

Program EVALPLOT
(Version 2018-1)

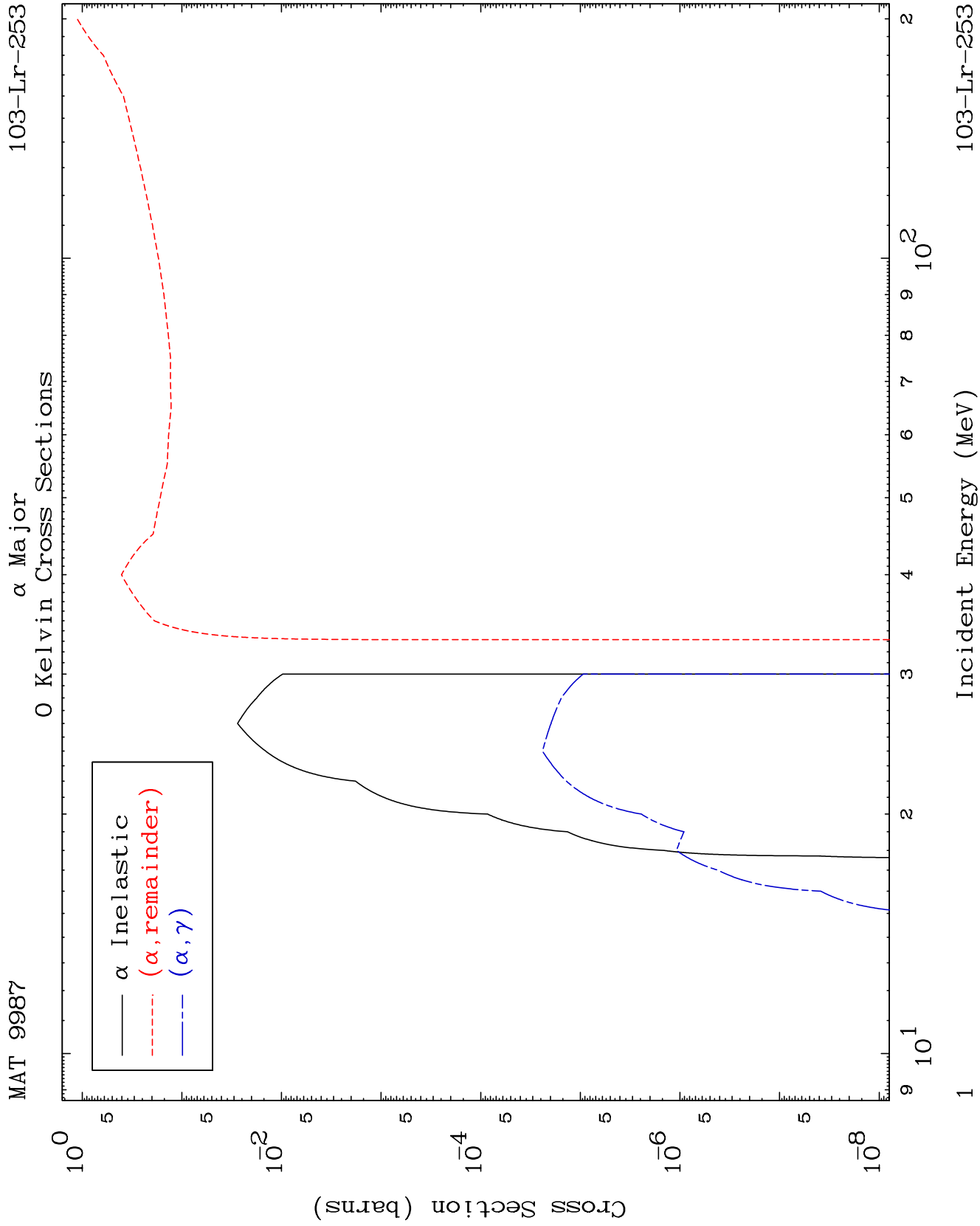
by

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Web: redcullen1.net/HOMEPAGE.NEW

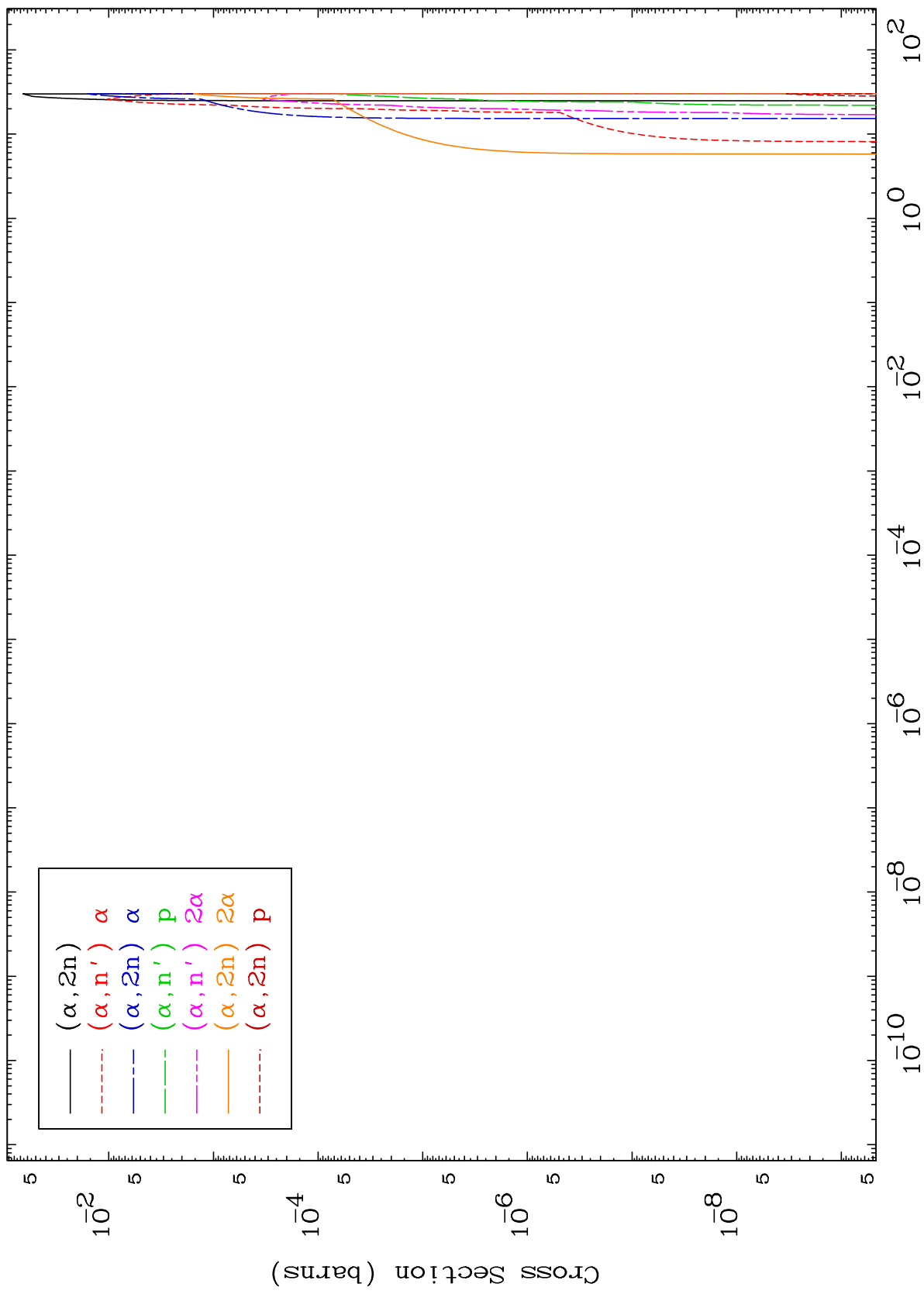
Press Mouse Button to Start



MAT 9987

α Neutron Production
0 Kelvin Cross Sections

103-Lr-253



2

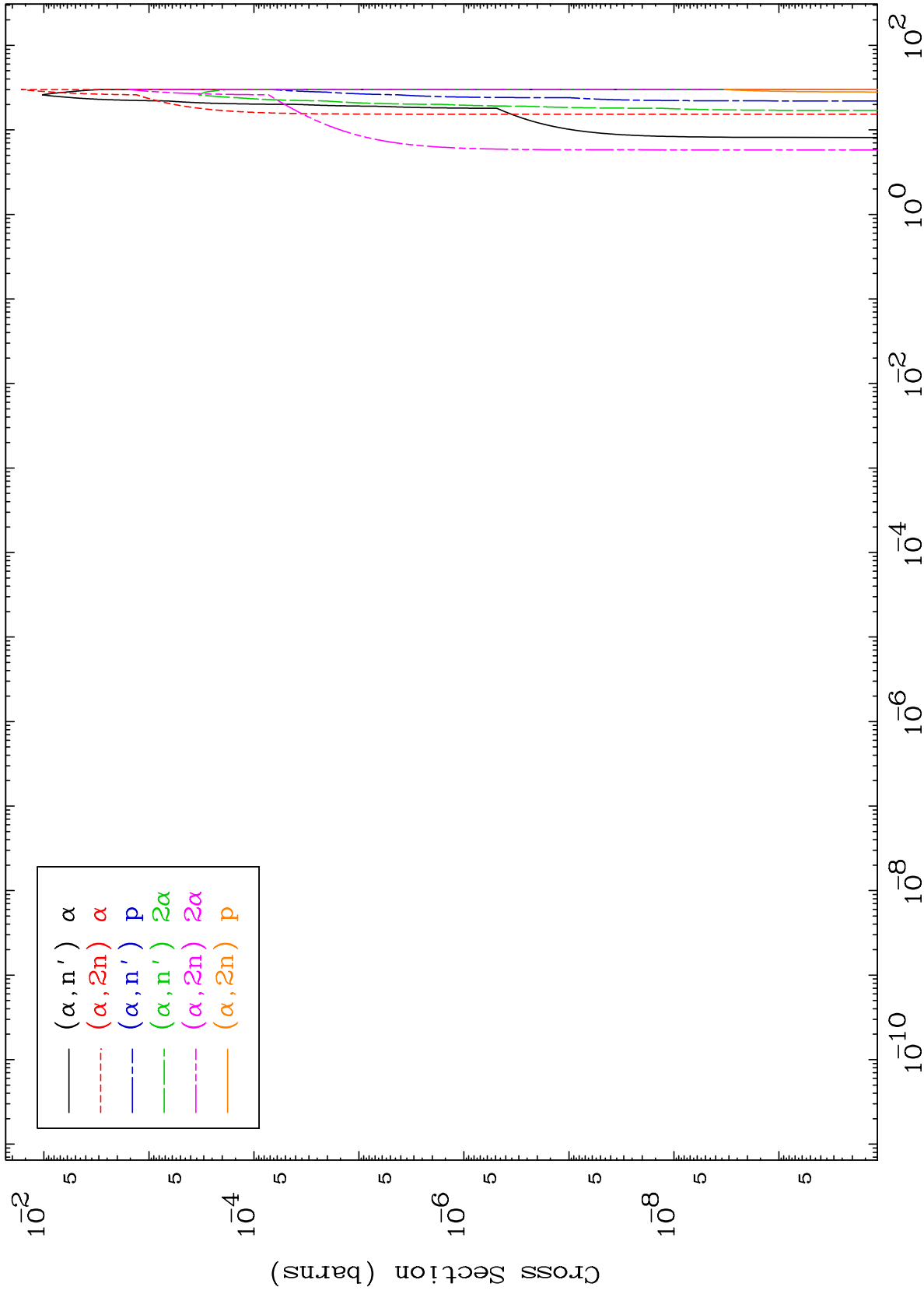
Incident Energy (MeV)

103-Lr-253

MAT 9987

α Charged Particle
0 Kelvin Cross Sections

103-Lr-253



3

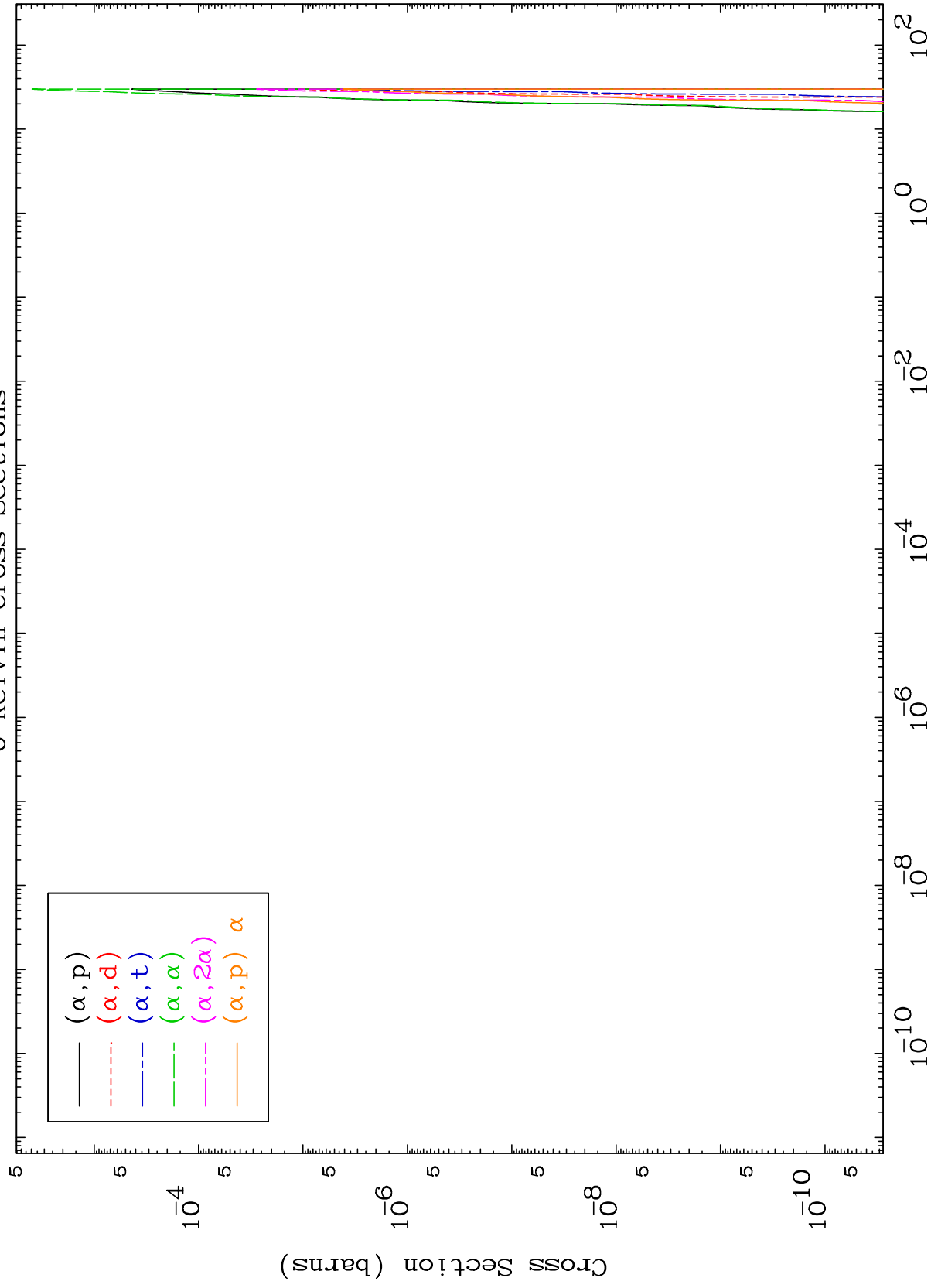
Incident Energy (MeV)

103-Lr-253

MAT 9987

α Charged Particle
0 Kelvin Cross Sections

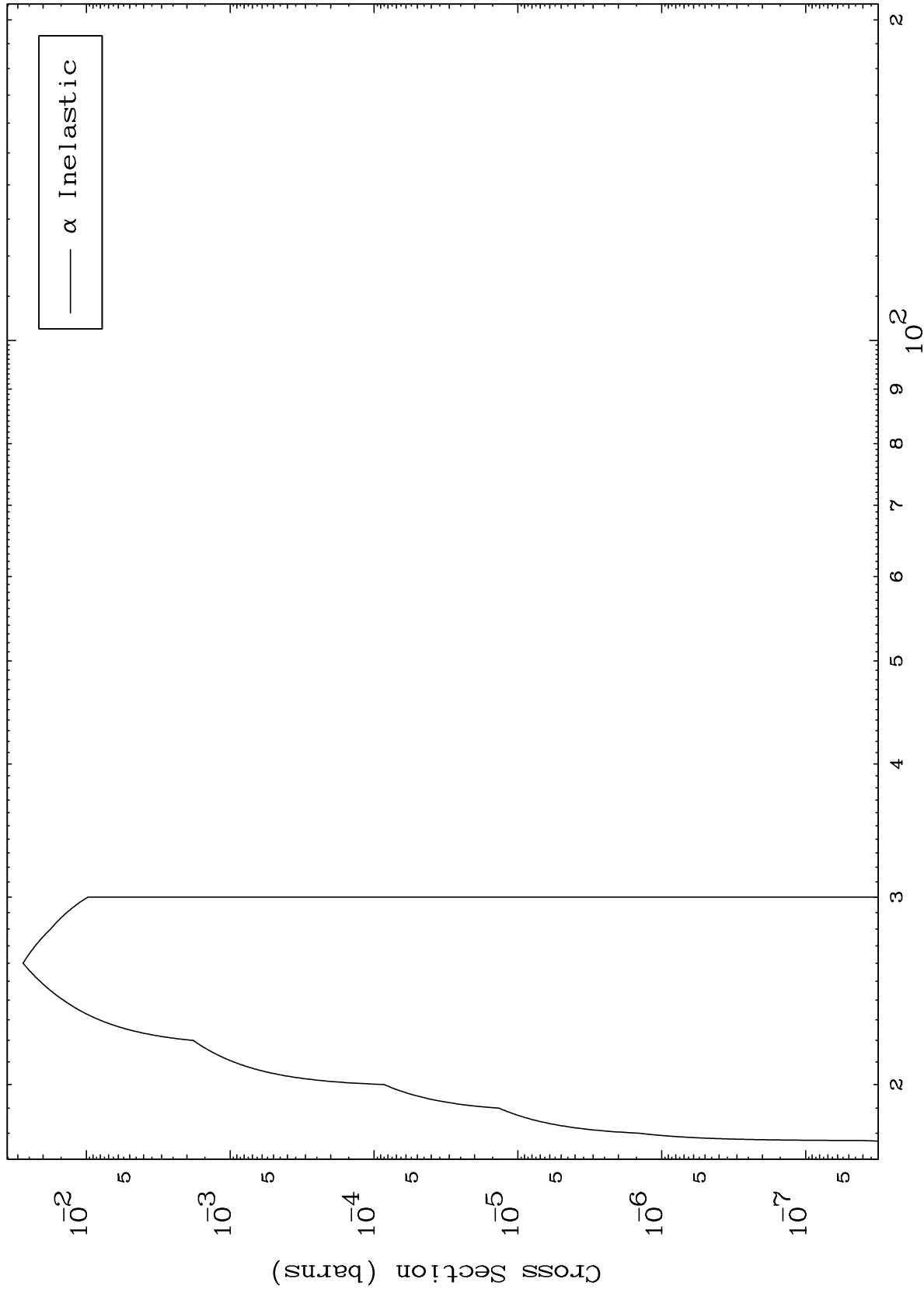
103-Lr-253



MAT 9987

(α, n') Level
0 Kelvin Cross Sections

103-Lr-253



5

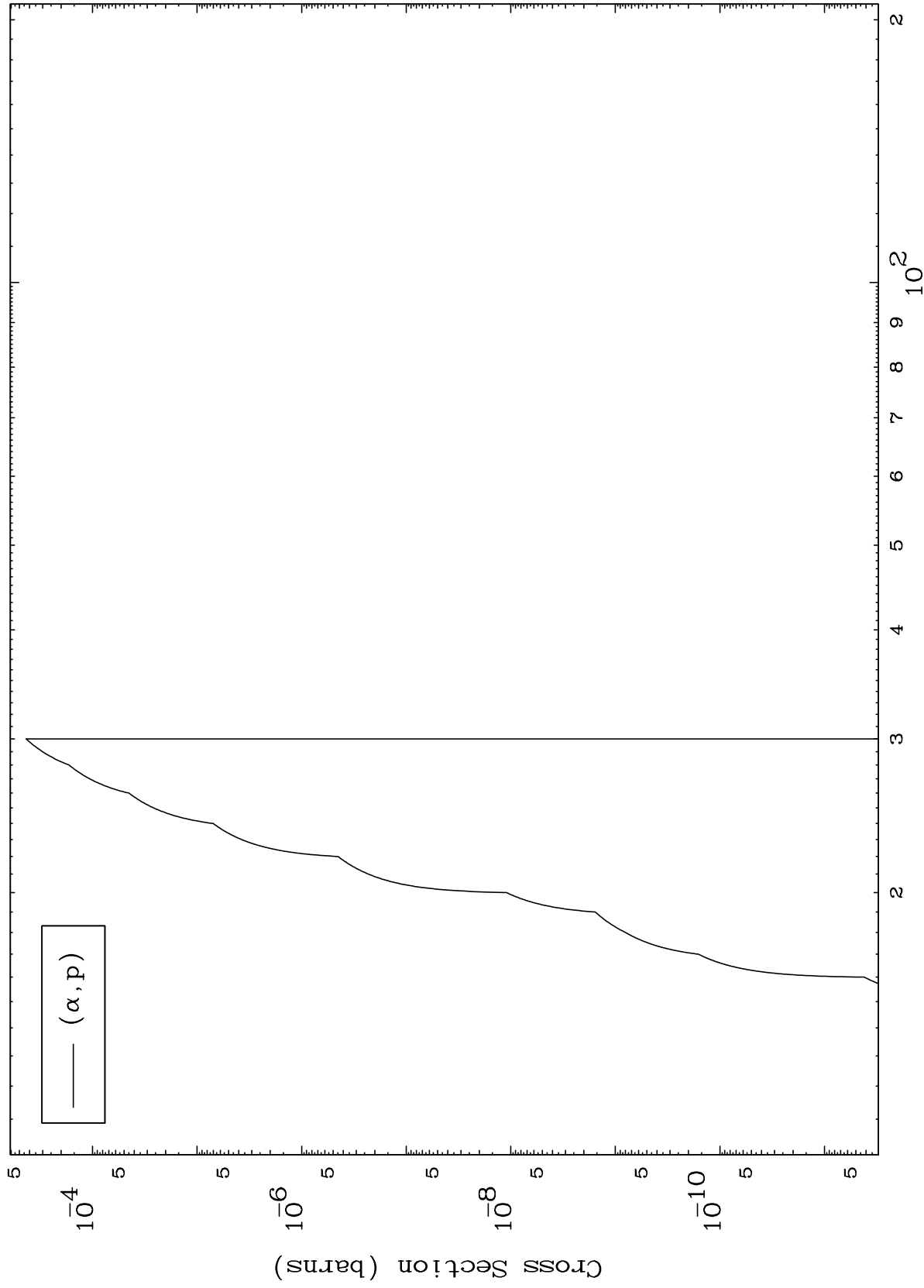
Incident Energy (MeV)

103-Lr-253

MAT 9987

103-Lr-253

(α ,p) Levels
0 Kelvin Cross Sections



6

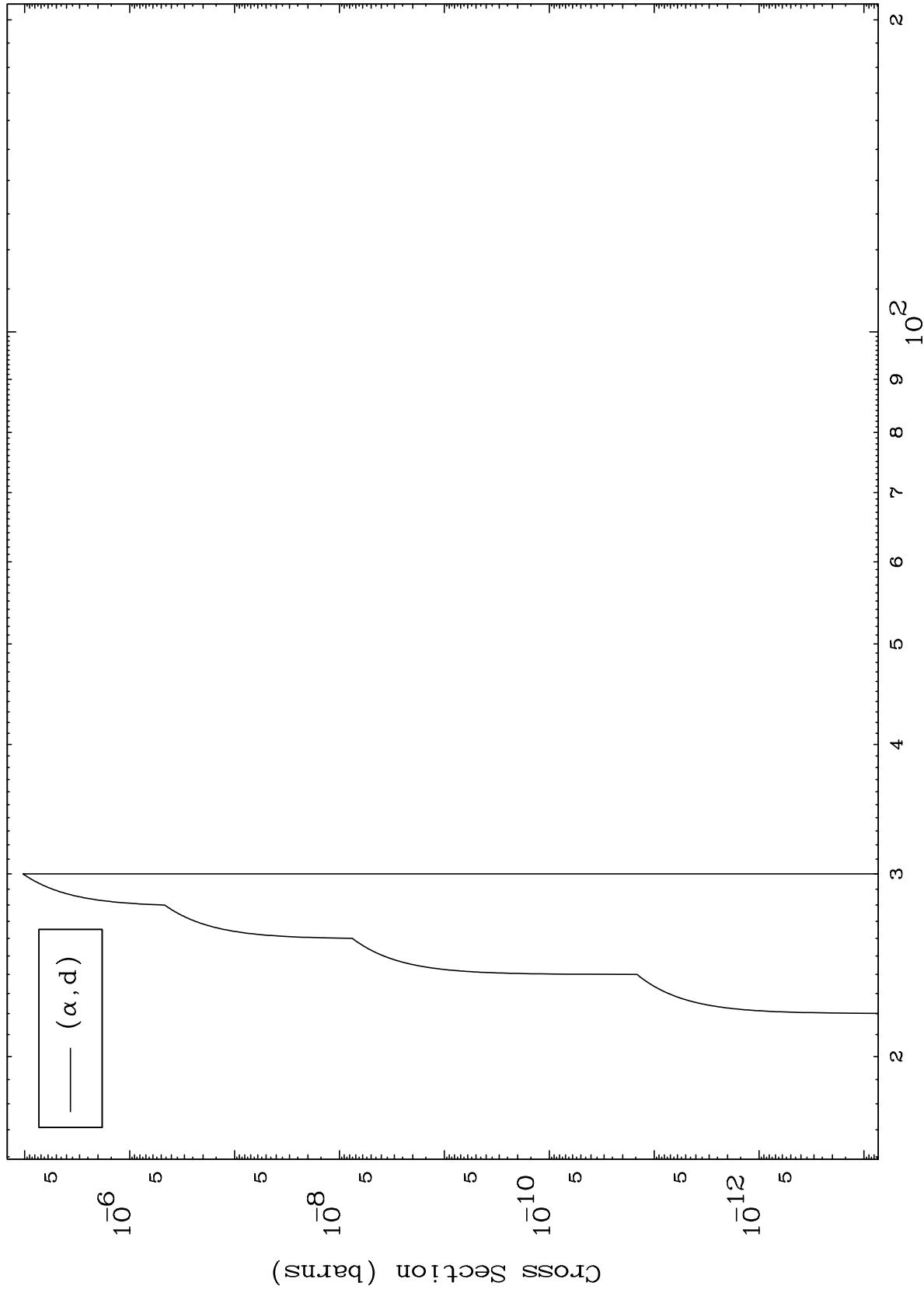
Incident Energy (MeV)

103-Lr-253

MAT 9987

(α ,d) Levels
0 Kelvin Cross Sections

103-Lr-253



7

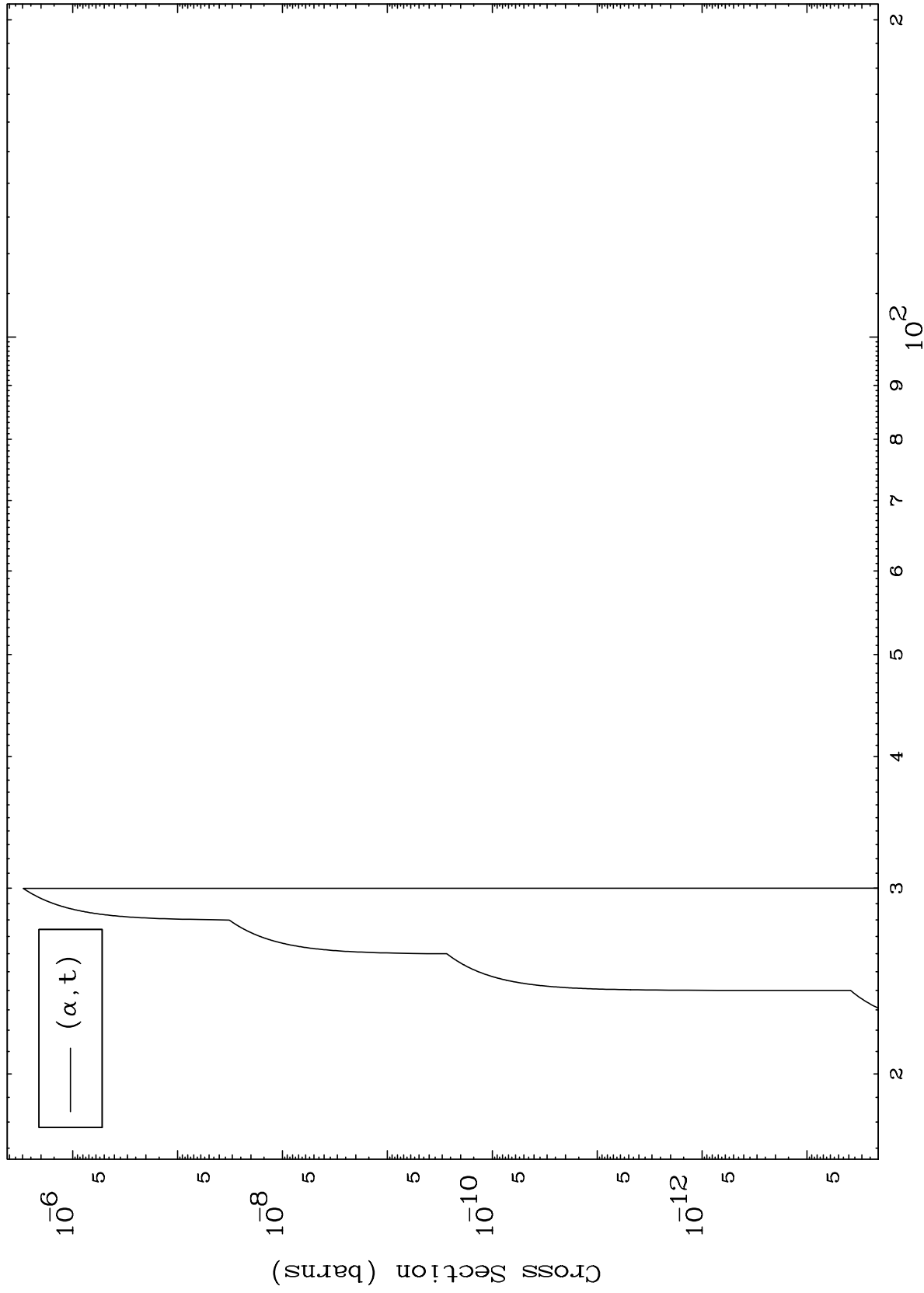
Incident Energy (MeV)

103-Lr-253

MAT 9987

(α ,t) Levels
0 Kelvin Cross Sections

103-Lr-253



8

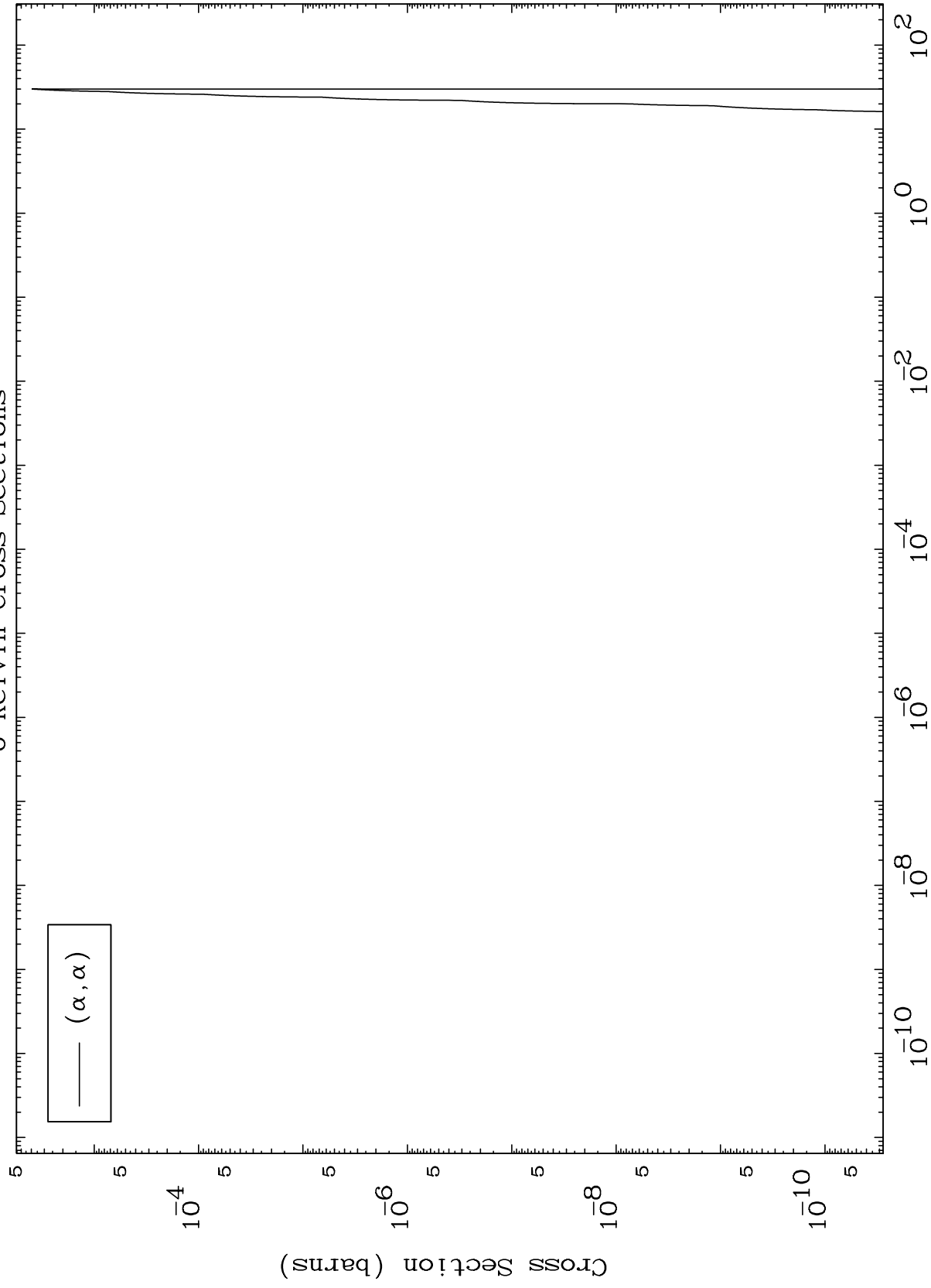
Incident Energy (MeV)

103-Lr-253

MAT 9987

(α, α) Levels
0 Kelvin Cross Sections

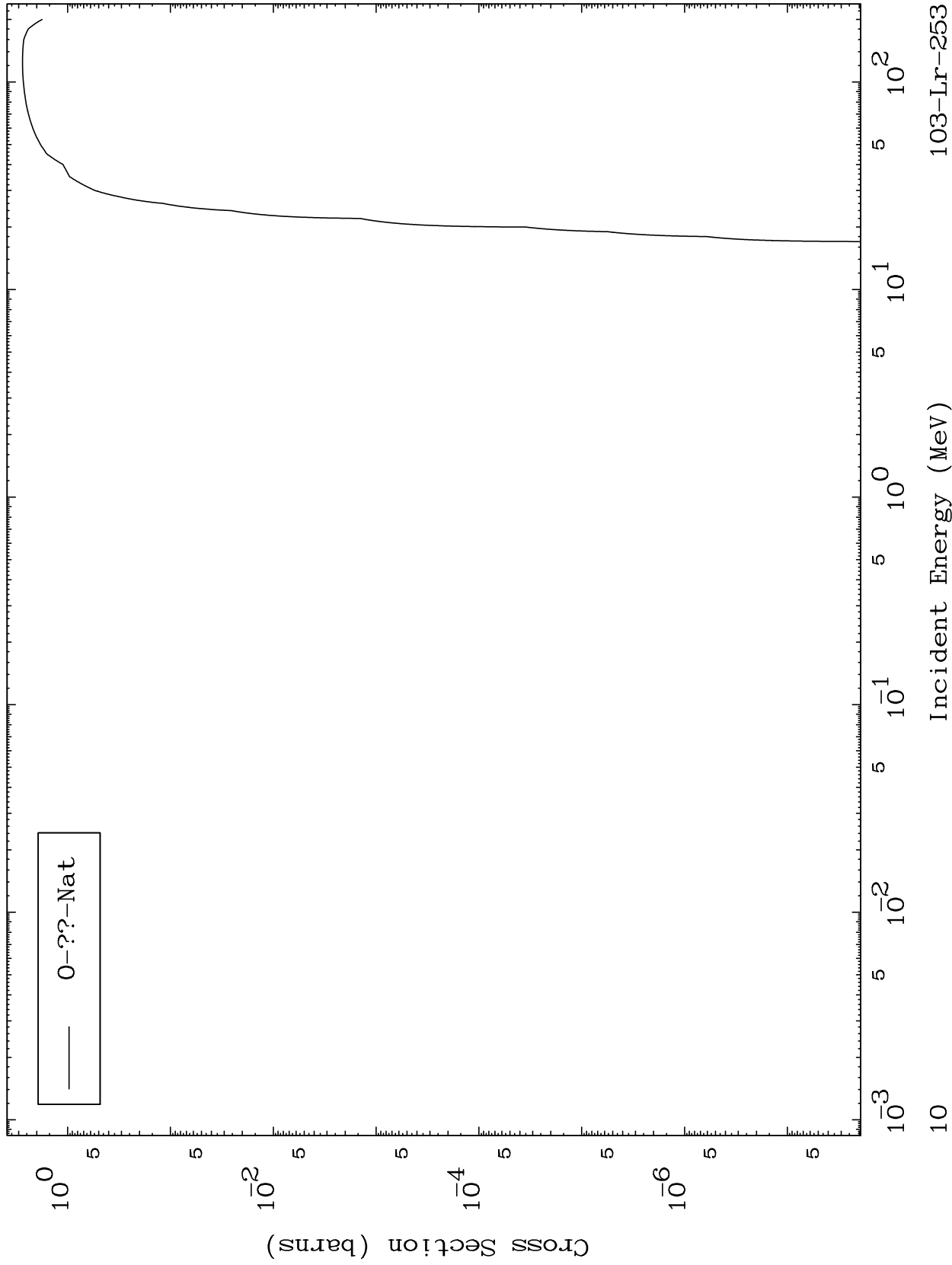
103-Lr-253



MAT 9987

103-Lr-253

α Fission
Radionuclide Production Cross Section

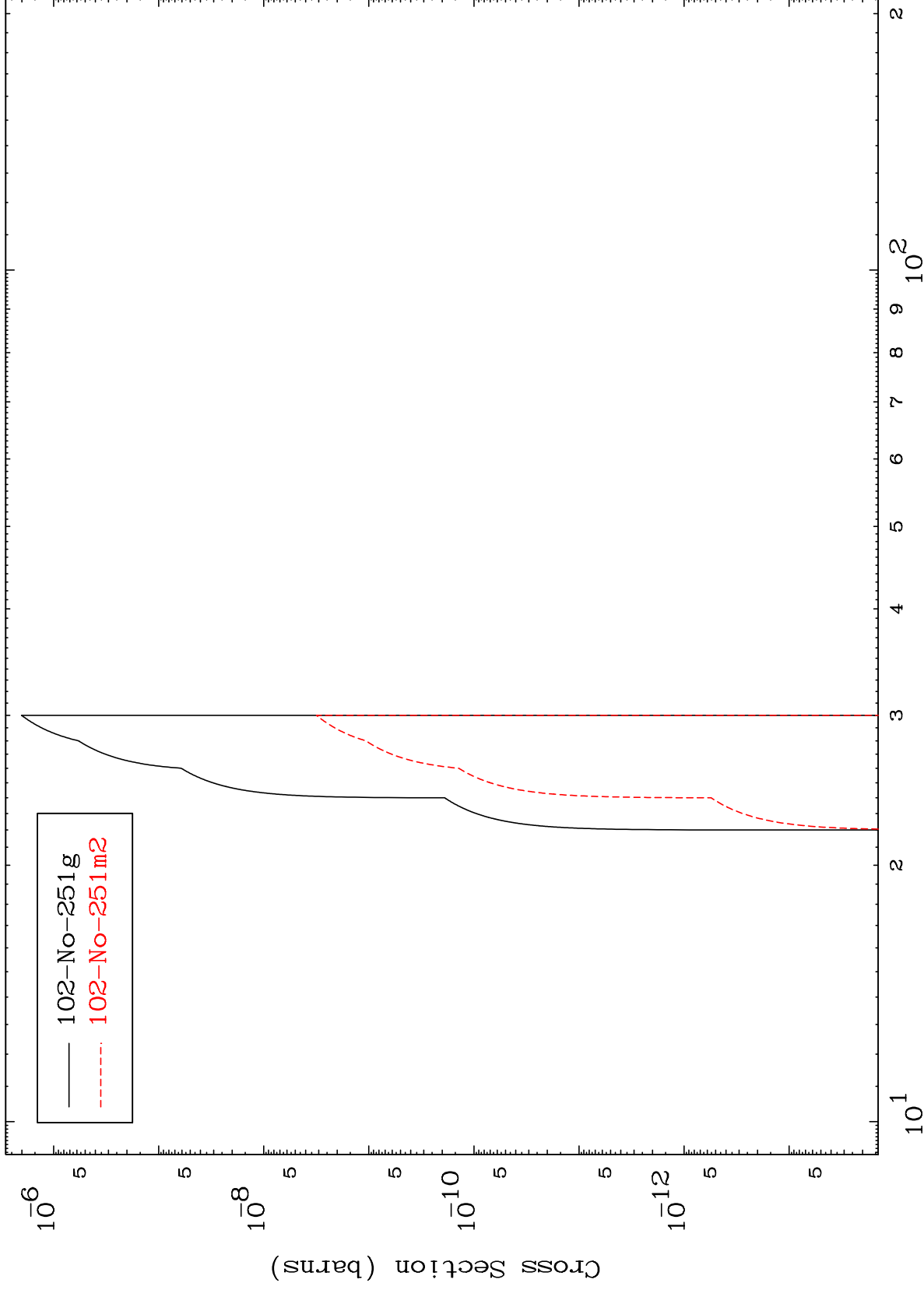


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(α, n') p α

103-Lr-253

Radionuclide Production Cross Section



11

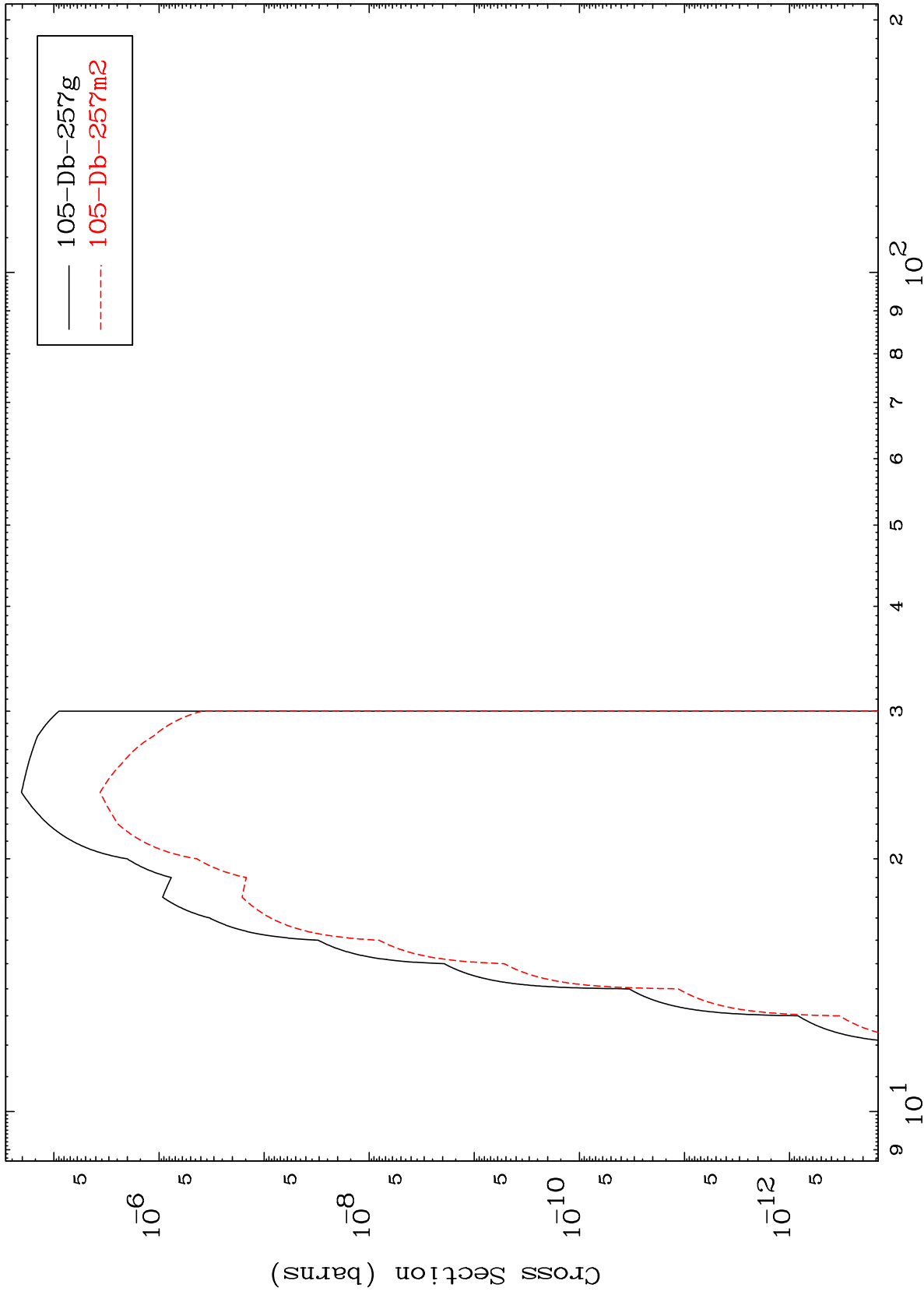
Incident Energy (MeV)

103-Lr-253

MAT 9987

103-Lr-253

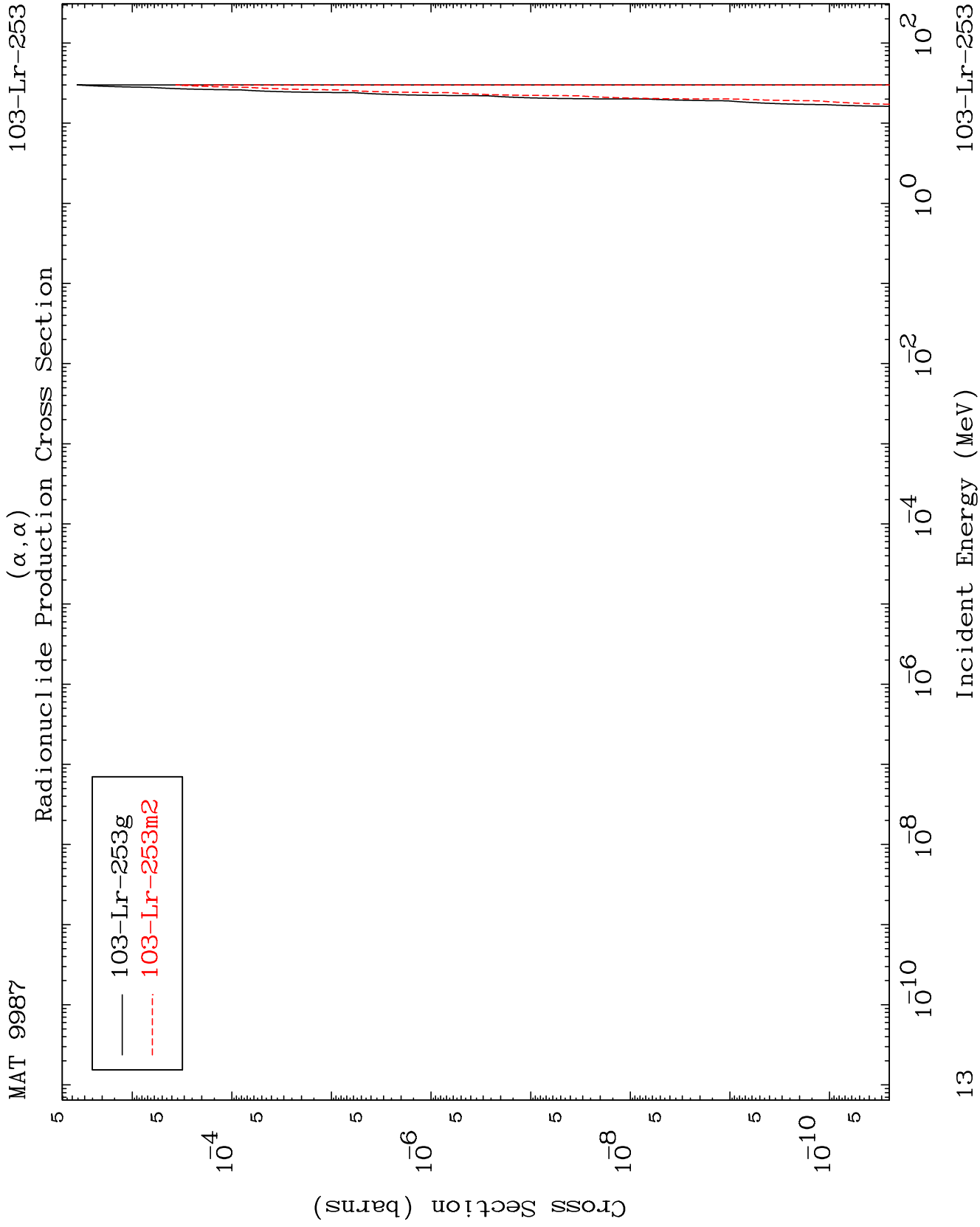
(α, γ)
Radionuclide Production Cross Section



12

Incident Energy (MeV)

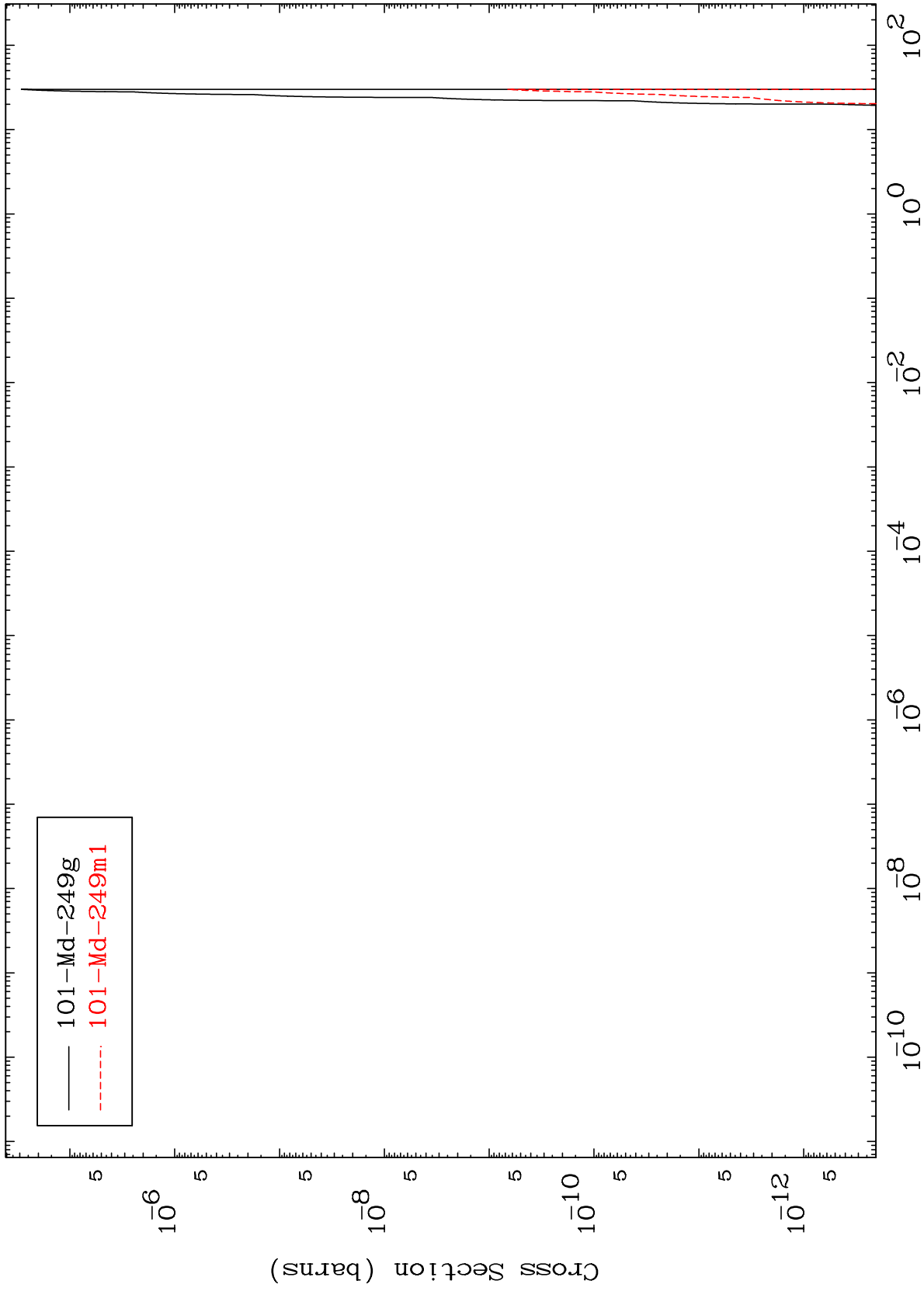
103-Lr-253



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Radionuclide Production Cross Section
($\alpha, 2\alpha$)

$^{103}\text{--}\text{Lr--}^{253}$



14

Incident Energy (MeV)

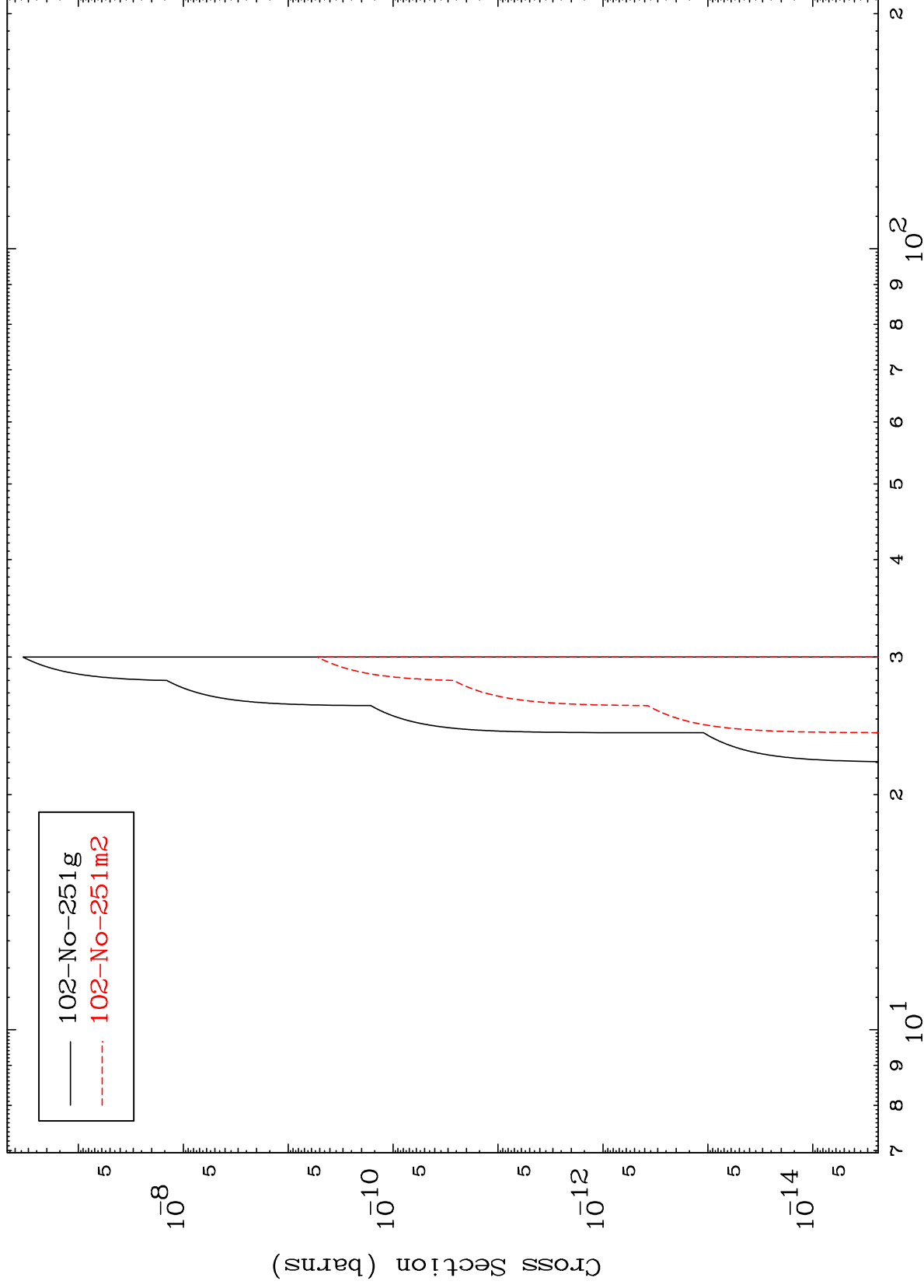
$^{103}\text{--}\text{Lr--}^{253}$

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(α ,d) α

103-Lr-253

Radionuclide Production Cross Section



15

Incident Energy (MeV)

103-Lr-253