

Program EVALPLOT  
(Version 2018-1)

by

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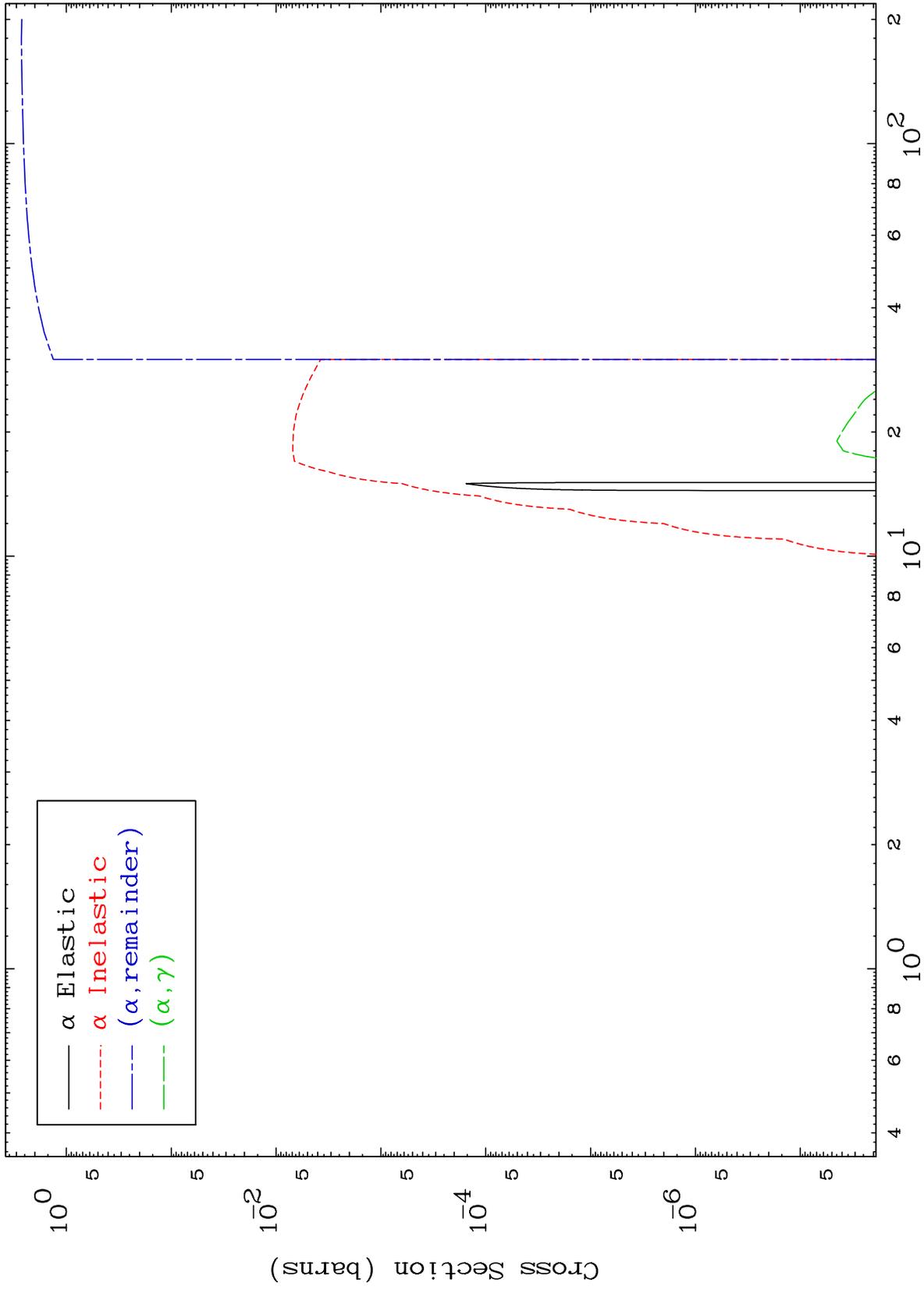
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 7152

0 Kelvin  $\alpha$  Major  
Cross Sections

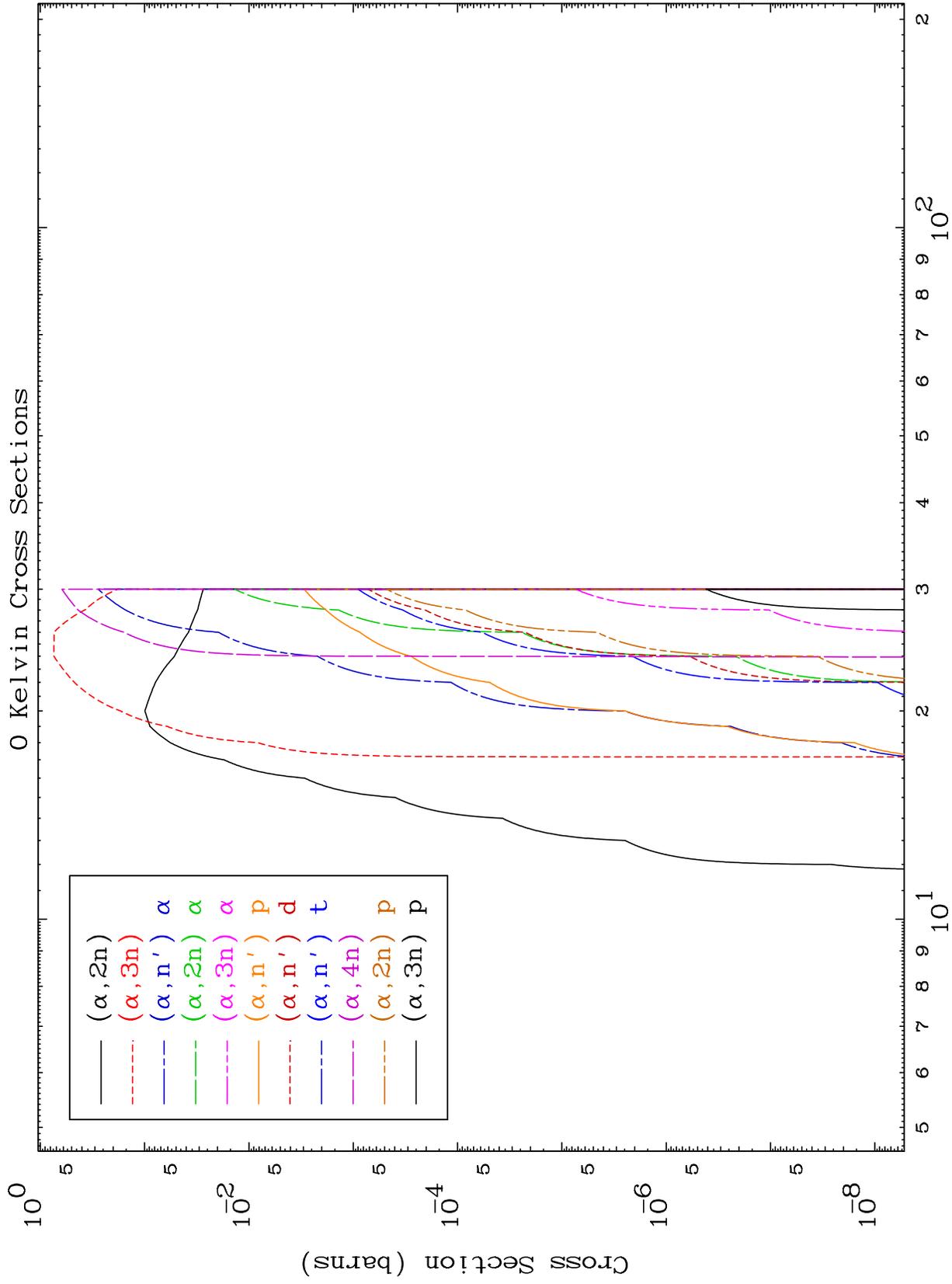
71-Lu-184

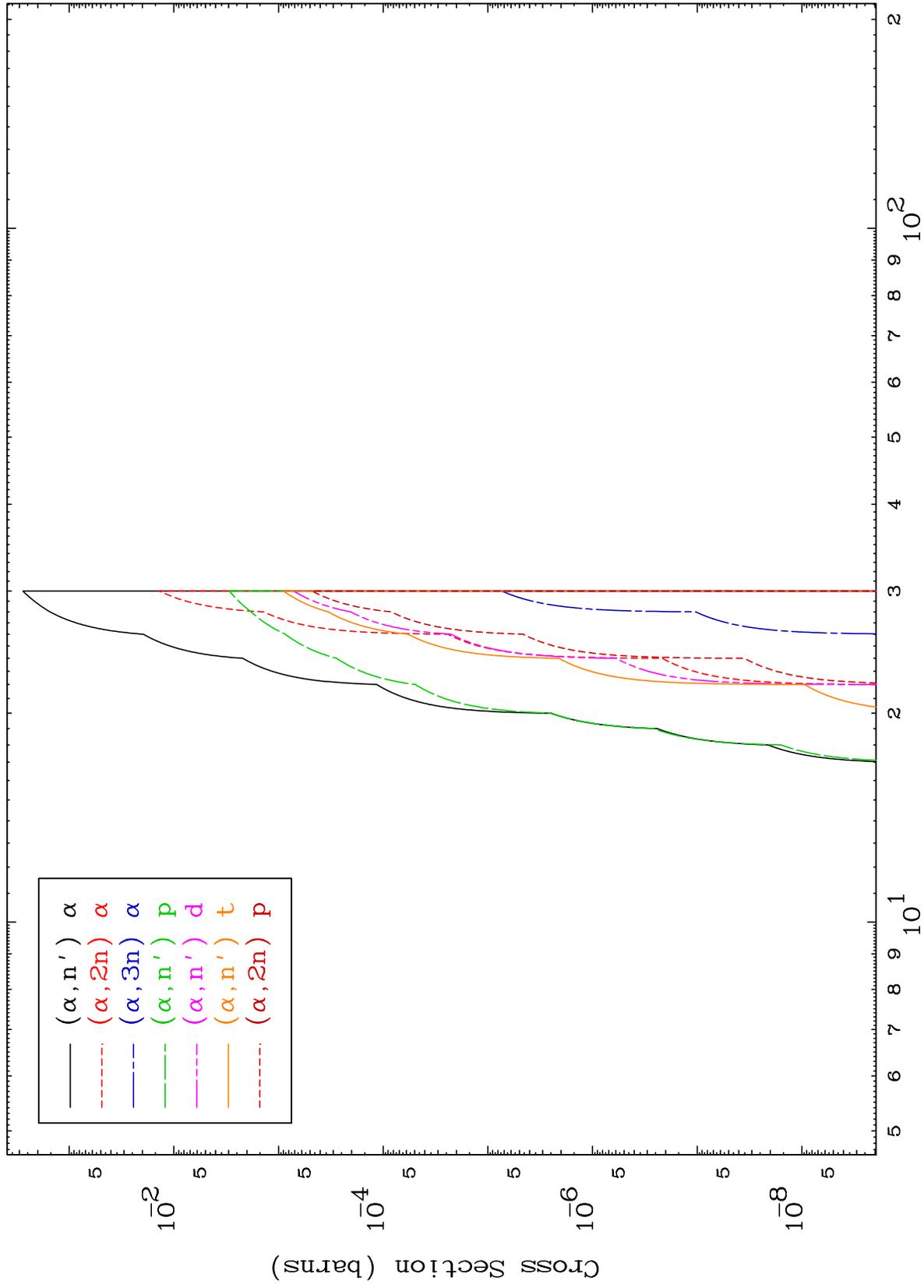


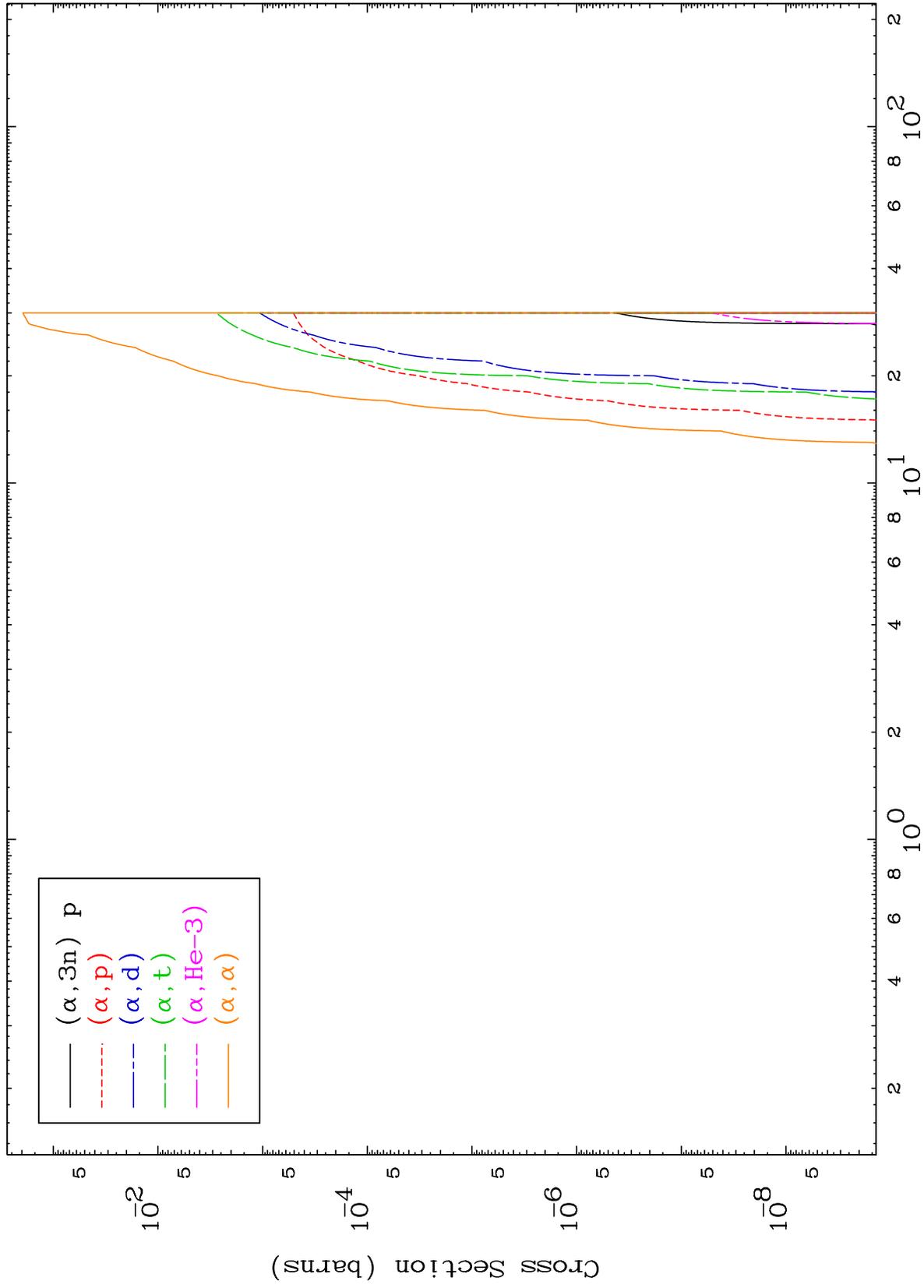
1

Incident Energy (MeV)

71-Lu-184



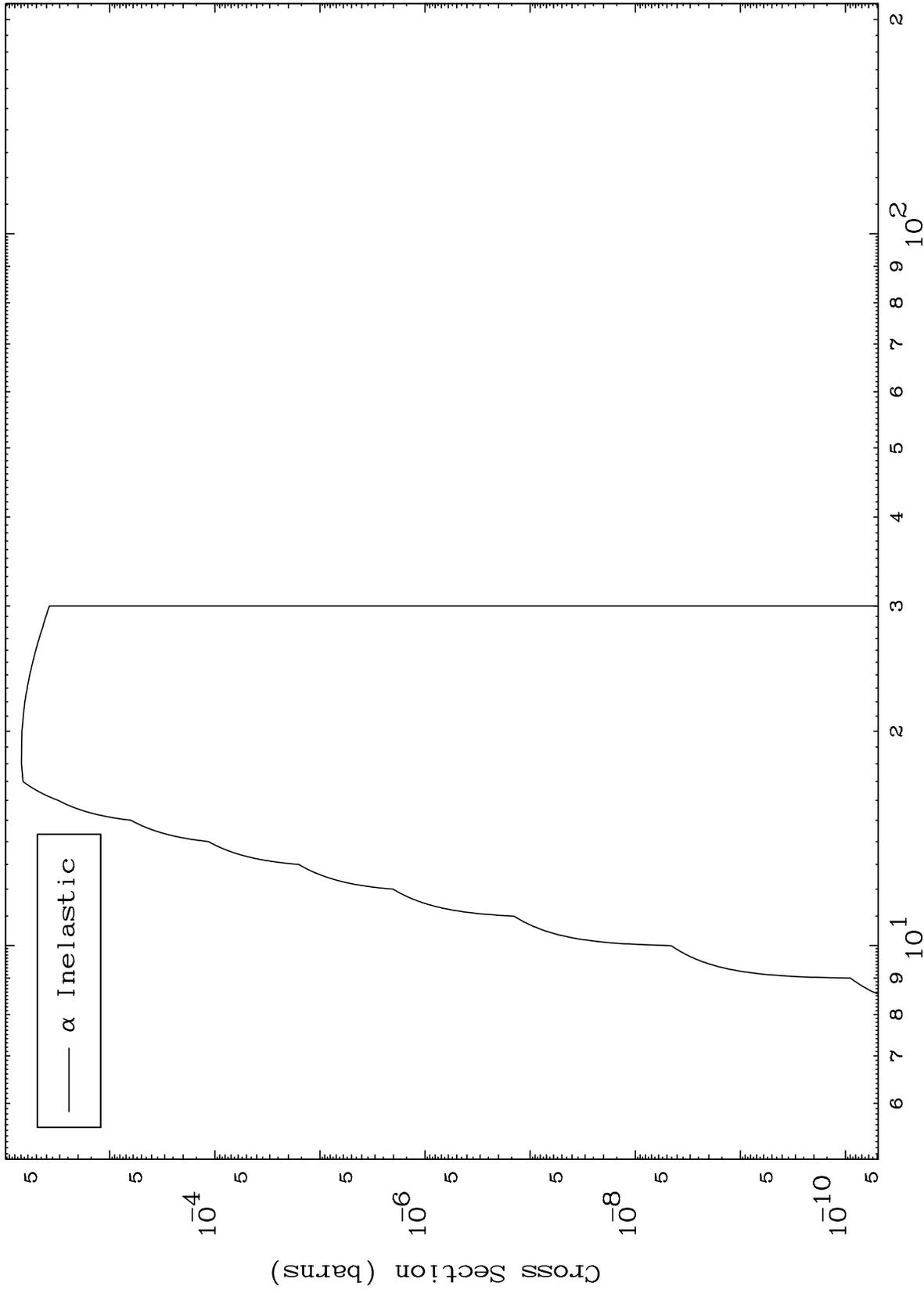




MAT 7152

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

71-Lu-184



5

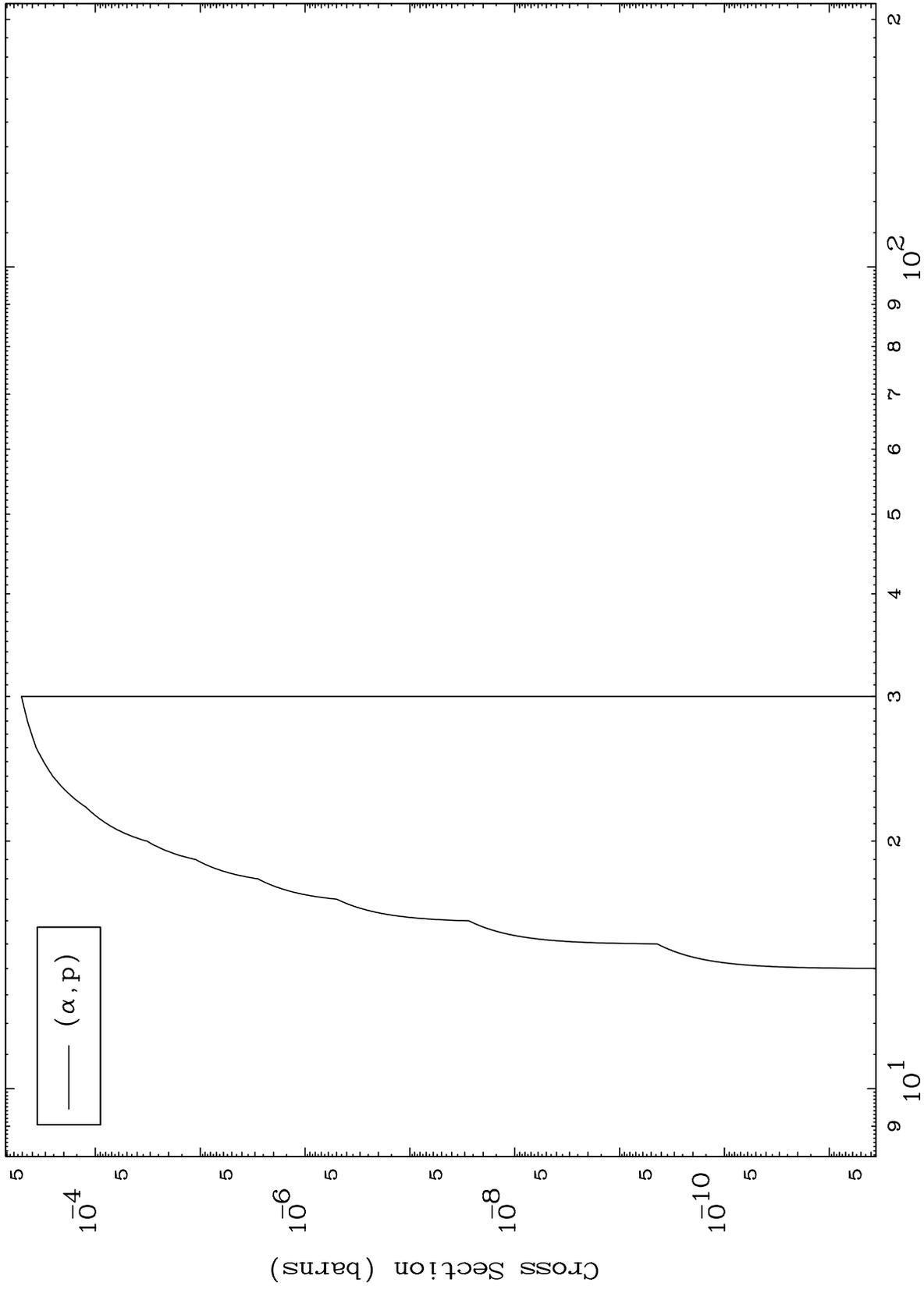
Incident Energy (MeV)

71-Lu-184

MAT 7152

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

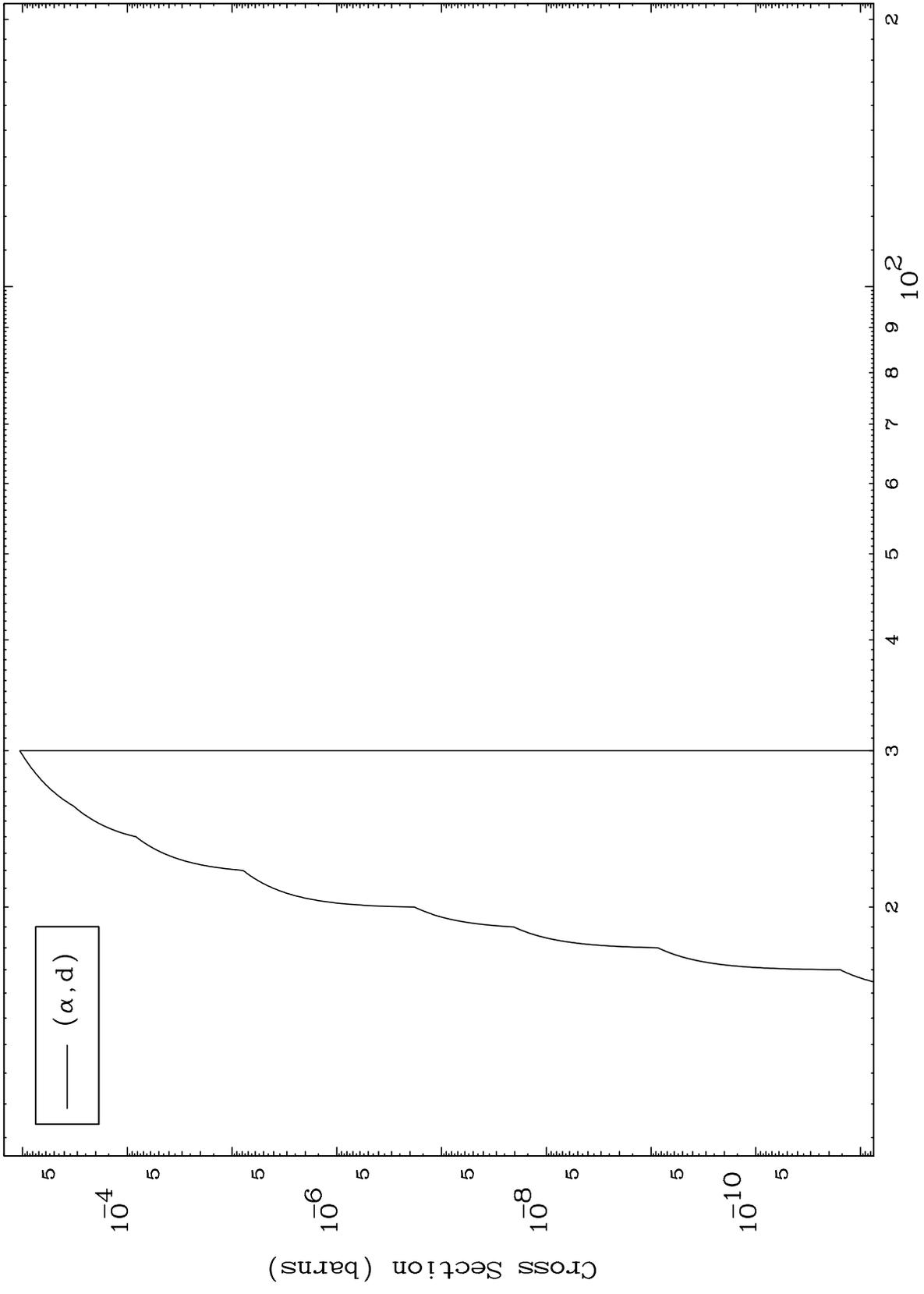
71-Lu-184

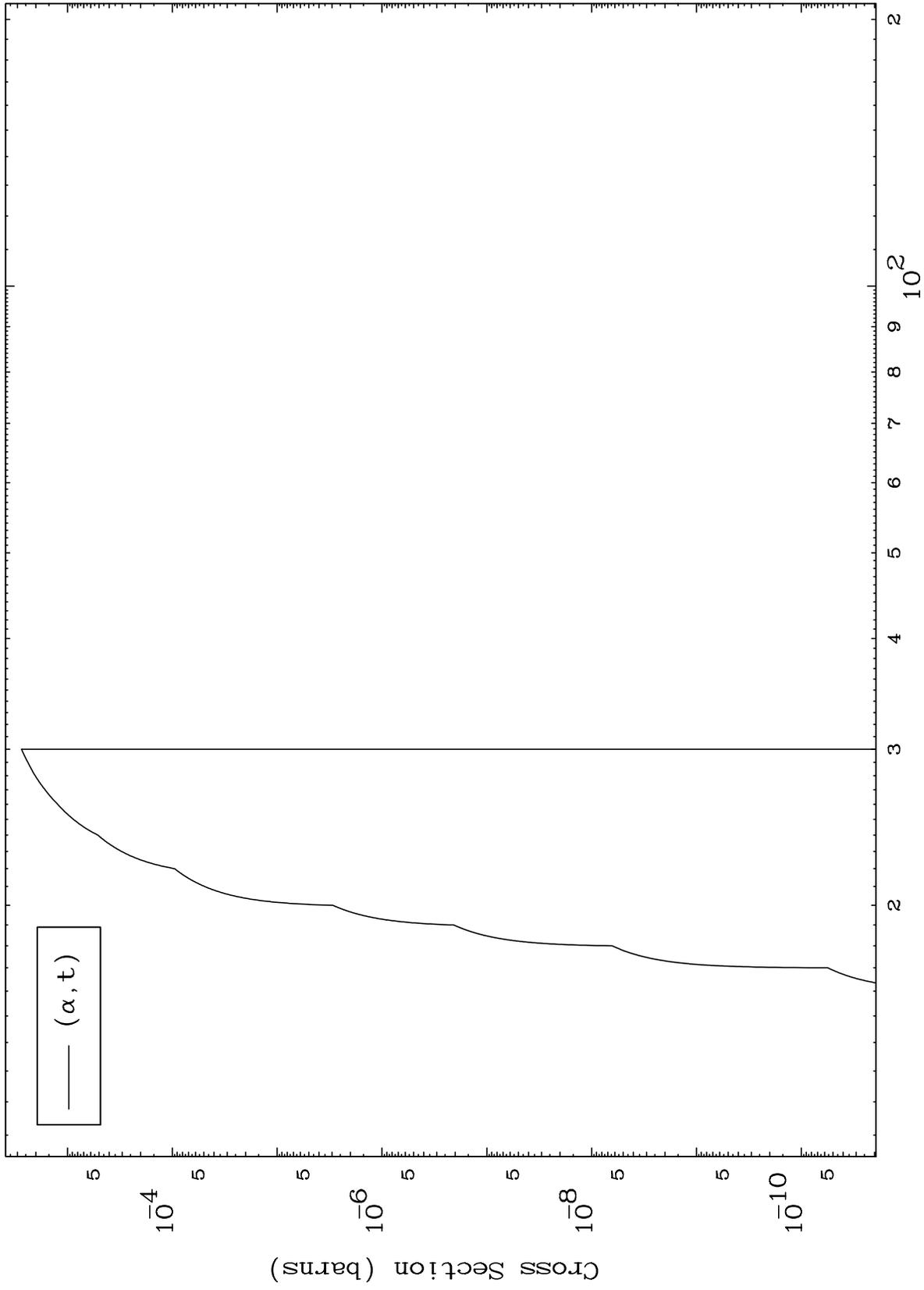


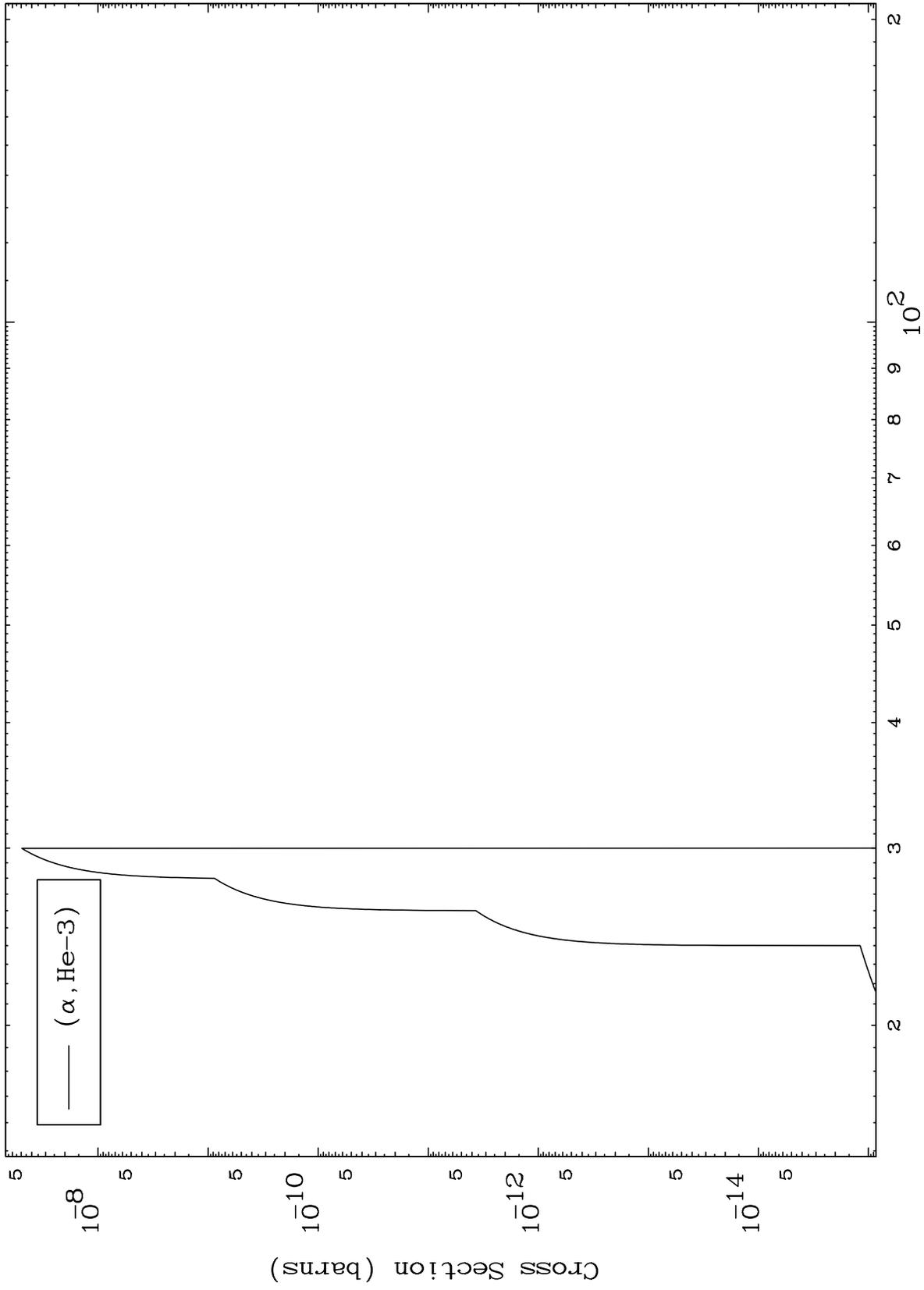
6

Incident Energy (MeV)

71-Lu-184



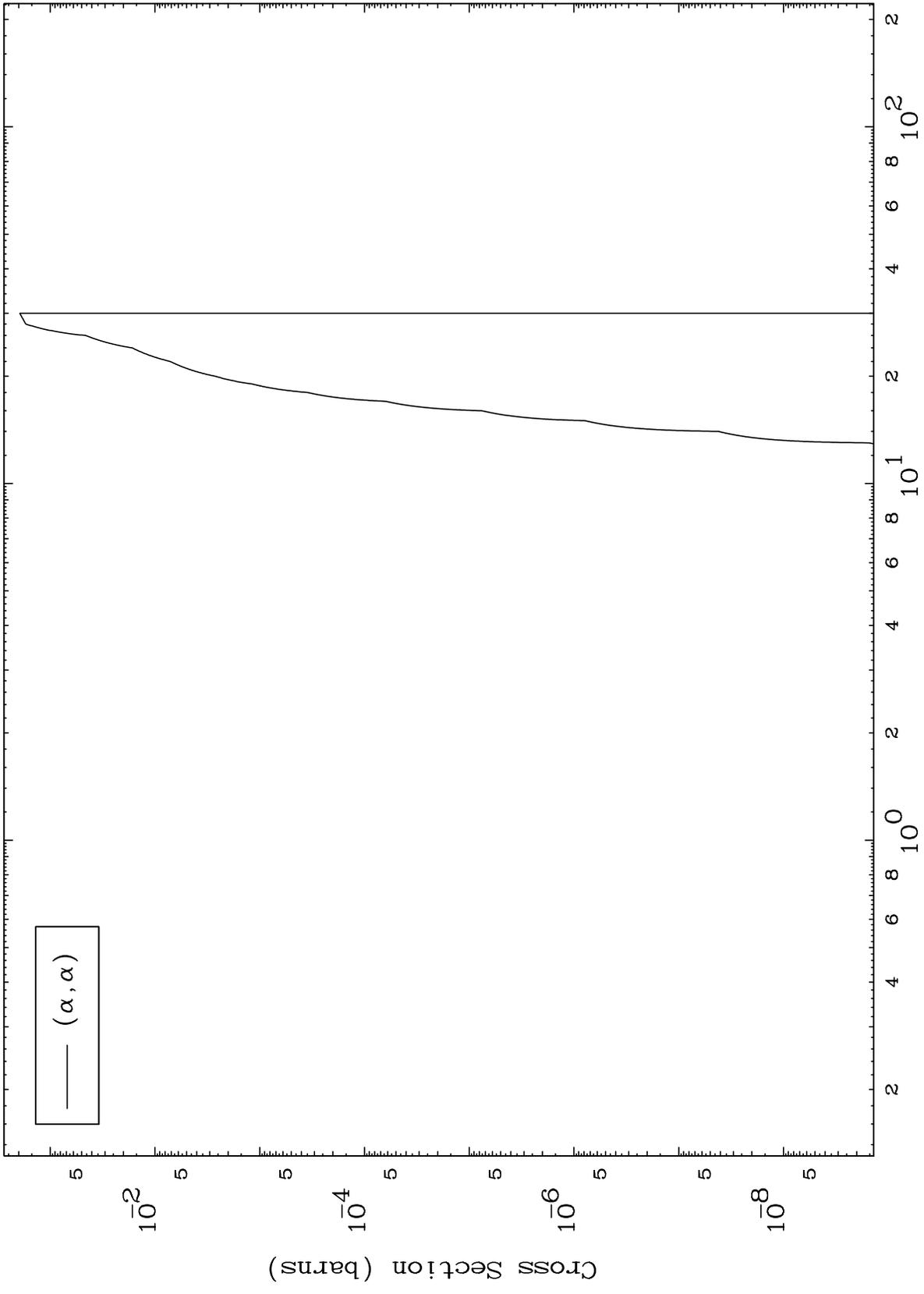




MAT 7152

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

71-Lu-184

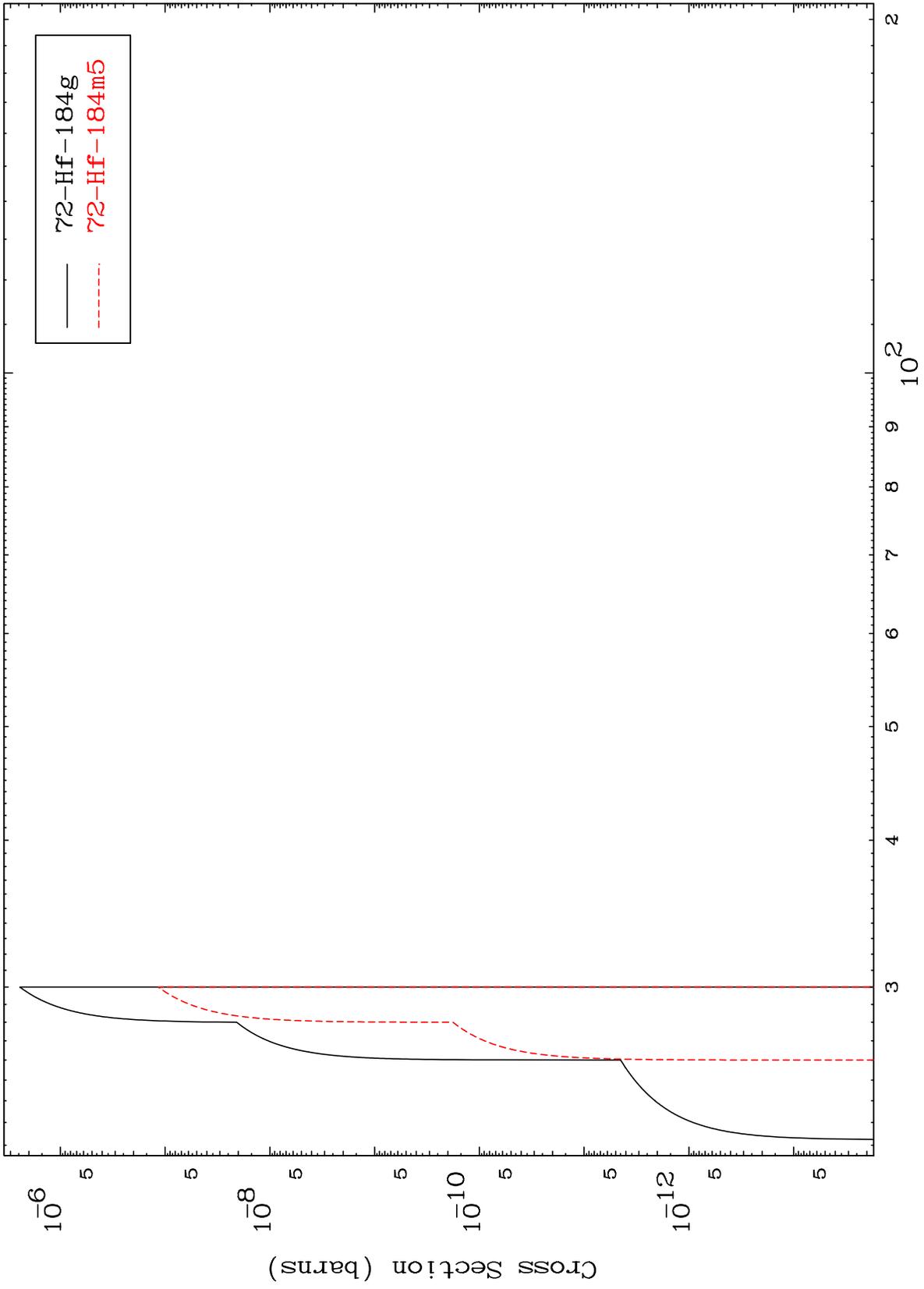


10

Incident Energy (MeV)

71-Lu-184

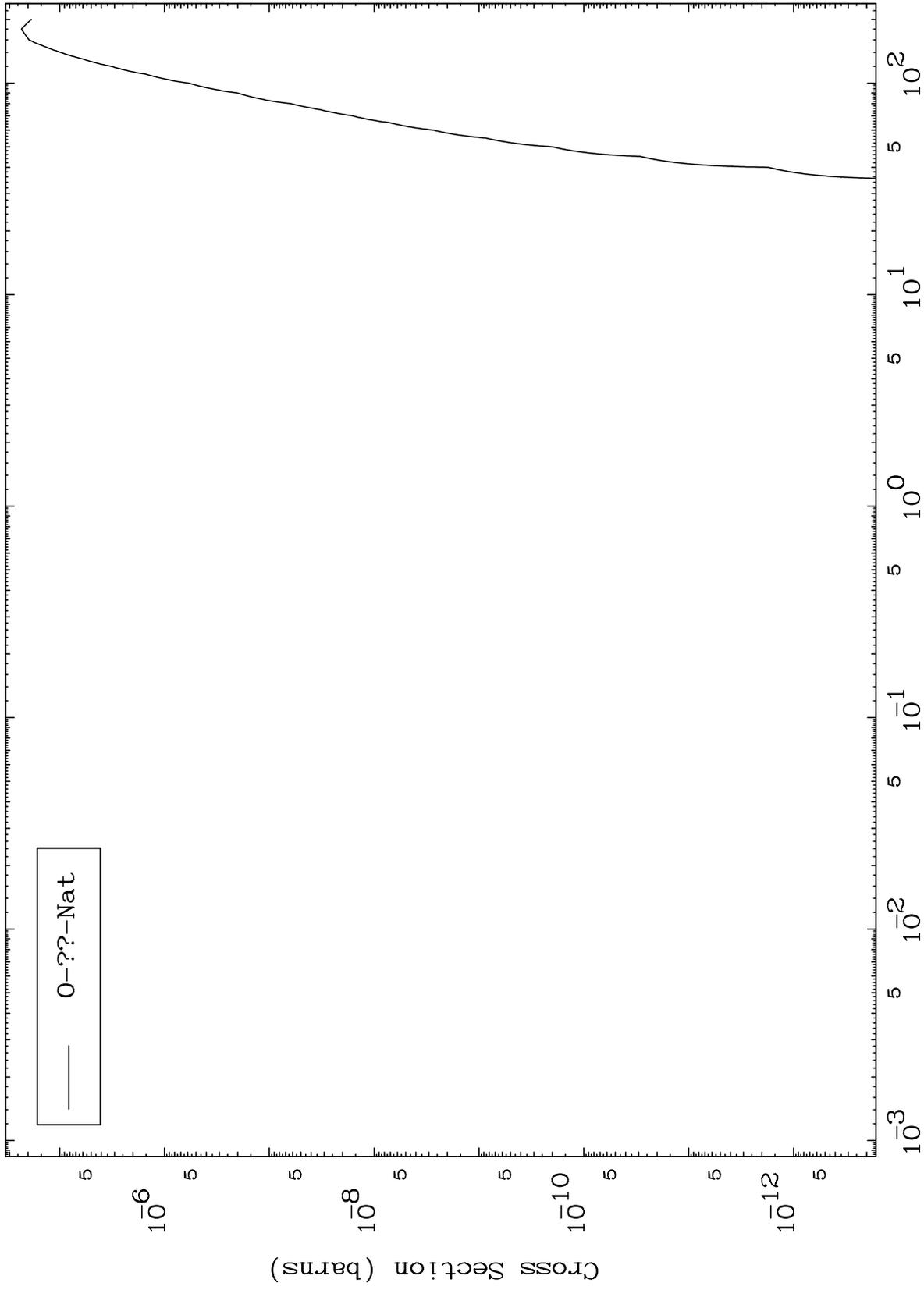
Radionuclide Production Cross Section



MAT 7152

$\alpha$  Fission  
Radionuclide Production Cross Section

71-Lu-184



12

Incident Energy (MeV)

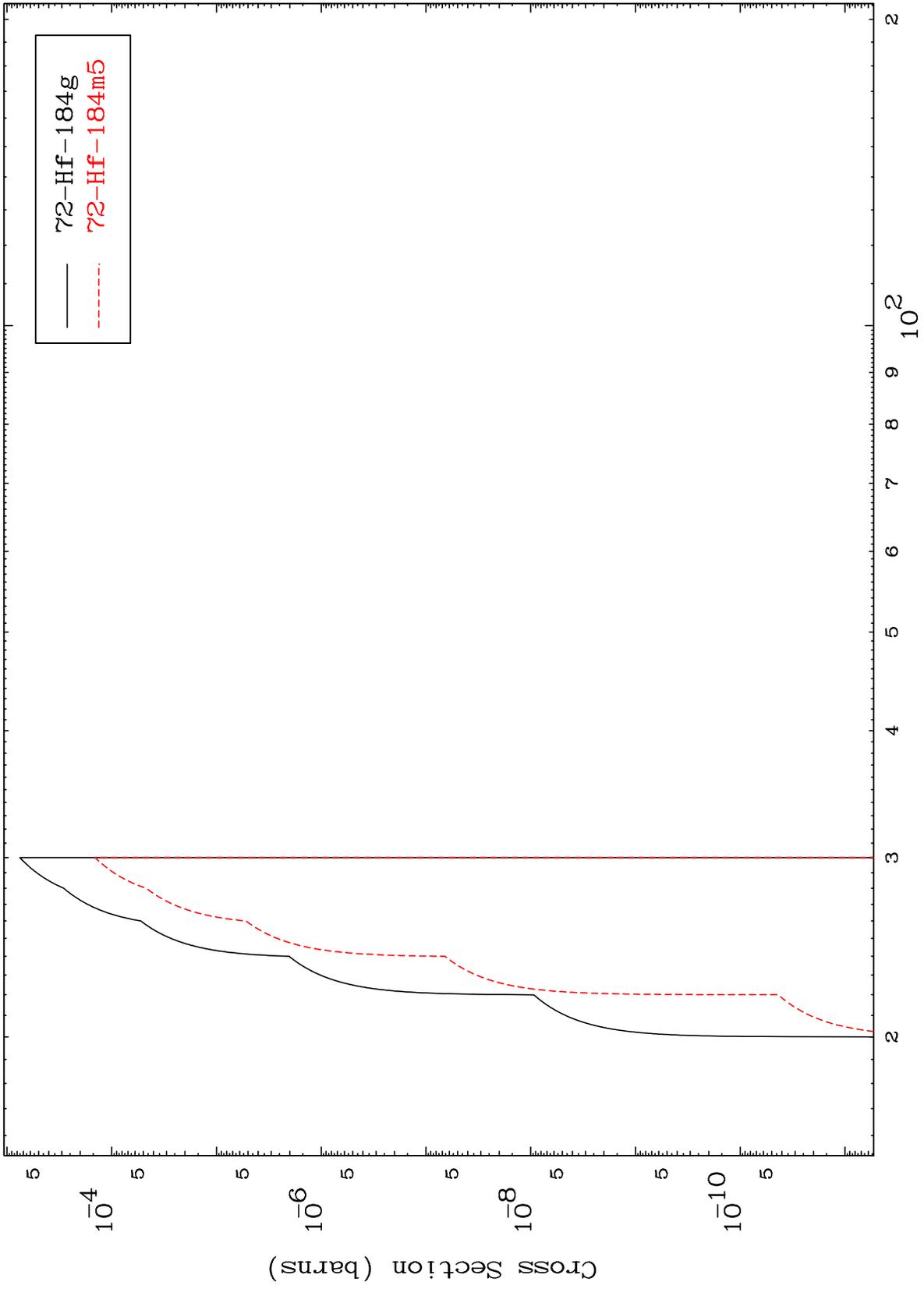
71-Lu-184

MAT 7152

( $\alpha, n'$ ) t

71-Lu-184

Radionuclide Production Cross Section



13

Incident Energy (MeV)

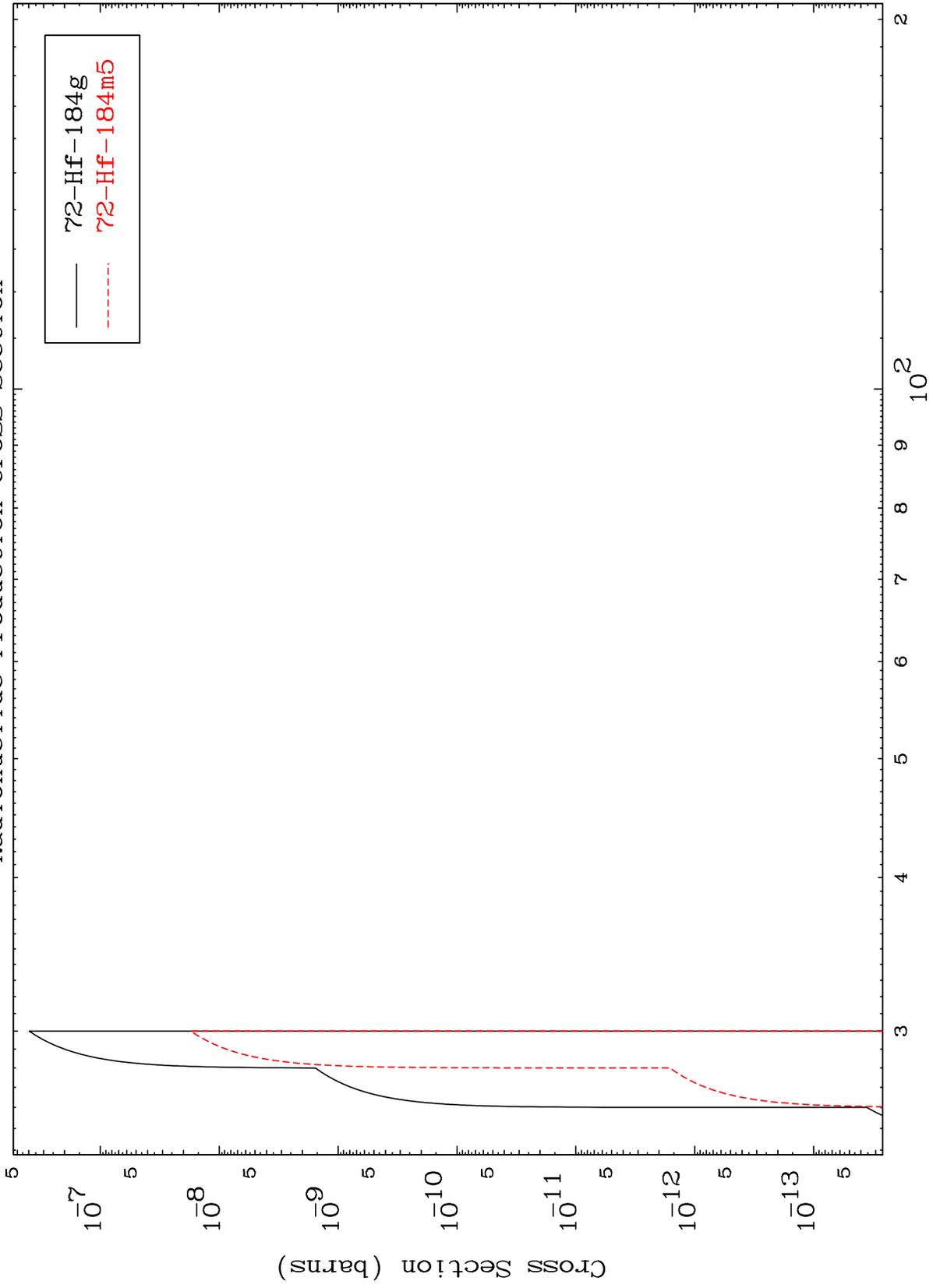
71-Lu-184

MAT 7152

( $\alpha, 3n$ ) p

71-Lu-184

Radionuclide Production Cross Section



14

Incident Energy (MeV)

71-Lu-184