

Program EVALPLOT
(Version 2018-1)

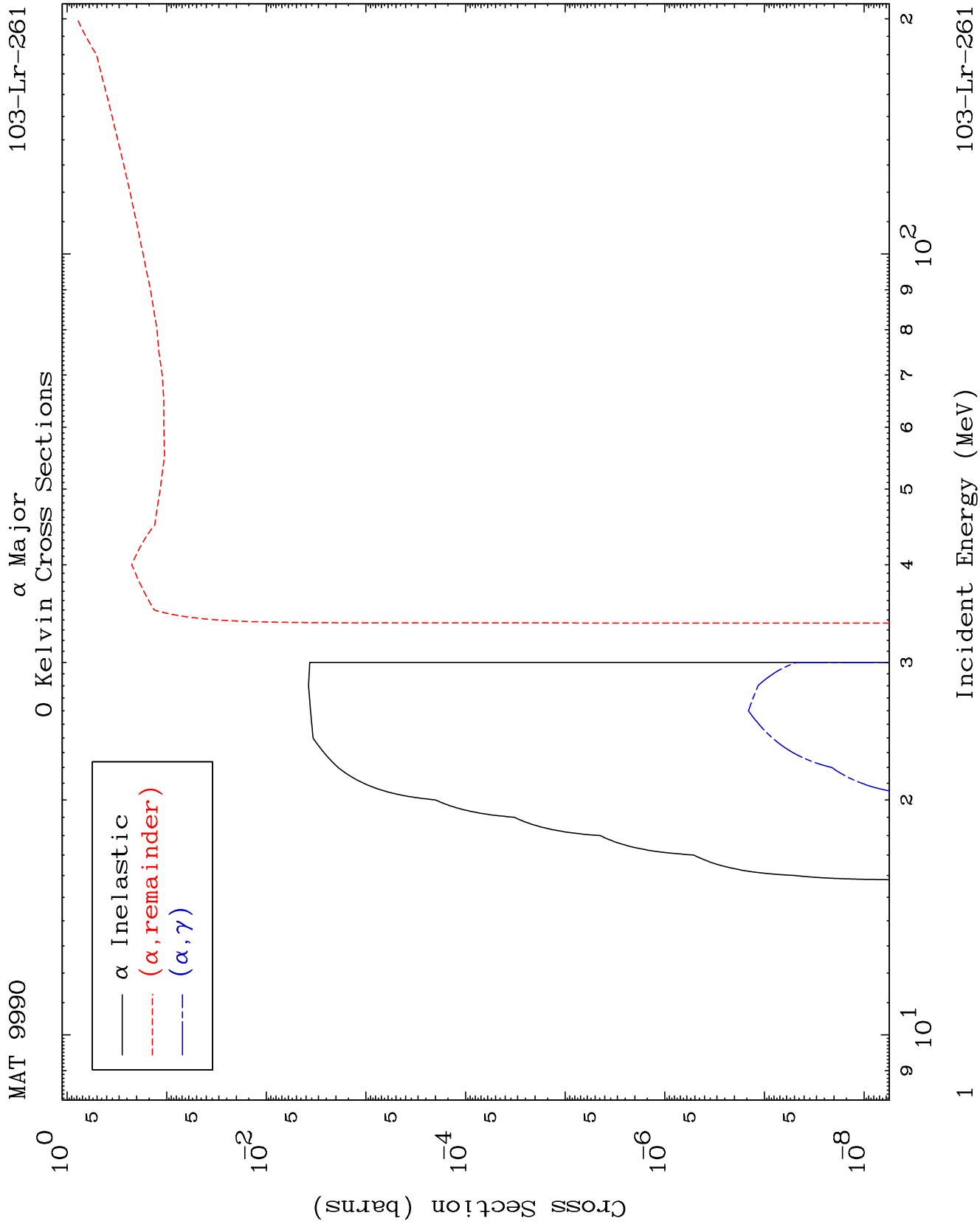
by

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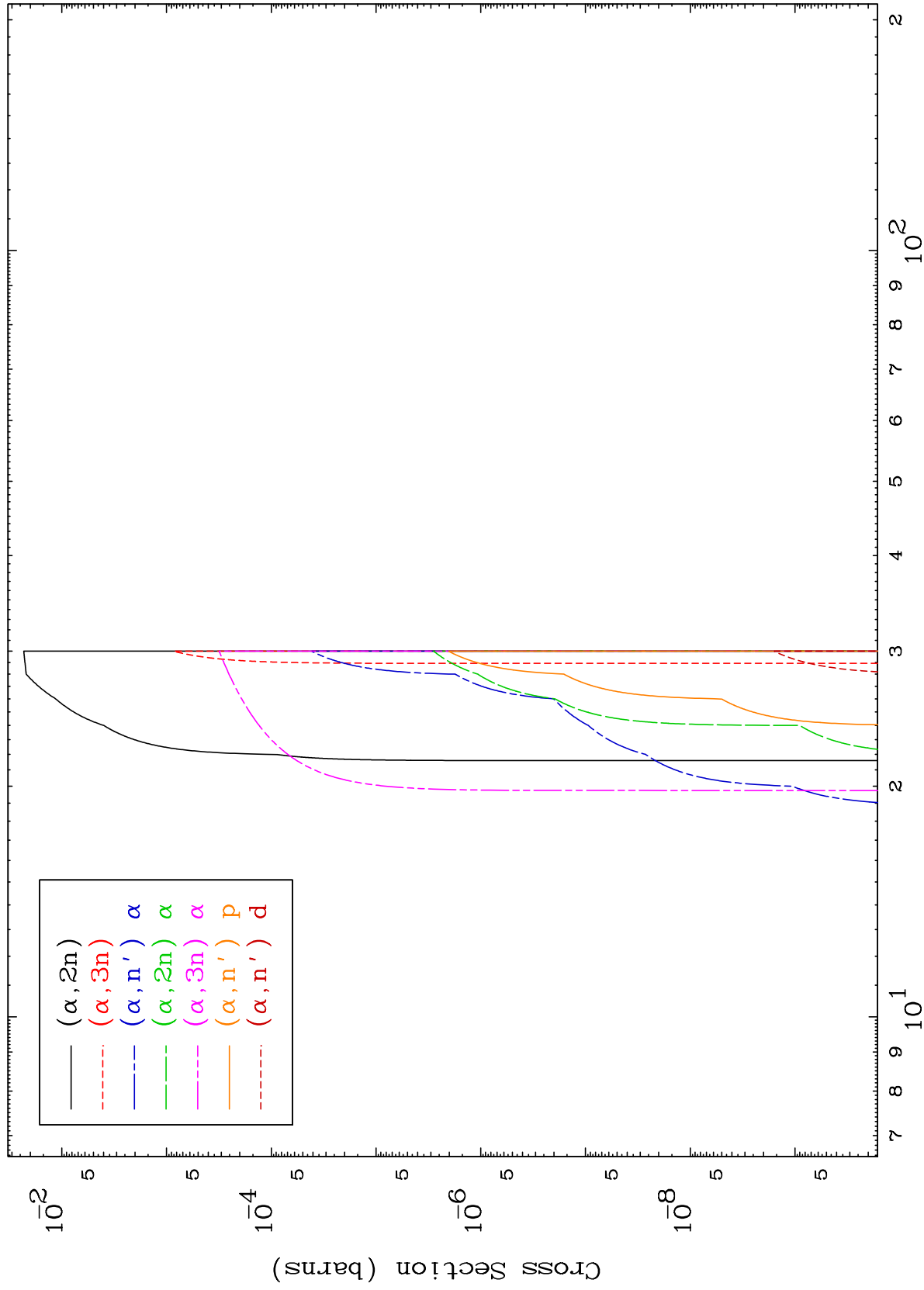
Press Mouse Button to Start



MAT 9990

α Neutron Production
0 Kelvin Cross Sections

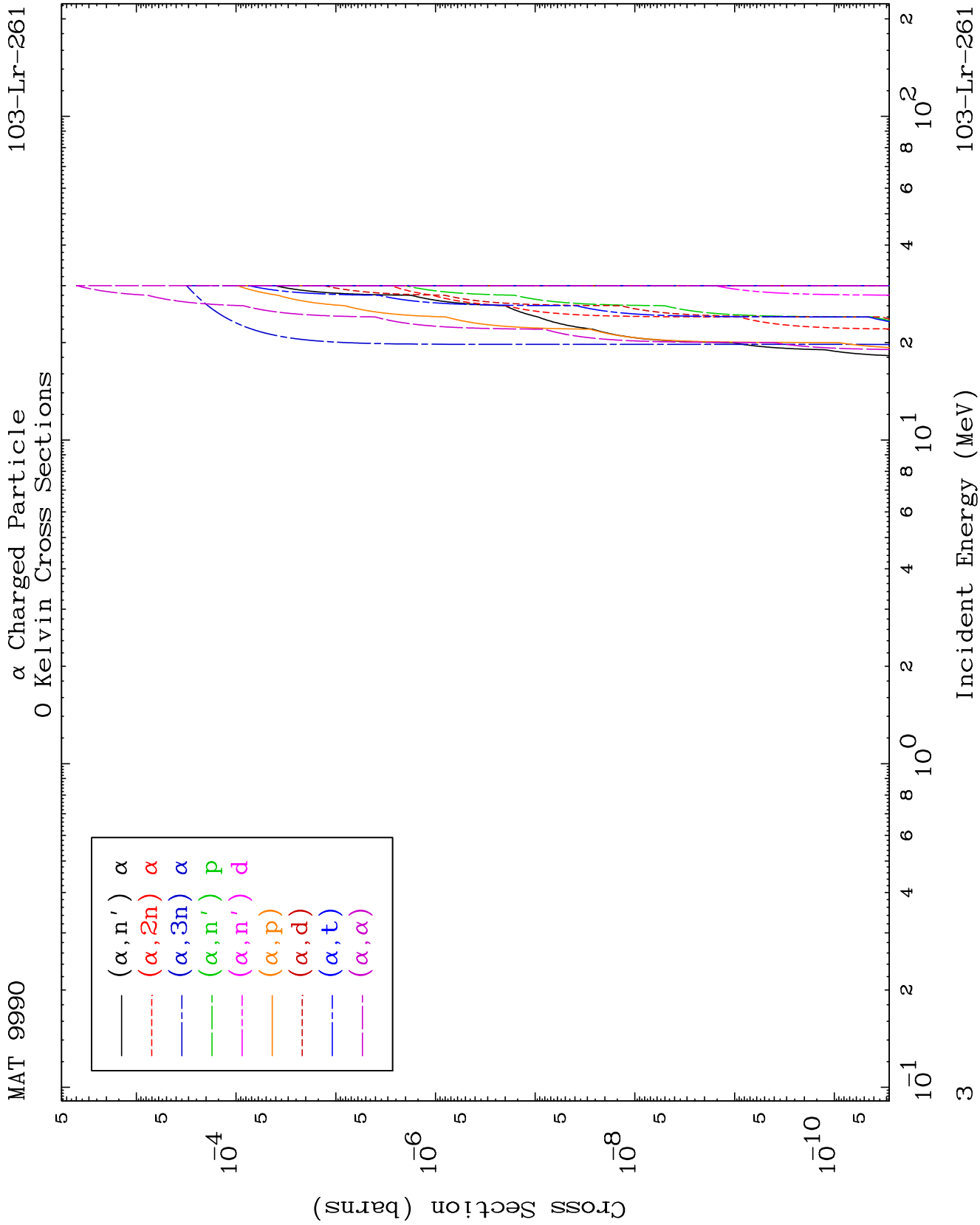
103-Lr-261



2

Incident Energy (MeV)

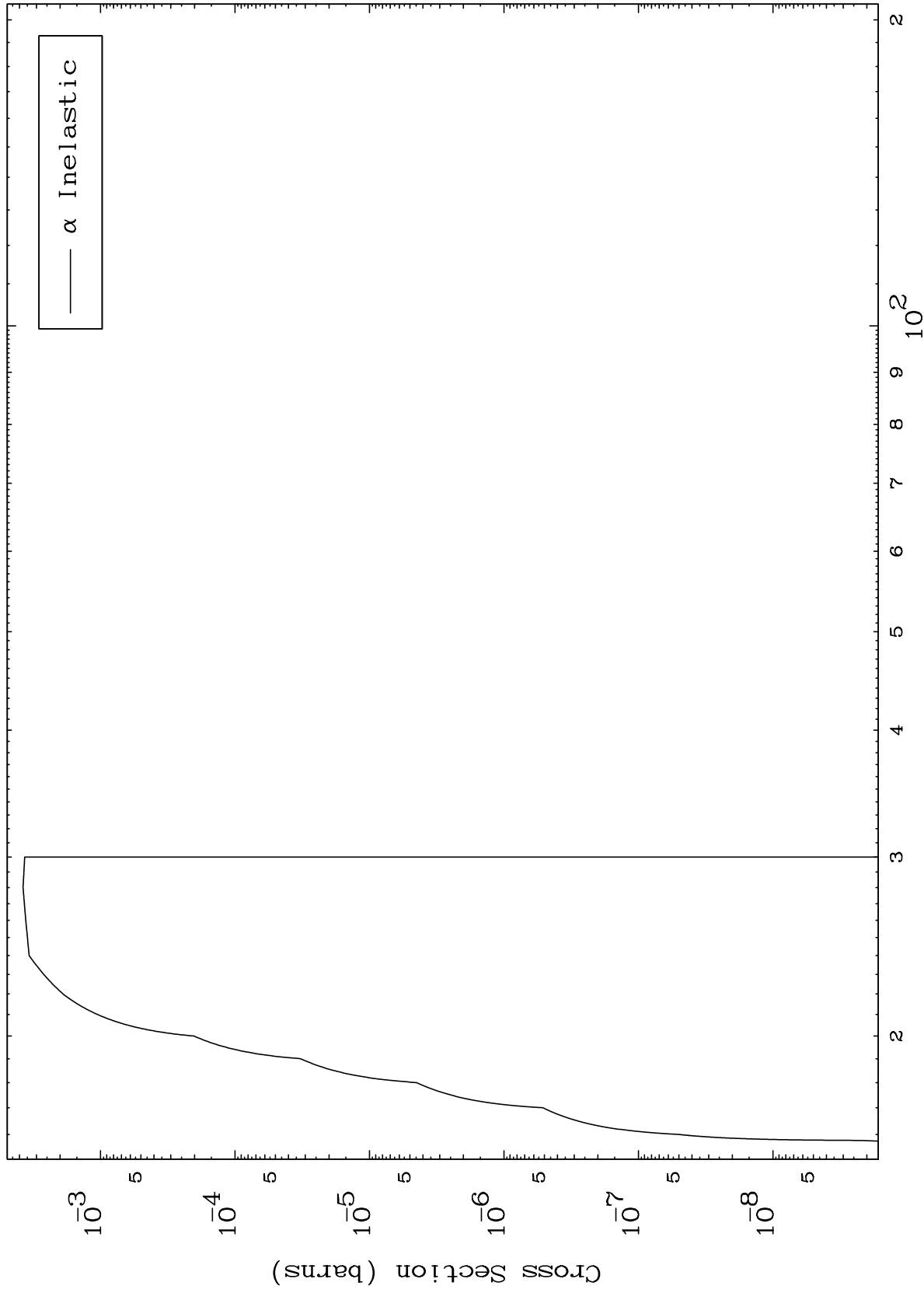
103-Lr-261



MAT 9990

(α, n') Level
0 Kelvin Cross Sections

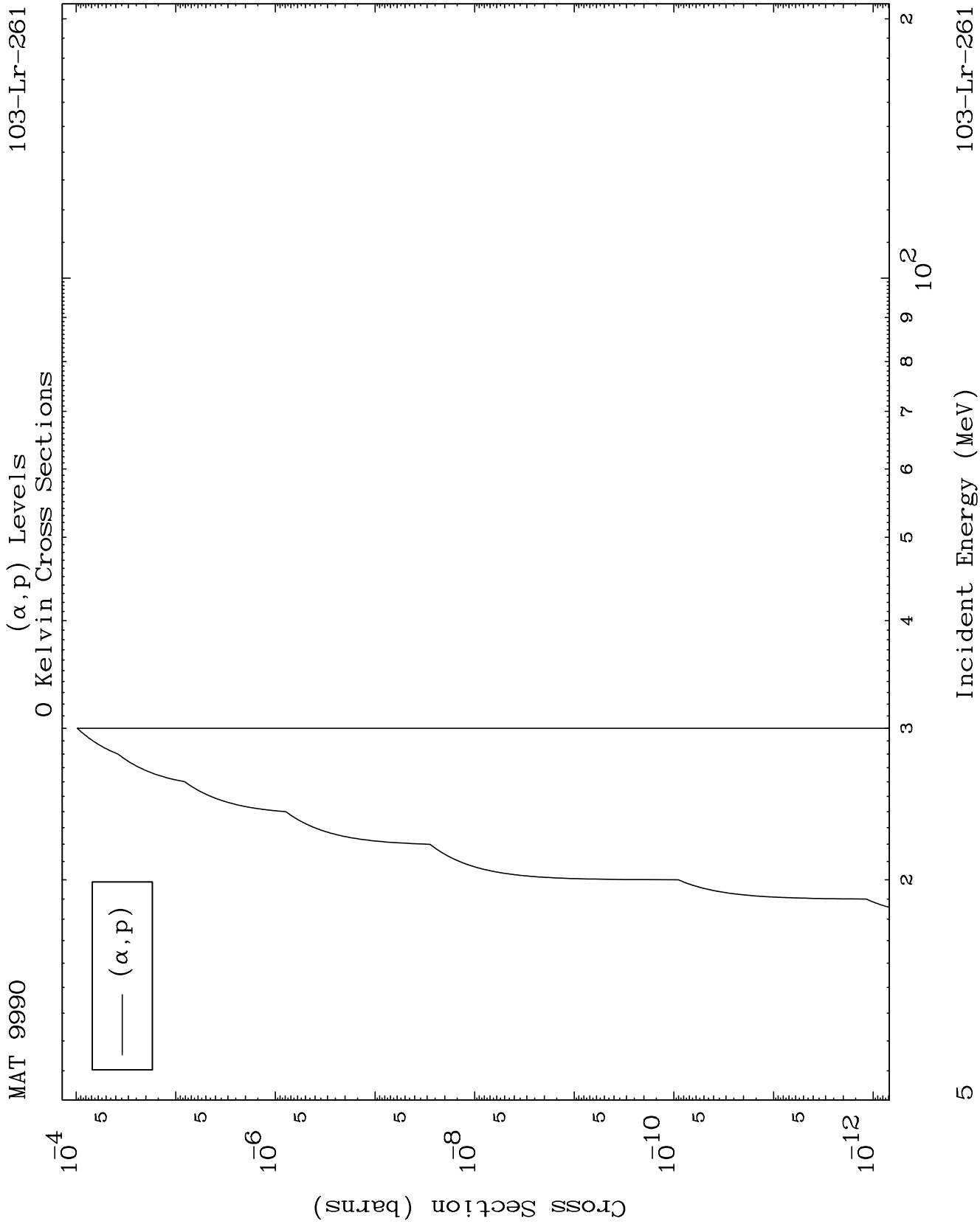
103-Lr-261



4

Incident Energy (MeV)

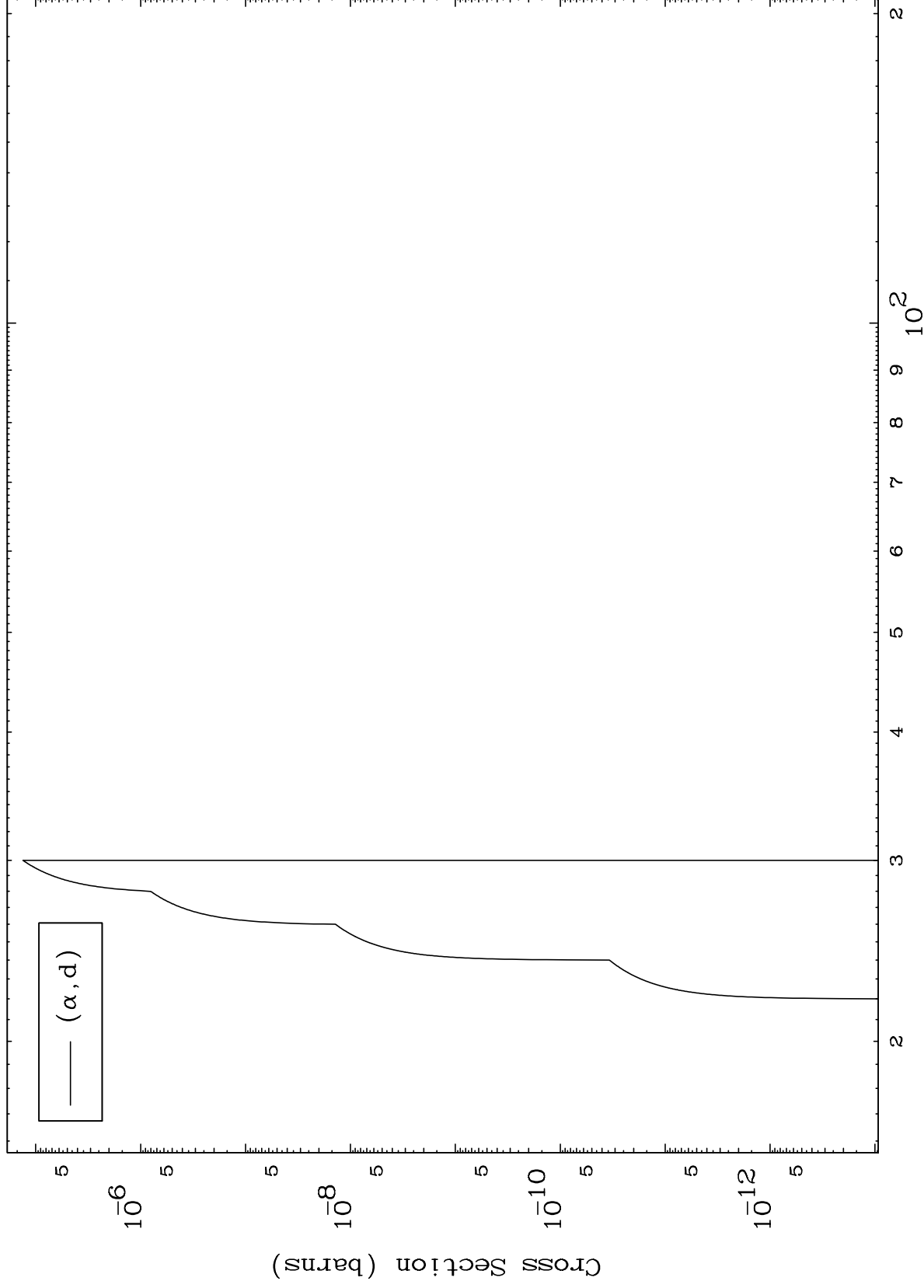
103-Lr-261



MAT 9990

(α ,d) Levels
0 Kelvin Cross Sections

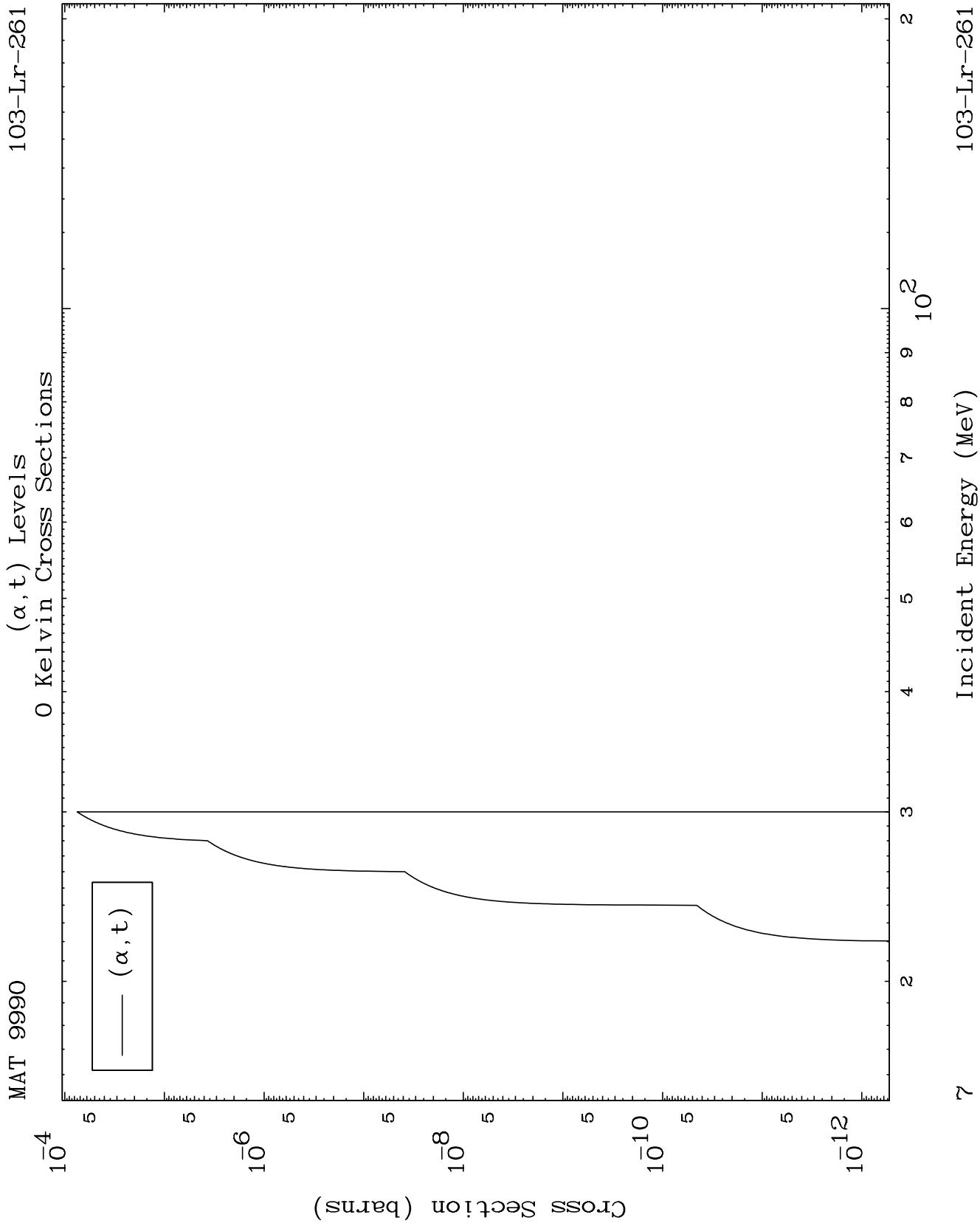
103-Lr-261



6

Incident Energy (MeV)

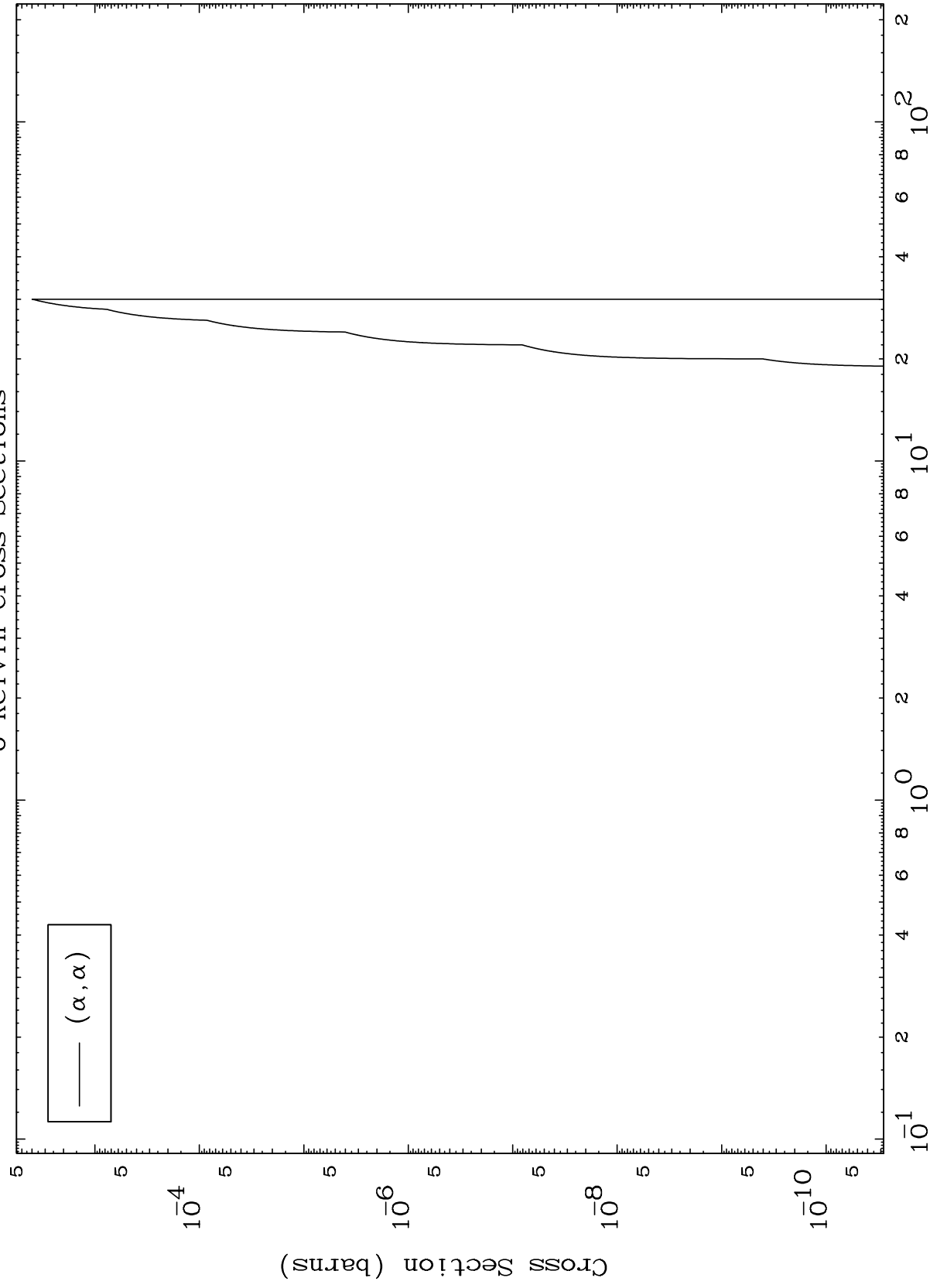
103-Lr-261



MAT 9990

(α, α) Levels
0 Kelvin Cross Sections

103-Lr-261



8

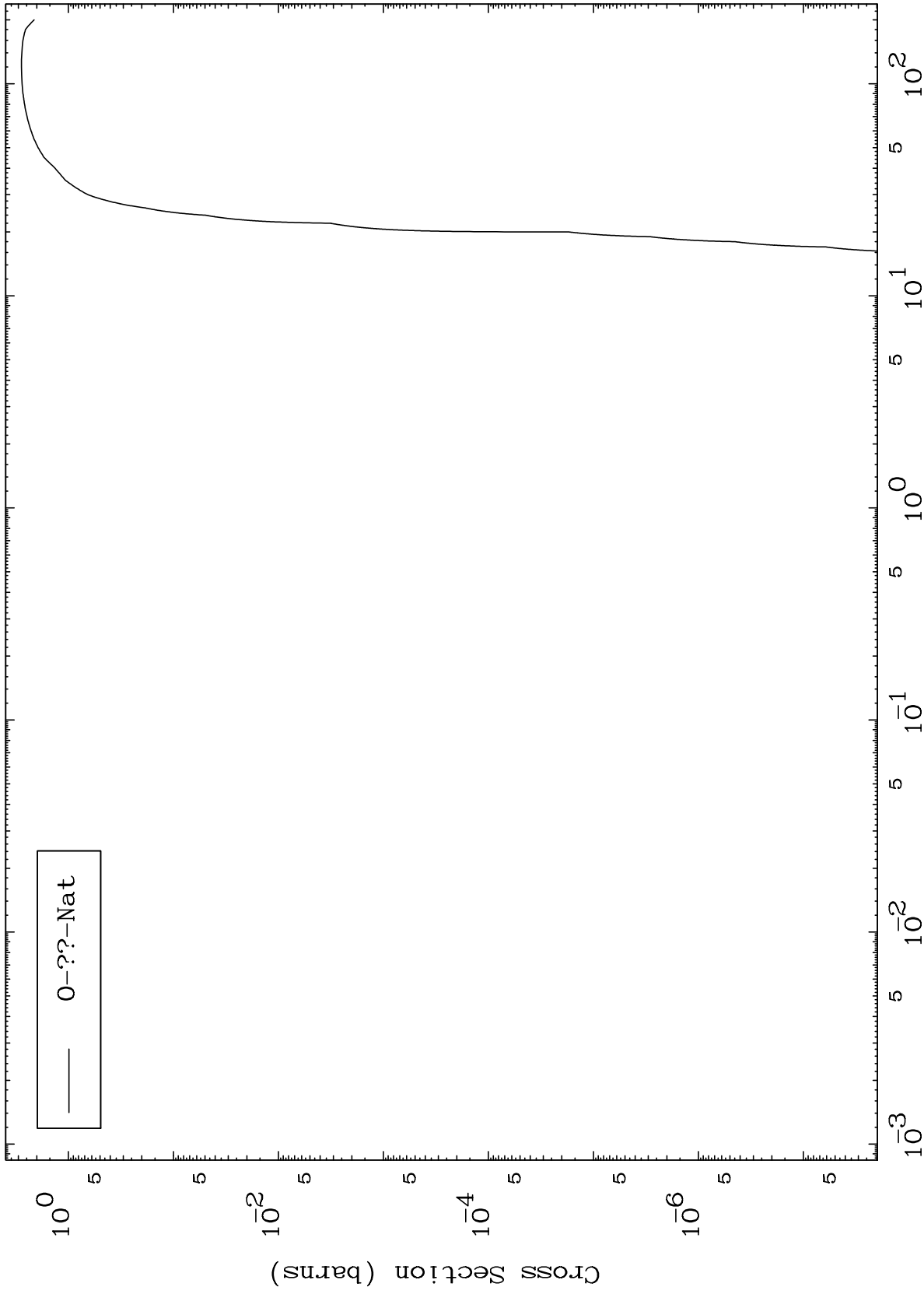
Incident Energy (MeV)

103-Lr-261

MAT 9990

α Fission
Radionuclide Production Cross Section

103-Lr-261



Incident Energy (MeV)

103-Lr-261