

Program EVALPLOT  
(Version 2018-1)

by

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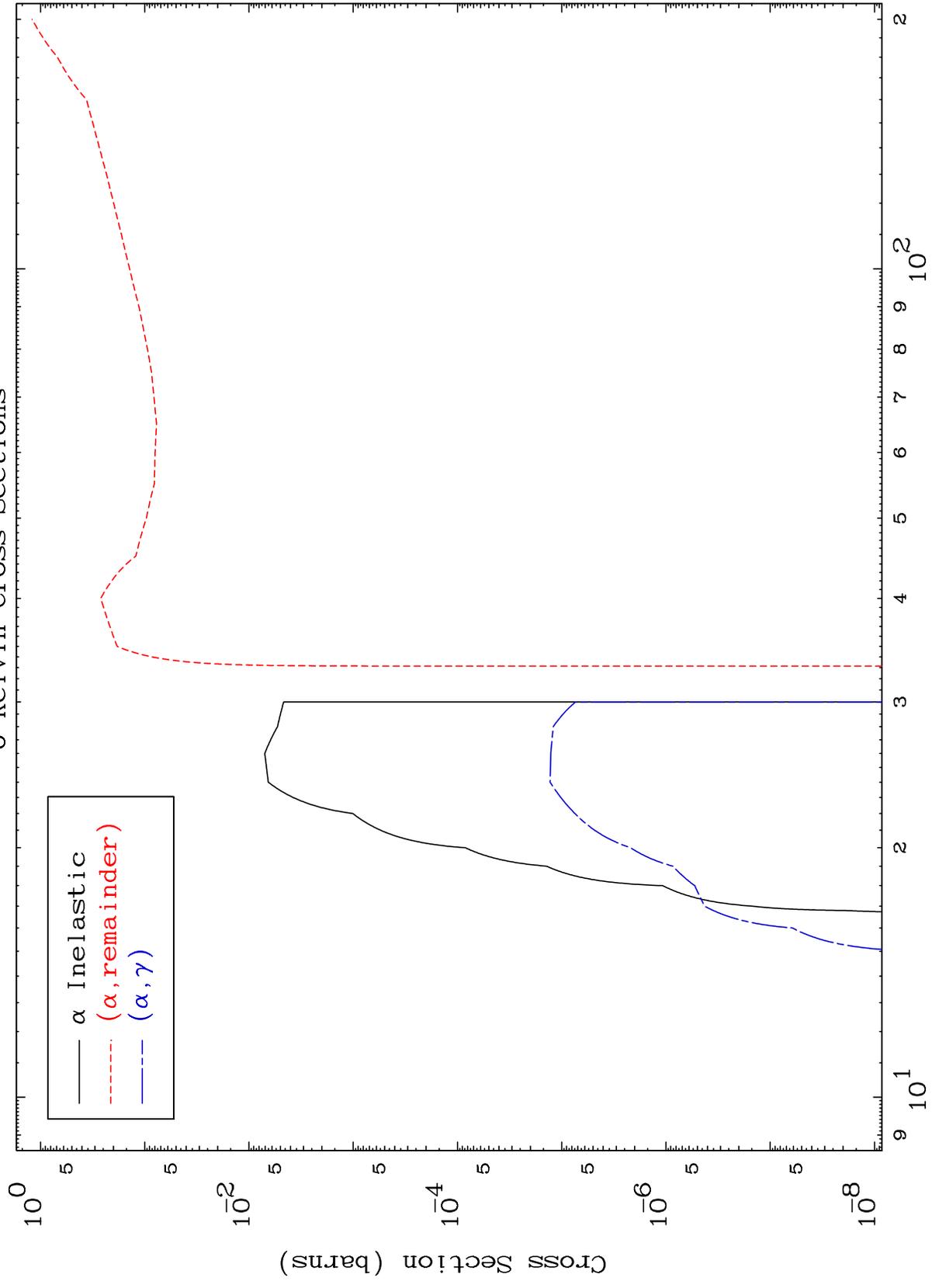
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9952

0 Kelvin  $\alpha$  Major  
Cross Sections

101-Md-246



1

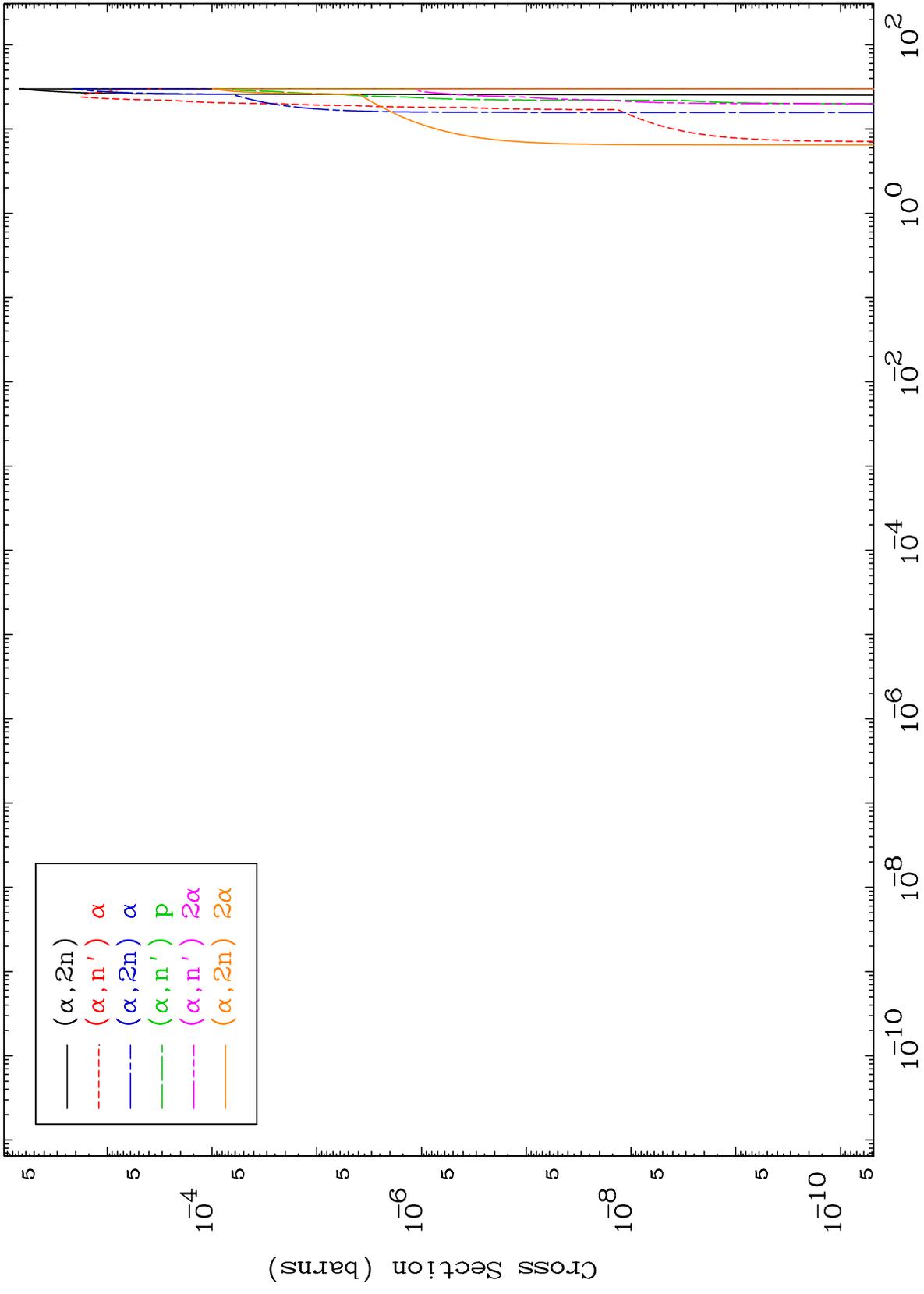
Incident Energy (MeV)

101-Md-246

MAT 9952

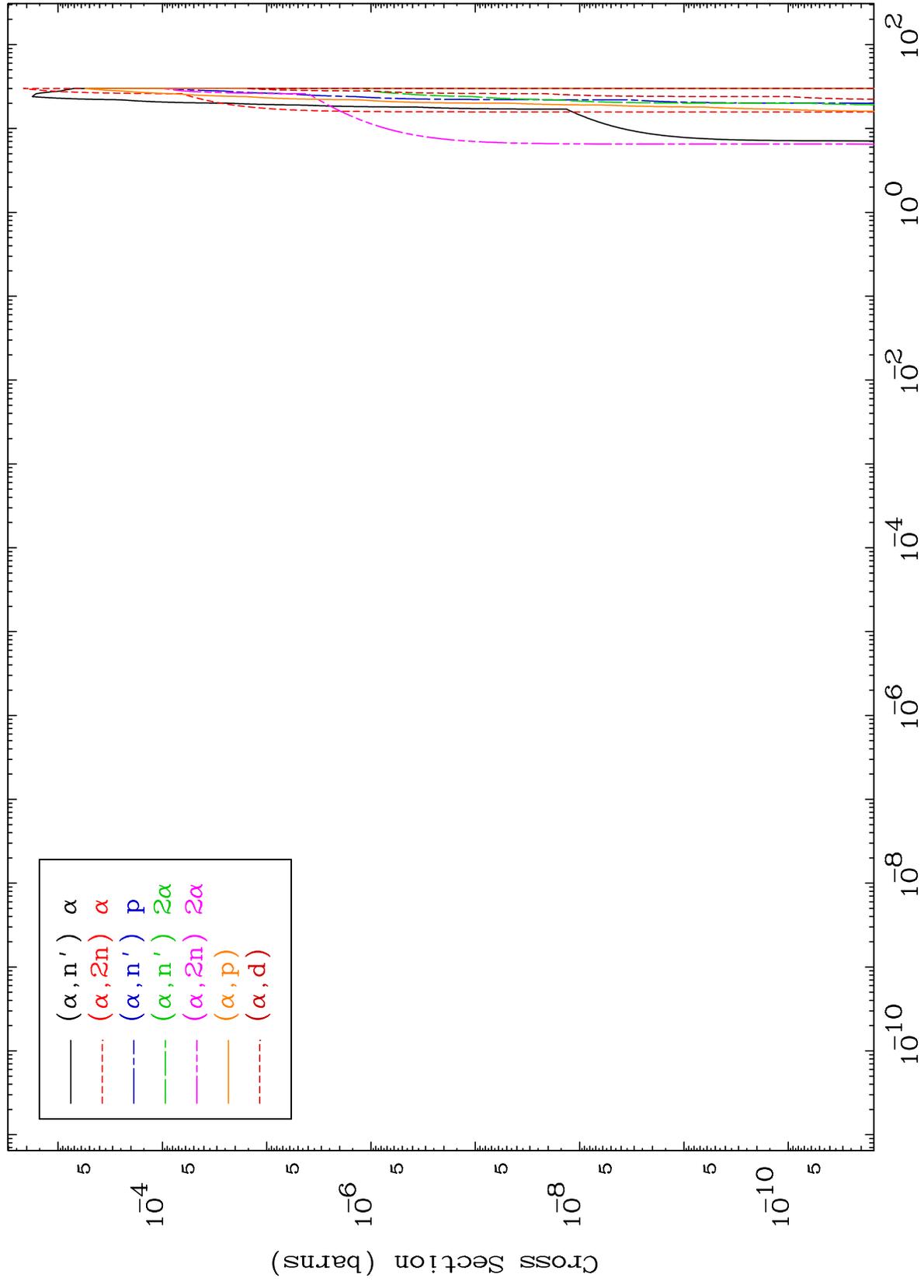
$\alpha$  Neutron Production  
0 Kelvin Cross Sections

101-Md-246



2

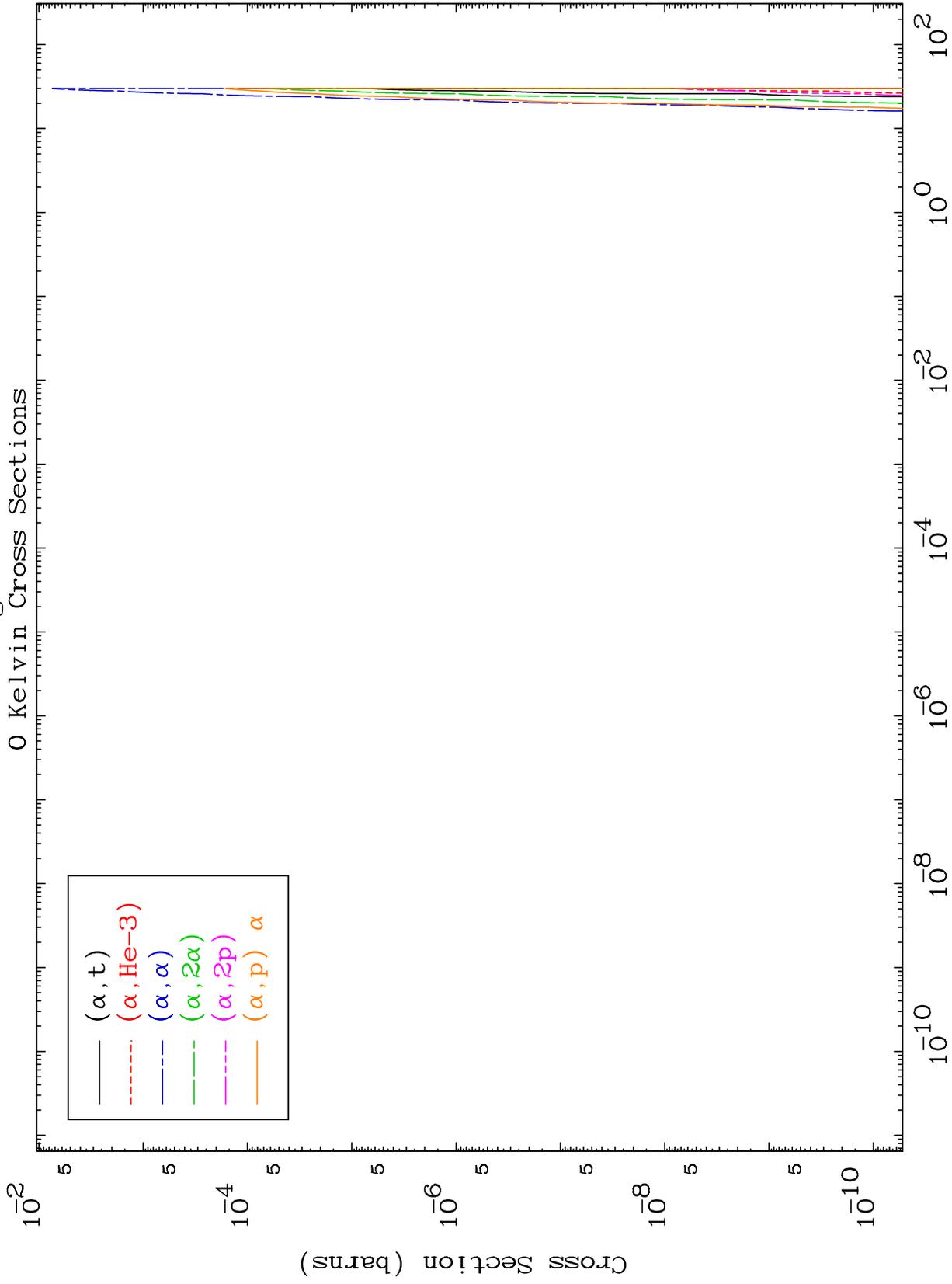
101-Md-246



MAT 9952

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

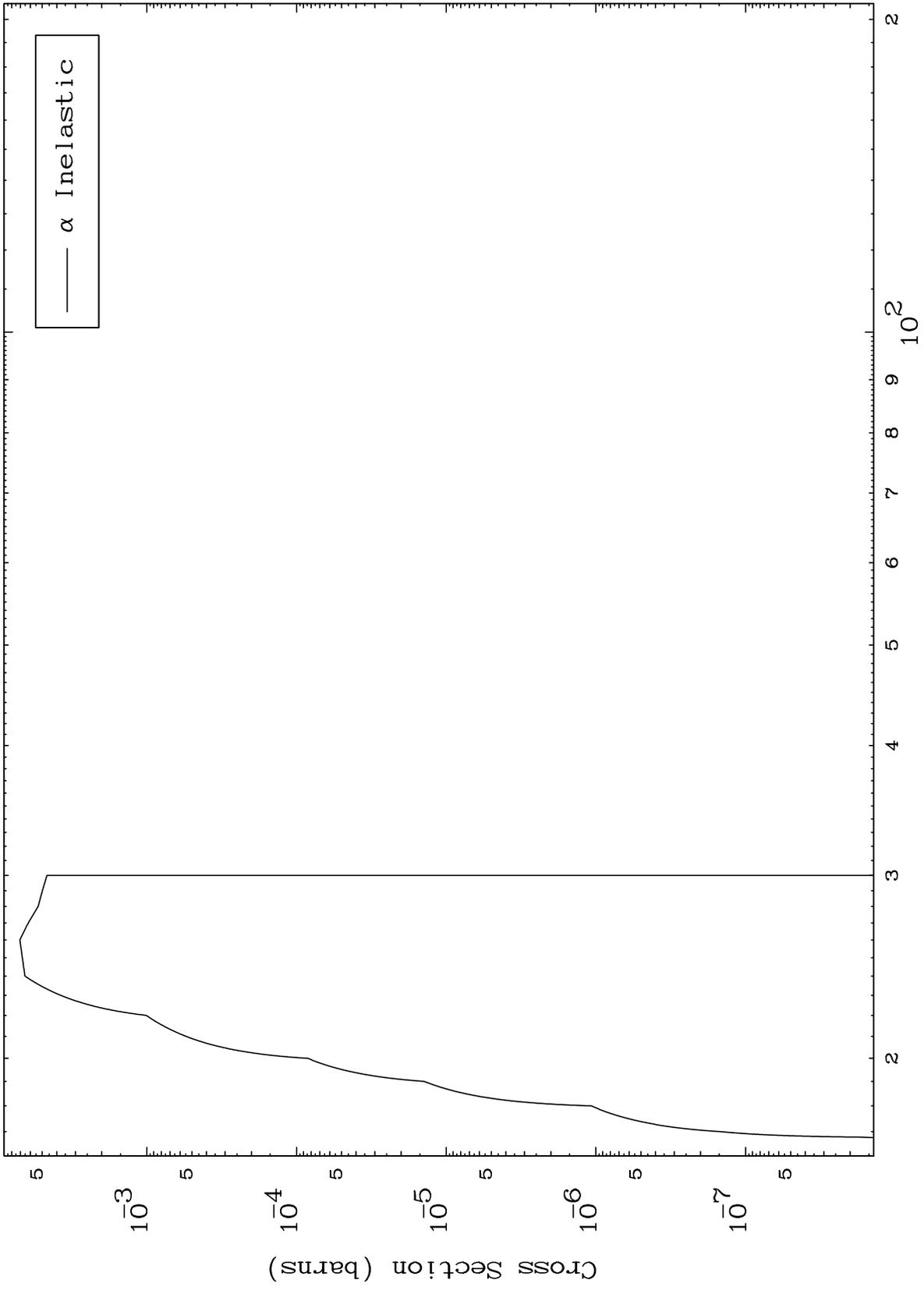
101-Md-246



MAT 9952

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

101-Md-246



5

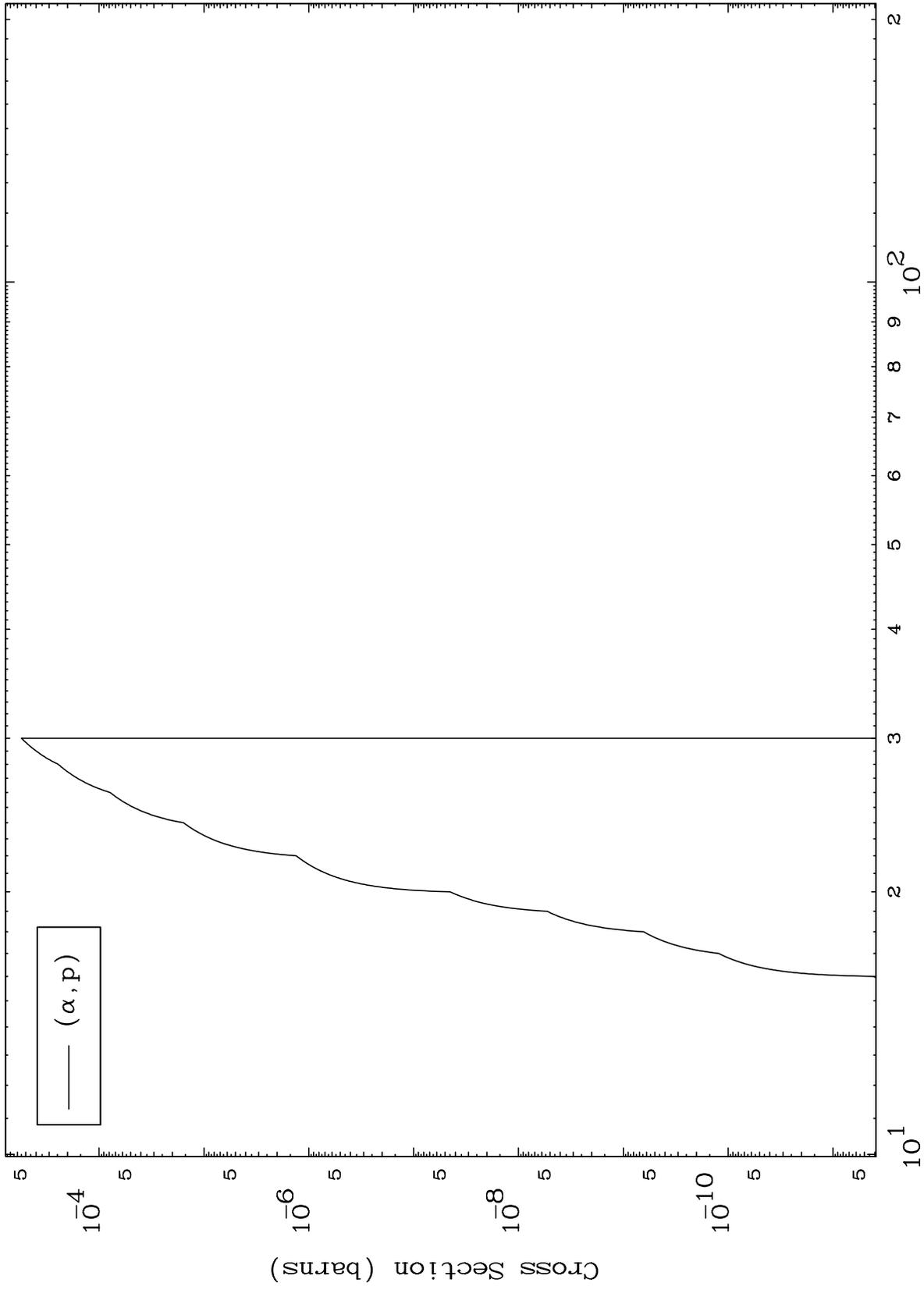
Incident Energy (MeV)

101-Md-246

MAT 9952

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

101-Md-246



Incident Energy (MeV)

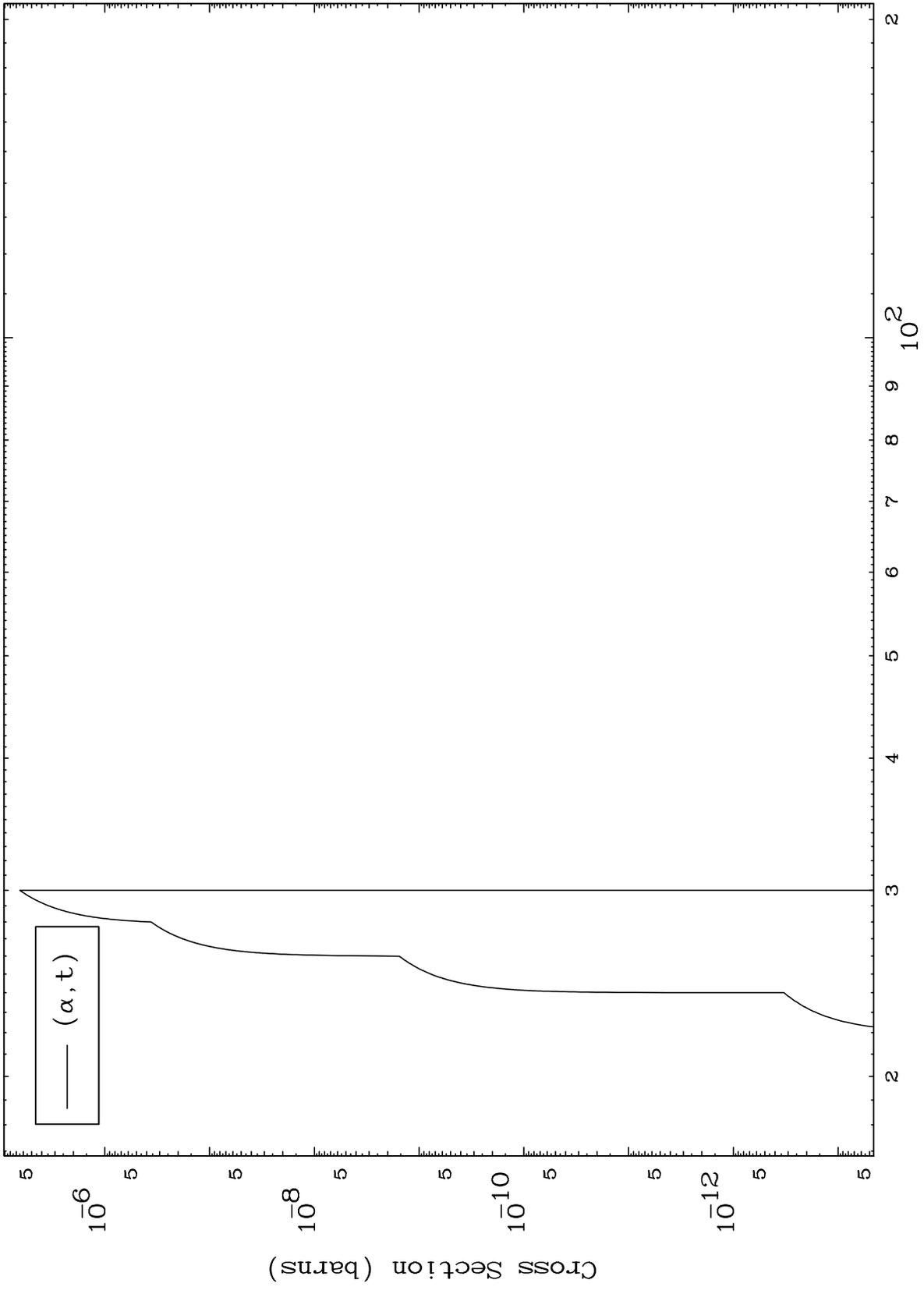
101-Md-246



MAT 9952

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

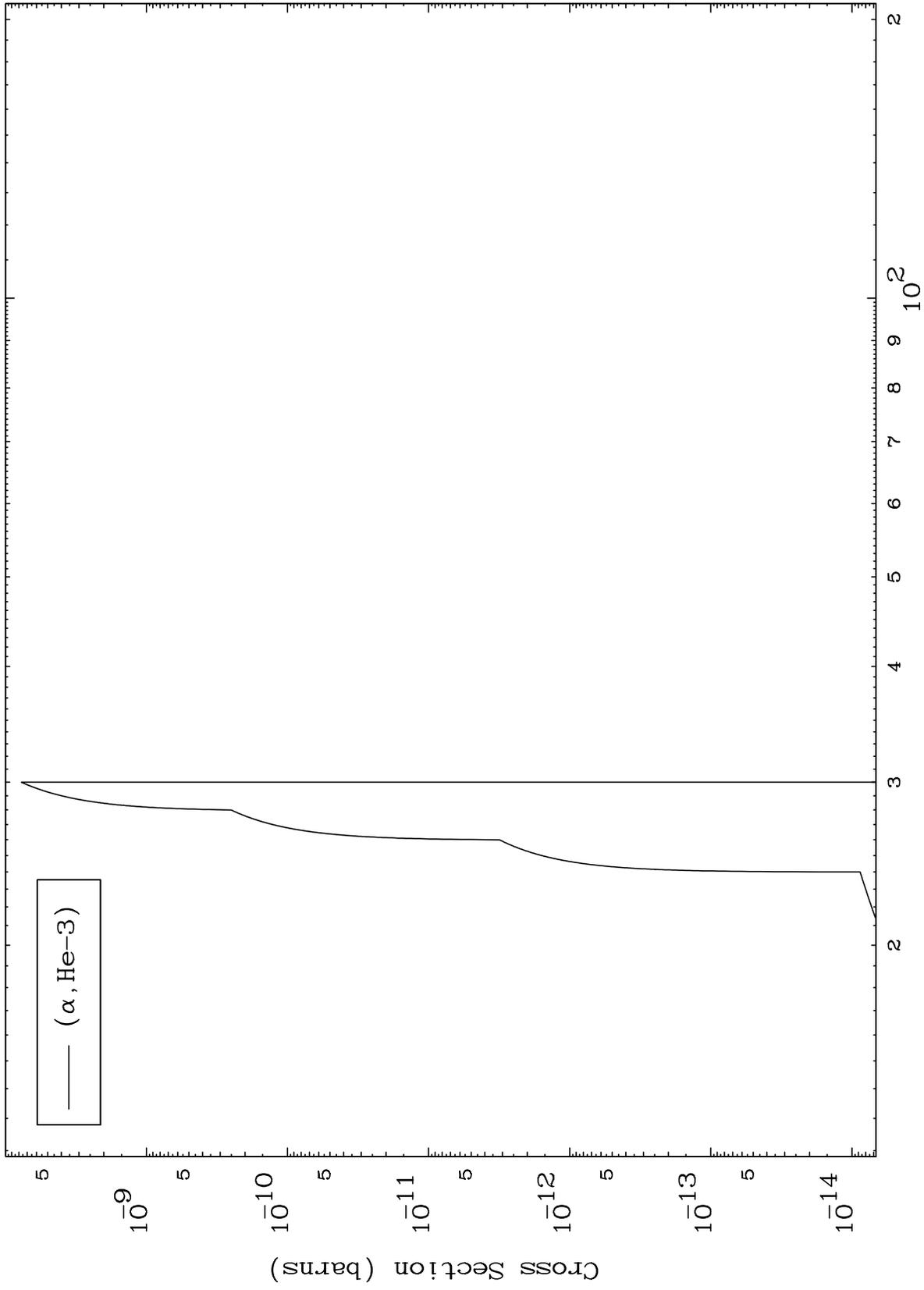
101-Md-246



8

Incident Energy (MeV)

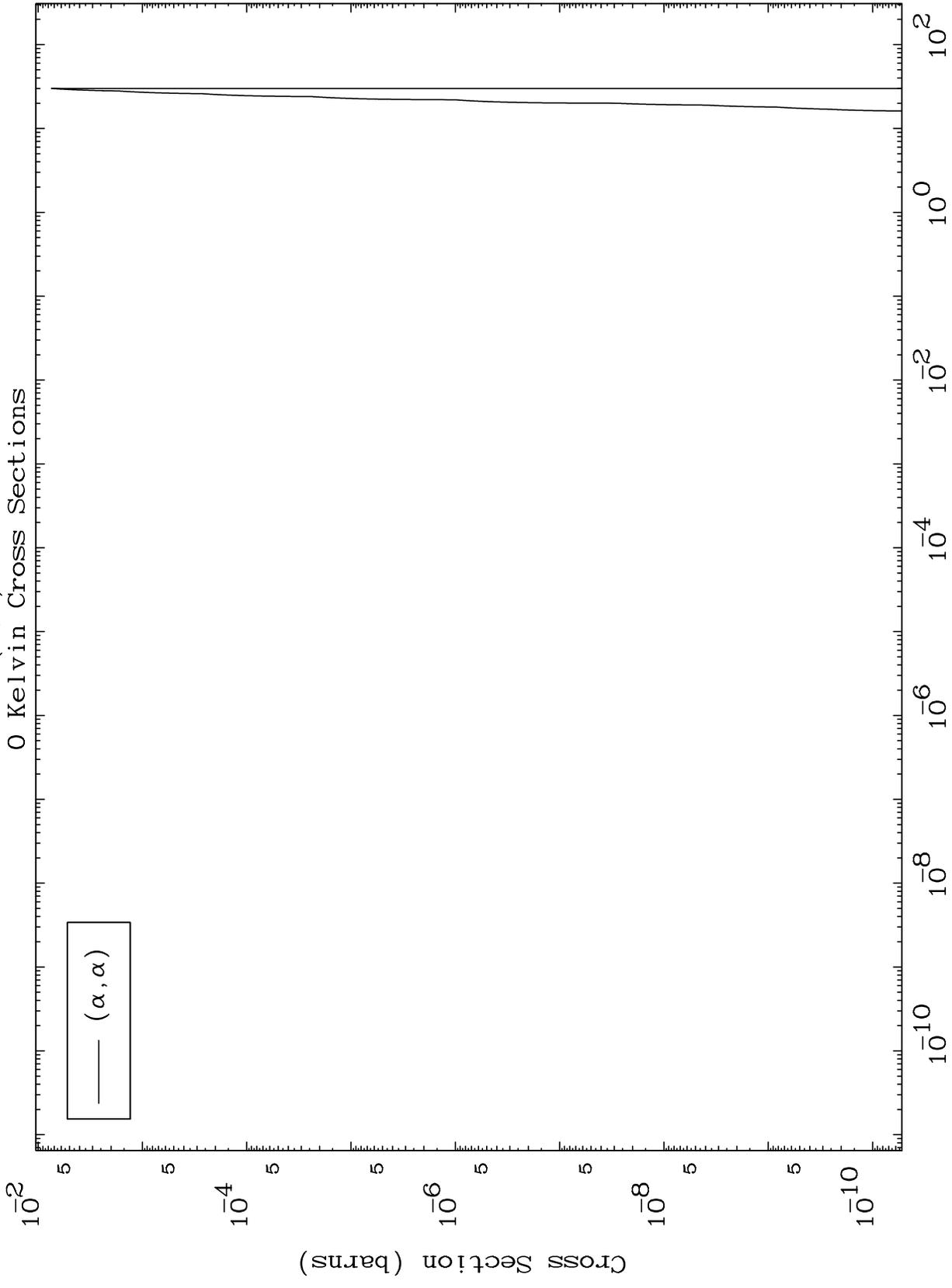
101-Md-246



MAT 9952

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

101-Md-246



10

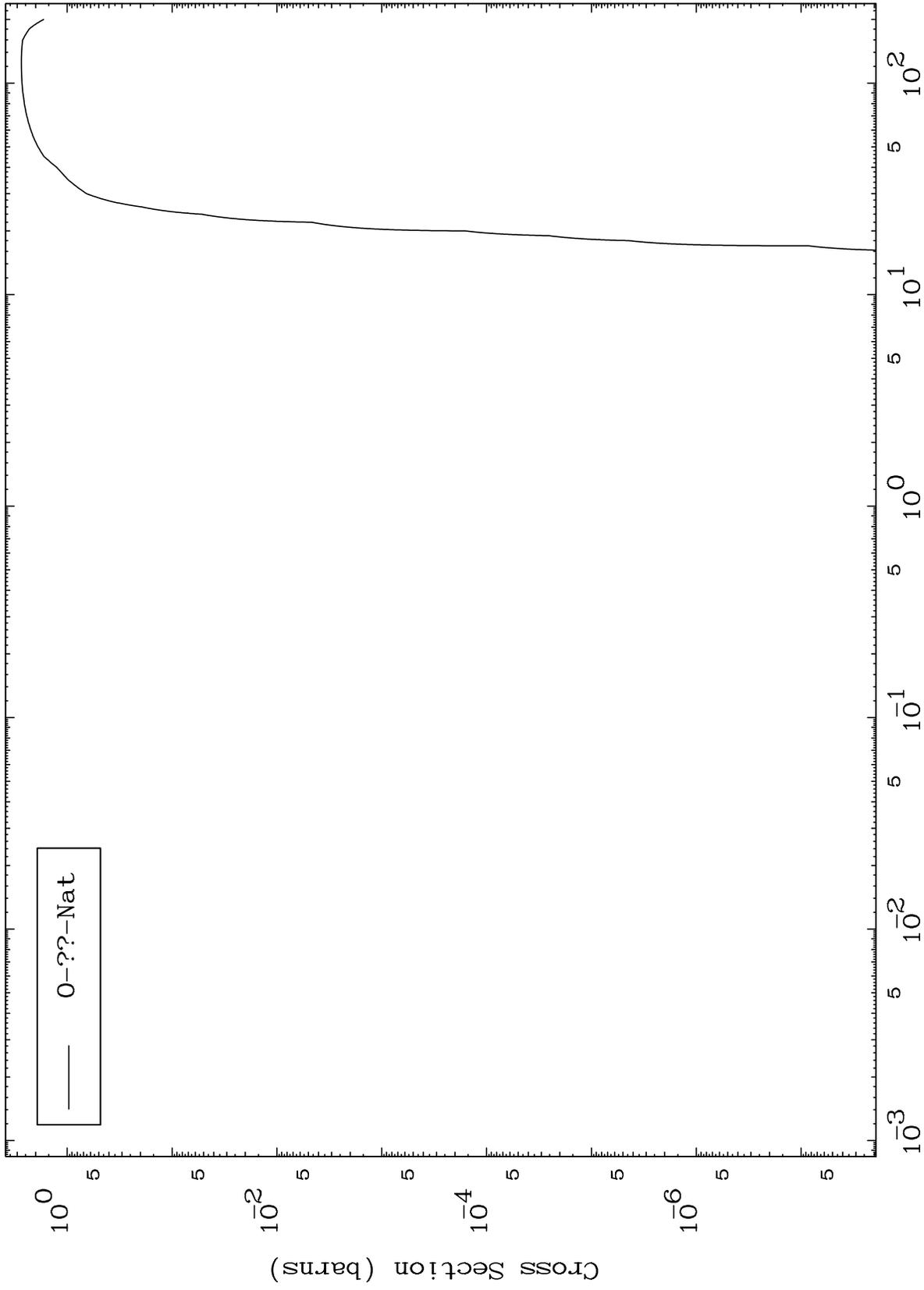
Incident Energy (MeV)

101-Md-246

MAT 9952

$\alpha$  Fission  
Radionuclide Production Cross Section

101-Md-246



11

Incident Energy (MeV)

101-Md-246

