

Program EVALPLOT  
(Version 2018-1)

by

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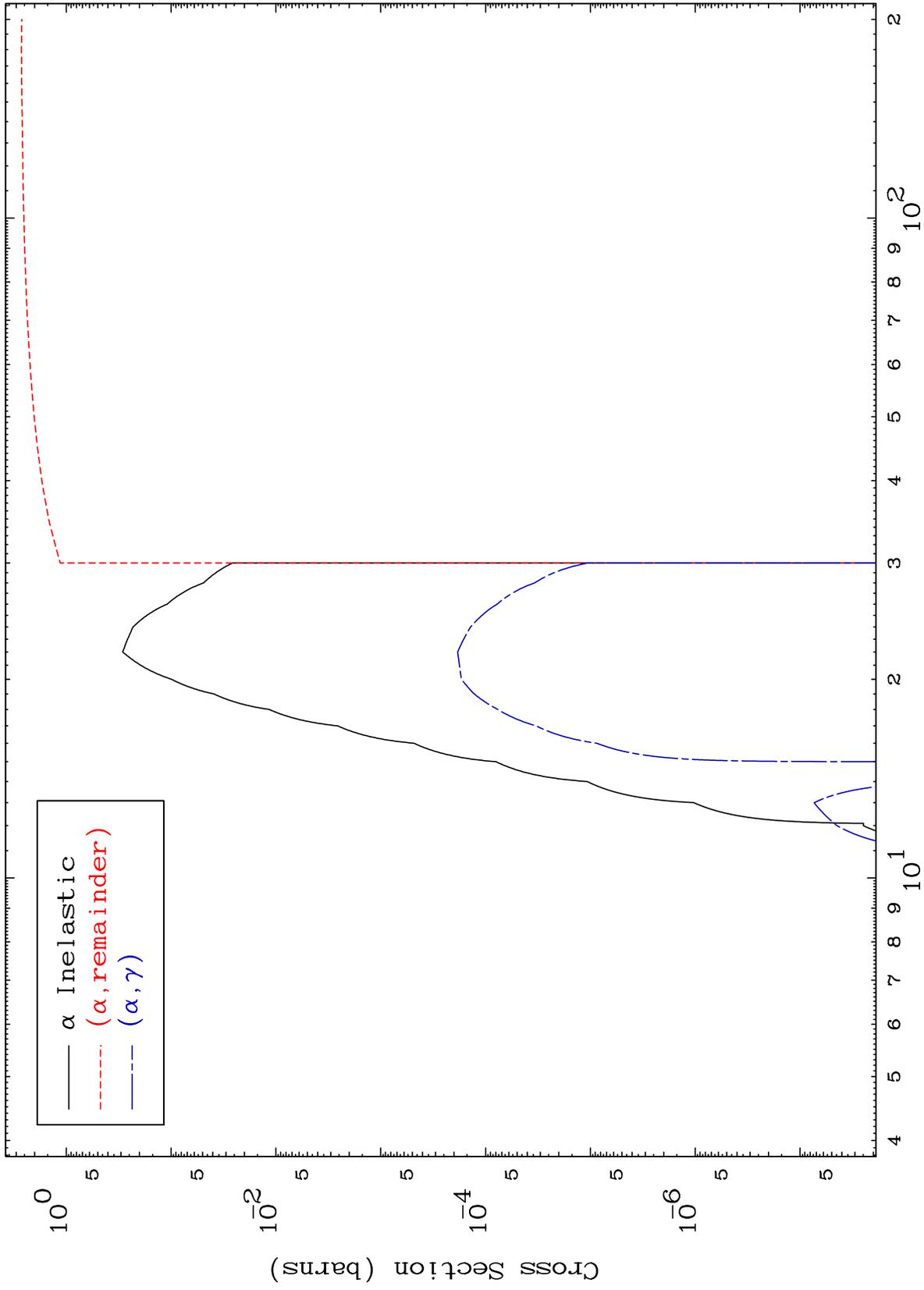
Press Mouse Button to Start

MAT 7710

$\alpha$  Major

77-Ir-186

0 Kelvin Cross Sections



1

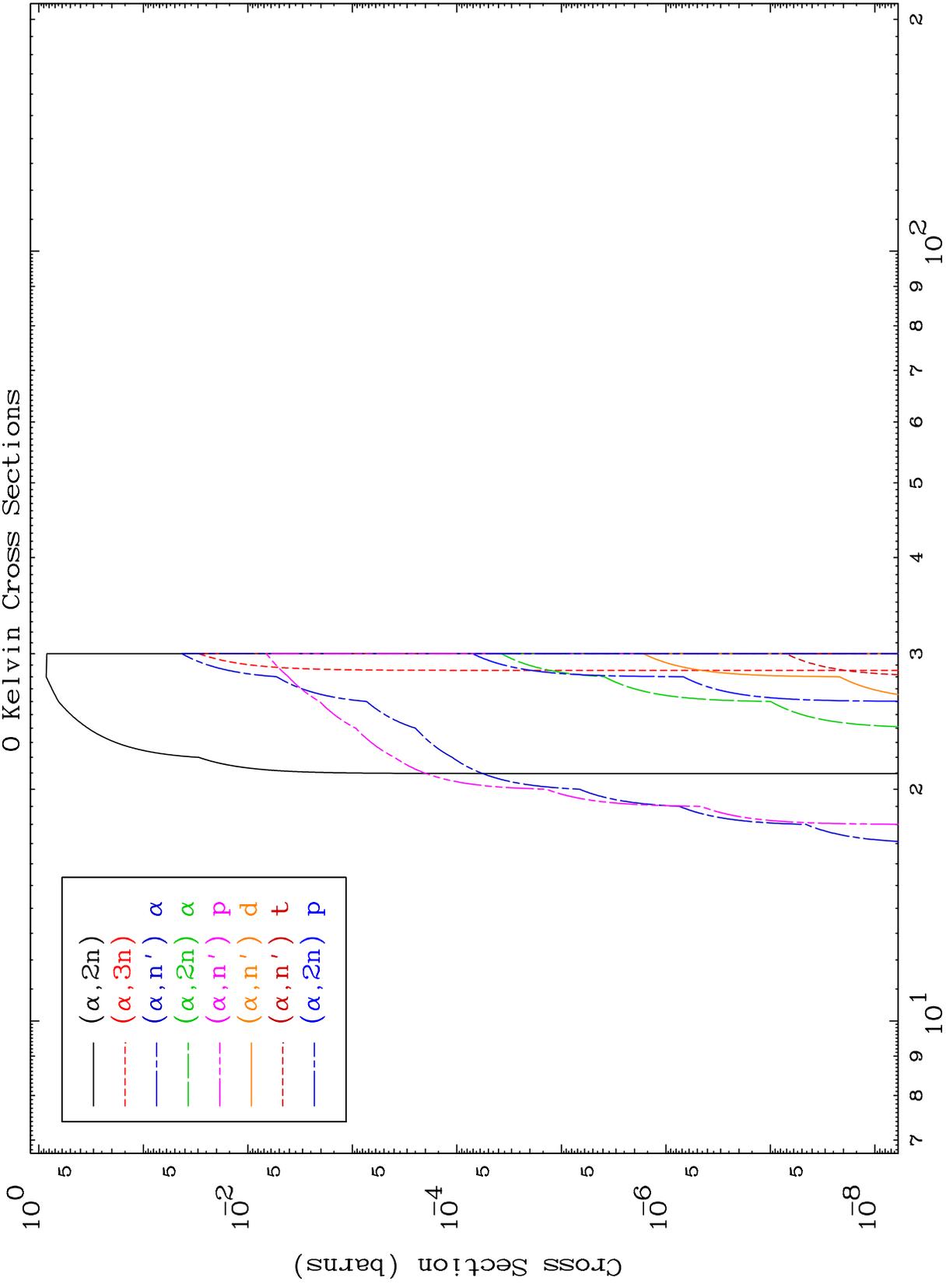
Incident Energy (MeV)

77-Ir-186

MAT 7710

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

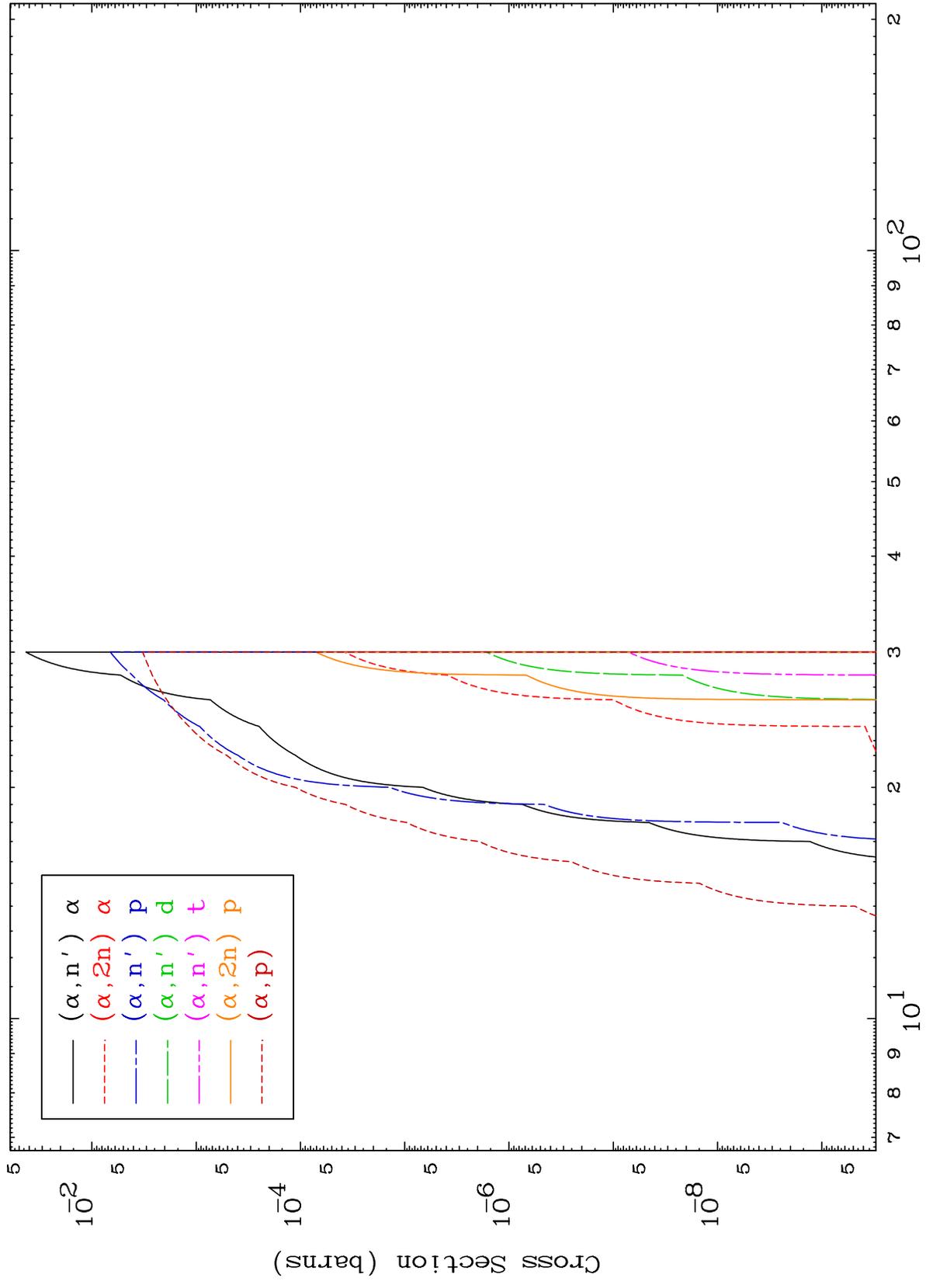
77-Ir-186



2

Incident Energy (MeV)

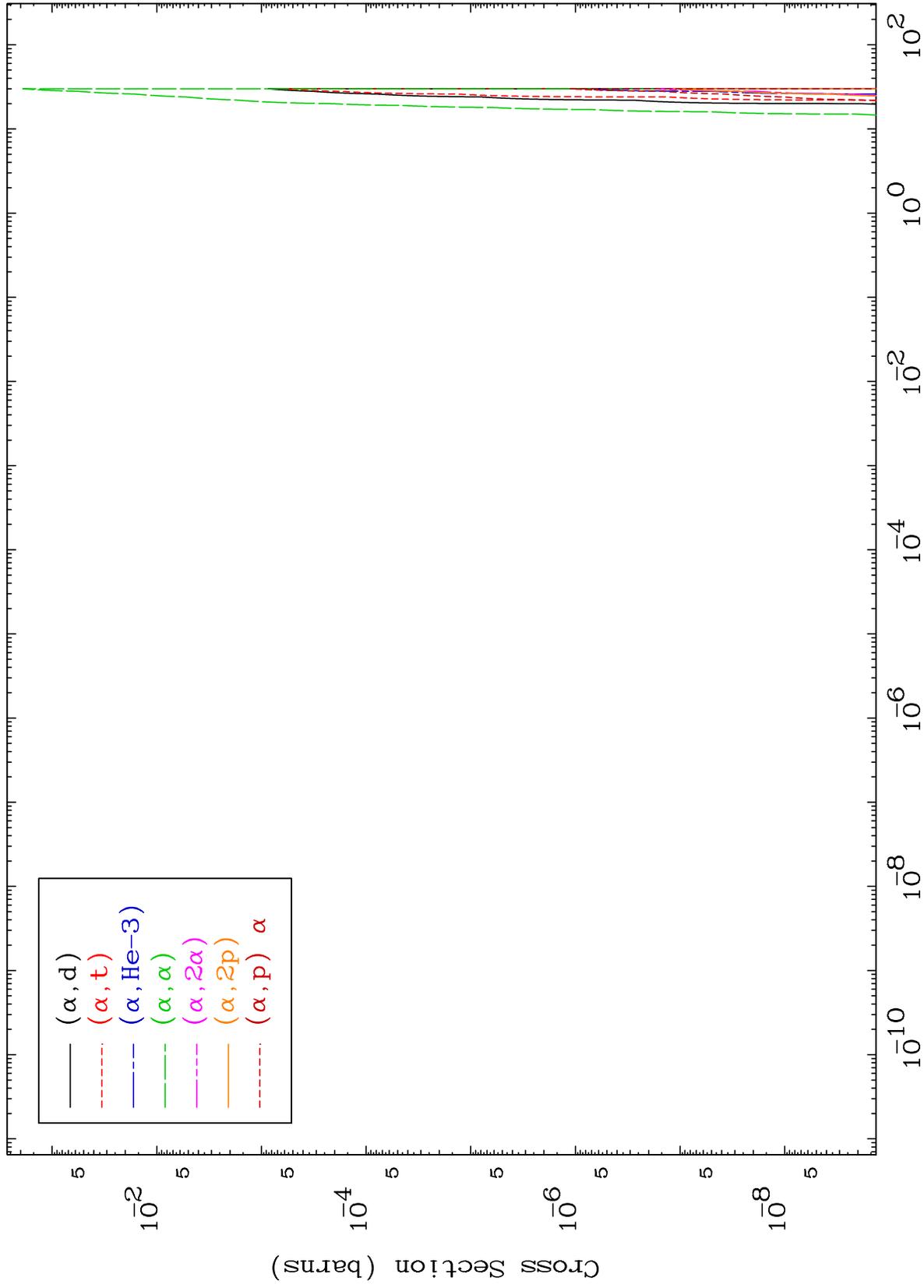
77-Ir-186



MAT 7710

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

77-Ir-186

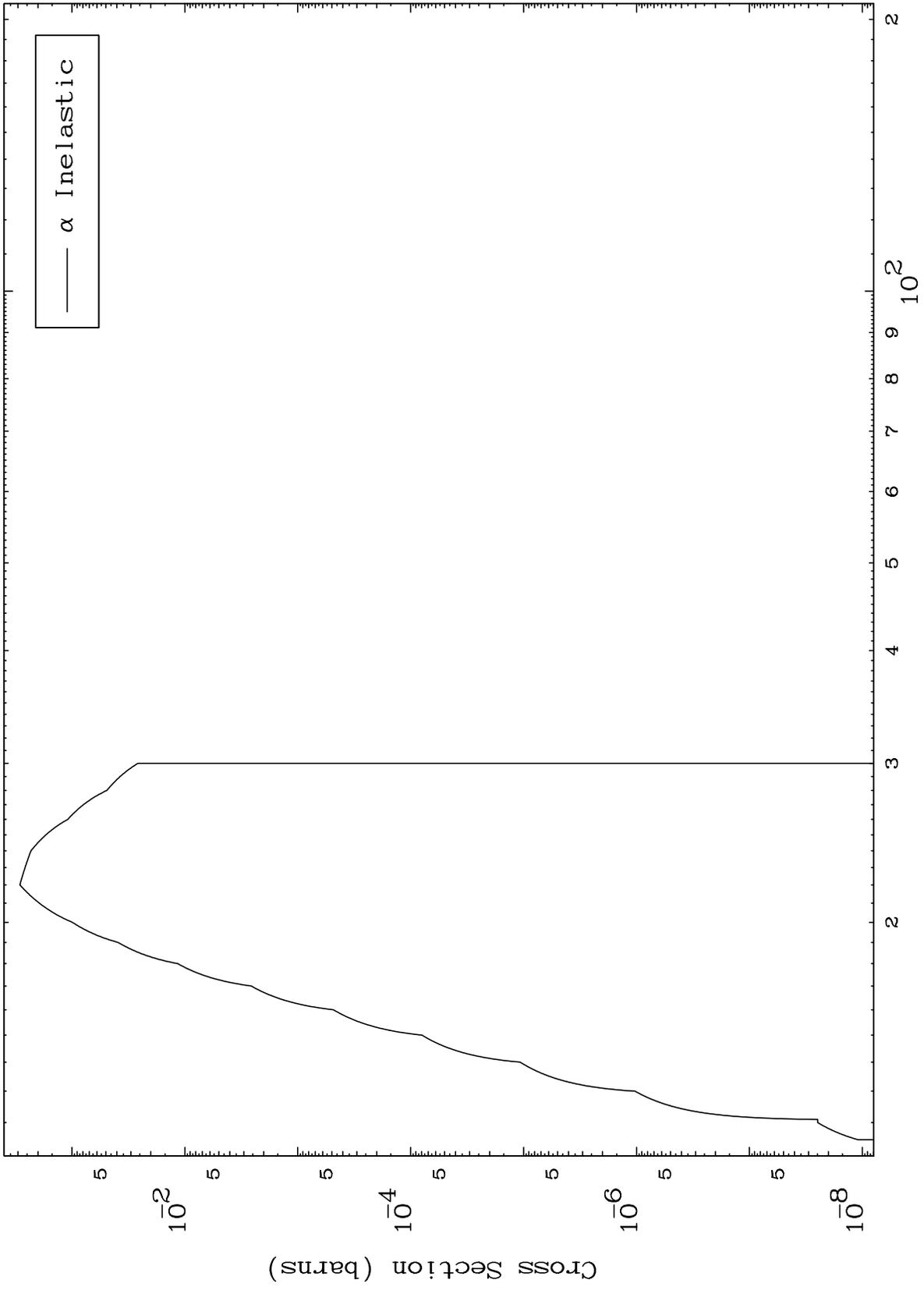


77-Ir-186

MAT 7710

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

77-Ir-186



5

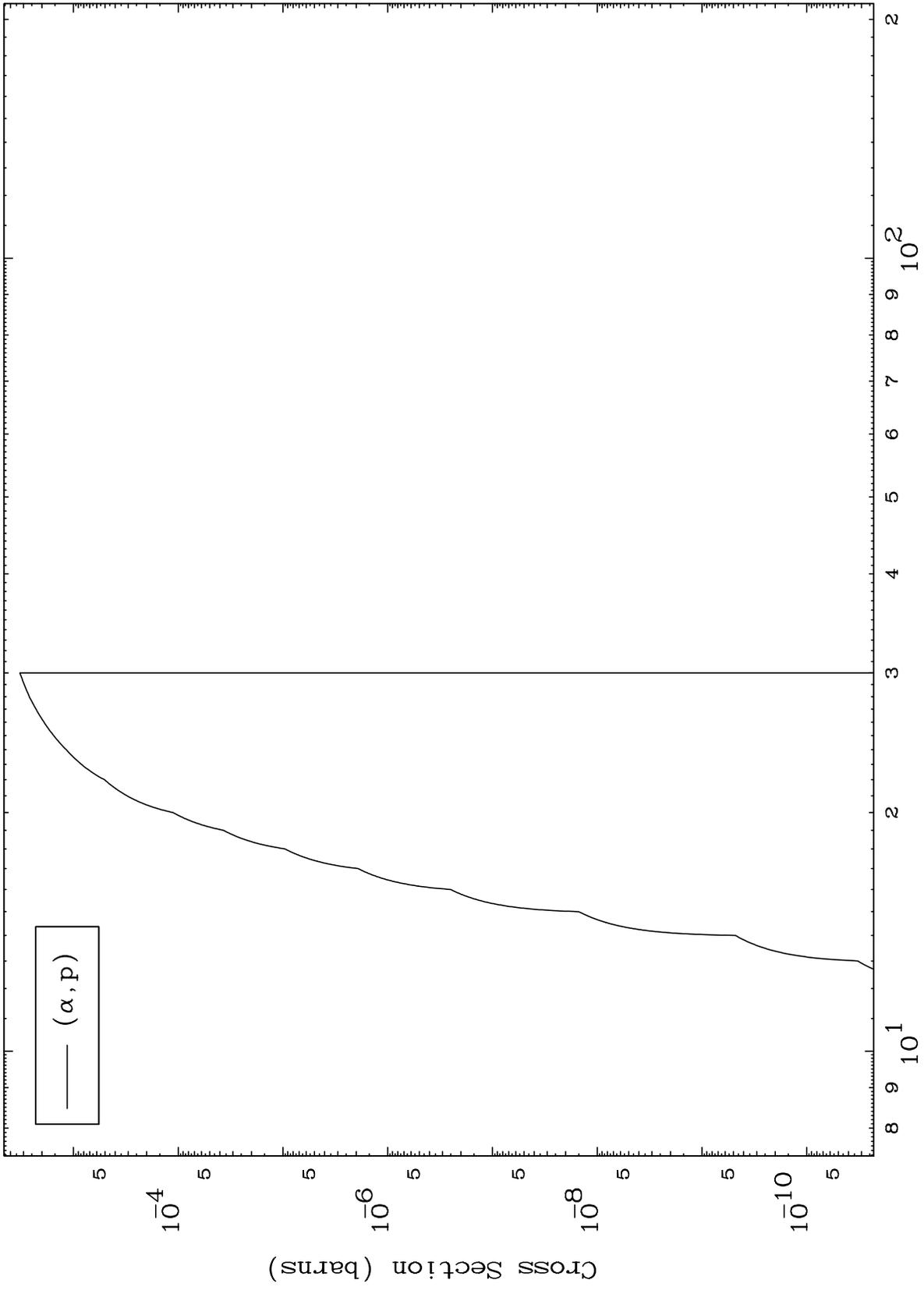
Incident Energy (MeV)

77-Ir-186

MAT 7710

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

77-Ir-186



6

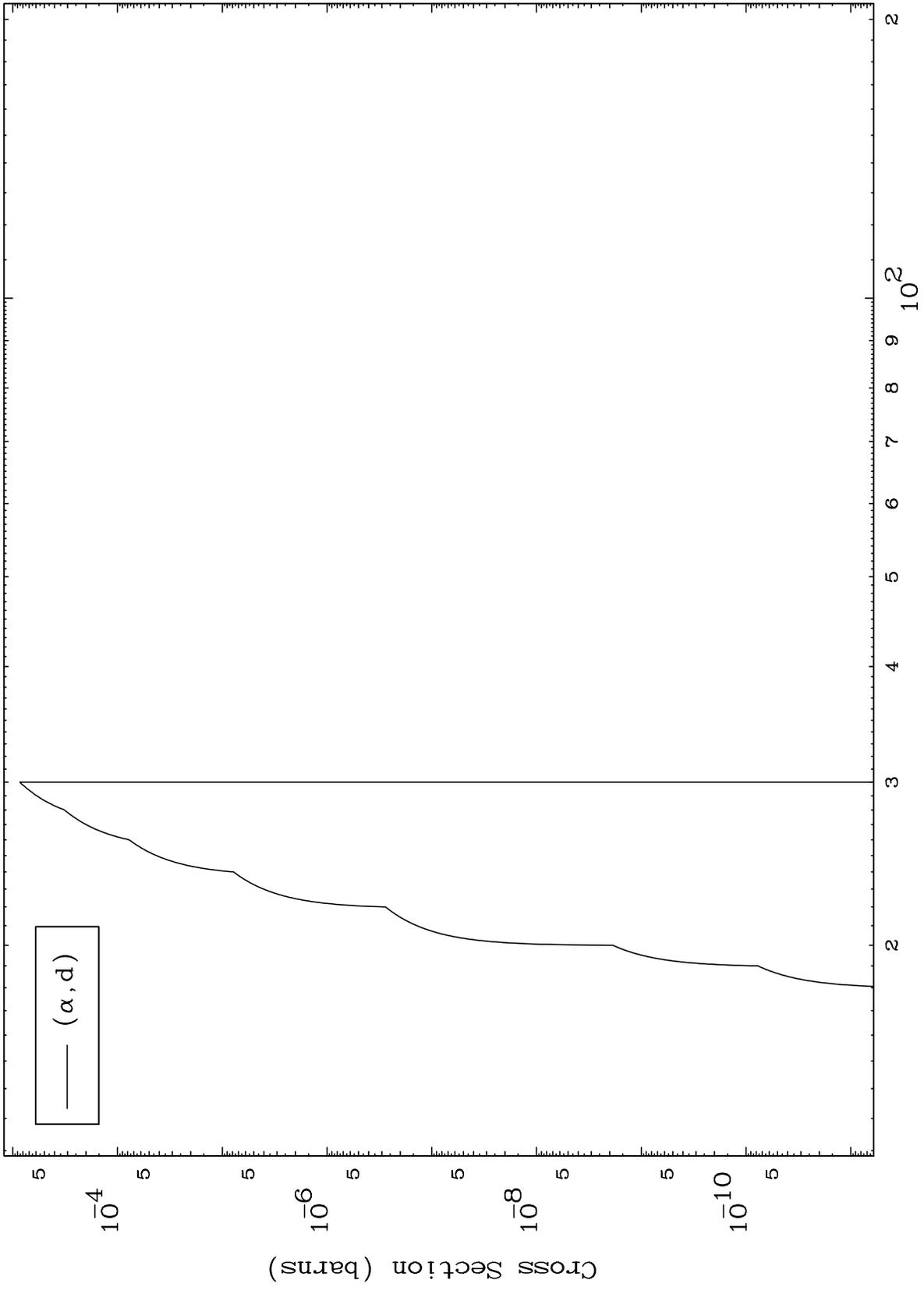
Incident Energy (MeV)

77-Ir-186

MAT 7710

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

77-Ir-186



7

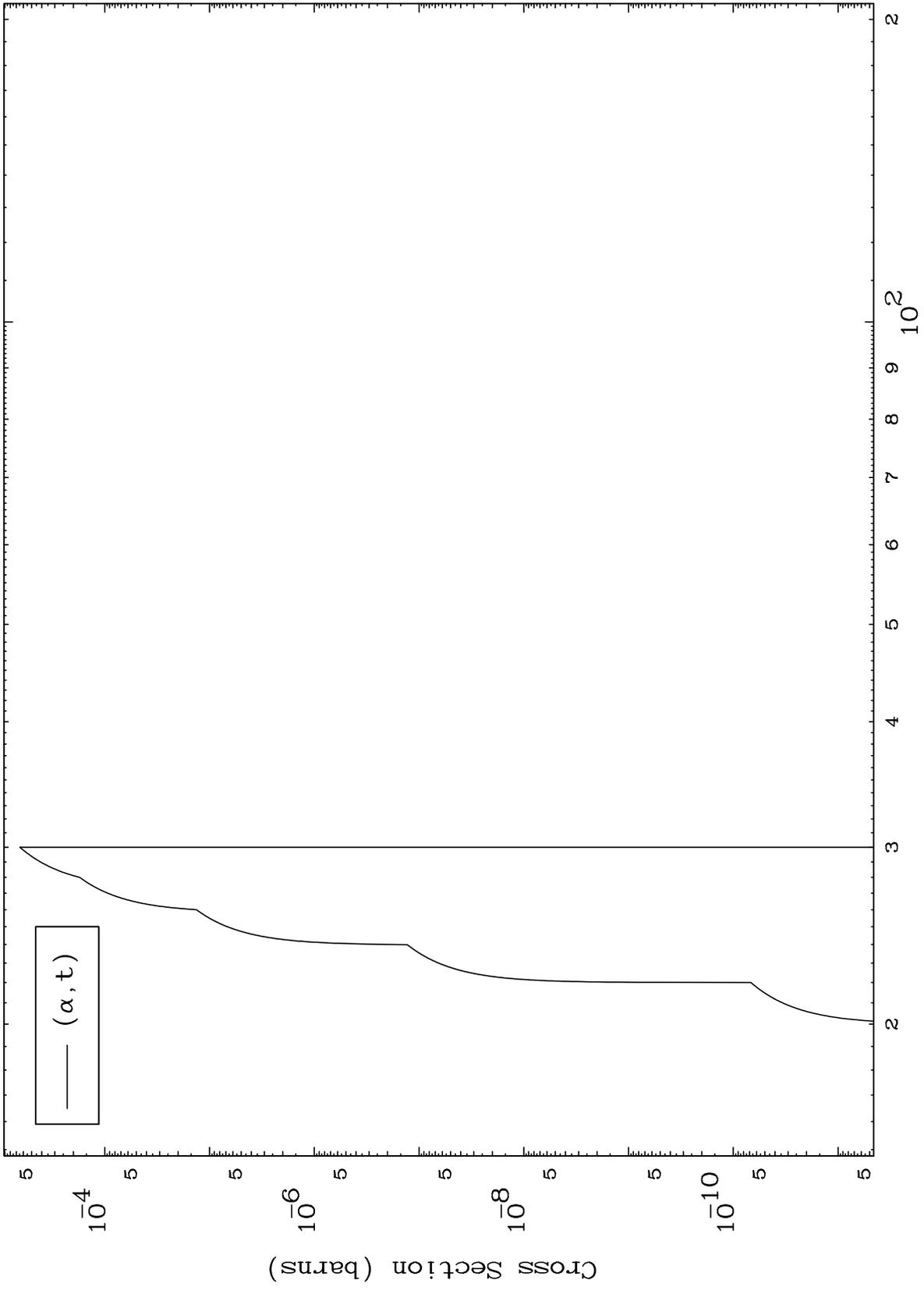
Incident Energy (MeV)

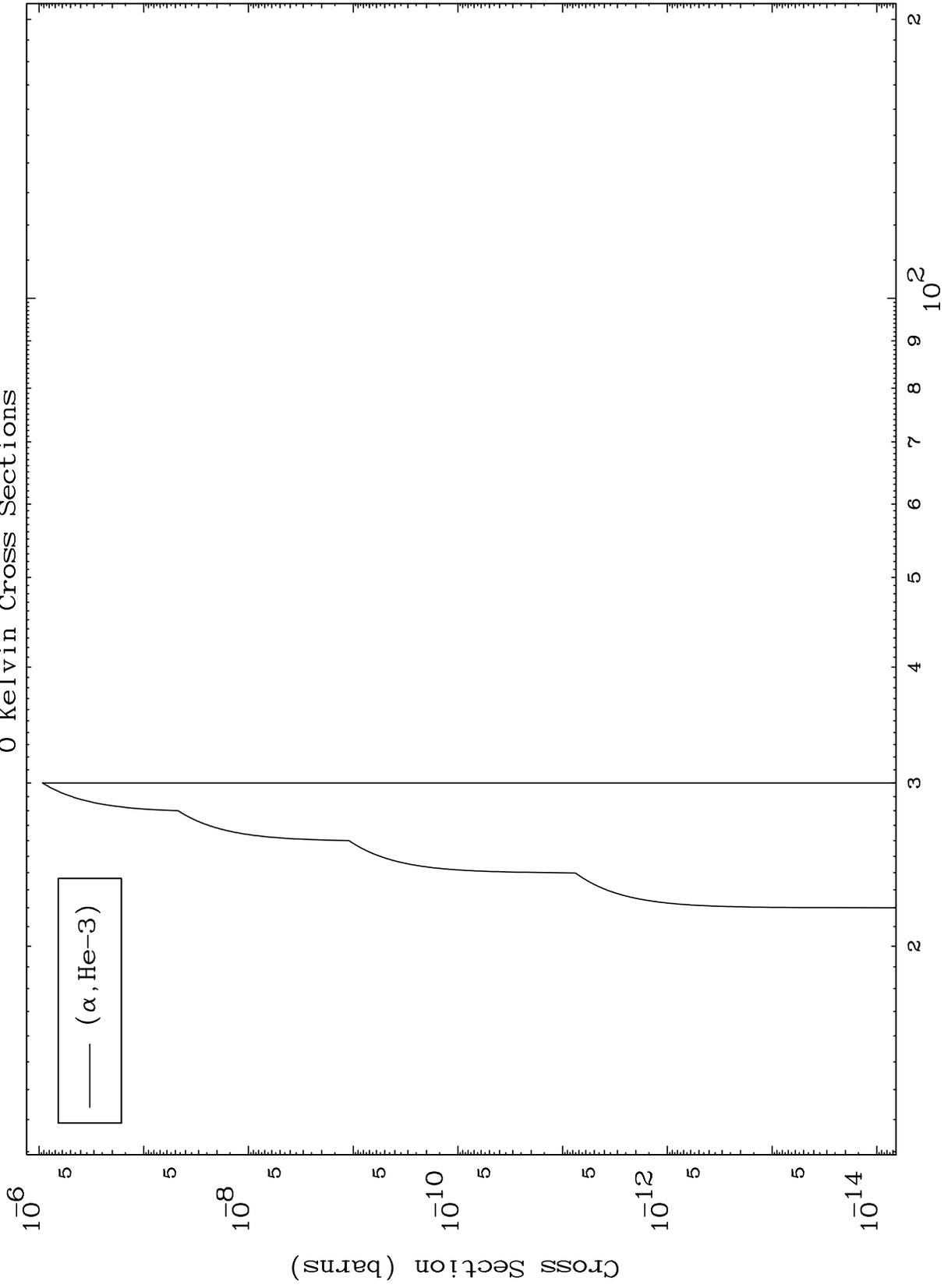
77-Ir-186

MAT 7710

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

77-Ir-186

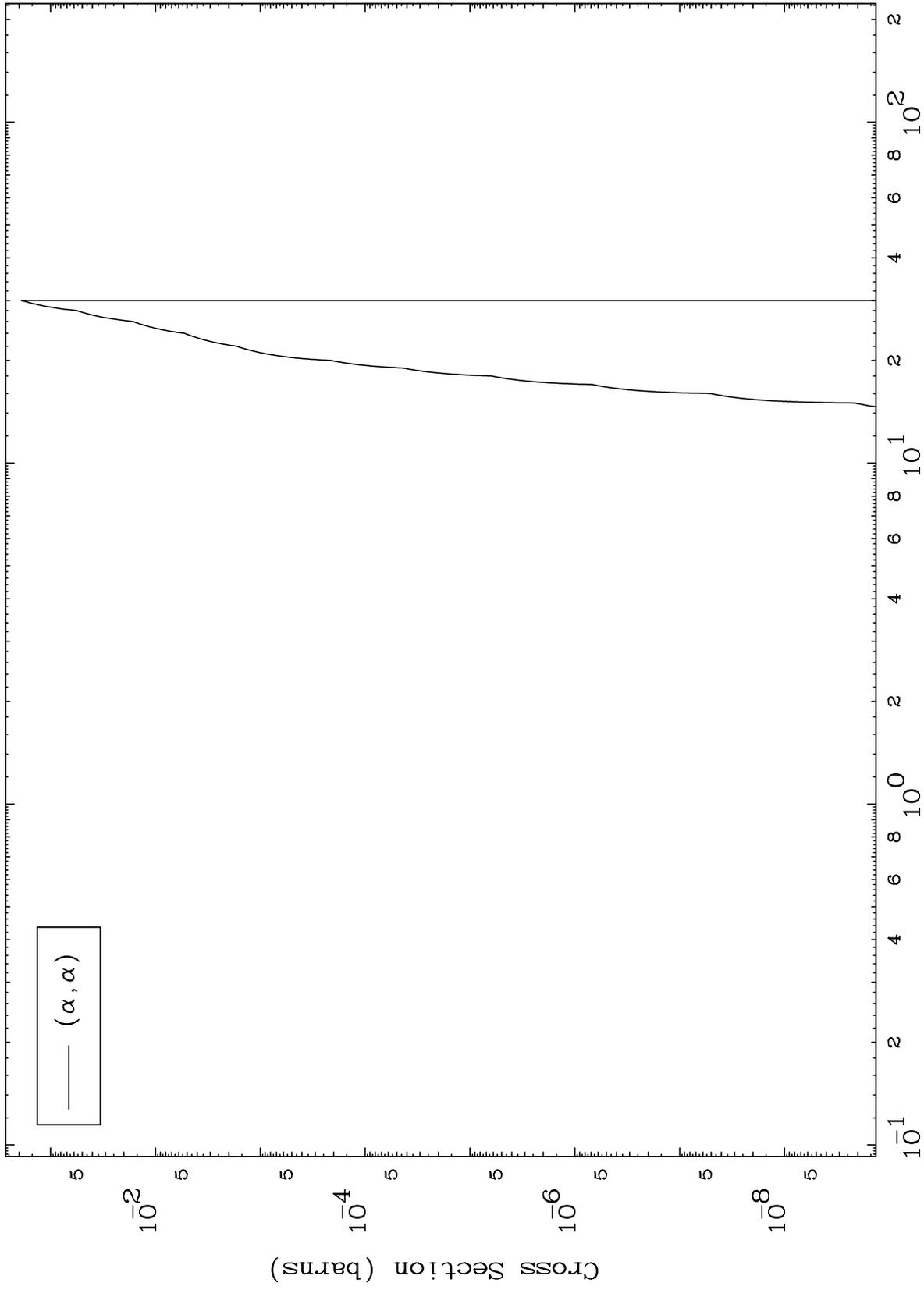




MAT 7710

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

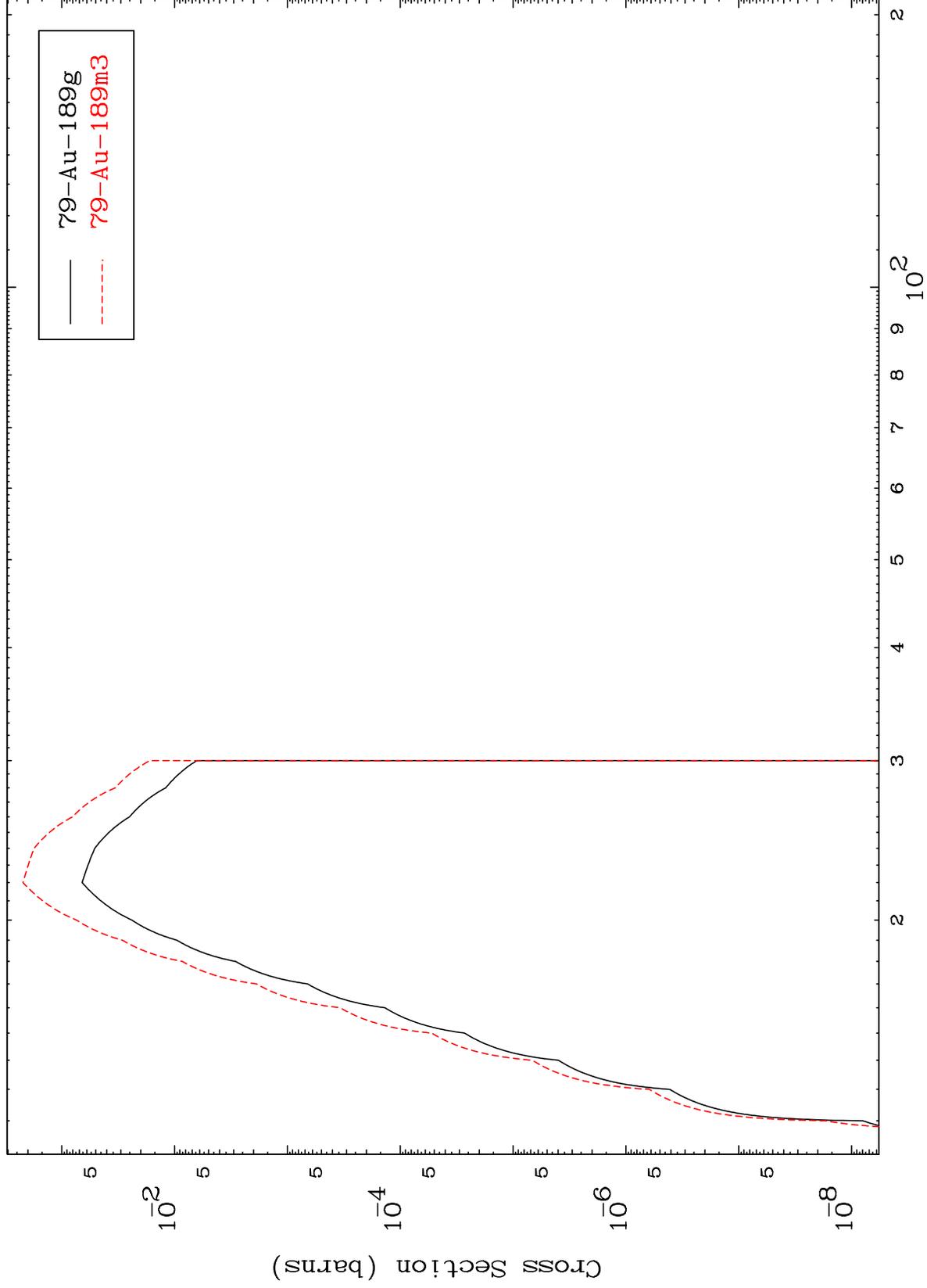
77-Ir-186



Incident Energy (MeV)

77-Ir-186

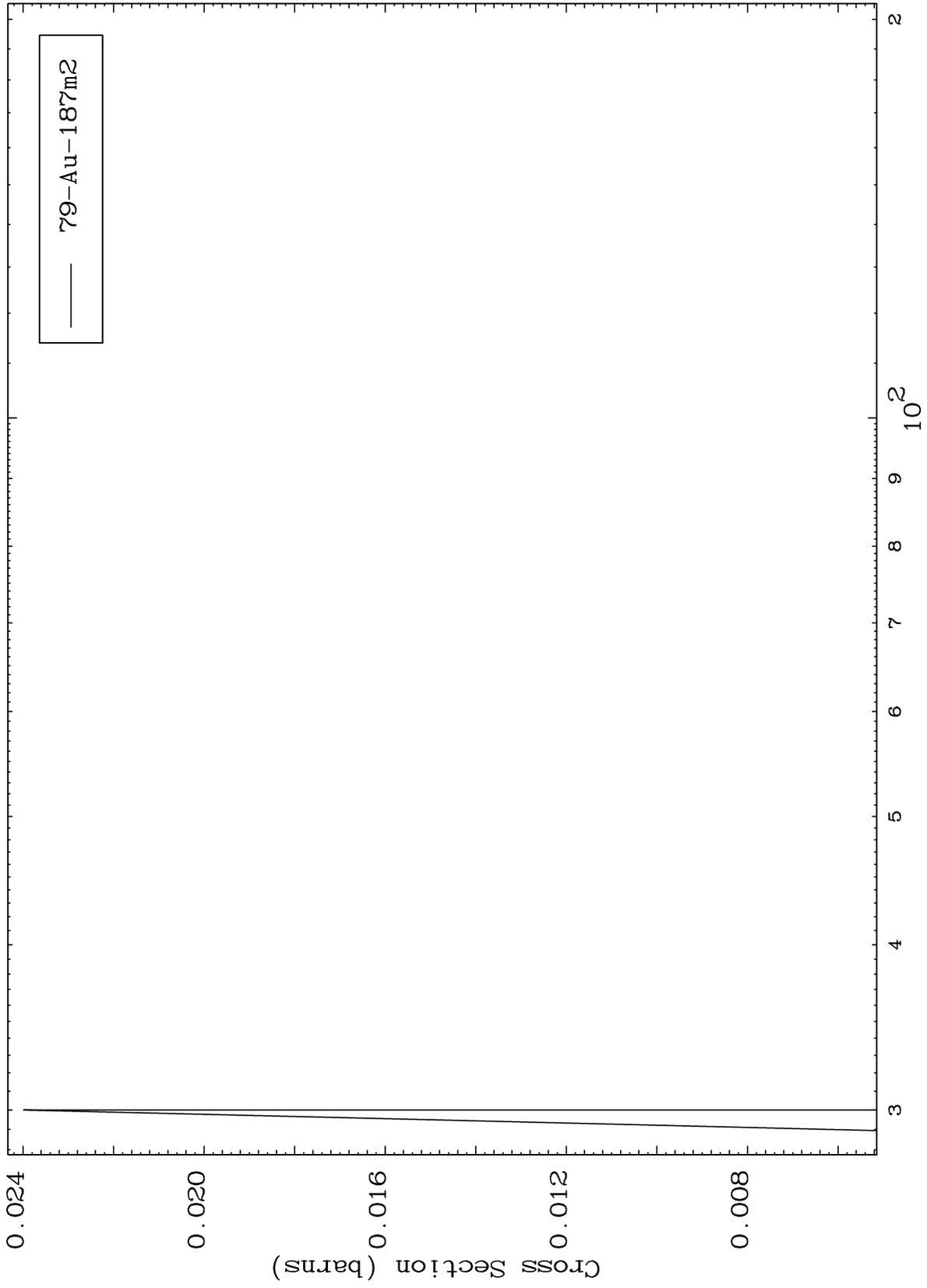
Radionuclide Production Cross Section



MAT 7710

77-Ir-186

( $\alpha, 3n$ )  
Radionuclide Production Cross Section



12

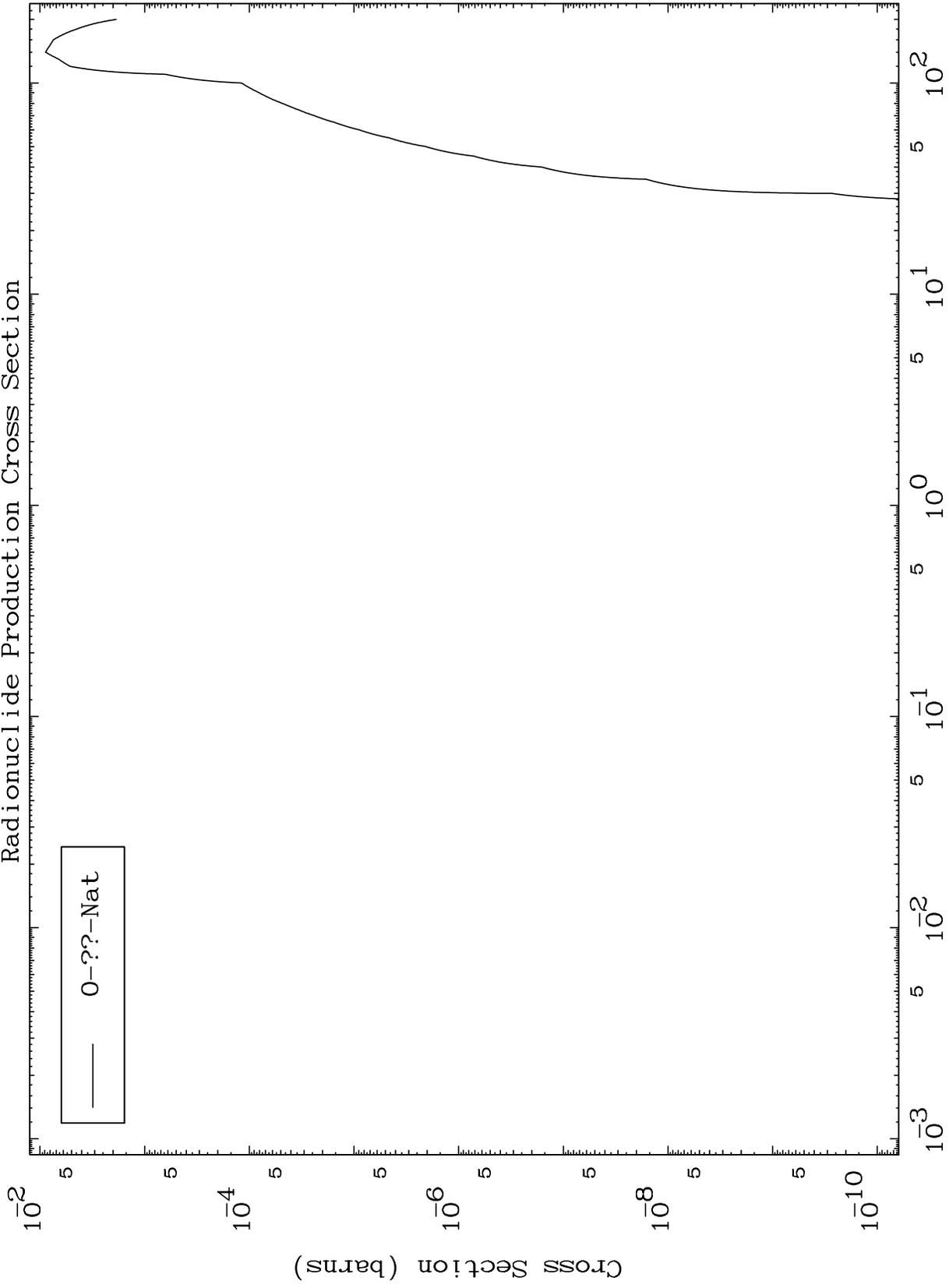
77-Ir-186

Incident Energy (MeV)

MAT 7710

$\alpha$  Fission  
Radionuclide Production Cross Section

77-Ir-186



13

Incident Energy (MeV)

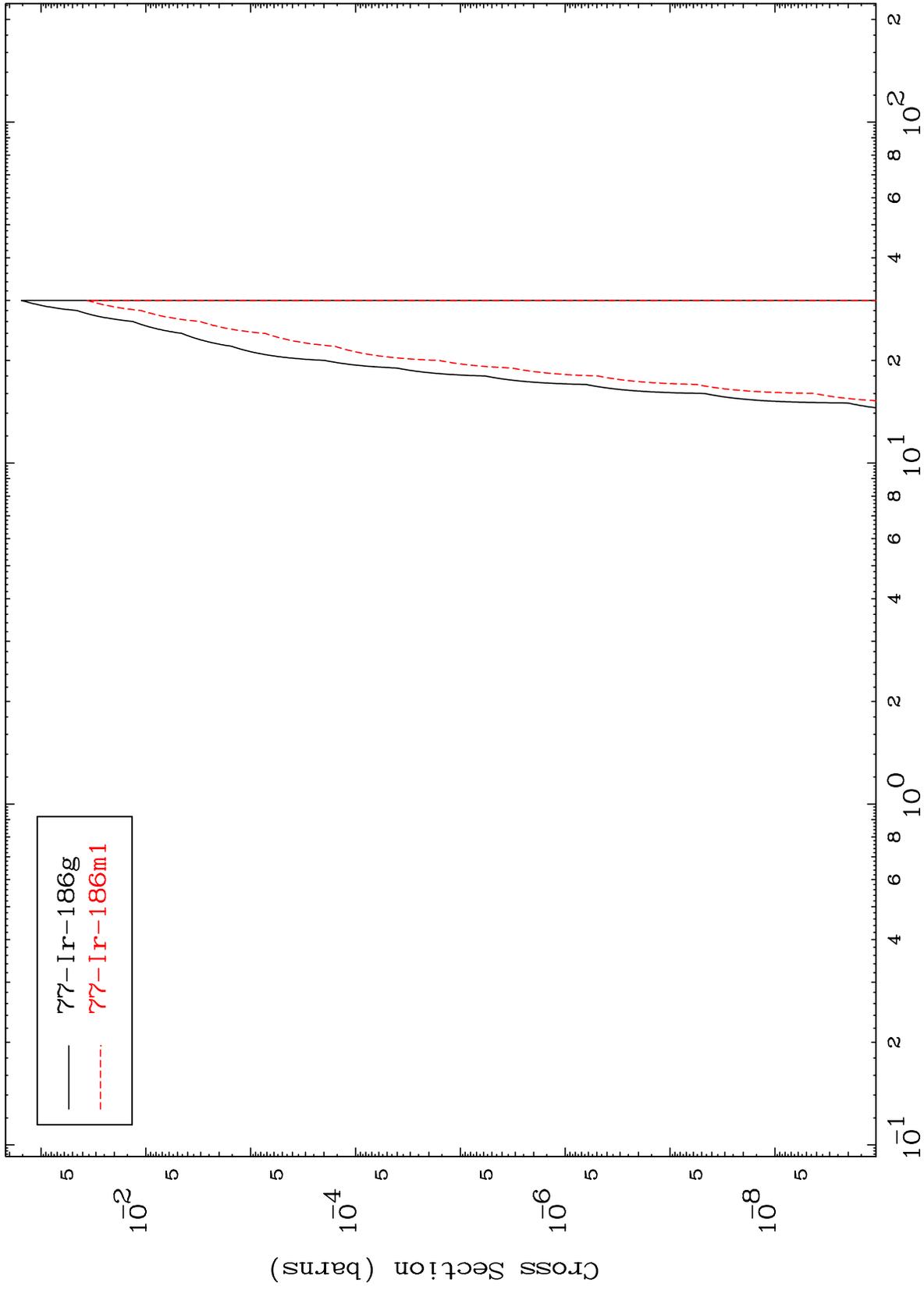
77-Ir-186

MAT 7710

( $\alpha, \alpha$ )

<sup>77</sup>Ir-186

Radionuclide Production Cross Section



14

Incident Energy (MeV)

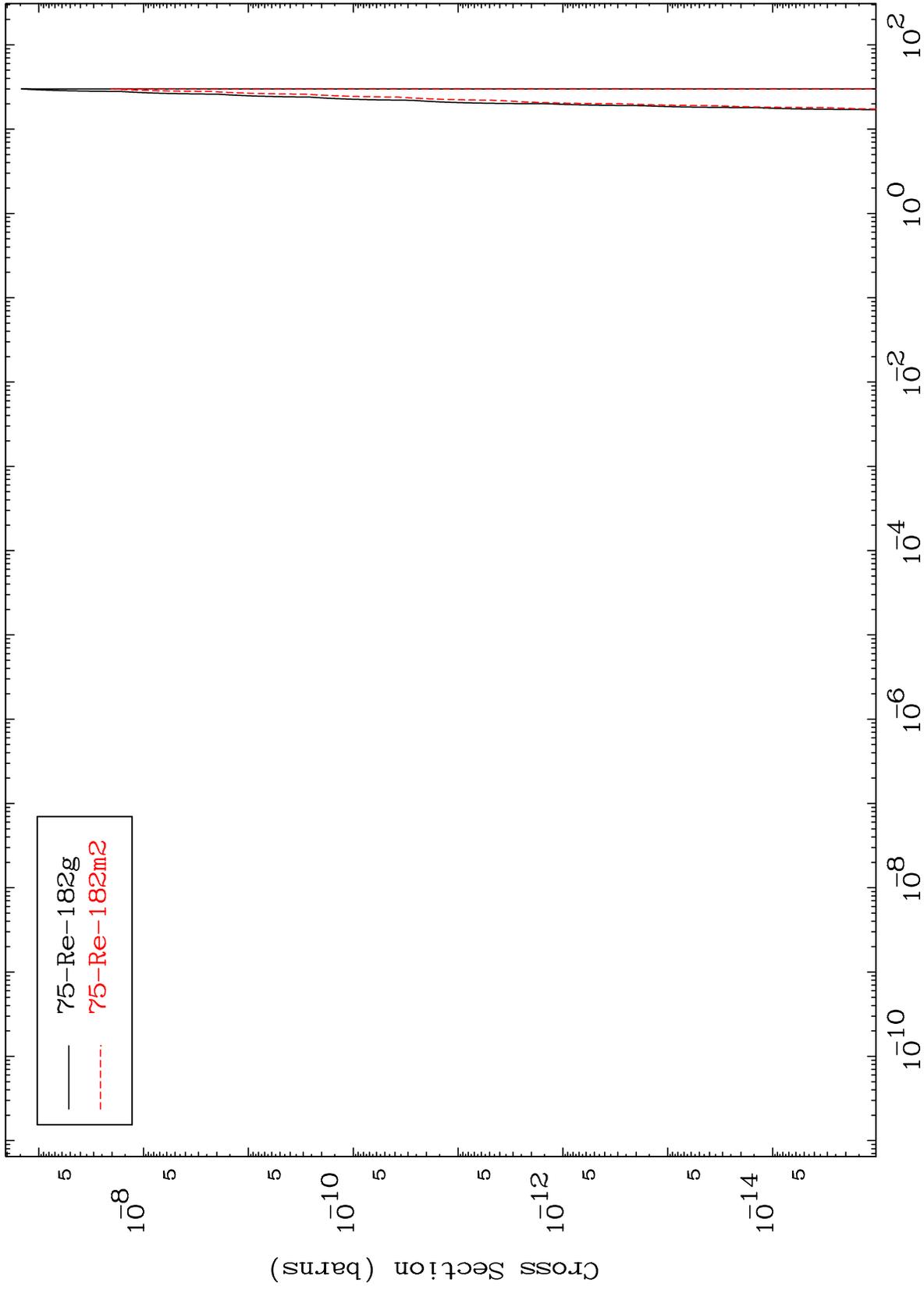
<sup>77</sup>Ir-186

MAT 7710

( $\alpha, 2\alpha$ )

77-Ir-186

Radionuclide Production Cross Section



15

Incident Energy (MeV)

77-Ir-186