

Program EVALPLOT  
(Version 2018-1)

by

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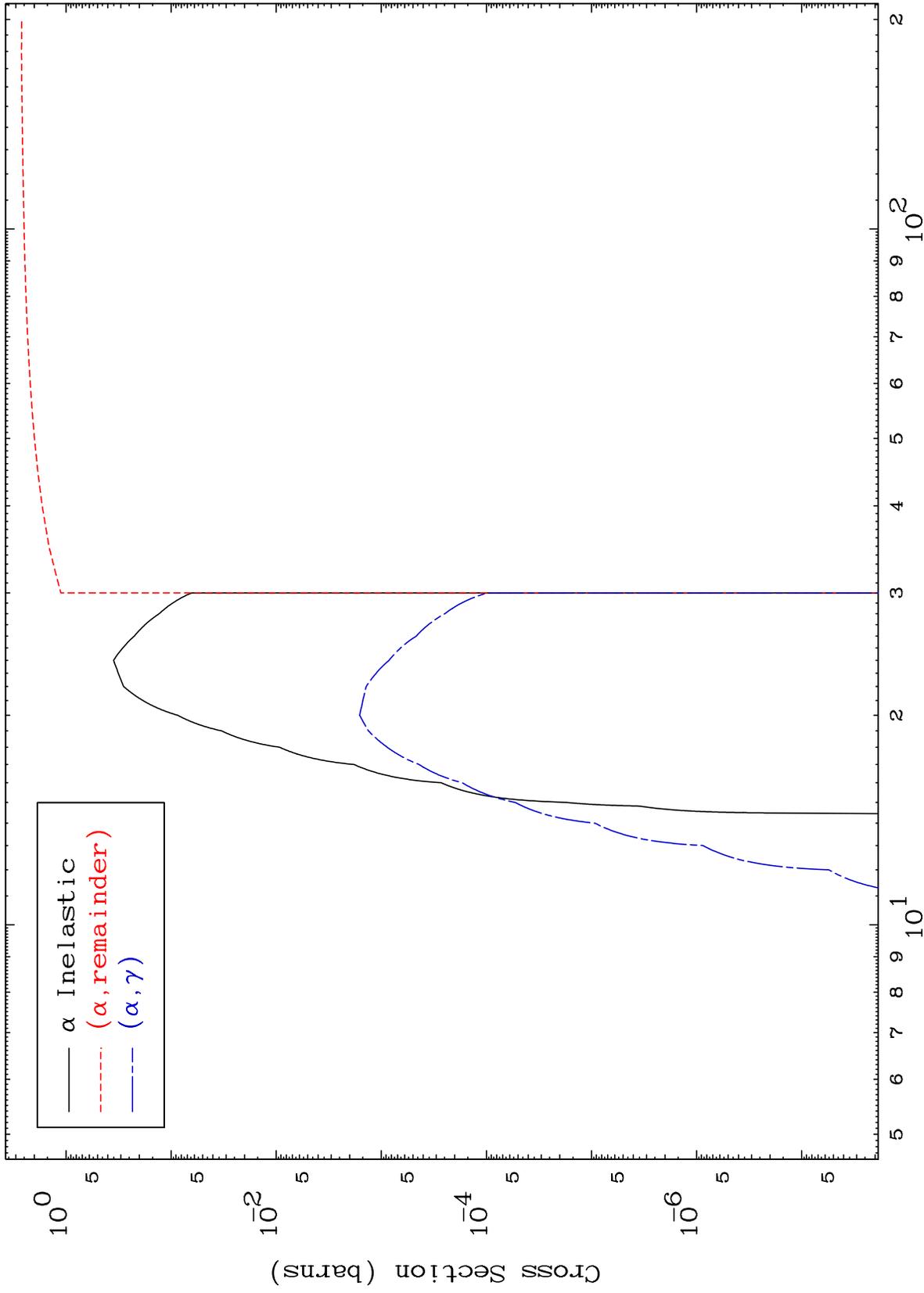
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 7701

0 Kelvin  $\alpha$  Major  
Cross Sections

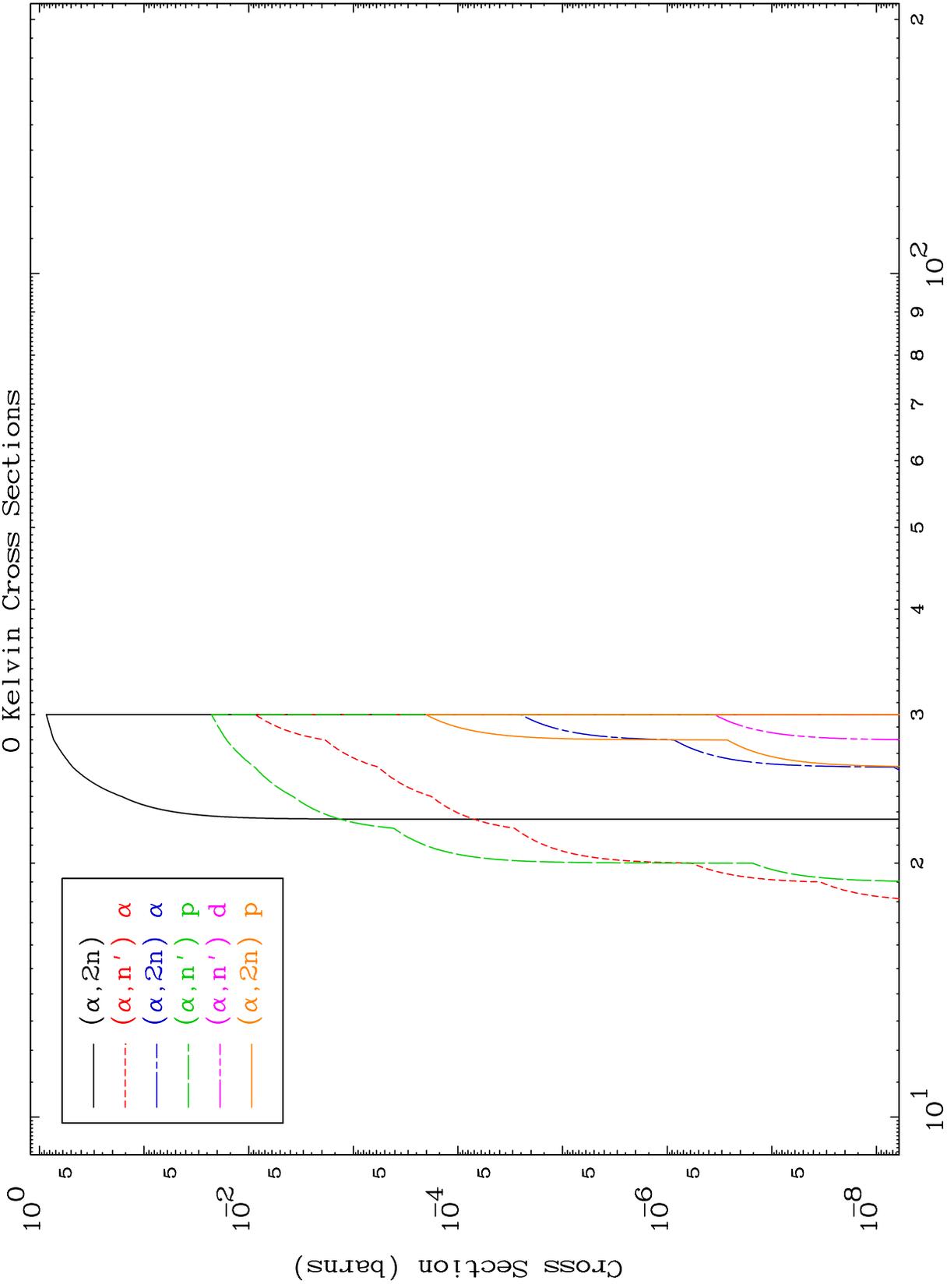
77-Ir-183



MAT 7701

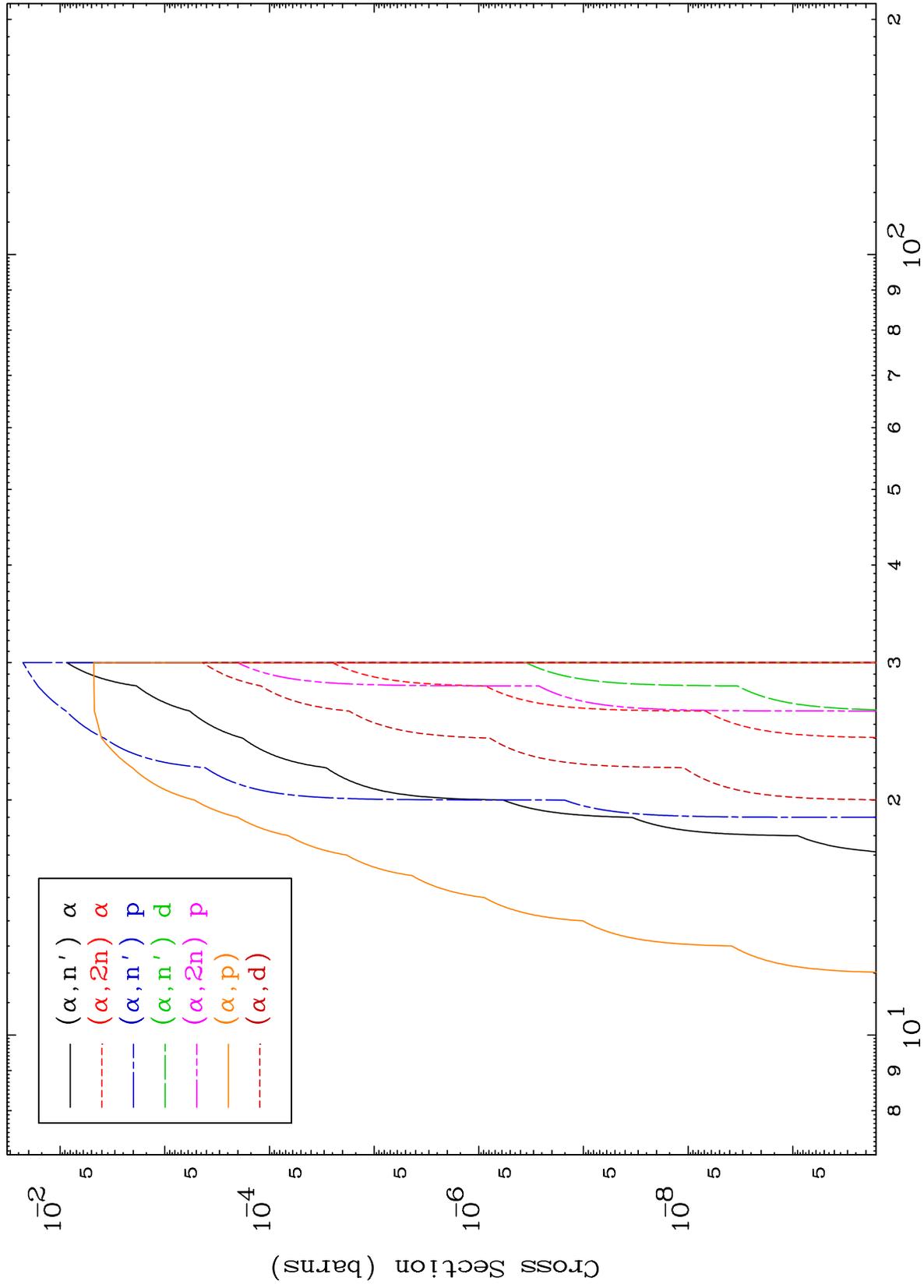
$\alpha$  Neutron Production  
0 Kelvin Cross Sections

77-Ir-183



Incident Energy (MeV)

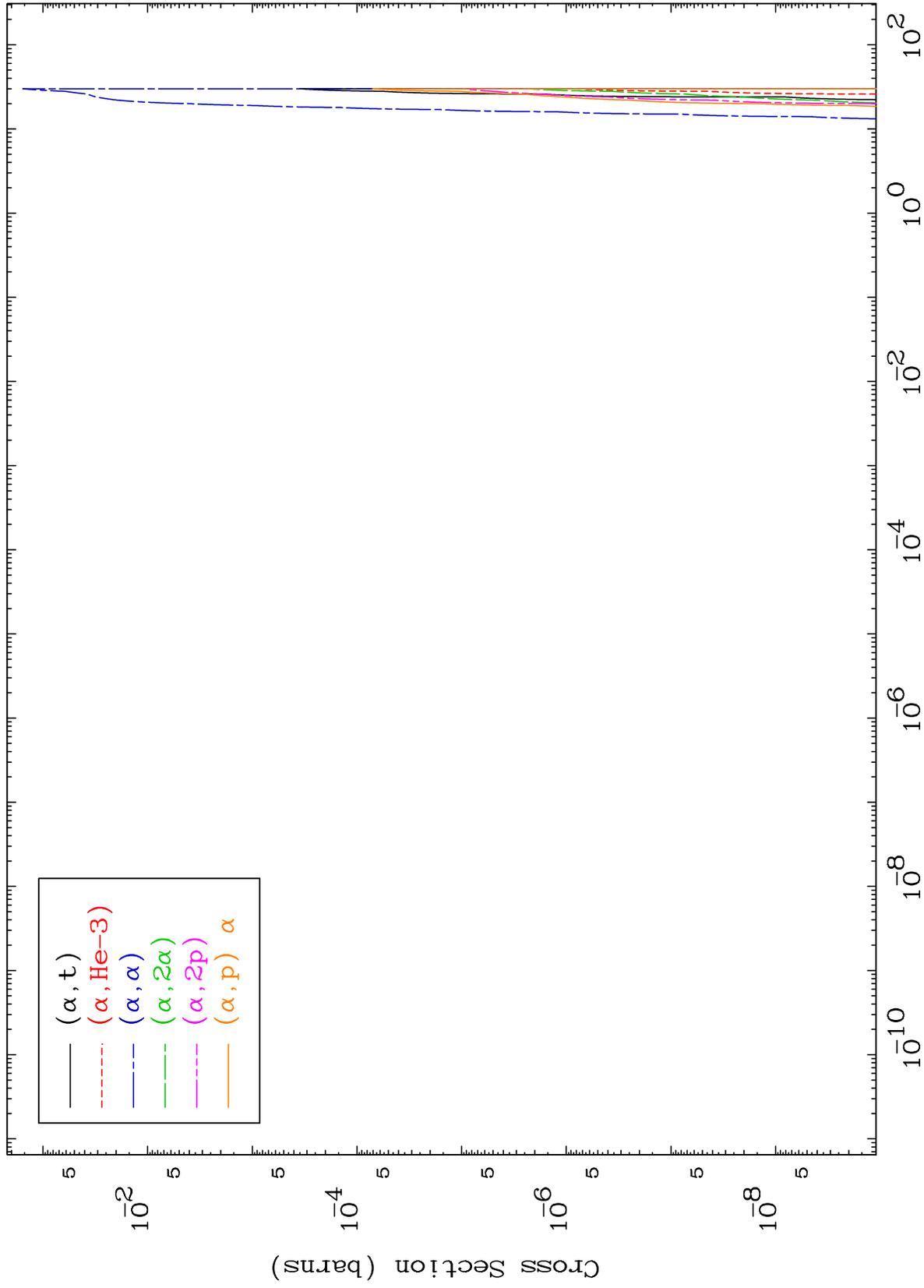
77-Ir-183

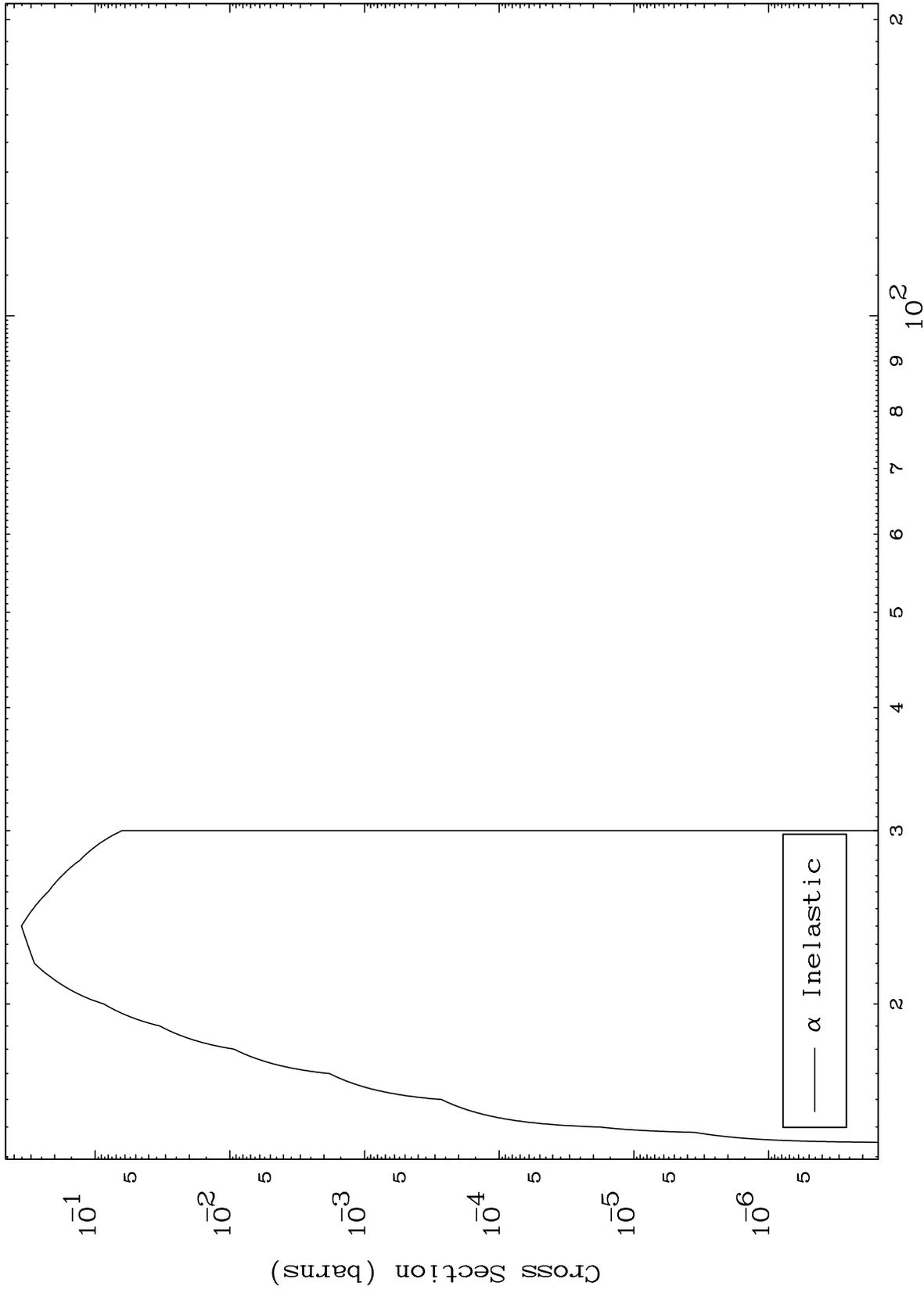


MAT 7701

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

77-Ir-183

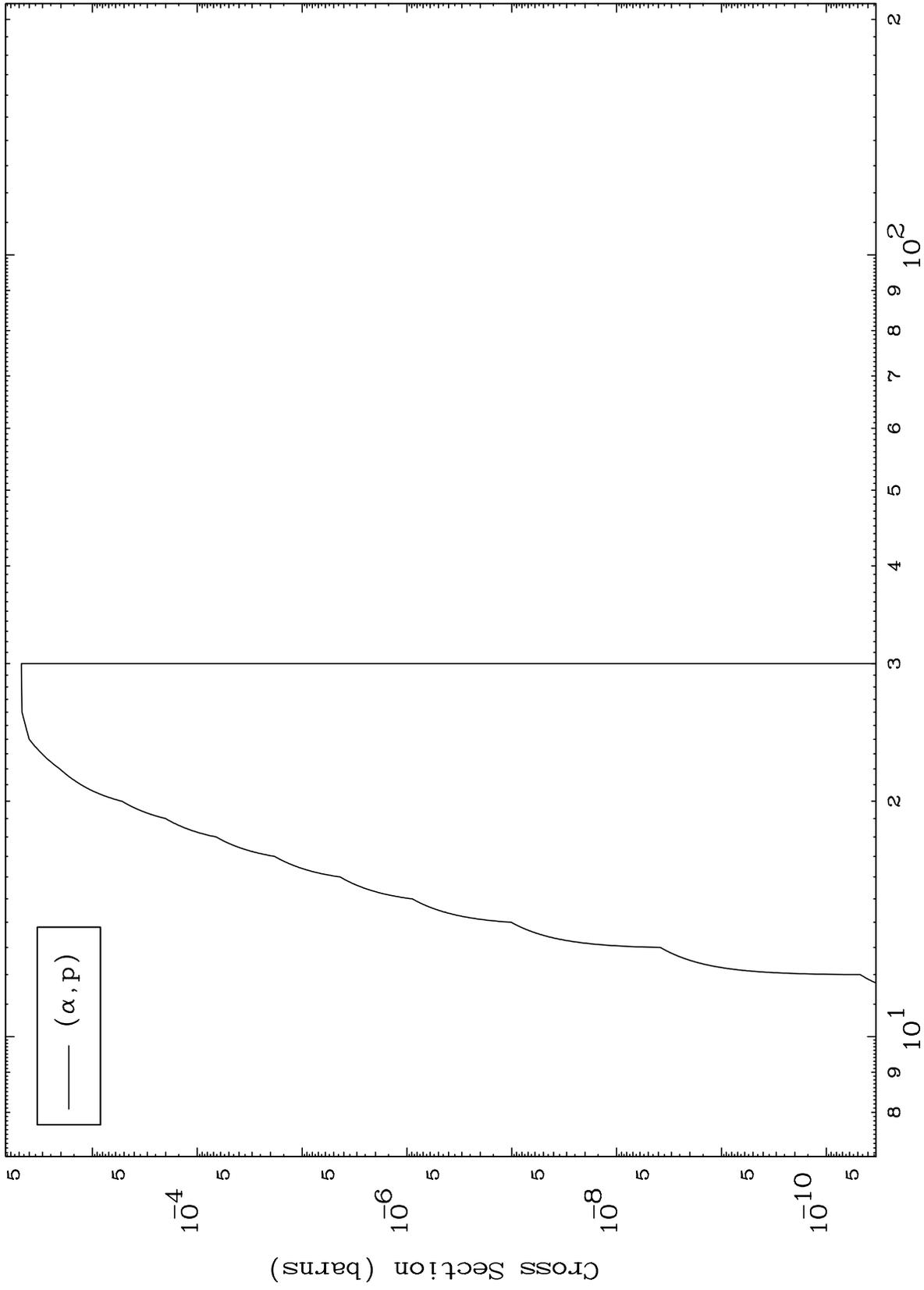




MAT 7701

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

77-Ir-183



6

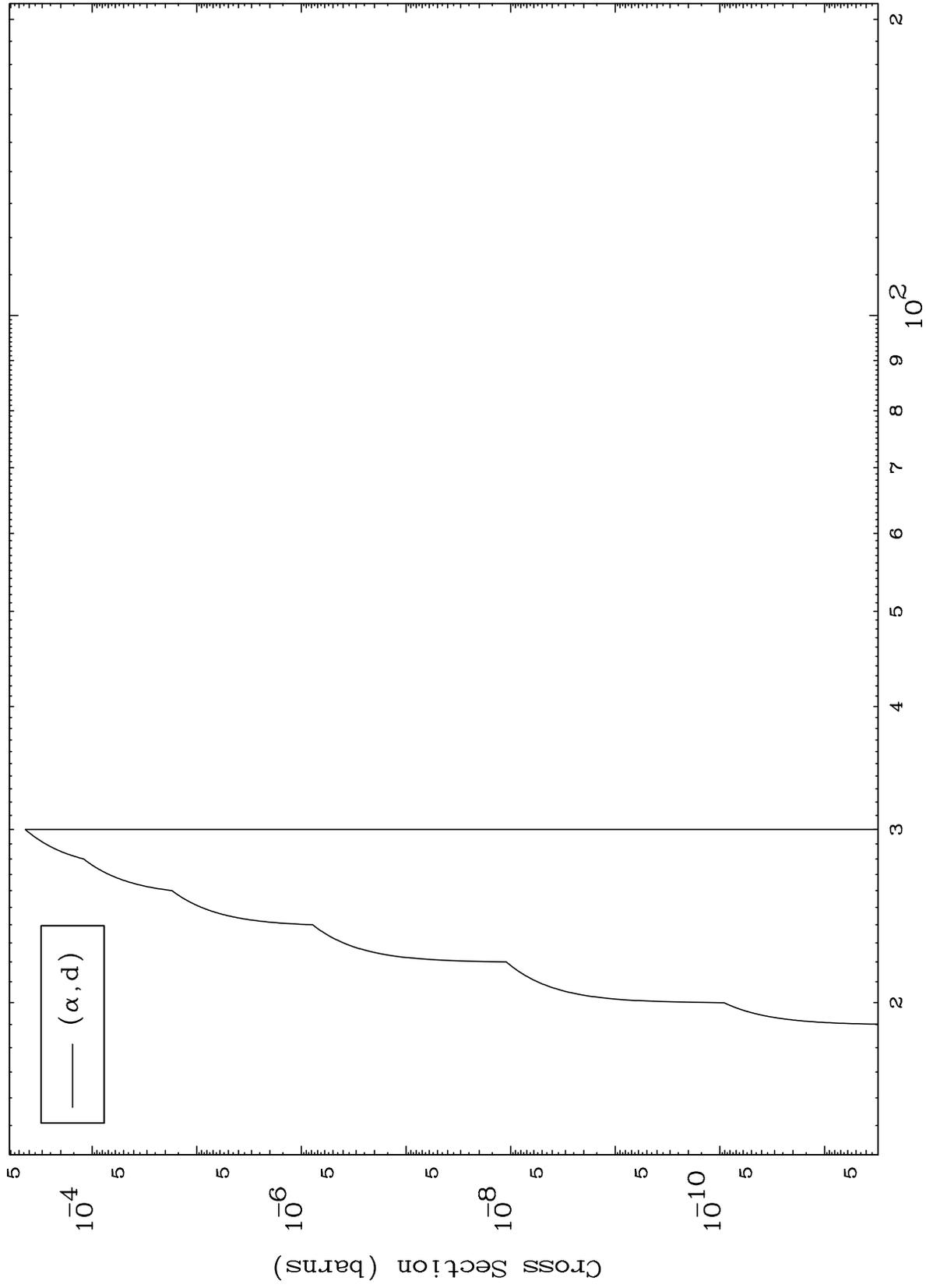
Incident Energy (MeV)

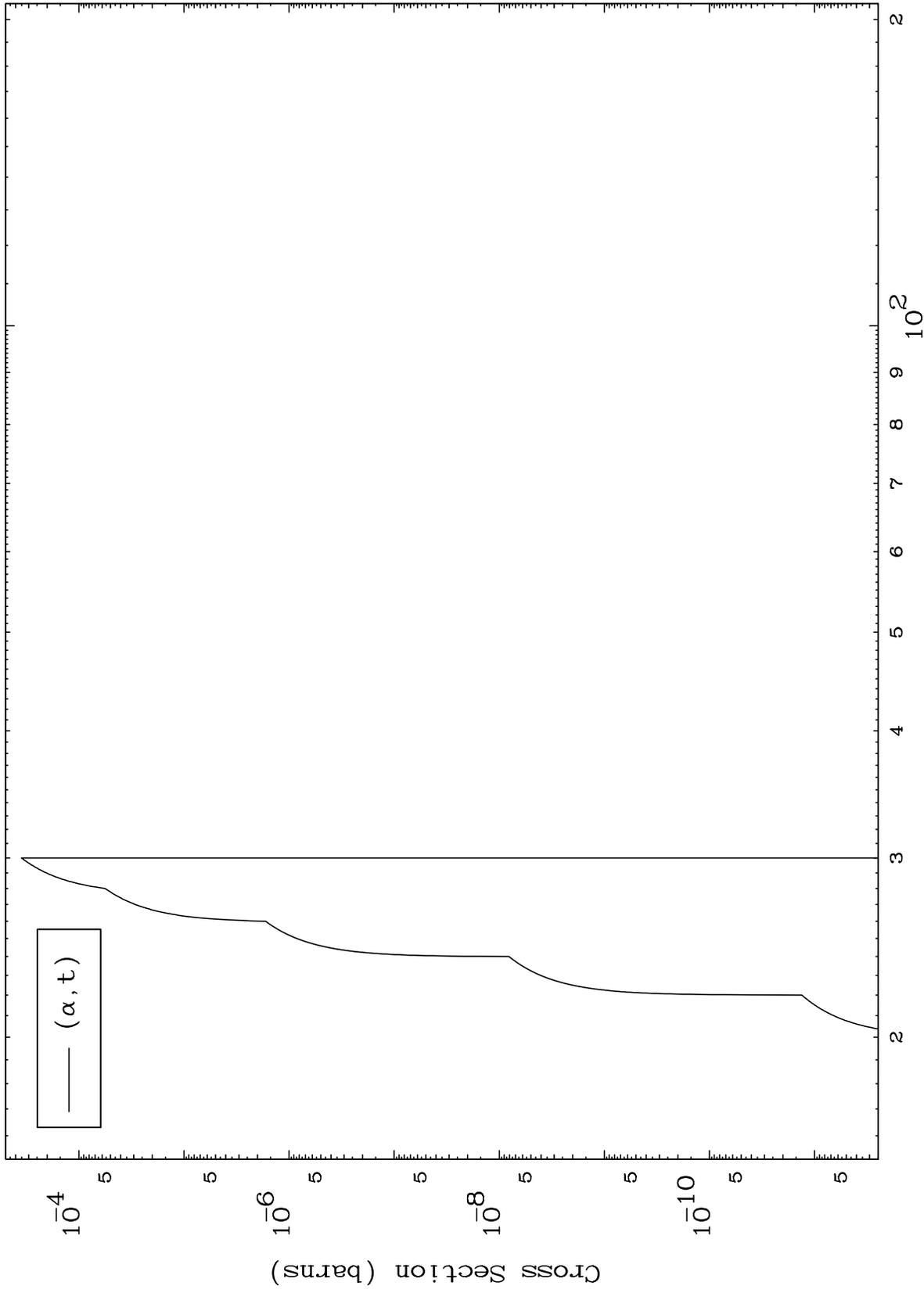
77-Ir-183

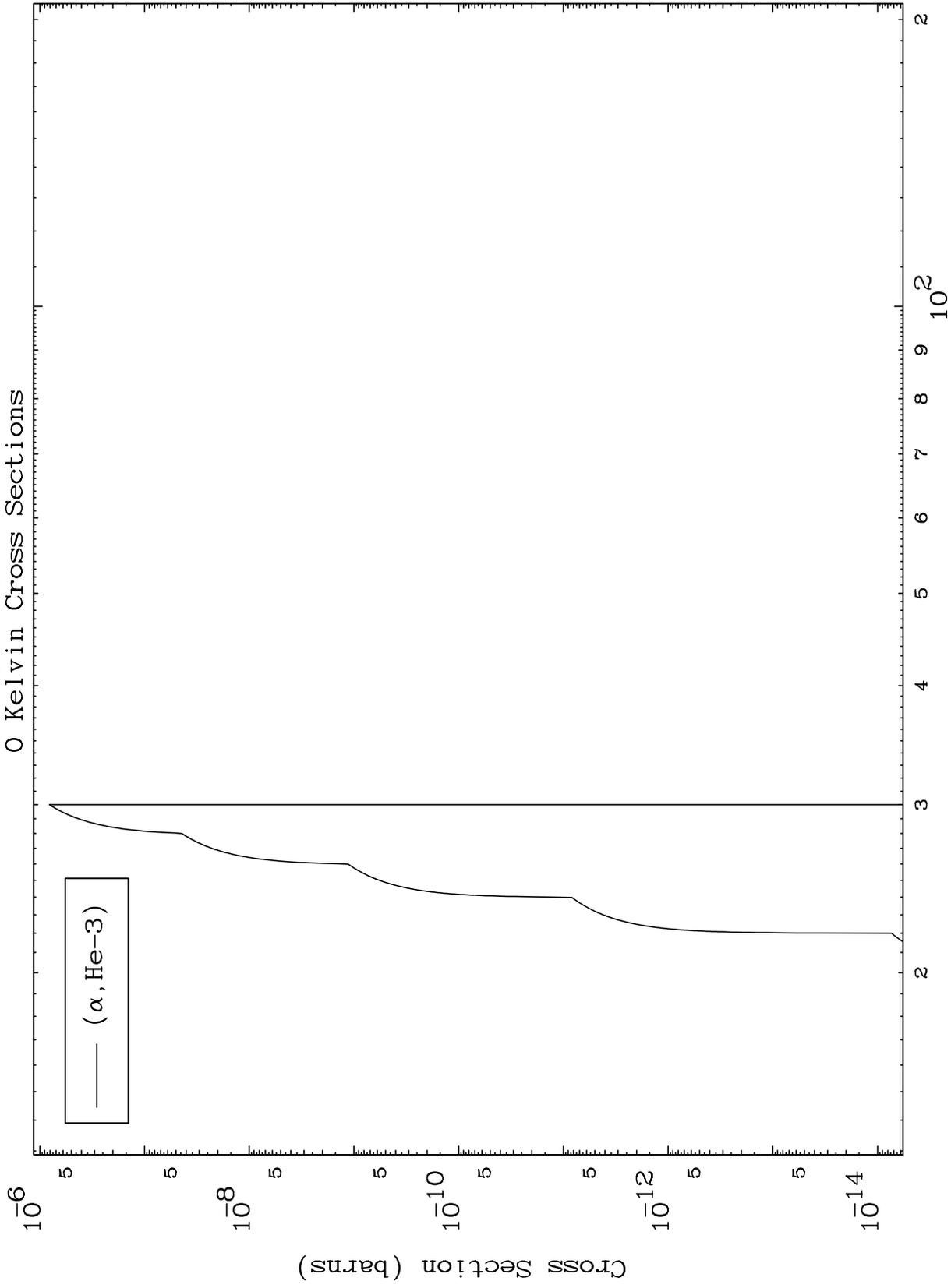
MAT 7701

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

77-Ir-183



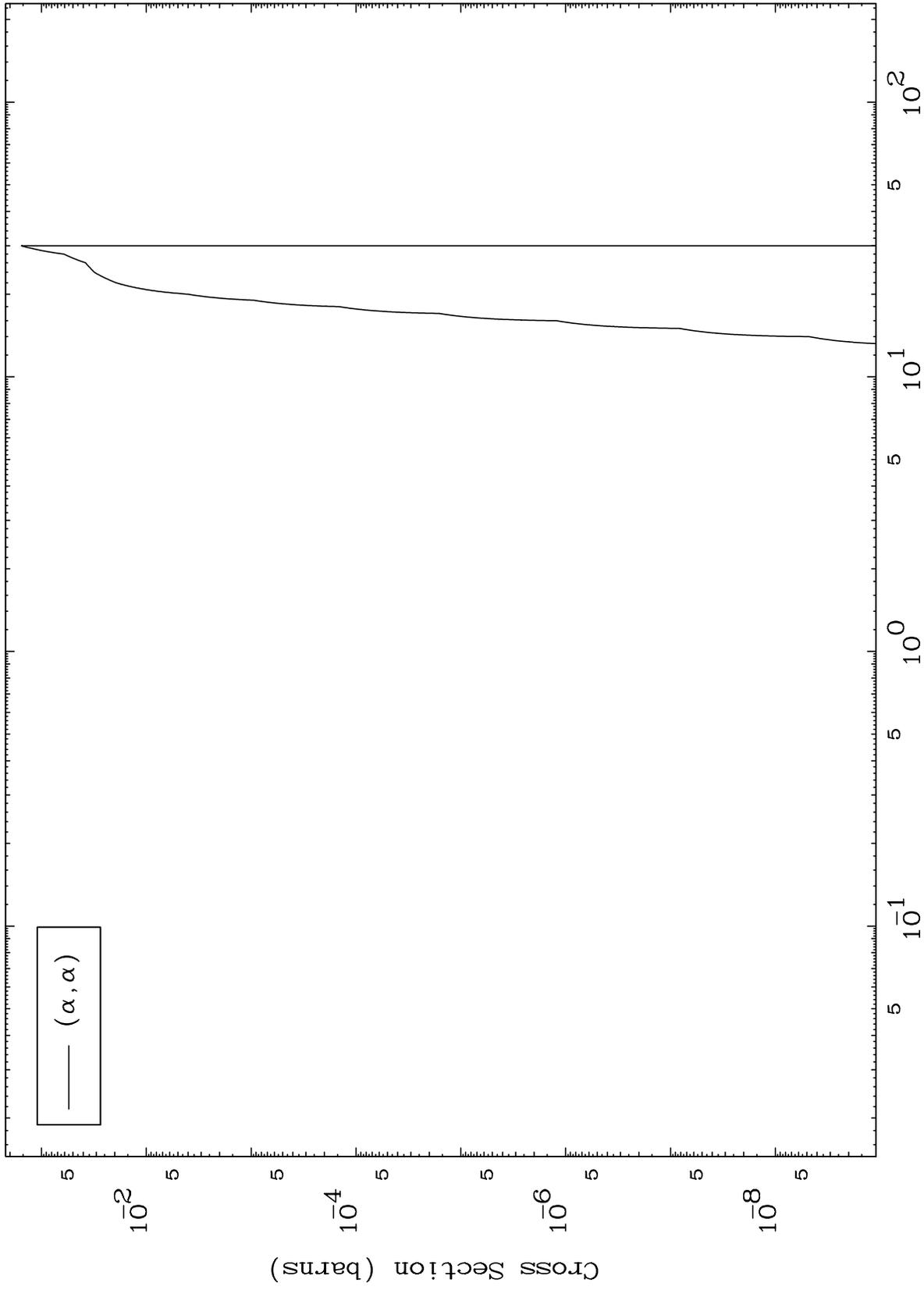




MAT 7701

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

77-Ir-183



10

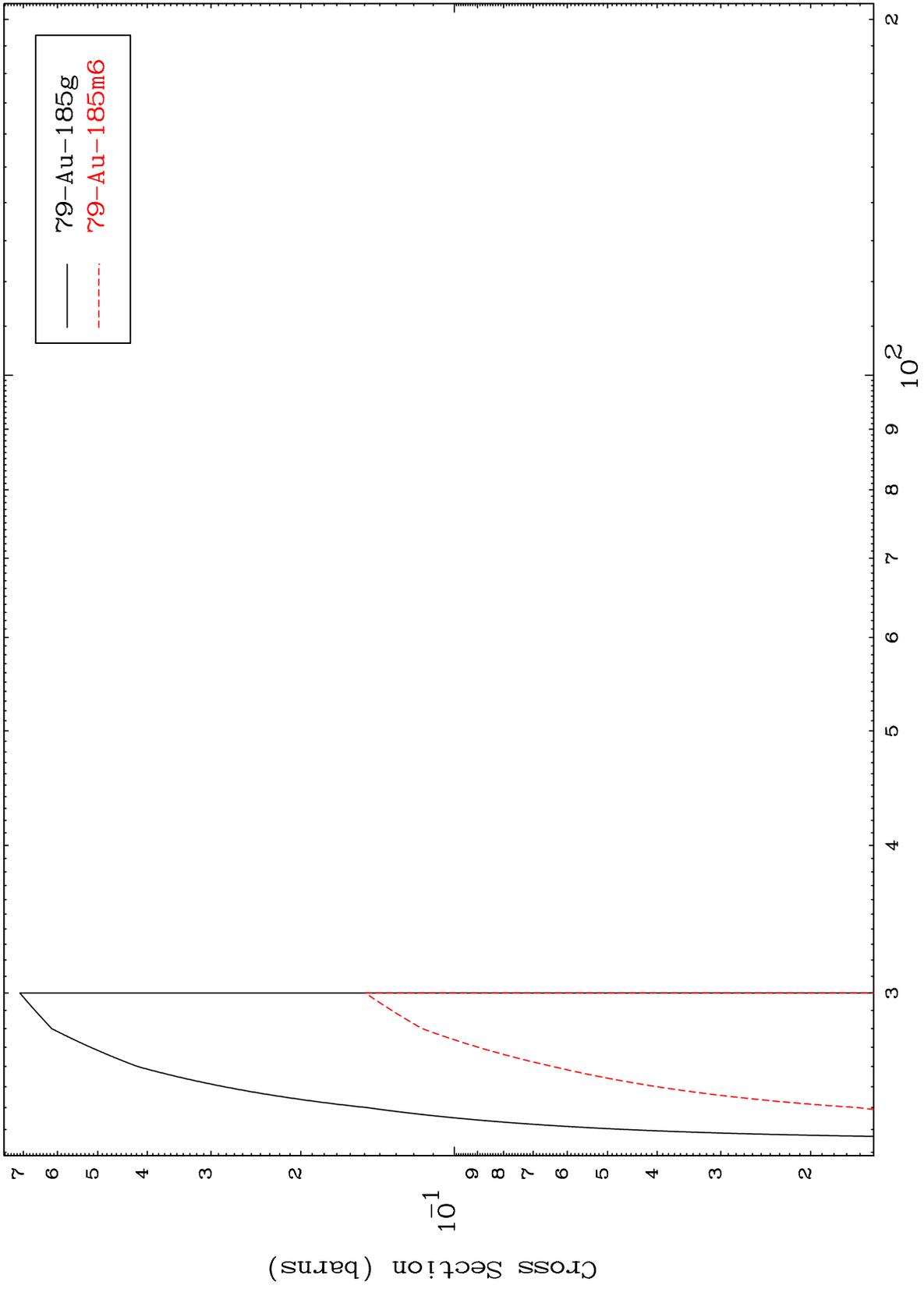
Incident Energy (MeV)

77-Ir-183

MAT 7701

77-Ir-183

( $\alpha, 2n$ )  
Radionuclide Production Cross Section



11

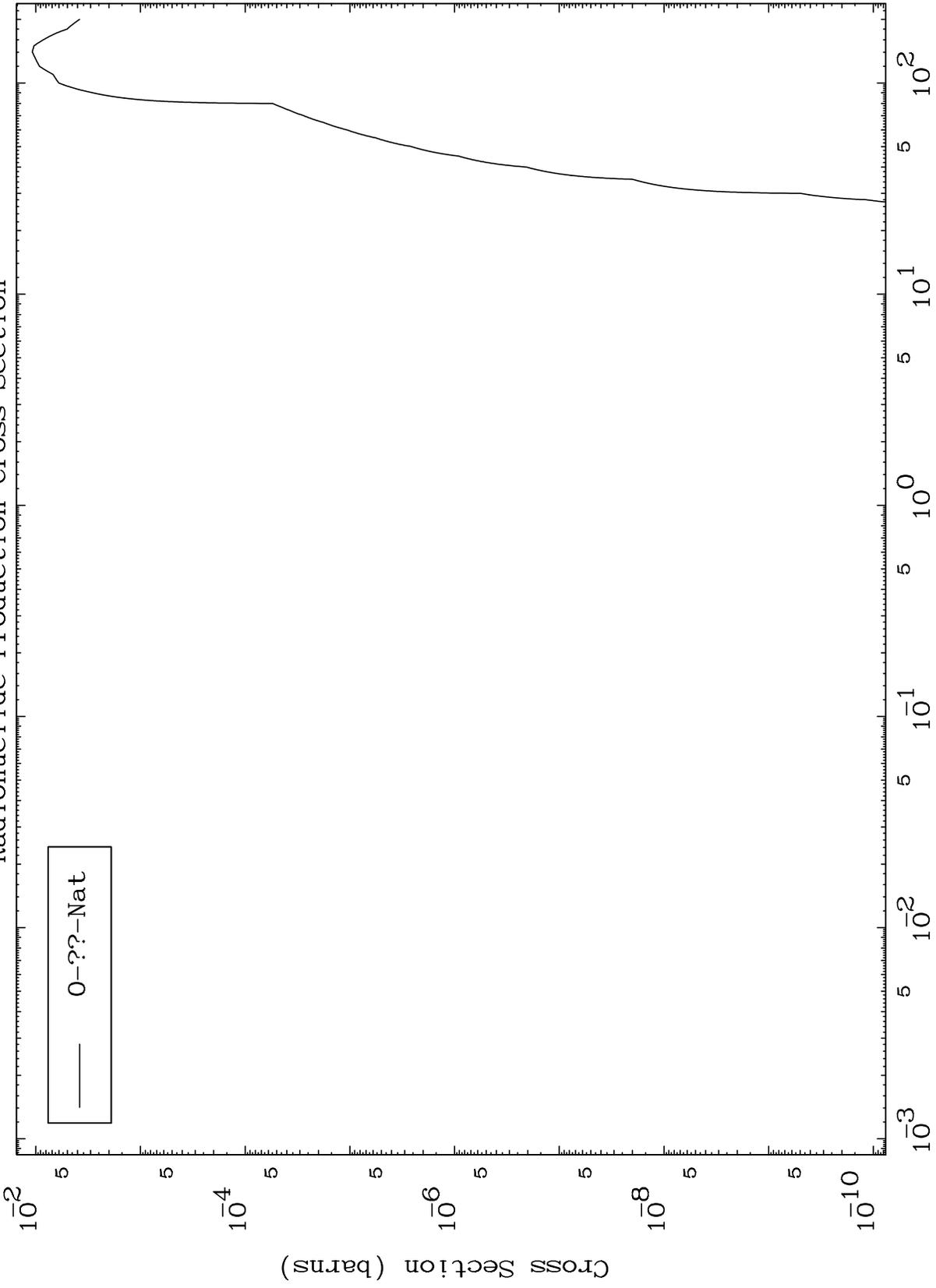
Incident Energy (MeV)

77-Ir-183

MAT 7701

$\alpha$  Fission  
Radionuclide Production Cross Section

77-Ir-183



12

Incident Energy (MeV)

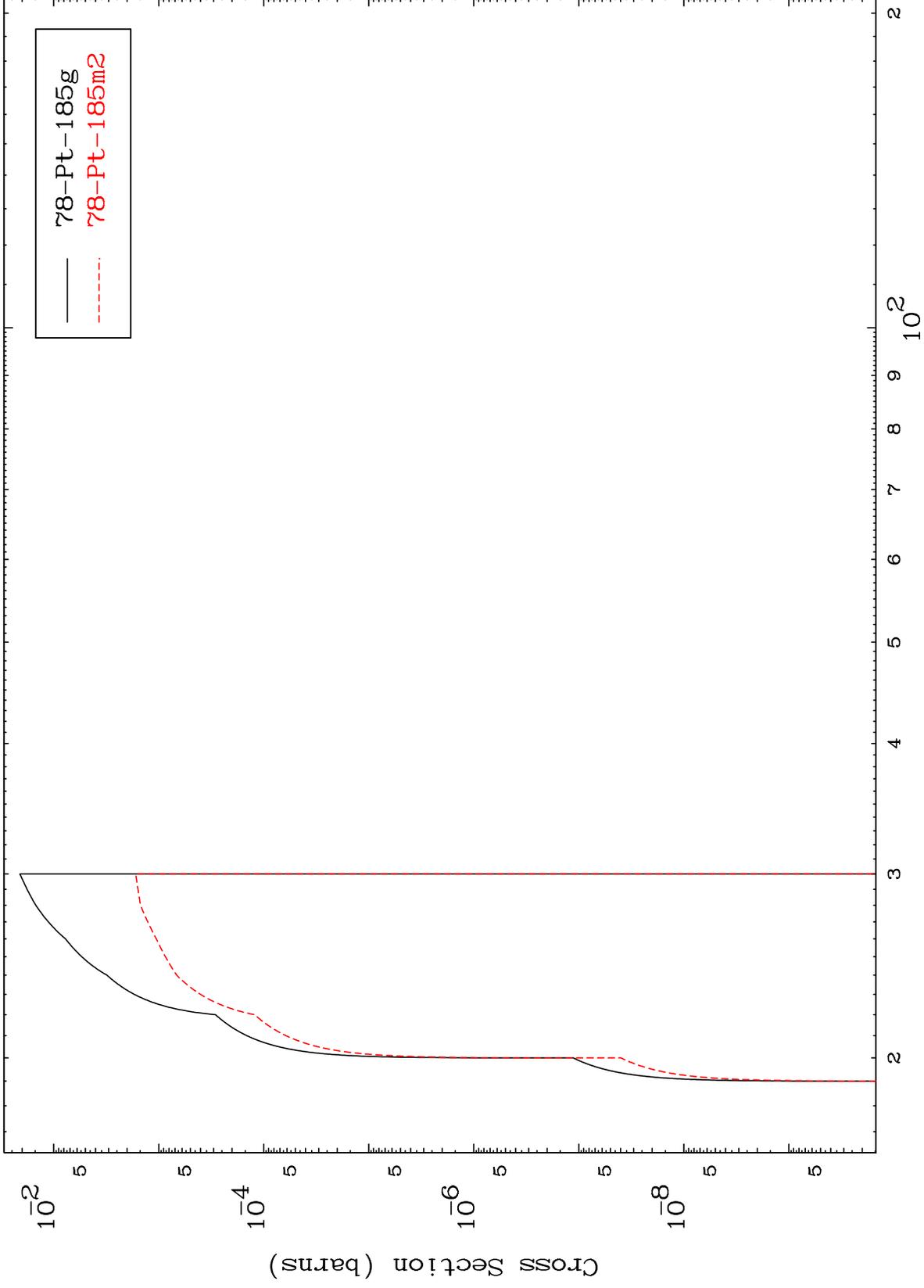
77-Ir-183

MAT 7701

$(\alpha, n')$  p

77-Ir-183

Radionuclide Production Cross Section



13

Incident Energy (MeV)

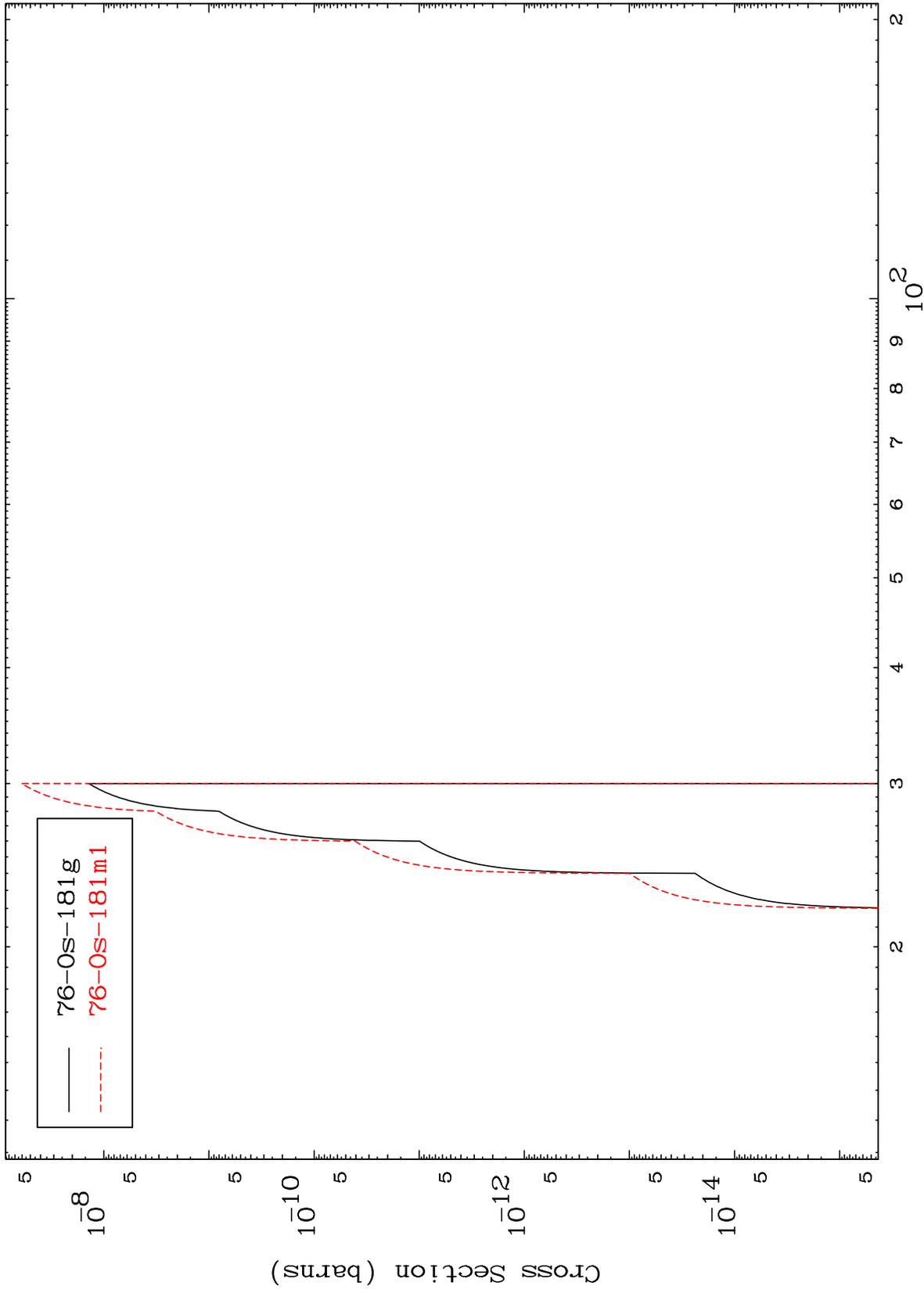
77-Ir-183

MAT 7701

( $\alpha, n'$ ) p  $\alpha$

77-Ir-183

Radionuclide Production Cross Section



14

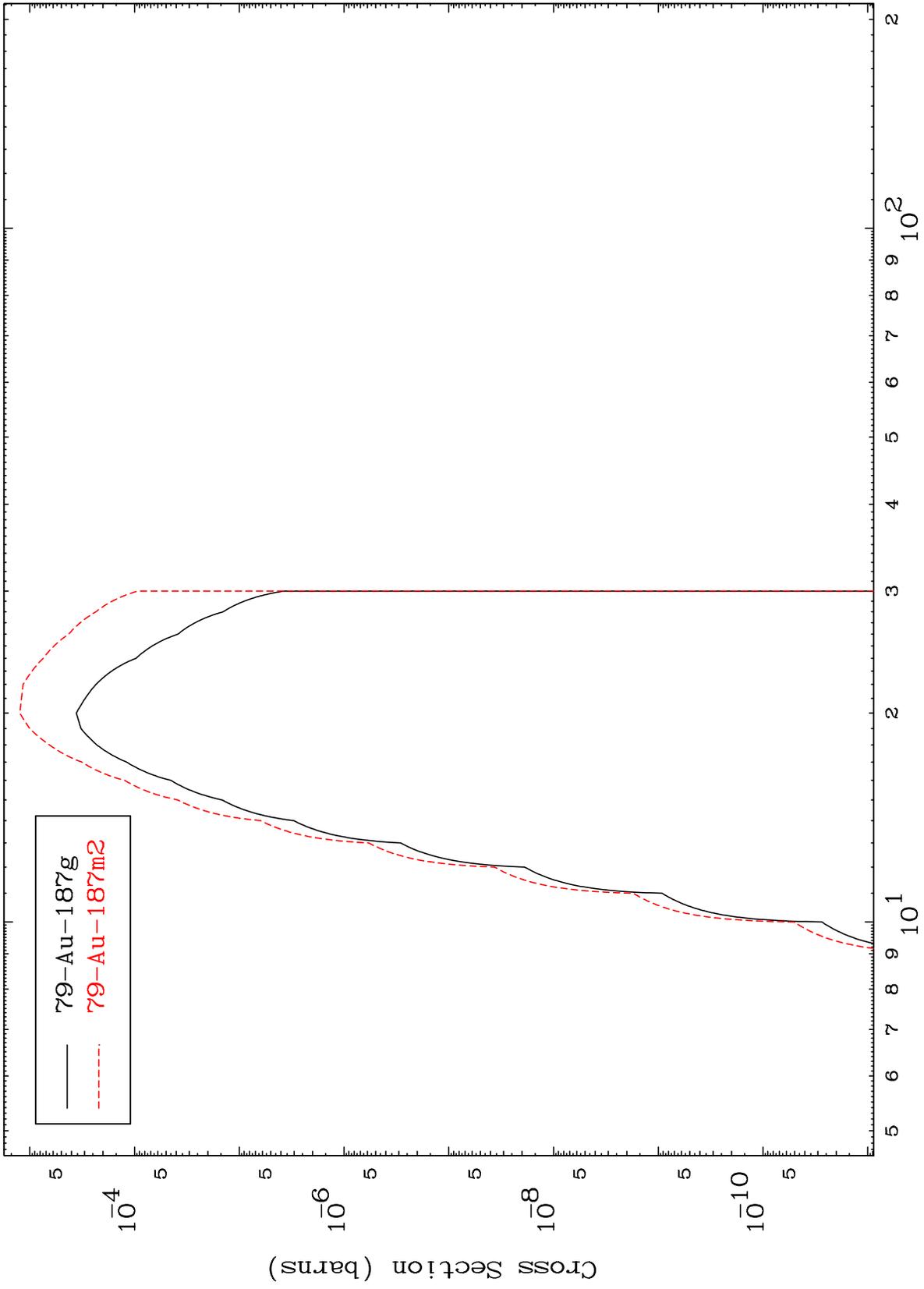
Incident Energy (MeV)

77-Ir-183

MAT 7701

77-Ir-183

( $\alpha, \gamma$ )  
Radionuclide Production Cross Section



15

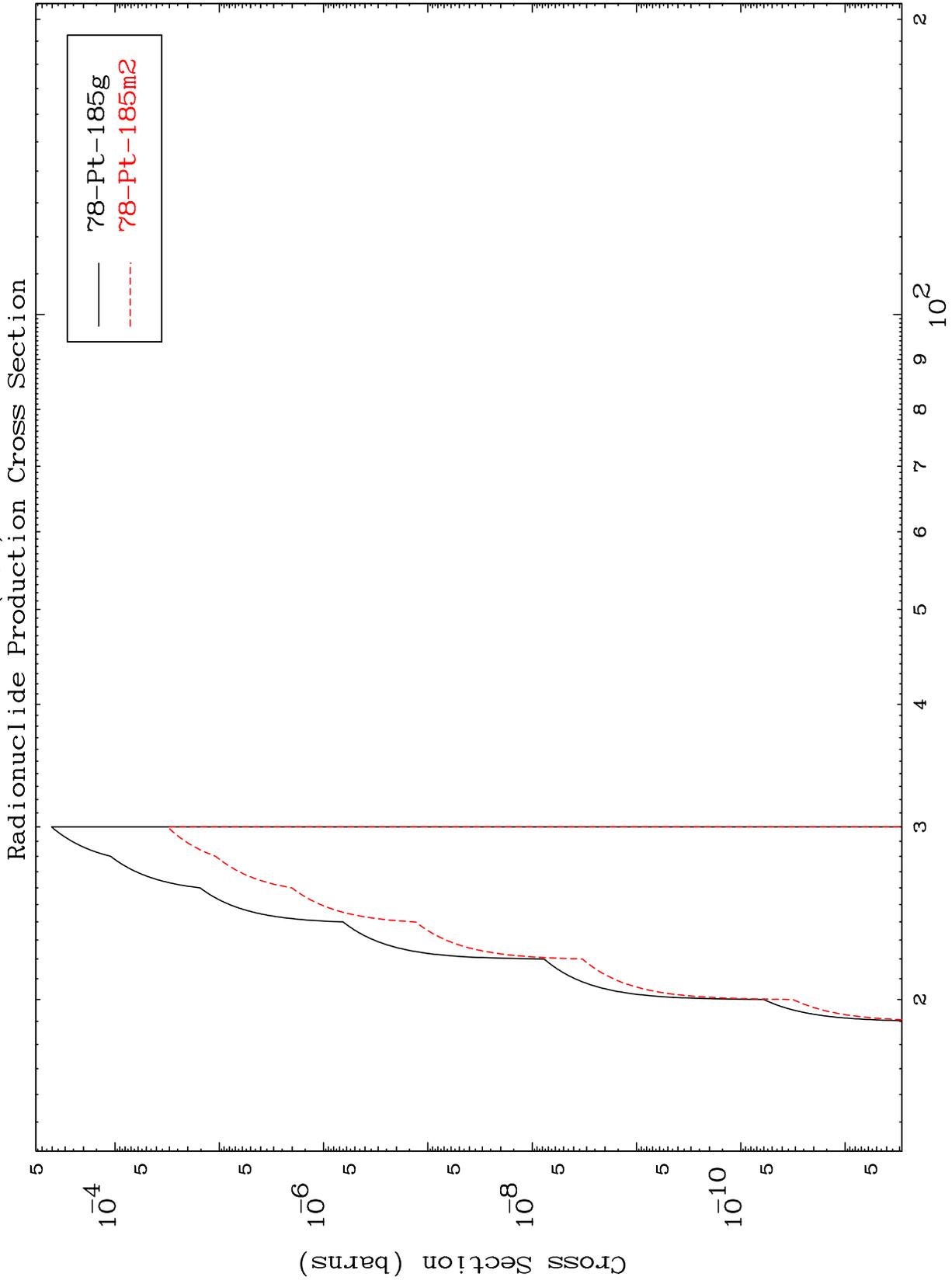
Incident Energy (MeV)

77-Ir-183

MAT 7701

( $\alpha, d$ )

77-Ir-183



16

Incident Energy (MeV)

77-Ir-183