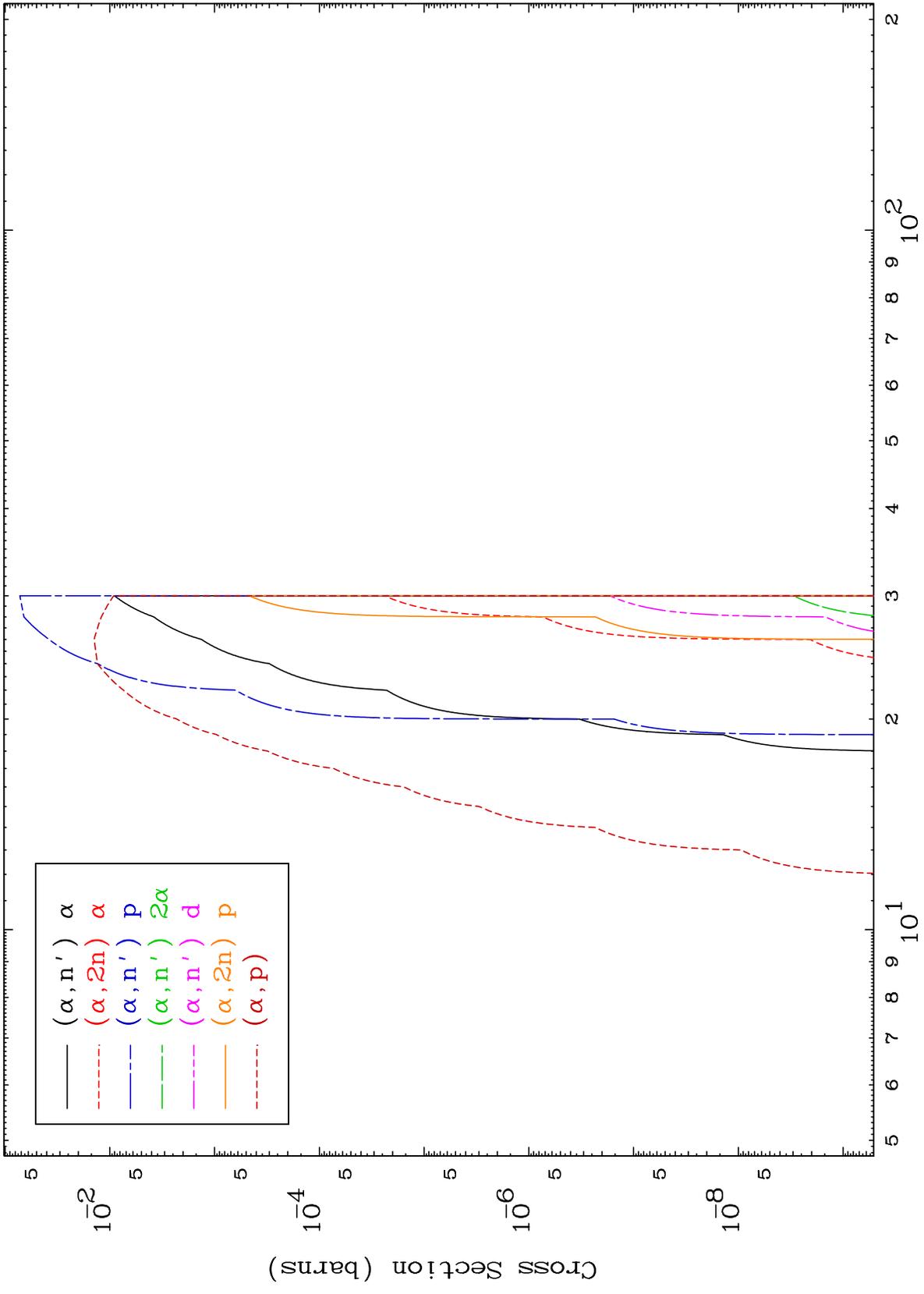
77-Ir-181



2

Incident Energy (MeV)

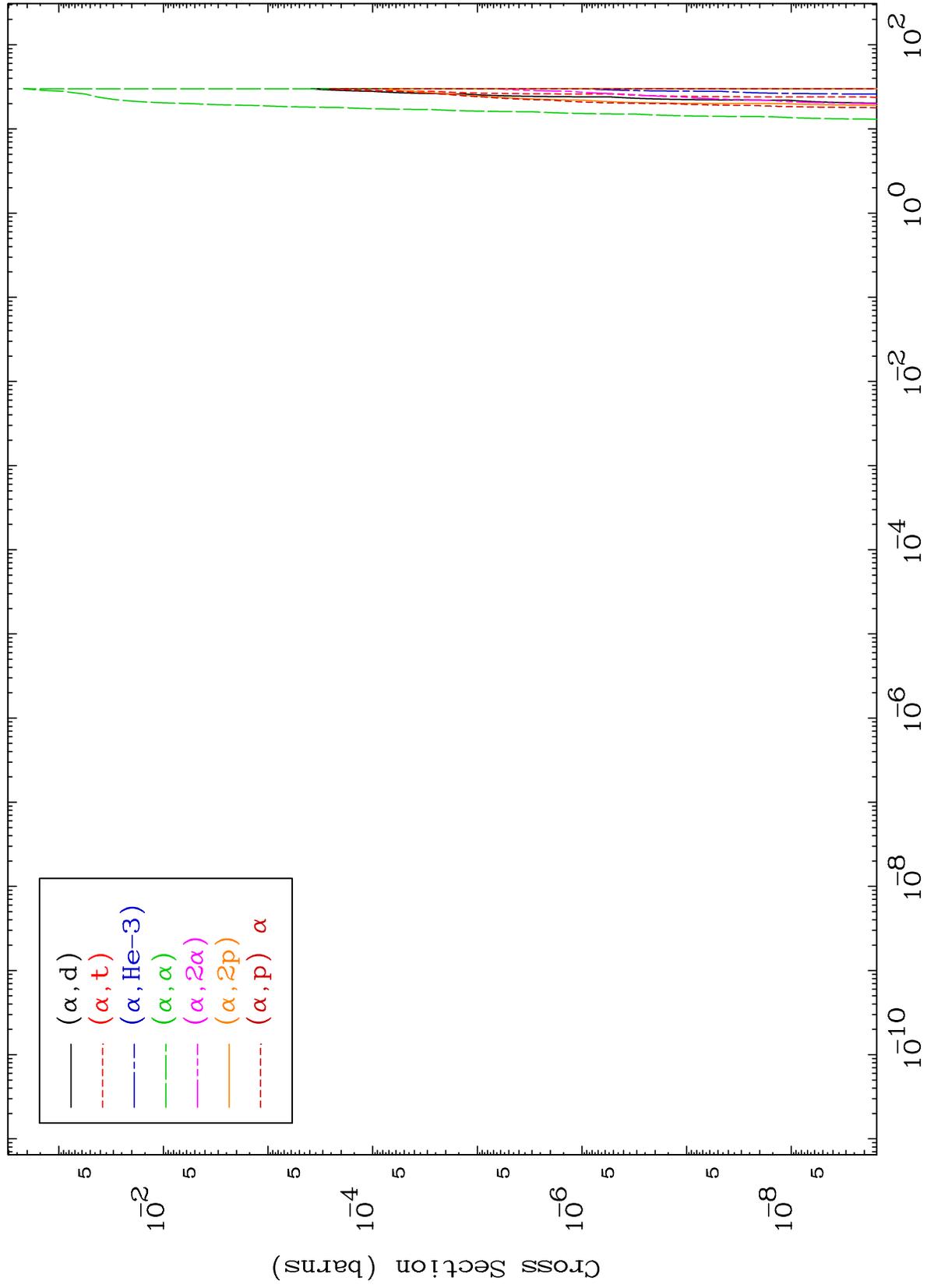
77-Ir-181



MAT 7695

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

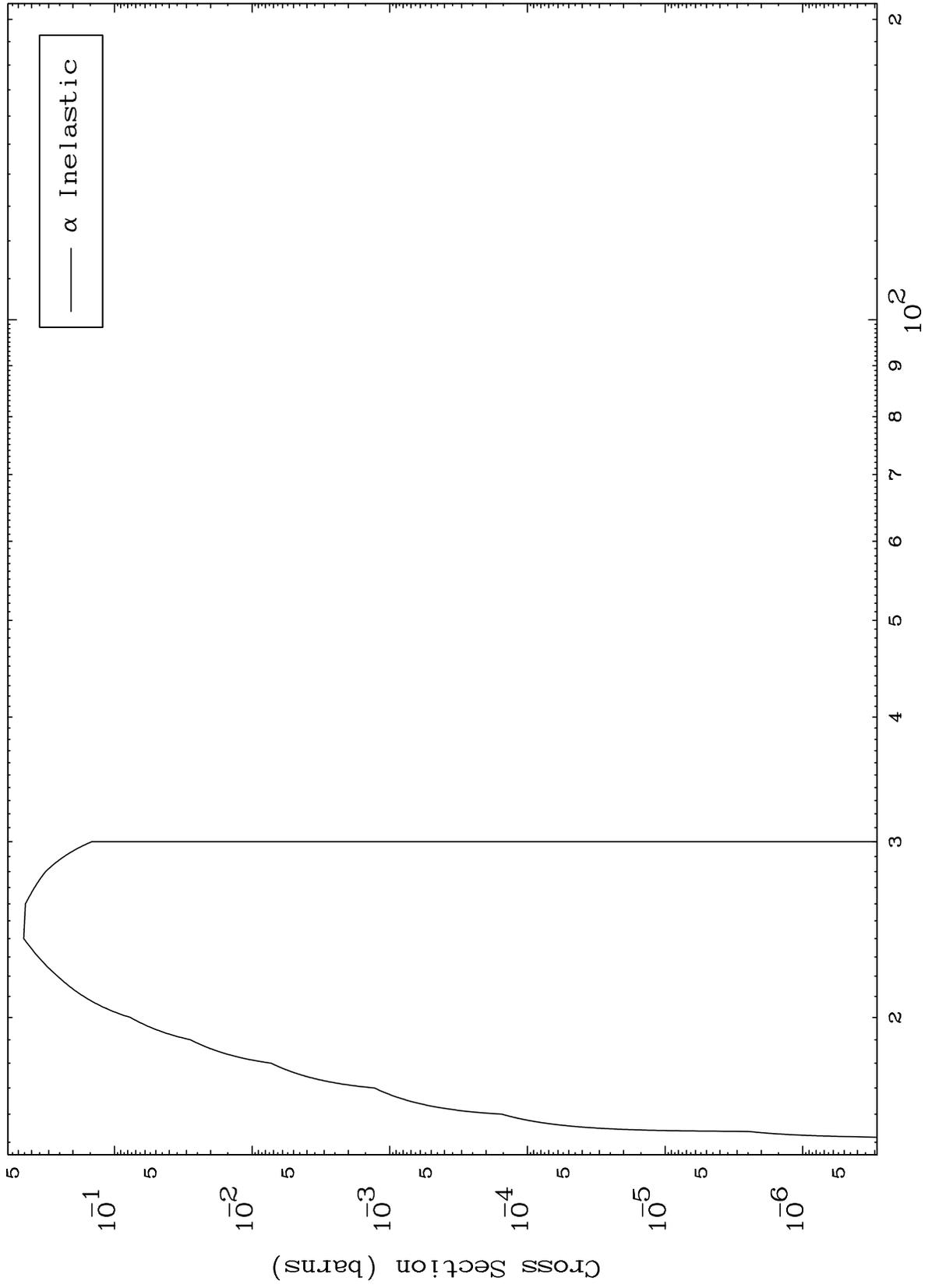
77-Ir-181



MAT 7695

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

77-Ir-181



5

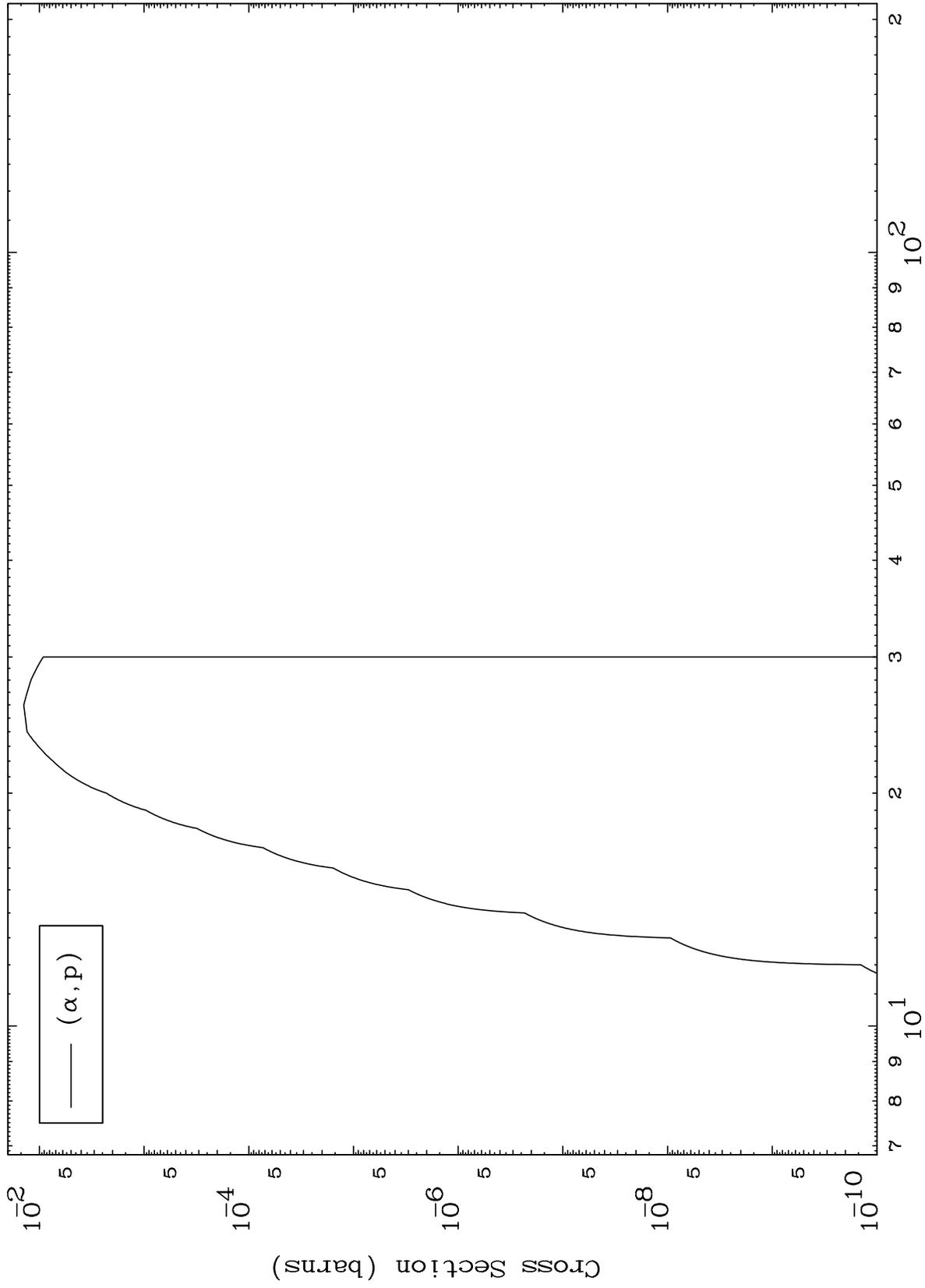
Incident Energy (MeV)

77-Ir-181

MAT 7695

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

77-Ir-181



6

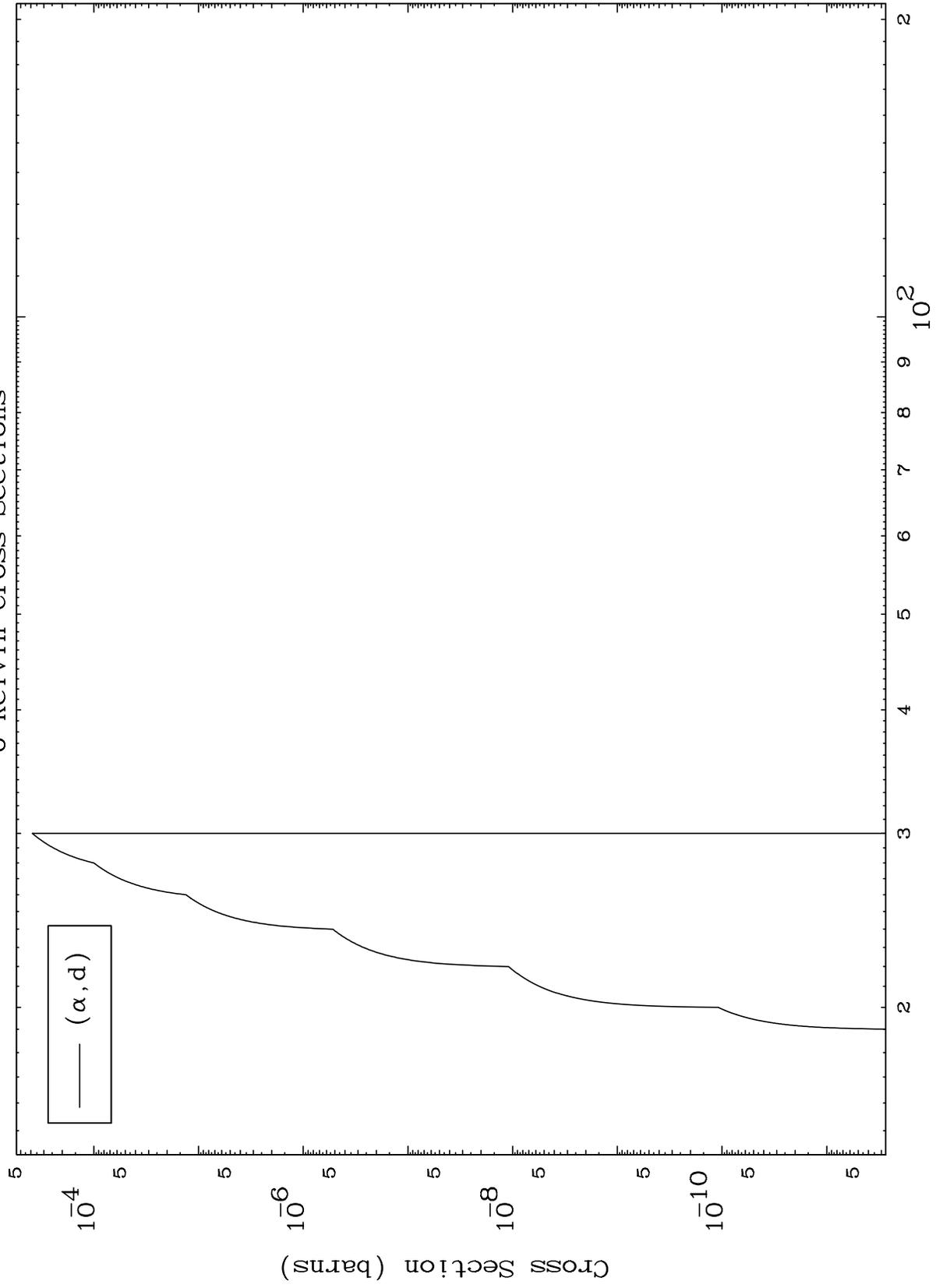
Incident Energy (MeV)

77-Ir-181

MAT 7695

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

<sup>77</sup>Ir-181



7

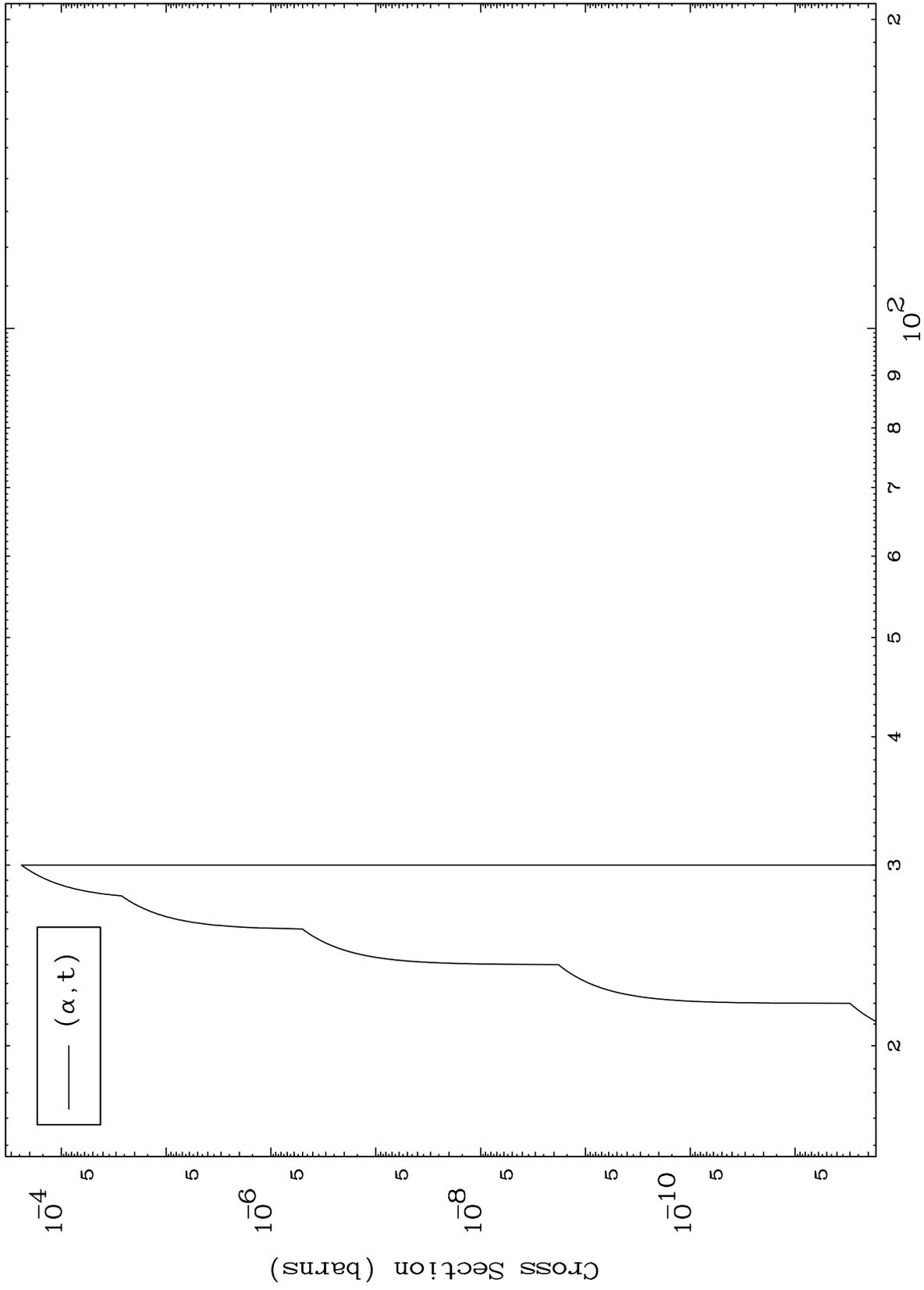
Incident Energy (MeV)

<sup>77</sup>Ir-181

MAT 7695

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

77-Ir-181



8

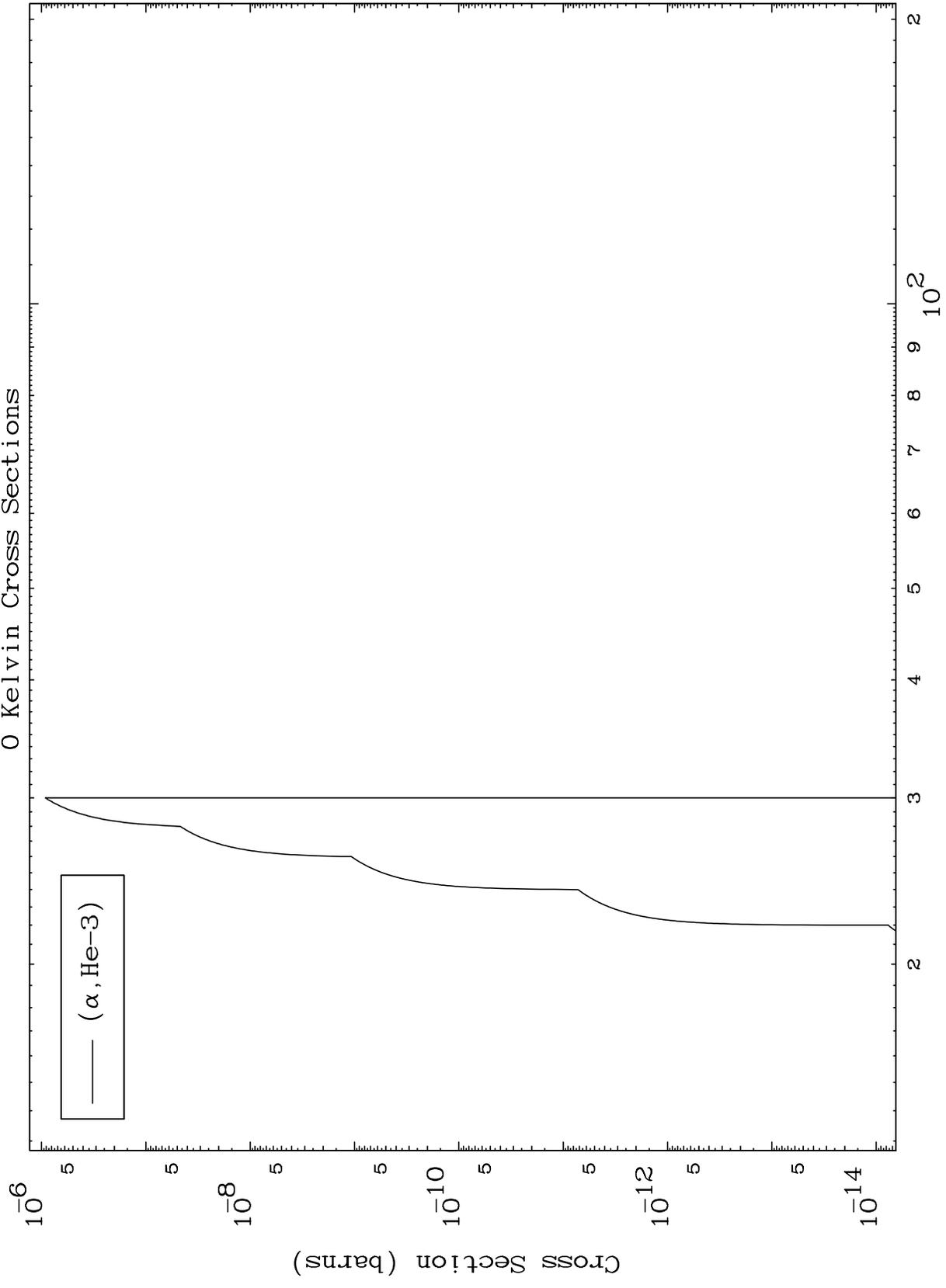
Incident Energy (MeV)

77-Ir-181

MAT 7695

( $\alpha$ , He3) Levels  
0 Kelvin Cross Sections

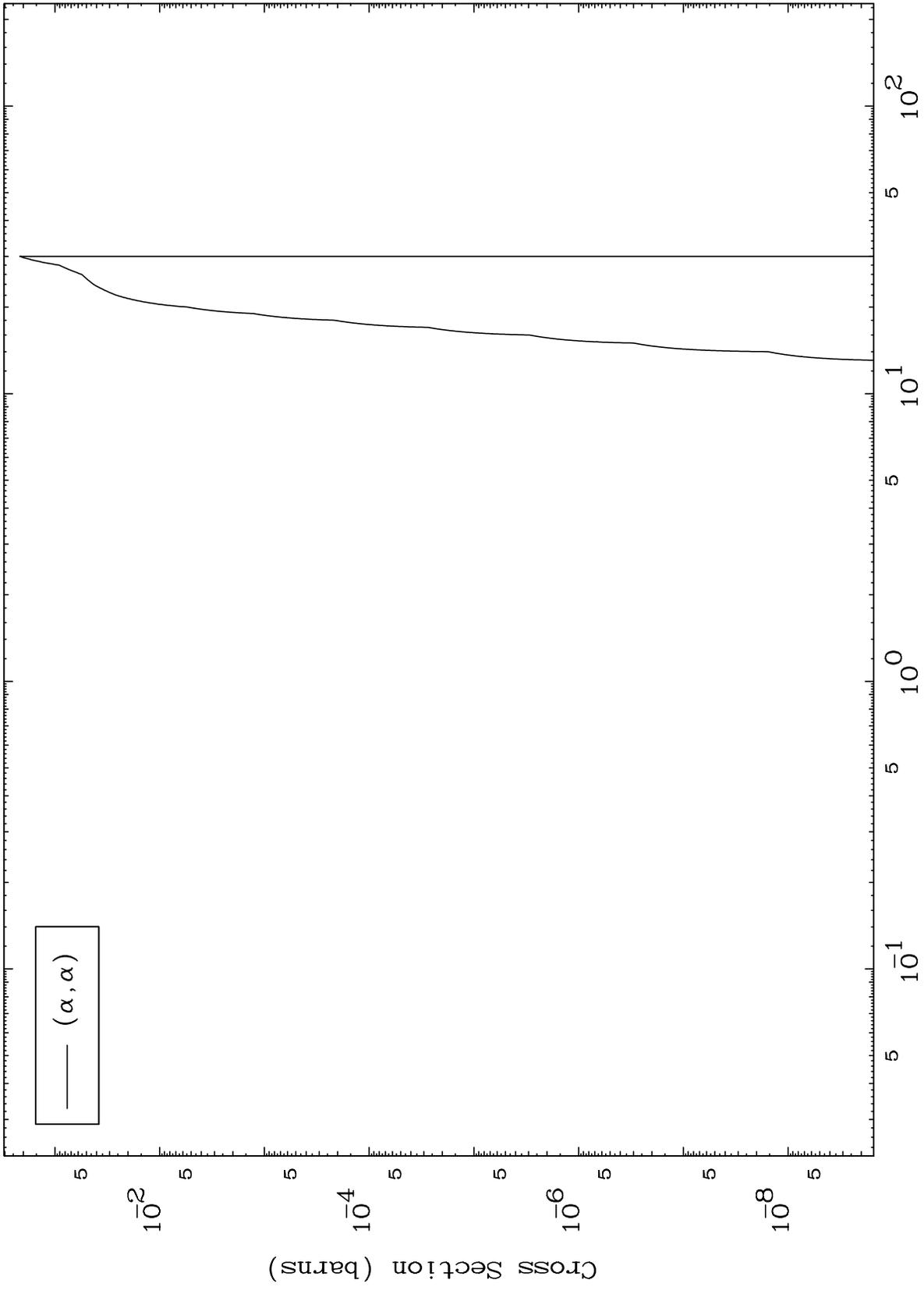
77-Ir-181



MAT 7695

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

77-Ir-181



10

Incident Energy (MeV)

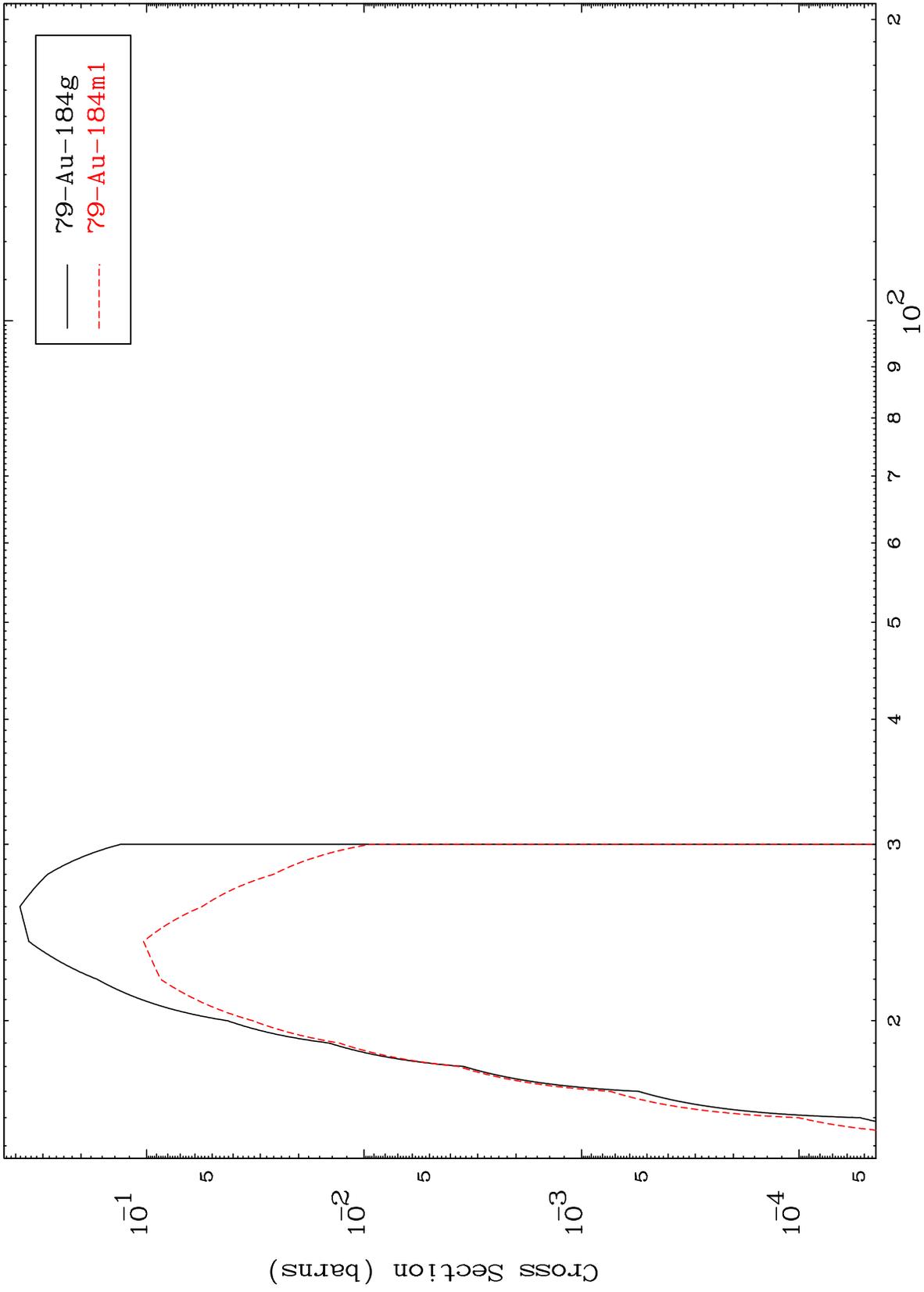
77-Ir-181

MAT 7695

$\alpha$  Inelastic

<sup>77</sup>Ir-181

Radionuclide Production Cross Section



11

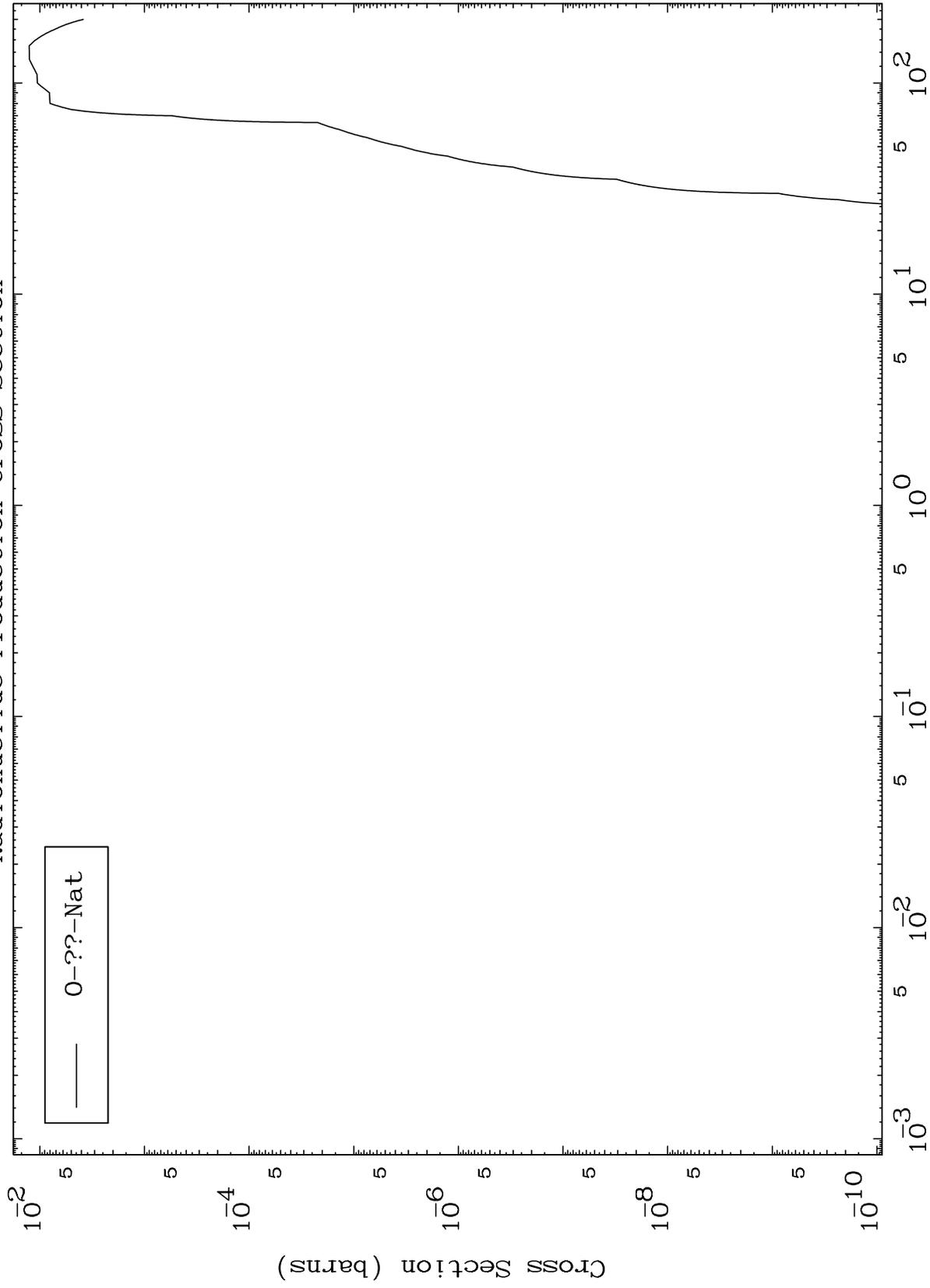
Incident Energy (MeV)

<sup>77</sup>Ir-181

MAT 7695

$\alpha$  Fission  
Radionuclide Production Cross Section

<sup>77</sup>Ir-181



12

Incident Energy (MeV)

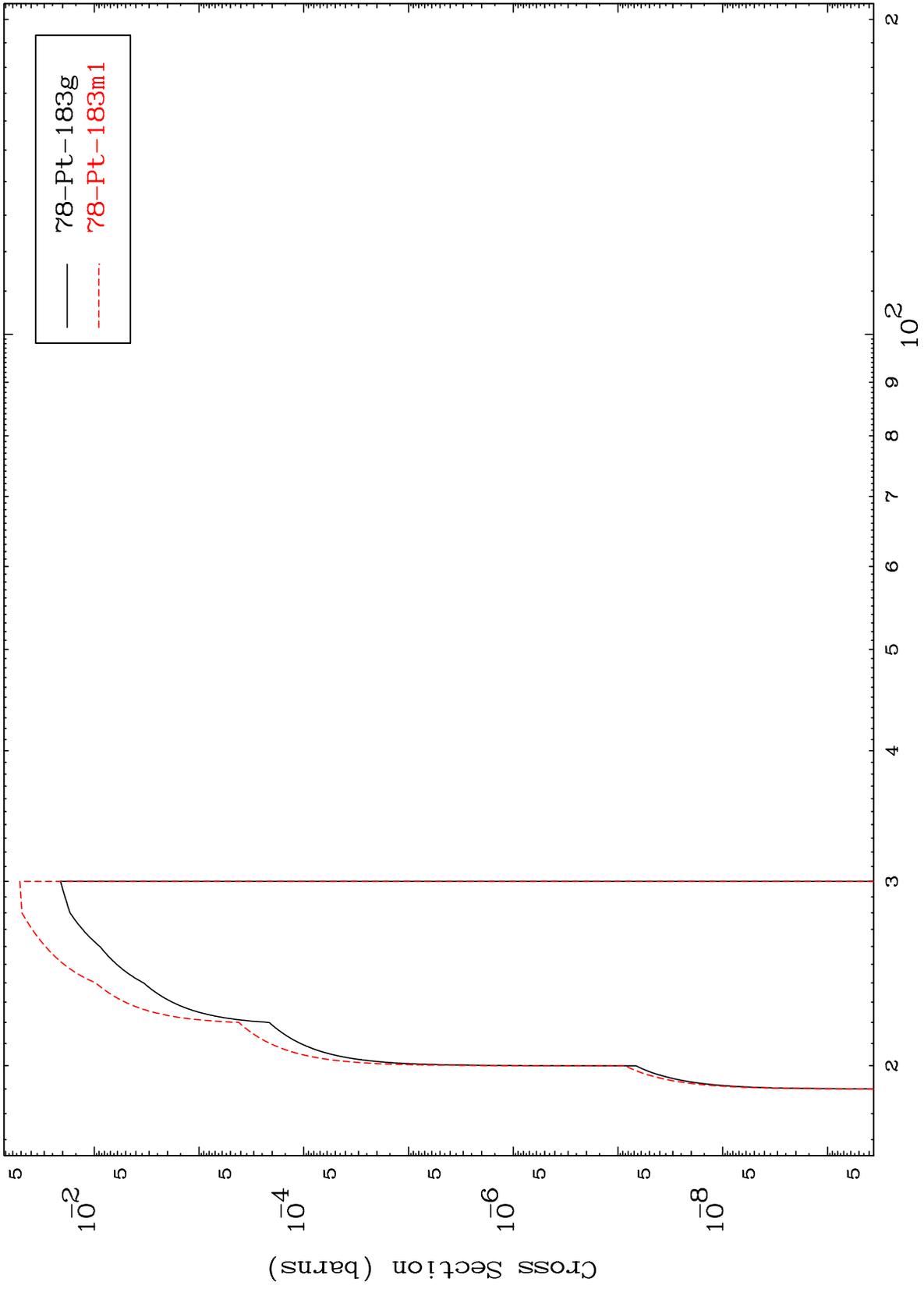
<sup>77</sup>Ir-181

MAT 7695

$(\alpha, n')$  p

<sup>77</sup>Ir-181

Radionuclide Production Cross Section



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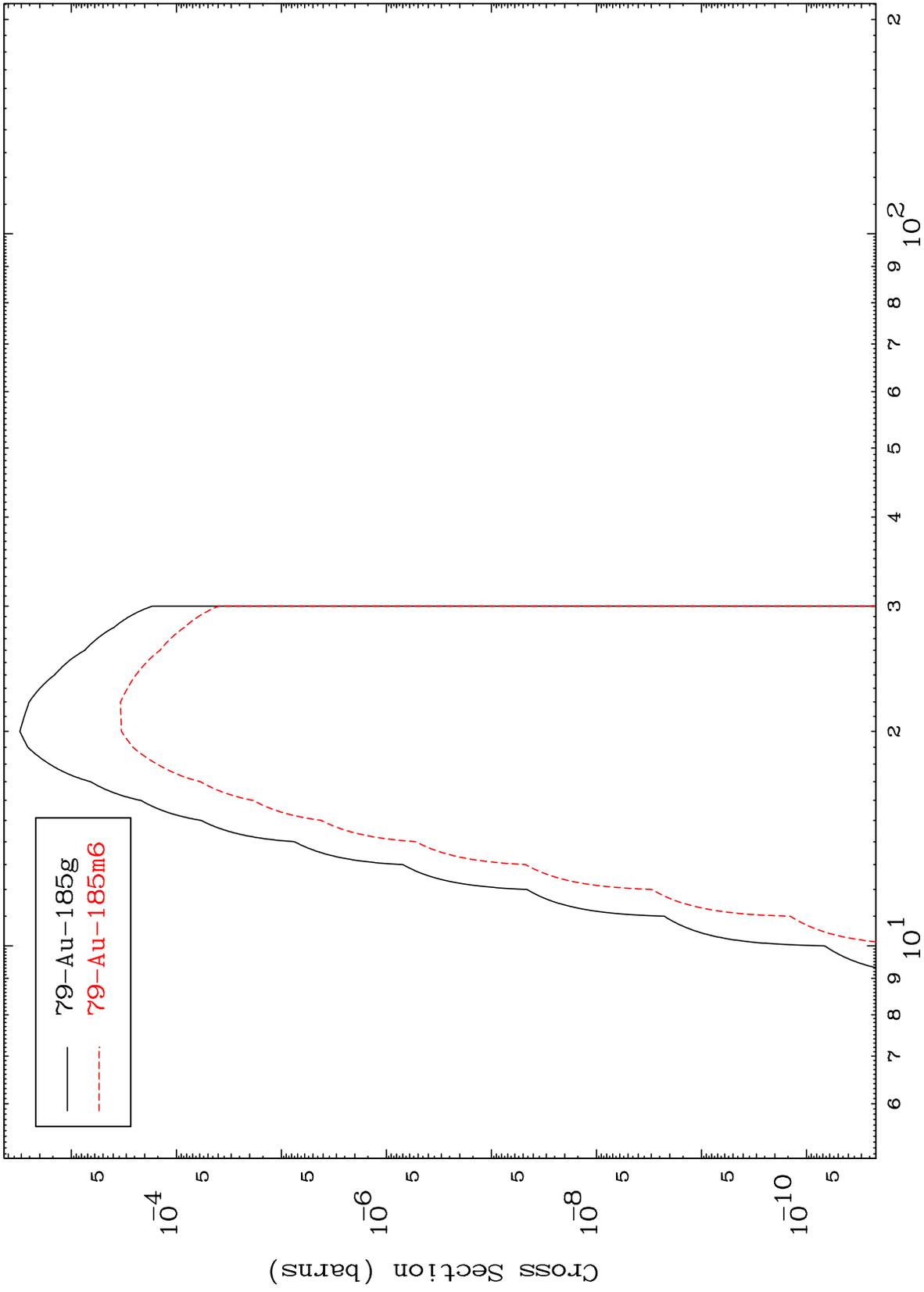
Incident Energy (MeV)

<sup>77</sup>Ir-181

MAT 7695

<sup>77</sup>Ir-181

( $\alpha, \gamma$ )  
Radionuclide Production Cross Section



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<sup>77</sup>Ir-181

