

Program EVALPLOT  
(Version 2018-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

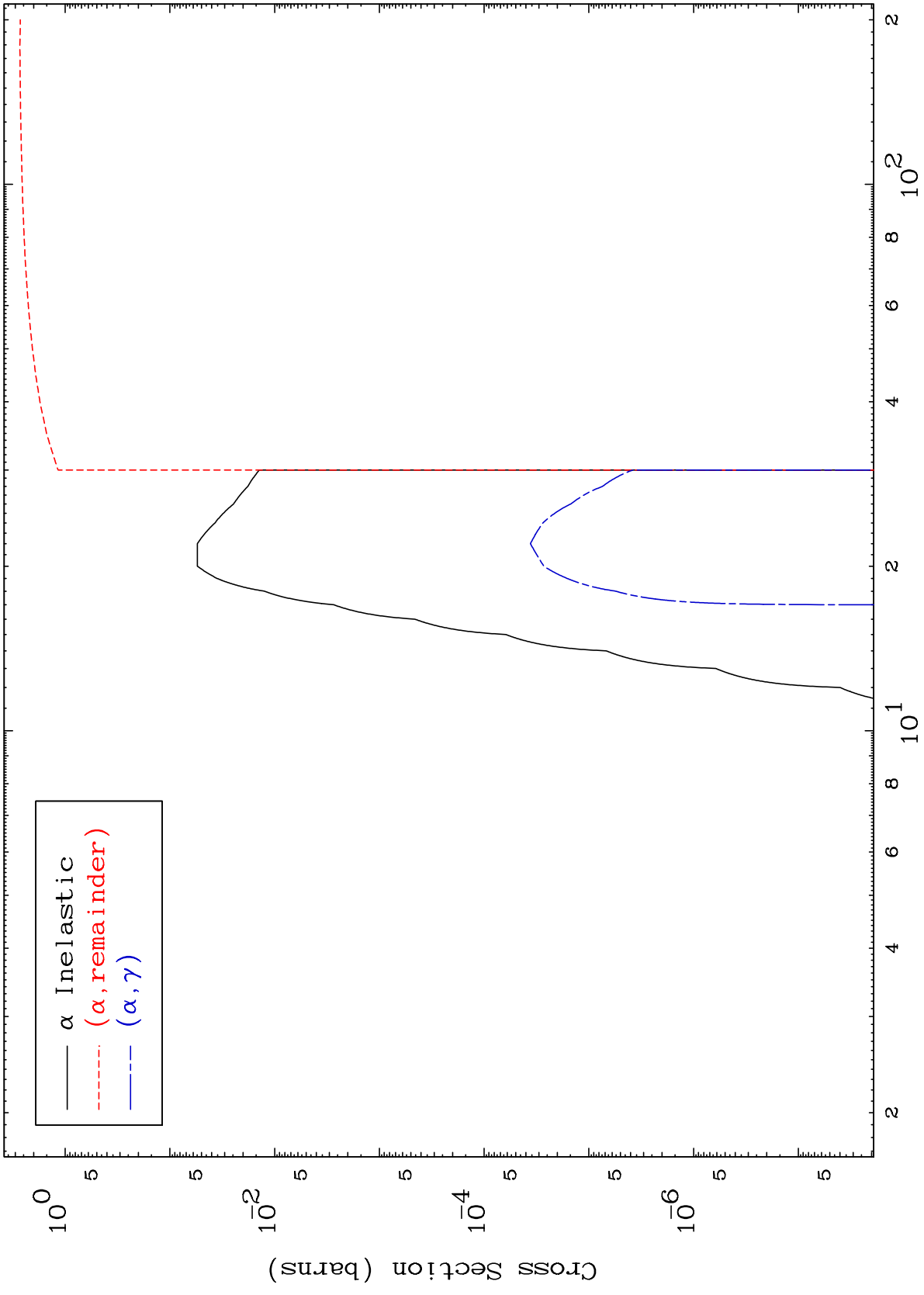
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 7724

0 Kelvin  $\alpha$  Major  
Cross Sections

77-Ir-190

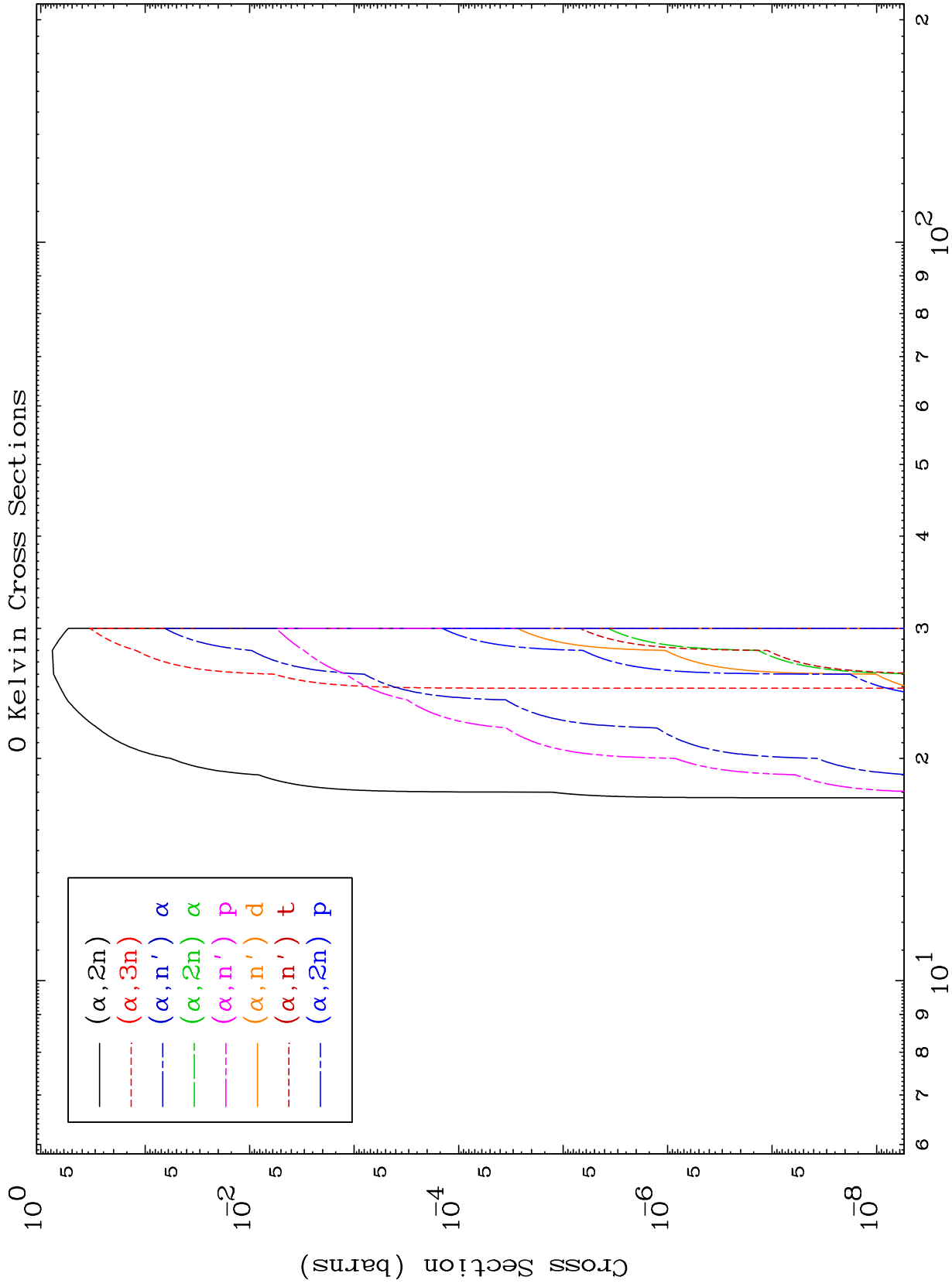


77-Ir-190

MAT 7724

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

77-Ir-190



2

Incident Energy (MeV)

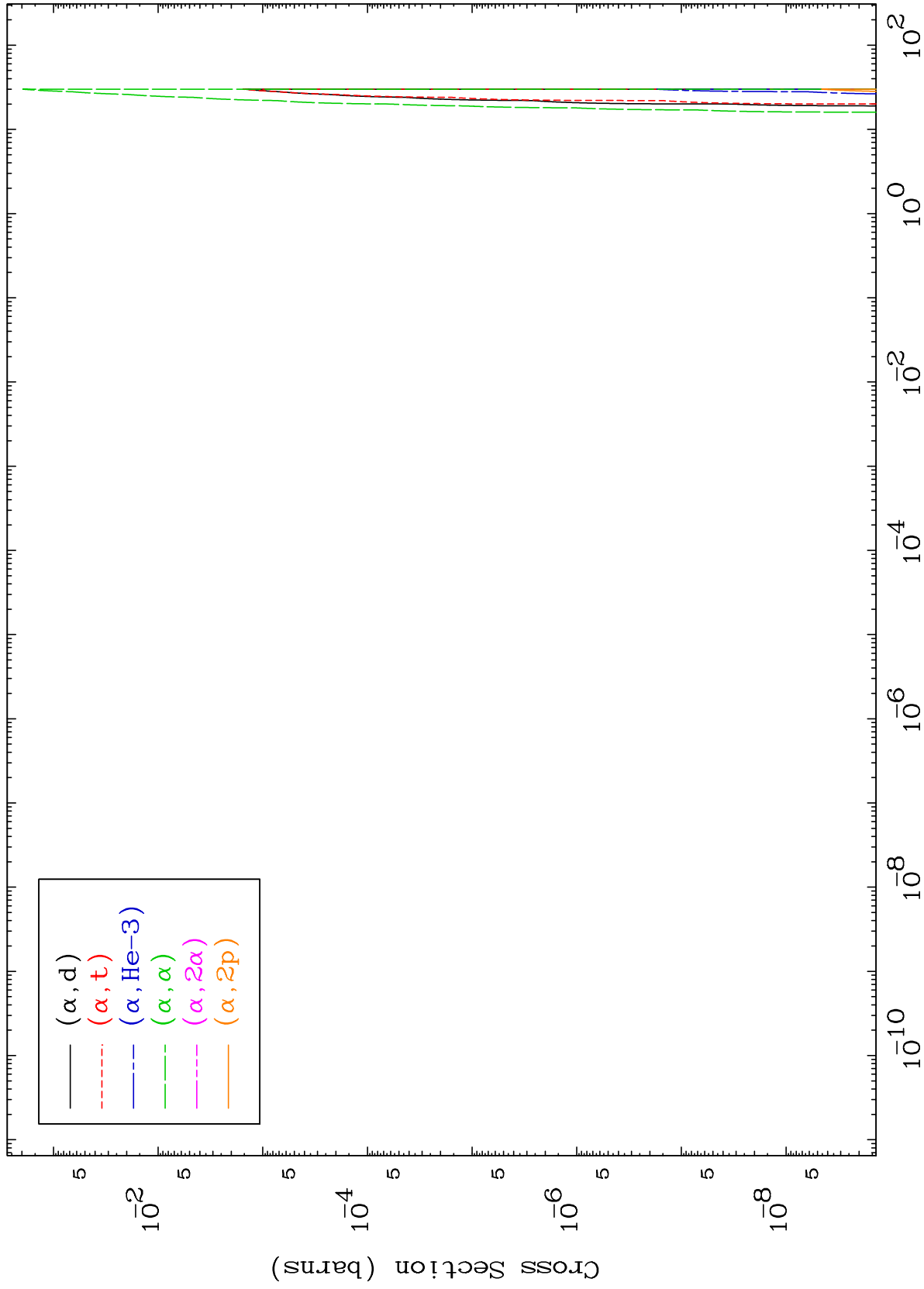
77-Ir-190



MAT 7724

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

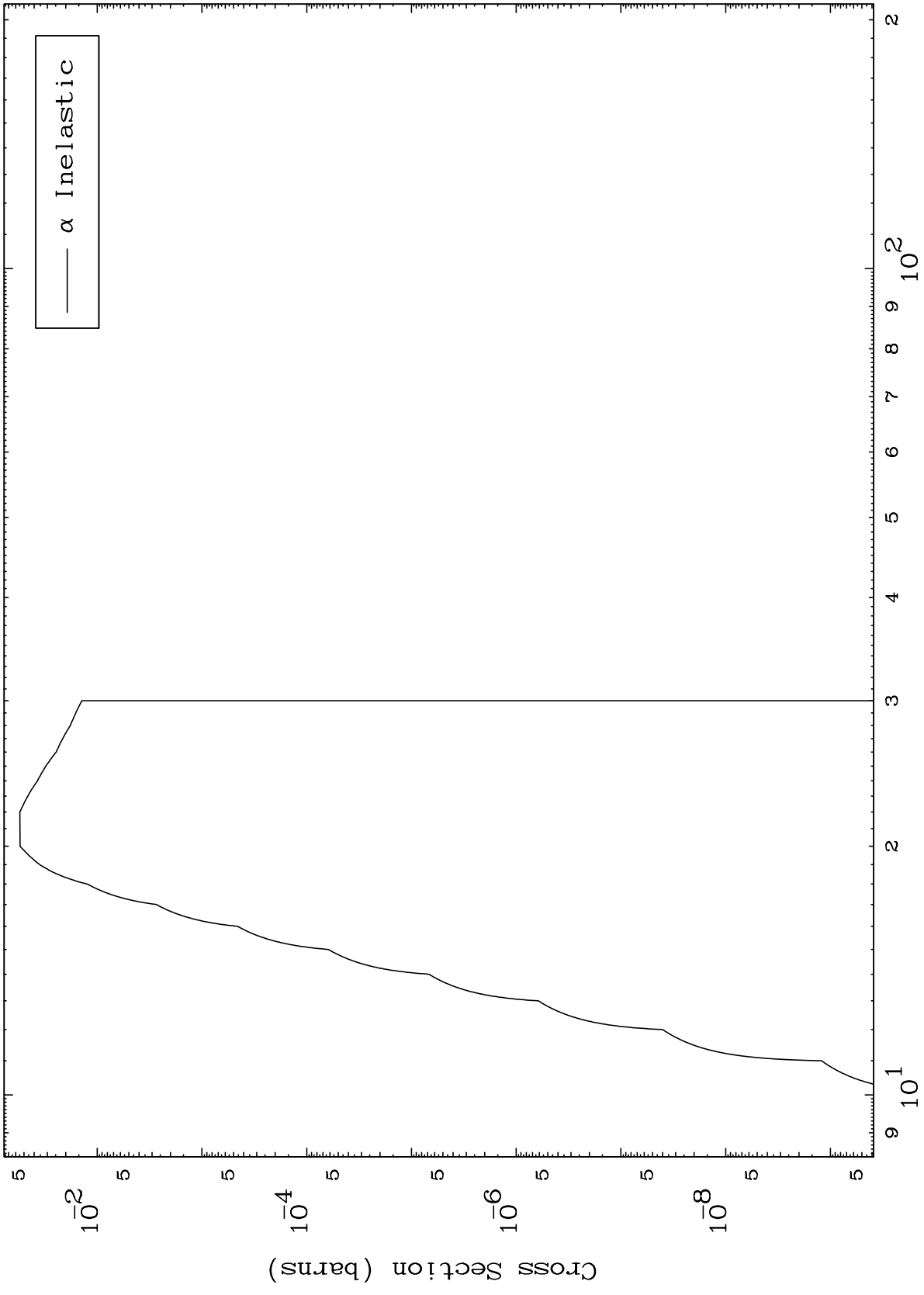
77-Ir-190



MAT 7724

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

77-Ir-190



77-Ir-190

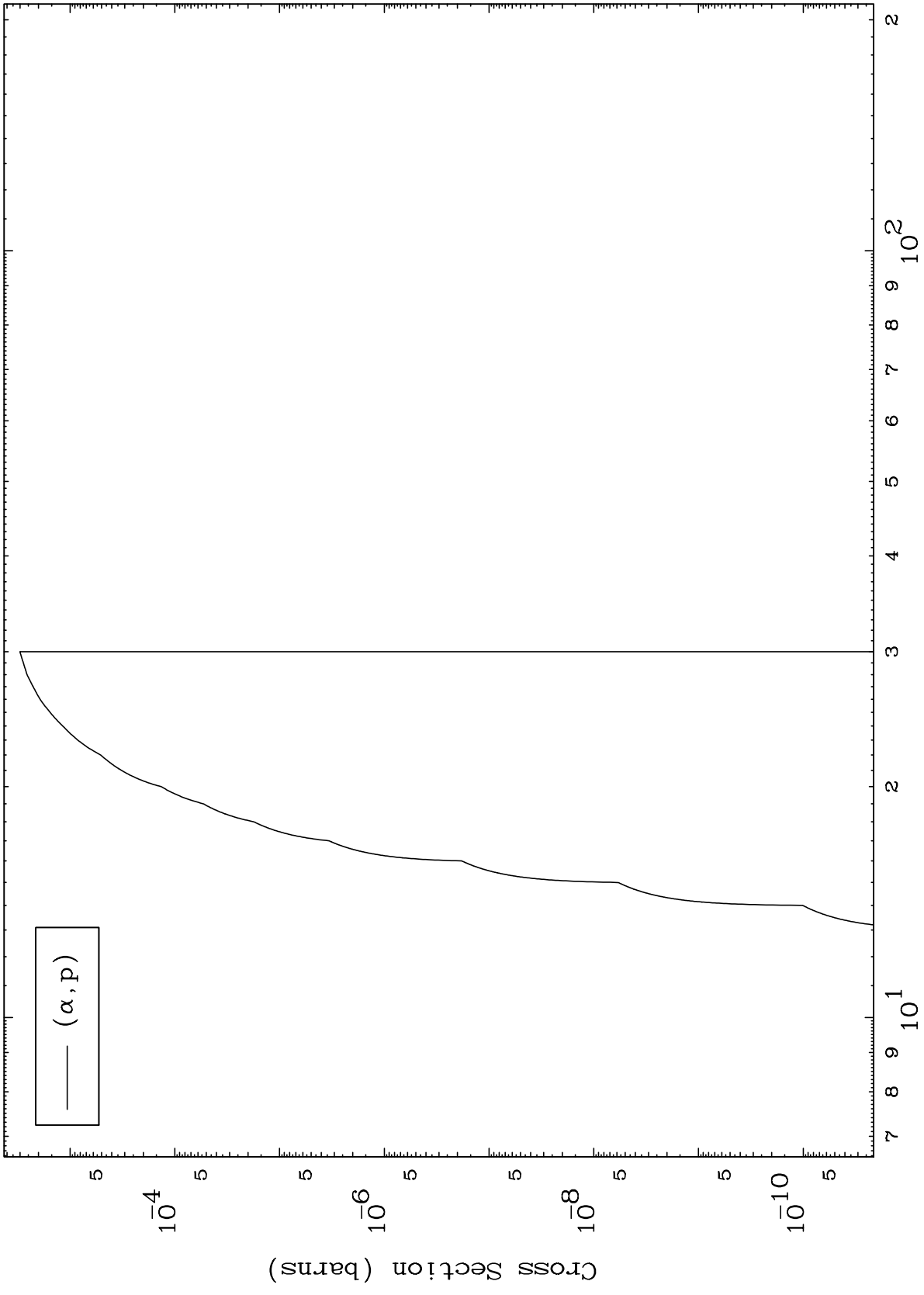
Incident Energy (MeV)

5

MAT 7724

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

77-Ir-190



6

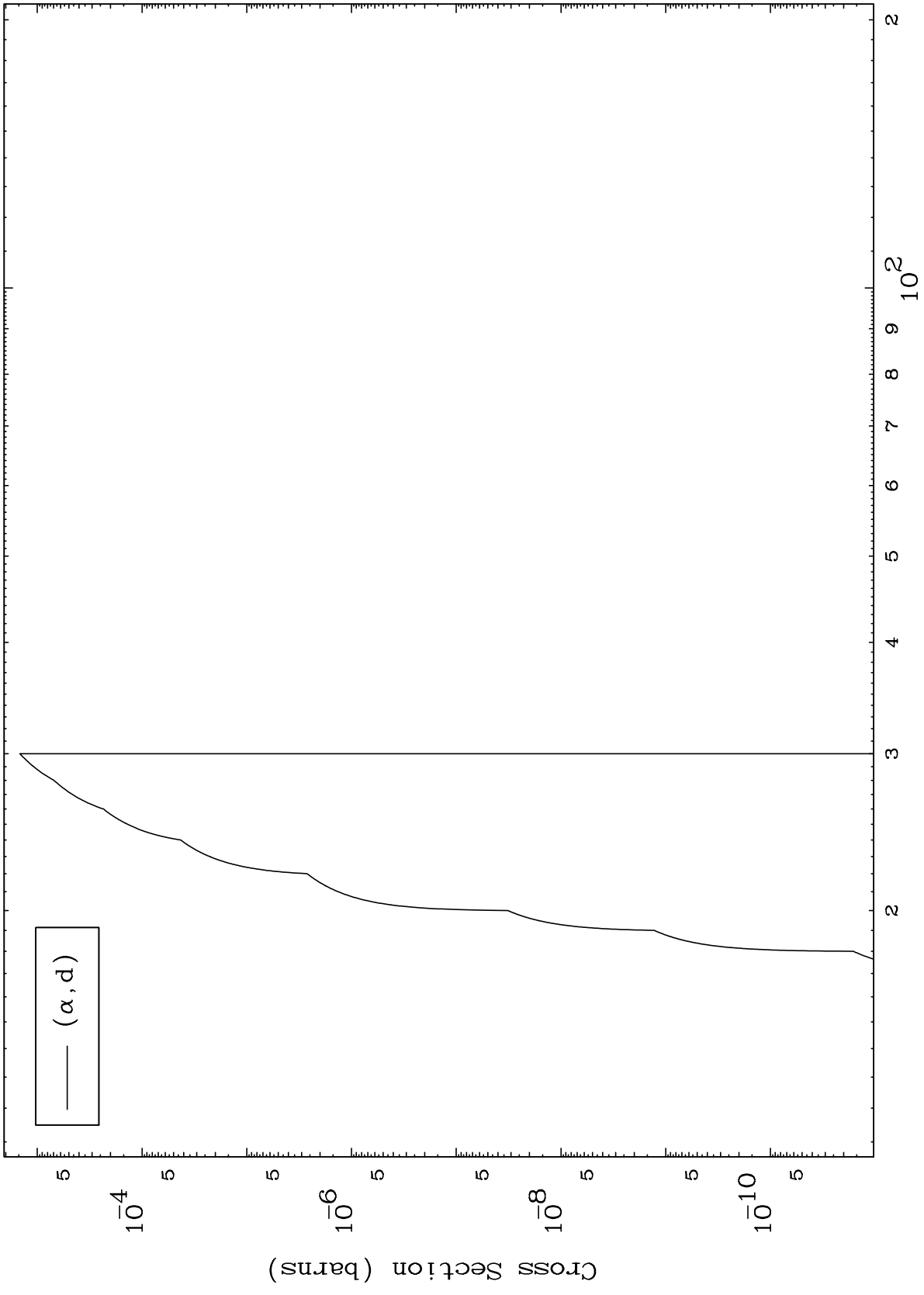
Incident Energy (MeV)

77-Ir-190

MAT 7724

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

77-Ir-190



7

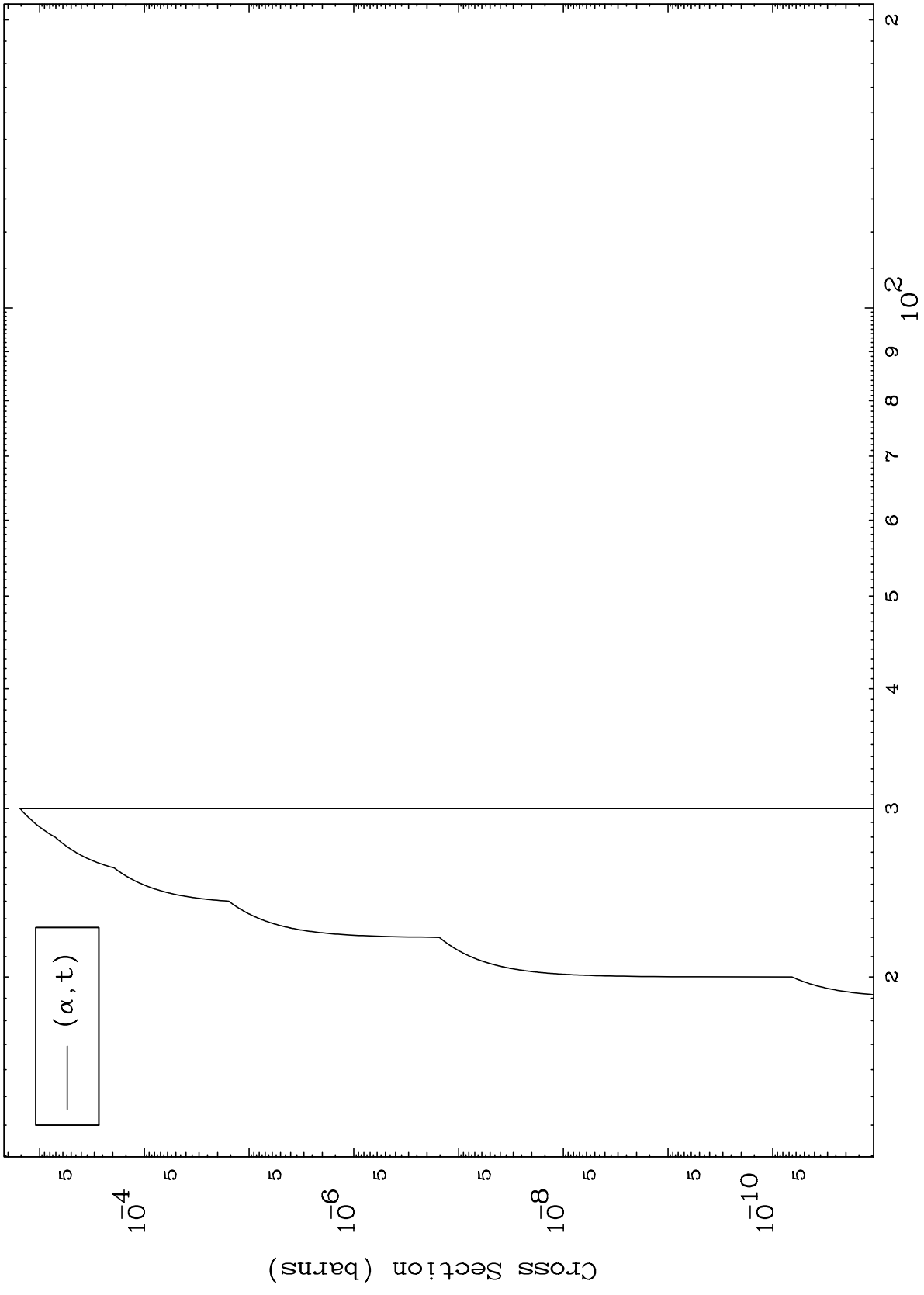
Incident Energy (MeV)

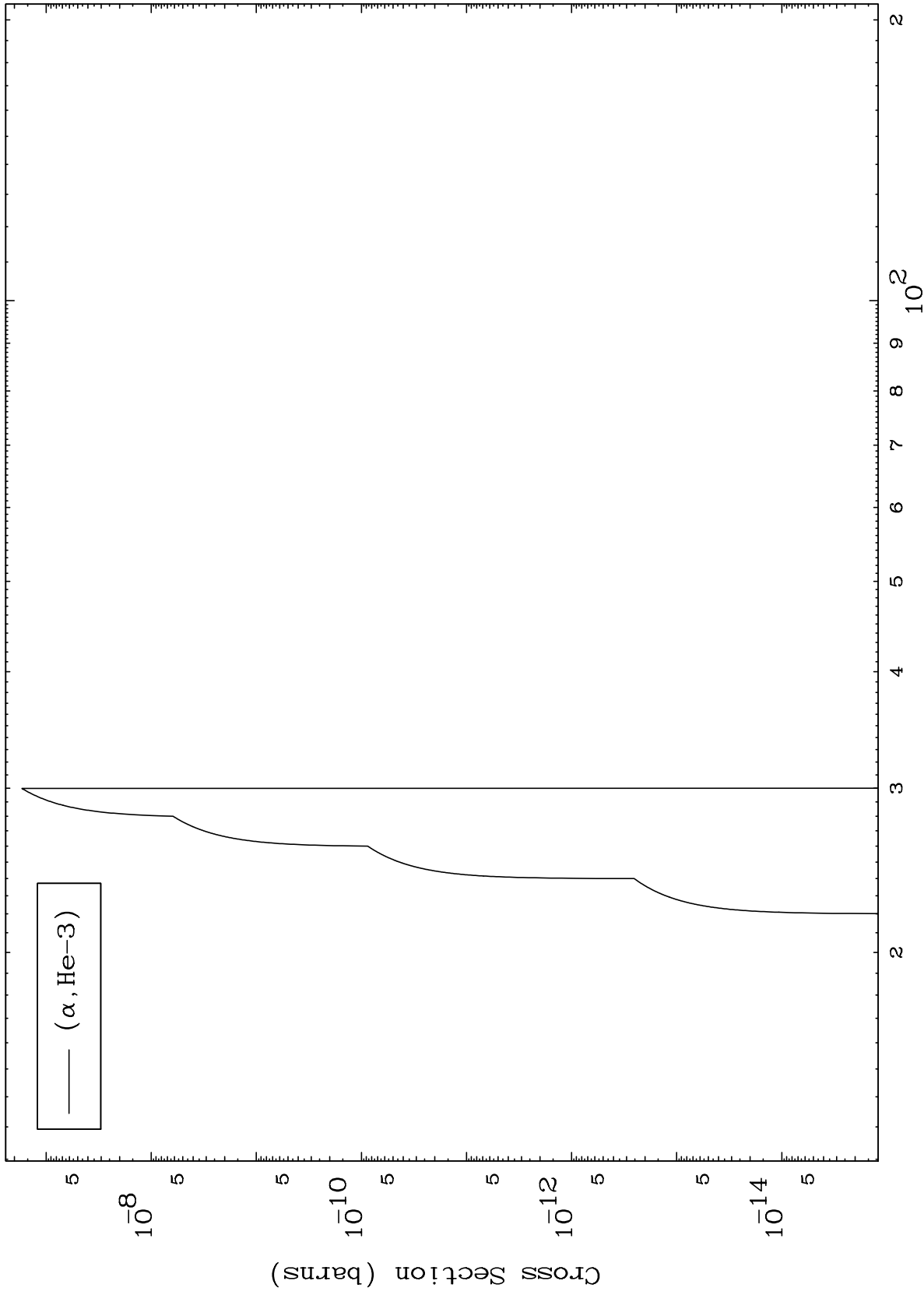
77-Ir-190

MAT 7724

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

77-Ir-190

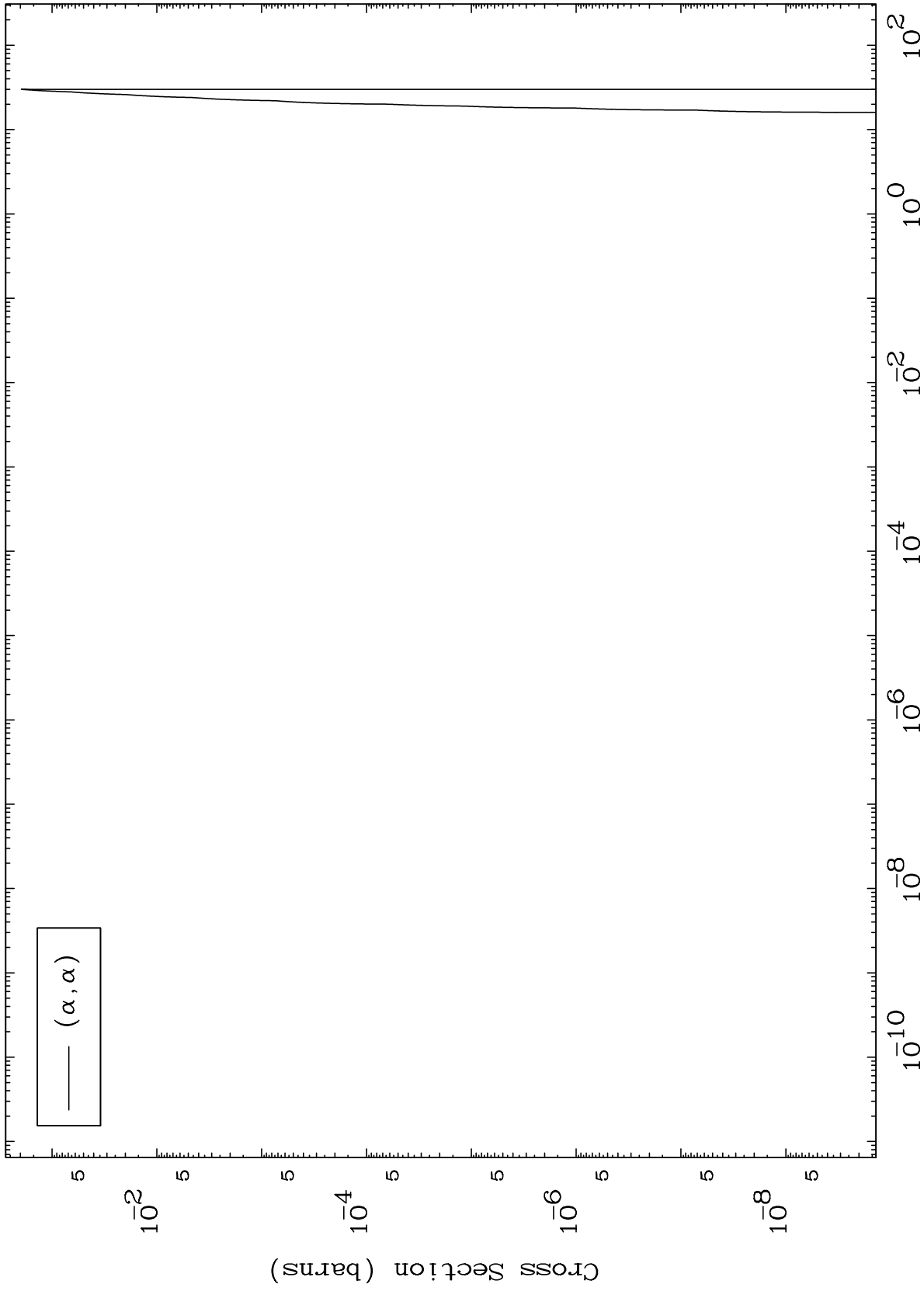




MAT 7724

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

77-Ir-190



10

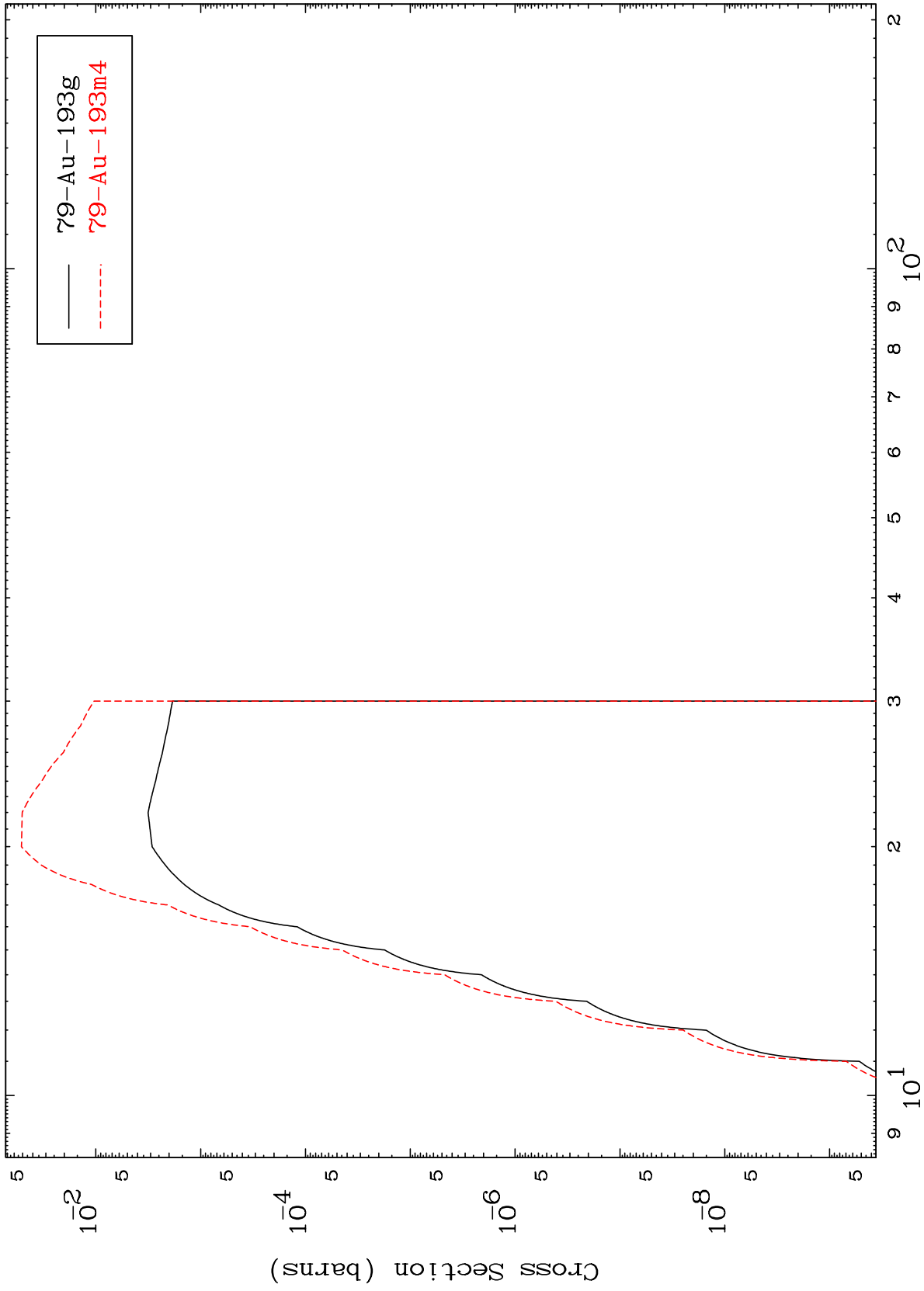
Incident Energy (MeV)

77-Ir-190

MAT 7724

77-Ir-190

$\alpha$  Inelastic  
Radionuclide Production Cross Section



11

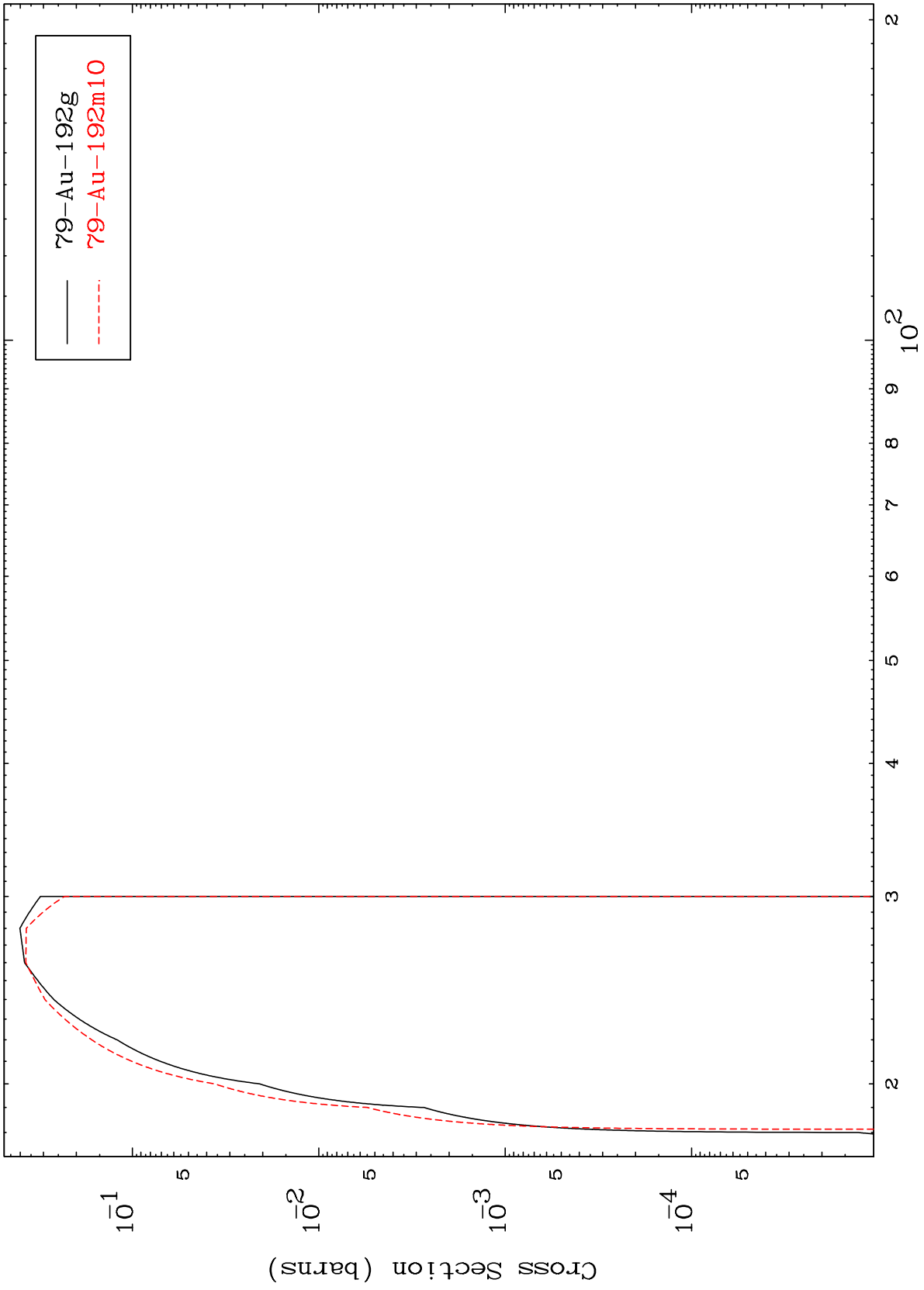
Incident Energy (MeV)

77-Ir-190

MAT 7724

77-Ir-190

( $\alpha, 2n$ )  
Radionuclide Production Cross Section



12

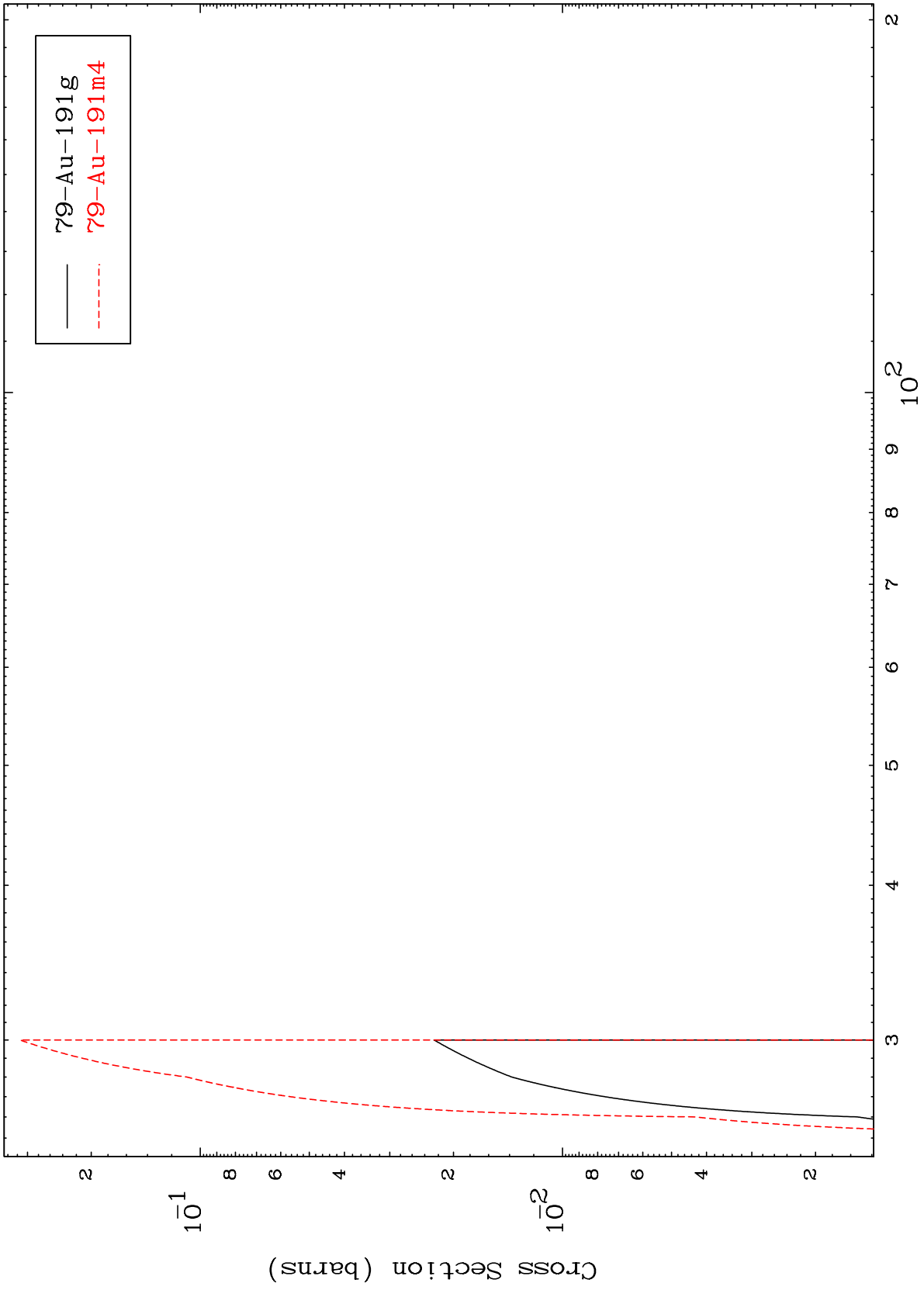
Incident Energy (MeV)

77-Ir-190

MAT 7724

77-Ir-190

( $\alpha, 3n$ )  
Radionuclide Production Cross Section



13

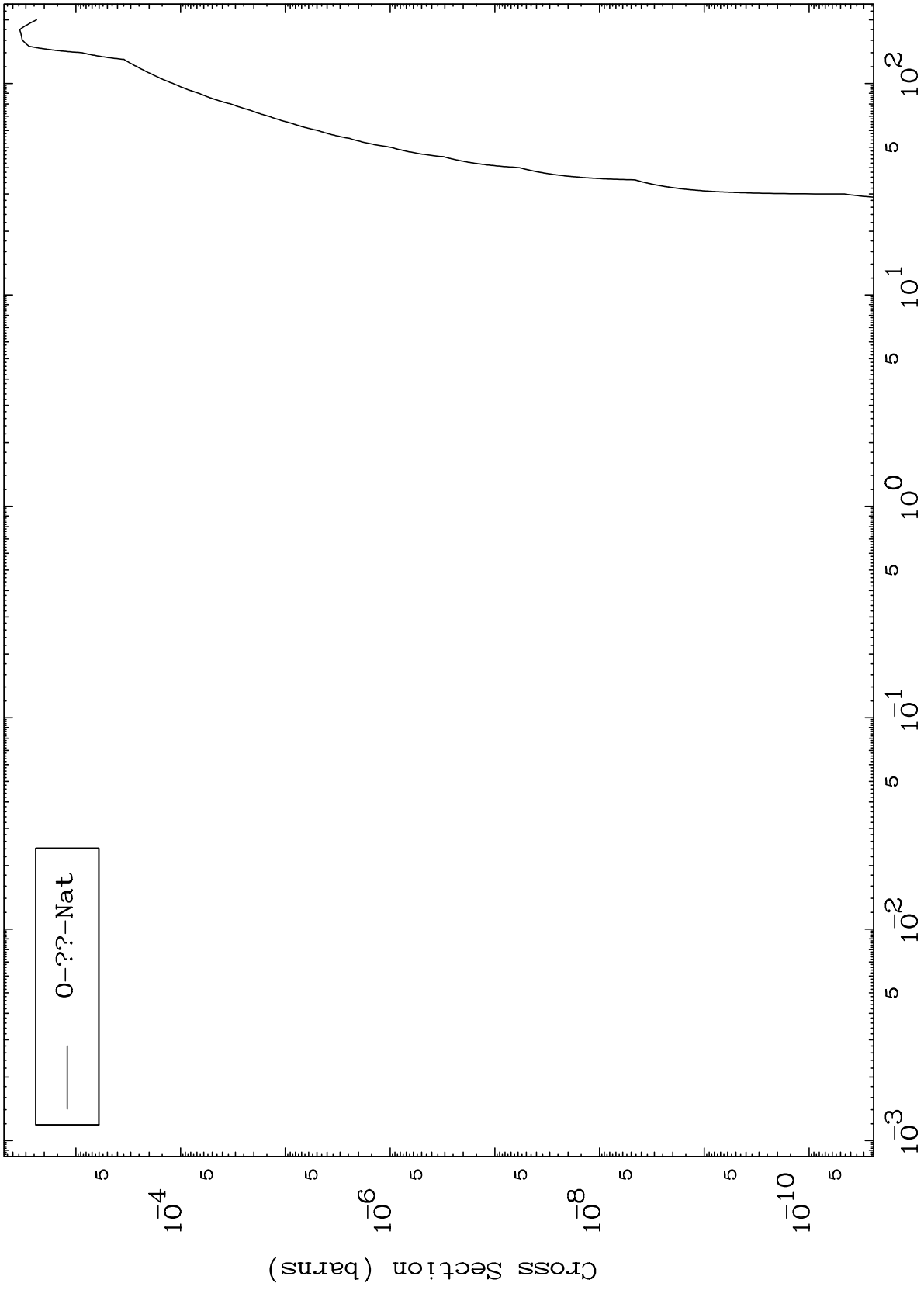
Incident Energy (MeV)

77-Ir-190

MAT 7724

$\alpha$  Fission  
Radionuclide Production Cross Section

77-Ir-190



14

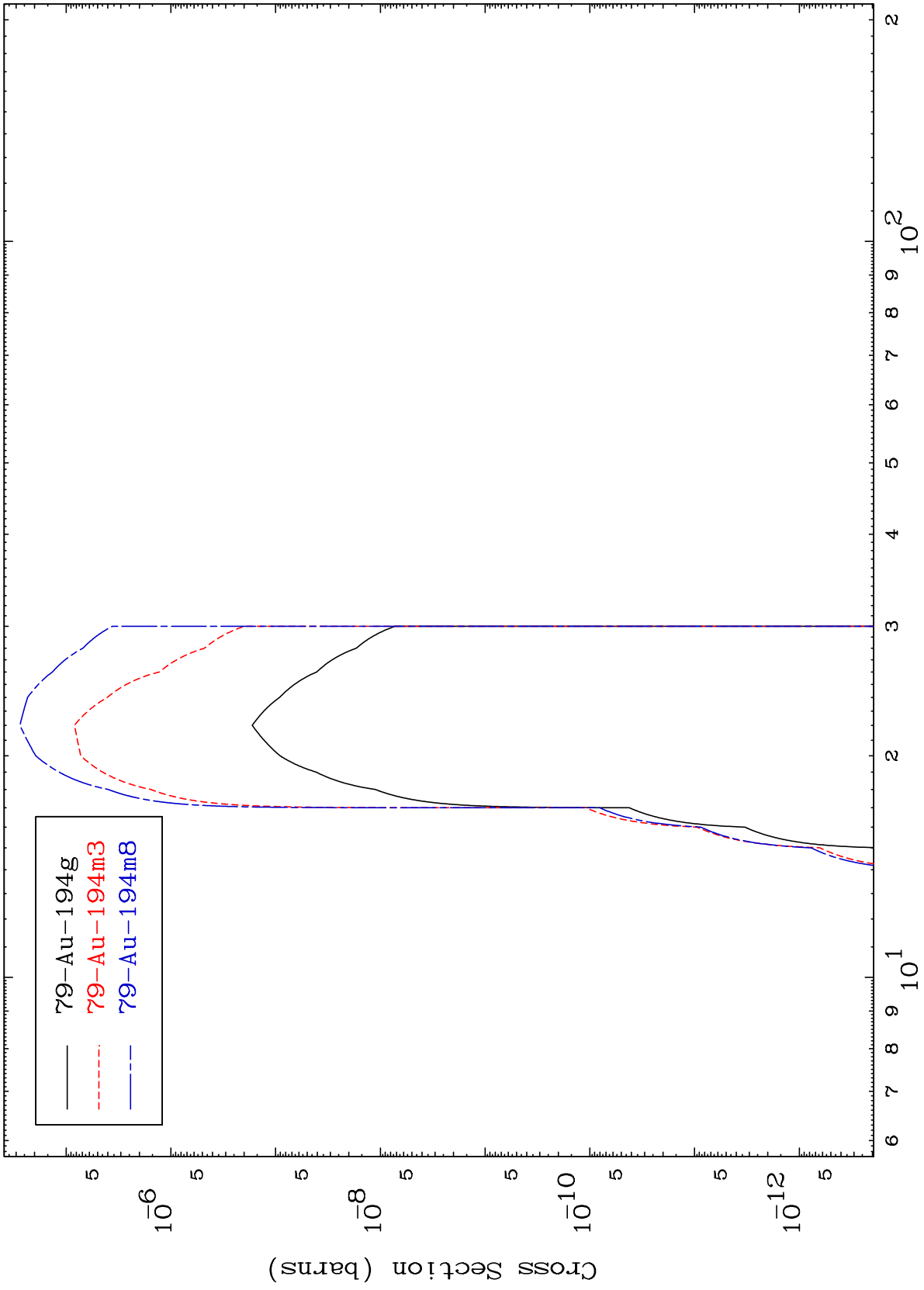
Incident Energy (MeV)

77-Ir-190

MAT 7724

<sup>77</sup>Ir-190

( $\alpha, \gamma$ )  
Radionuclide Production Cross Section



15

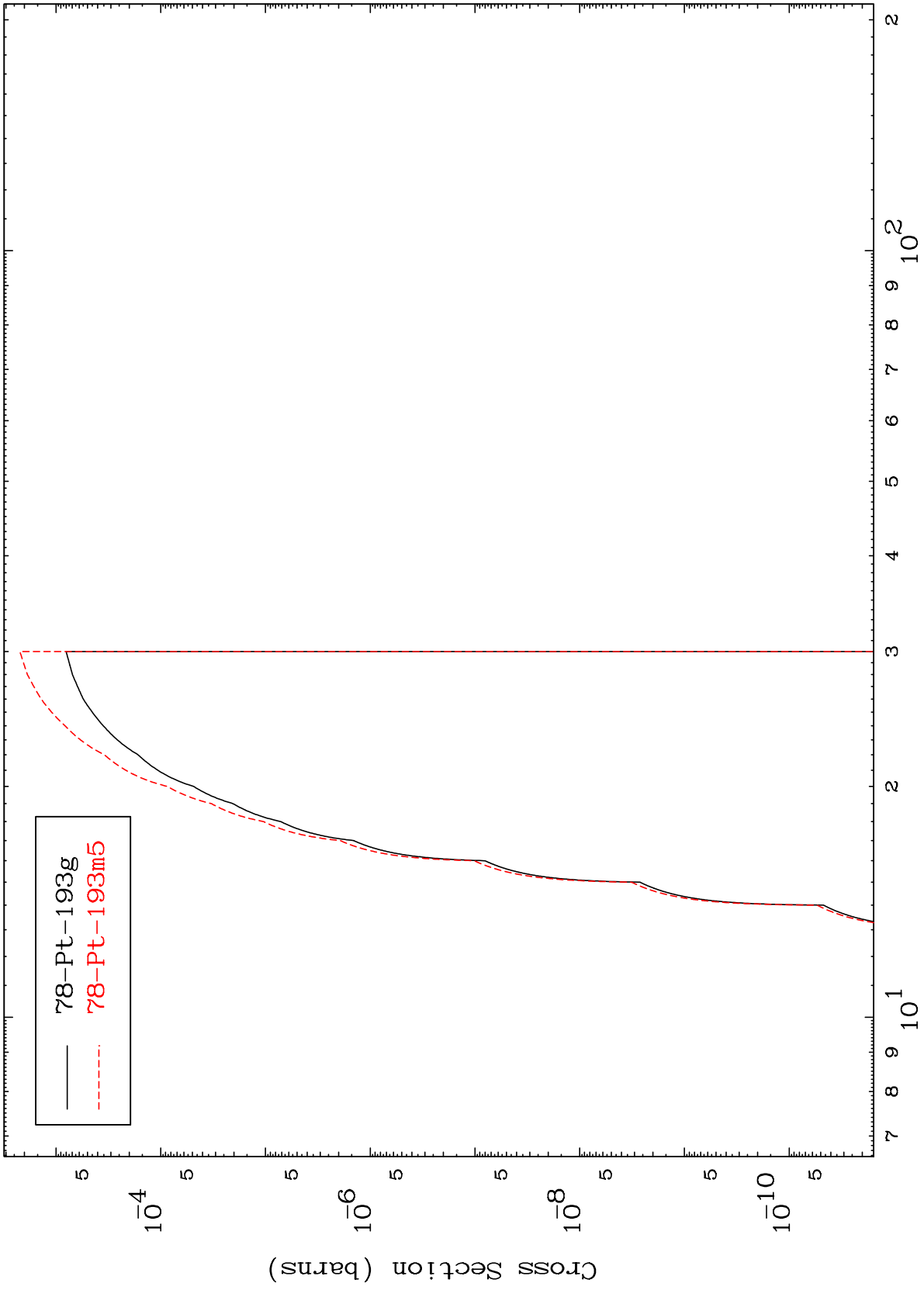
Incident Energy (MeV)

<sup>77</sup>Ir-190

MAT 7724

77-Ir-190

Radionuclide Production Cross Section



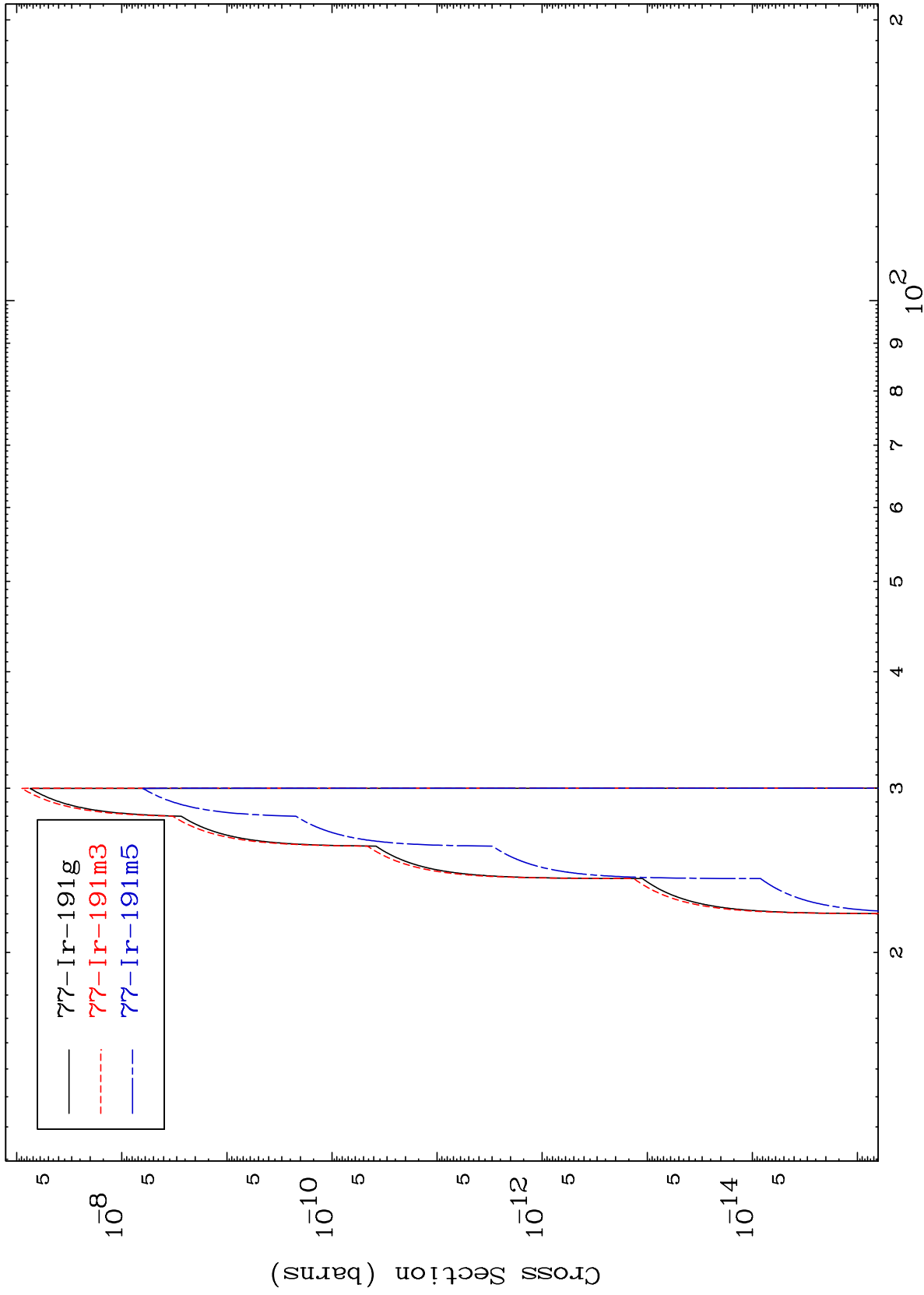
16

MAT 7724

( $\alpha, \text{He-3}$ )

<sup>77</sup>Ir-190

Radionuclide Production Cross Section



17

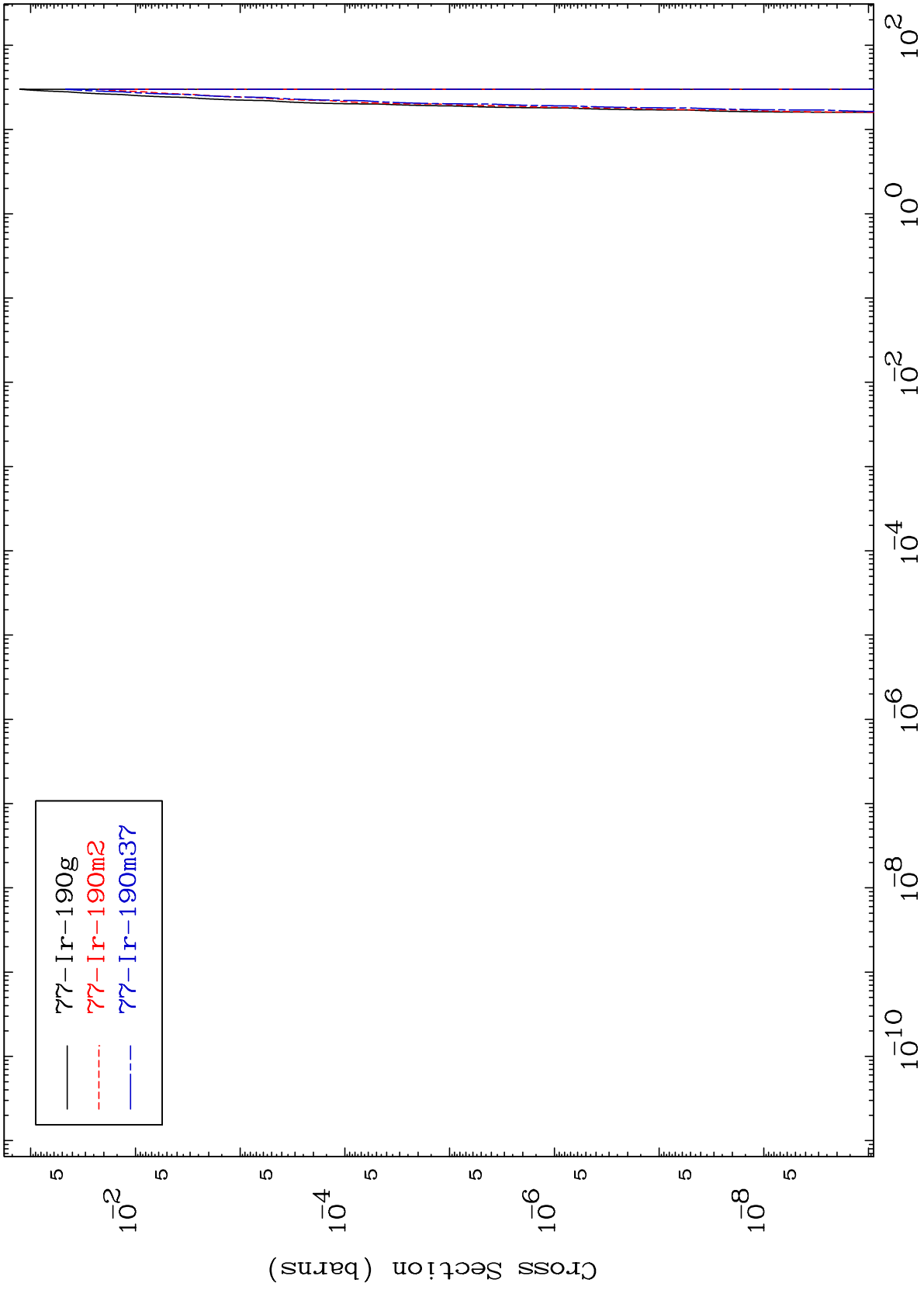
Incident Energy (MeV)

<sup>77</sup>Ir-190

MAT 7724

$(\alpha, \alpha)$   
Radionuclide Production Cross Section

$^{77}\text{Ir-190}$



18

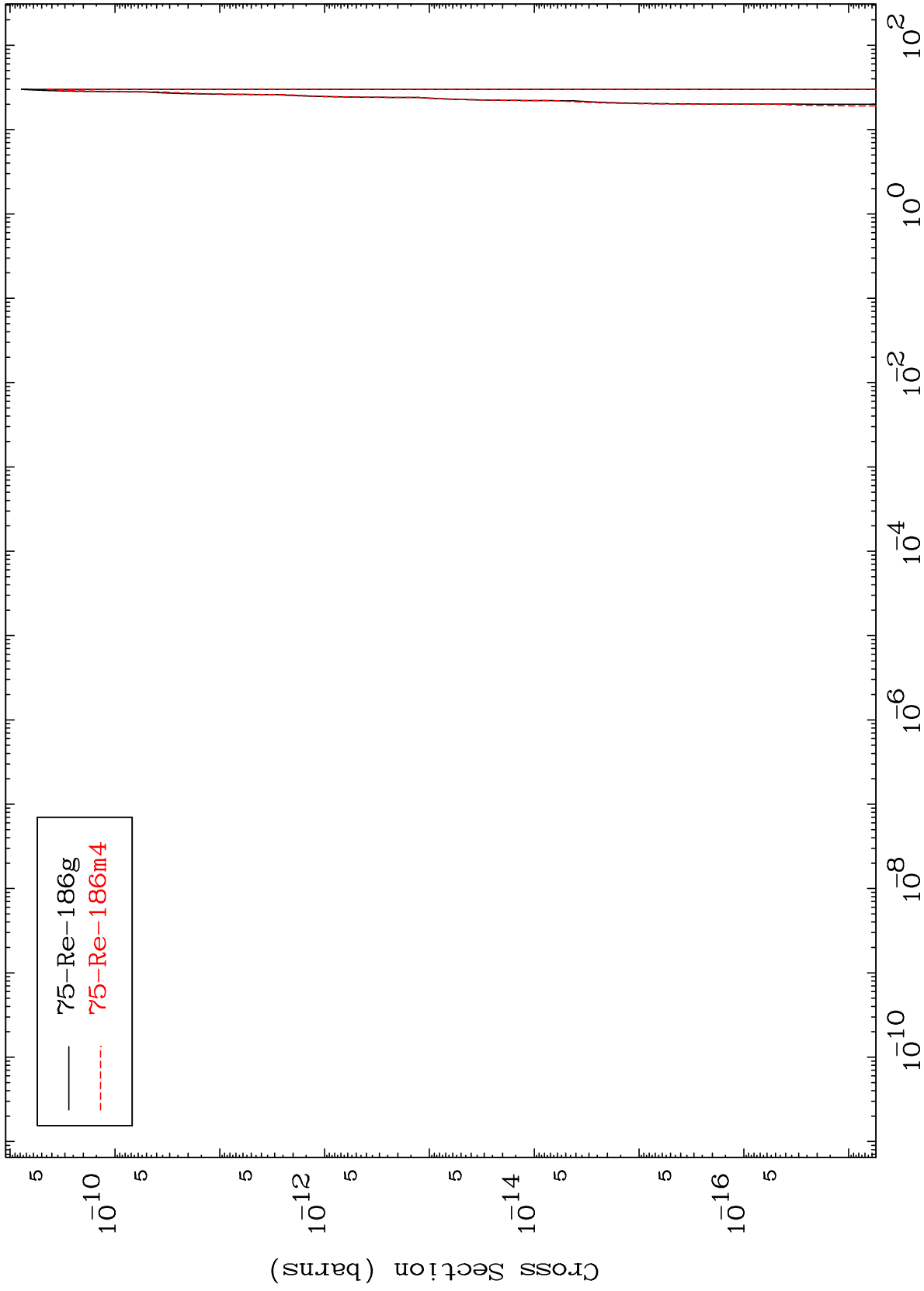
$^{77}\text{Ir-190}$

MAT 7724

( $\alpha, 2\alpha$ )

77-Ir-190

Radionuclide Production Cross Section



19

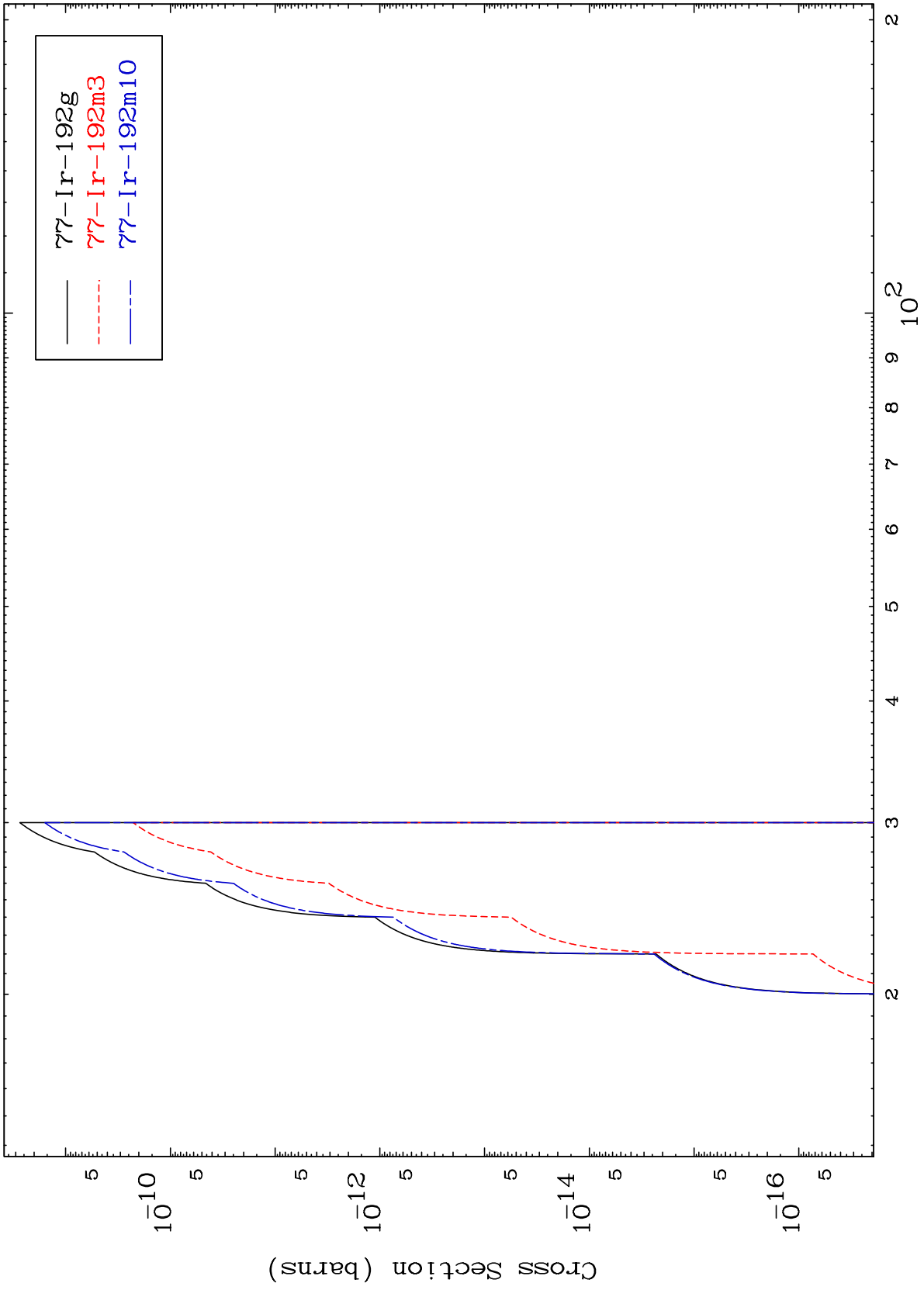
Incident Energy (MeV)

77-Ir-190

MAT 7724

77-Ir-190

( $\alpha, 2p$ )  
Radionuclide Production Cross Section



20

77-Ir-190

Incident Energy (MeV)

