

Program EVALPLOT  
(Version 2018-1)

by

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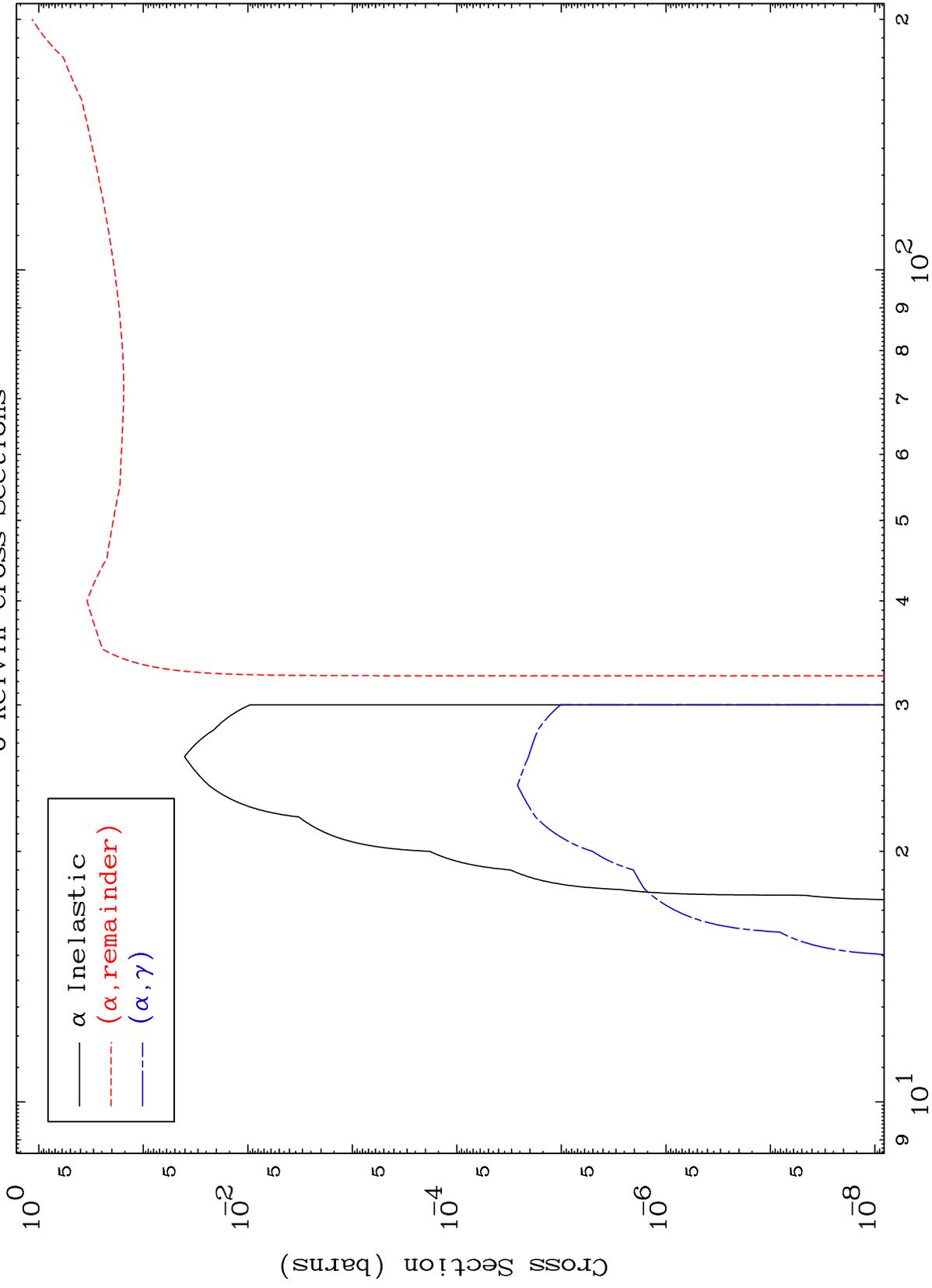
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9950

0 Kelvin  $\alpha$  Major  
Cross Sections

101-Md-249



101-Md-249

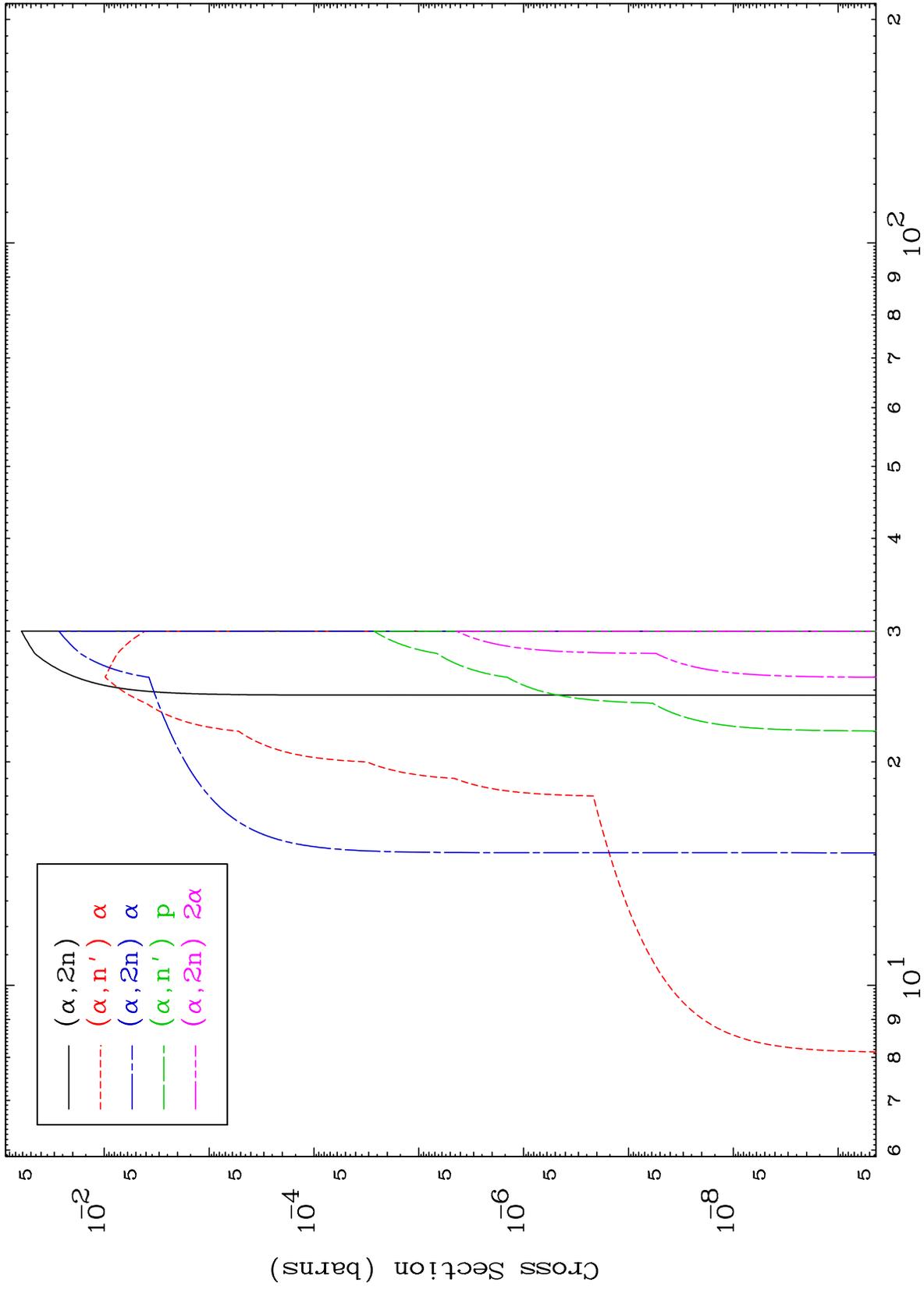
Incident Energy (MeV)

1

MAT 9950

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

101-Md-249



2

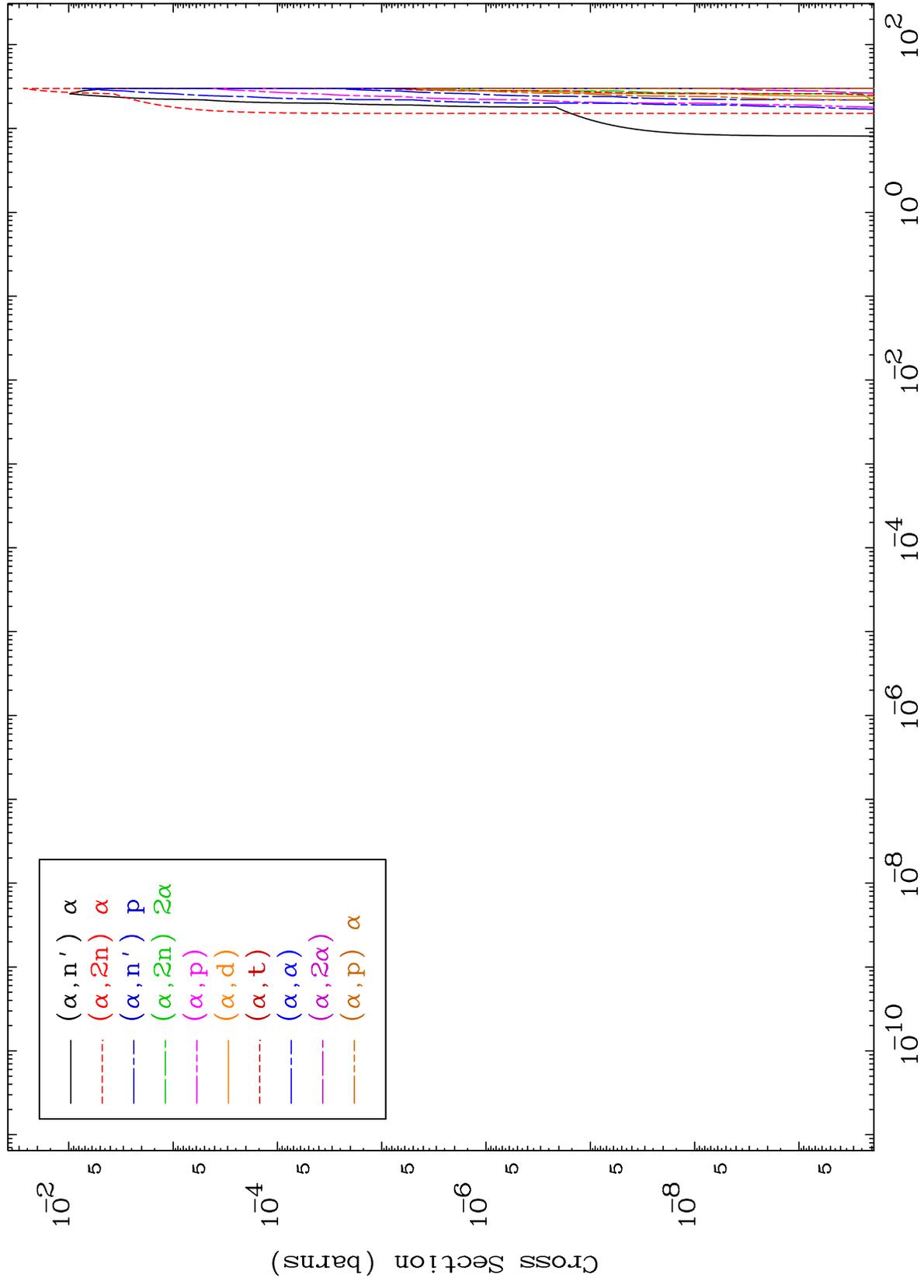
Incident Energy (MeV)

101-Md-249

MAT 9950

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

101-Md-249



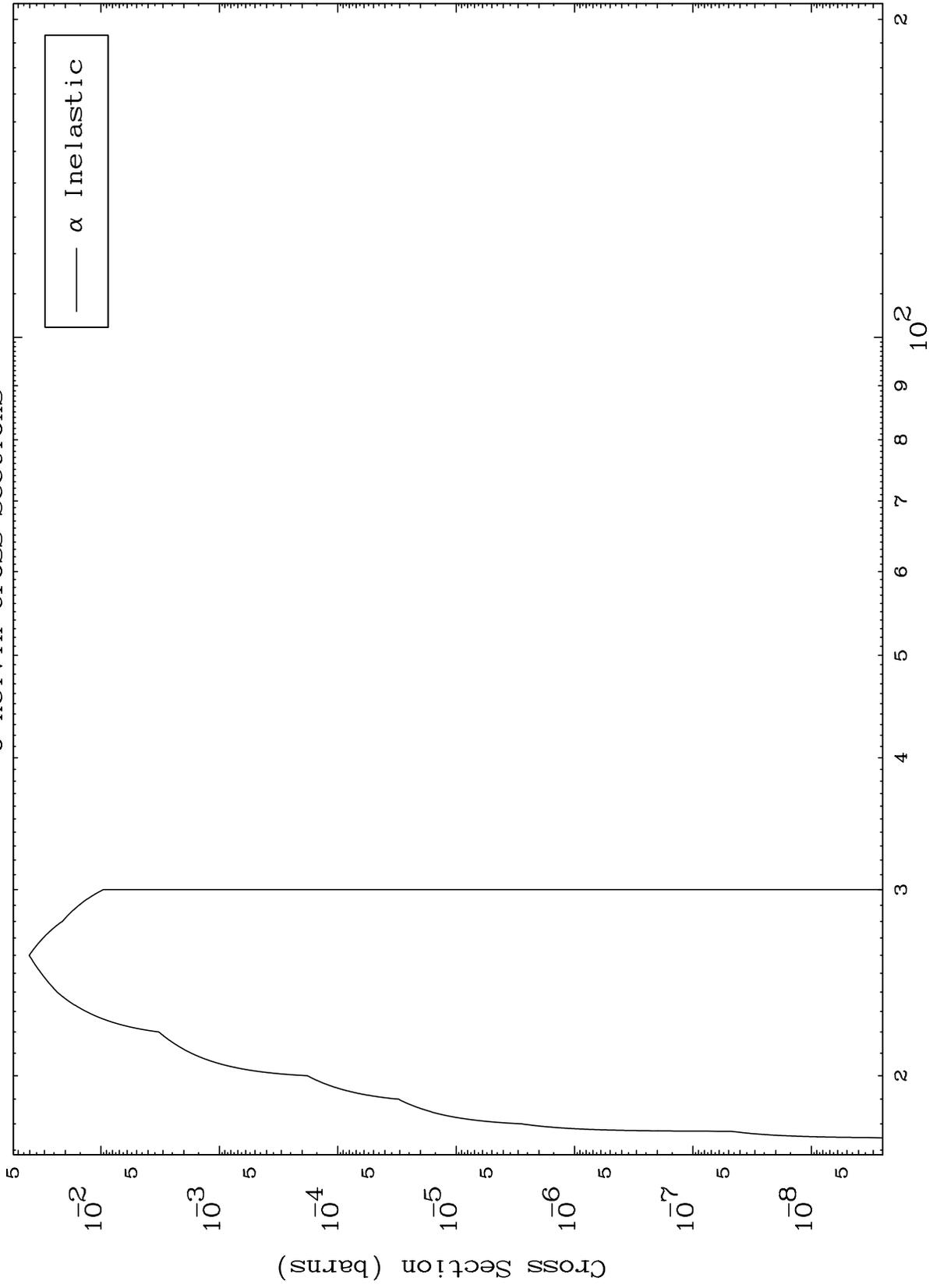
101-Md-249

Incident Energy (MeV)

MAT 9950

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

101-Md-249



4

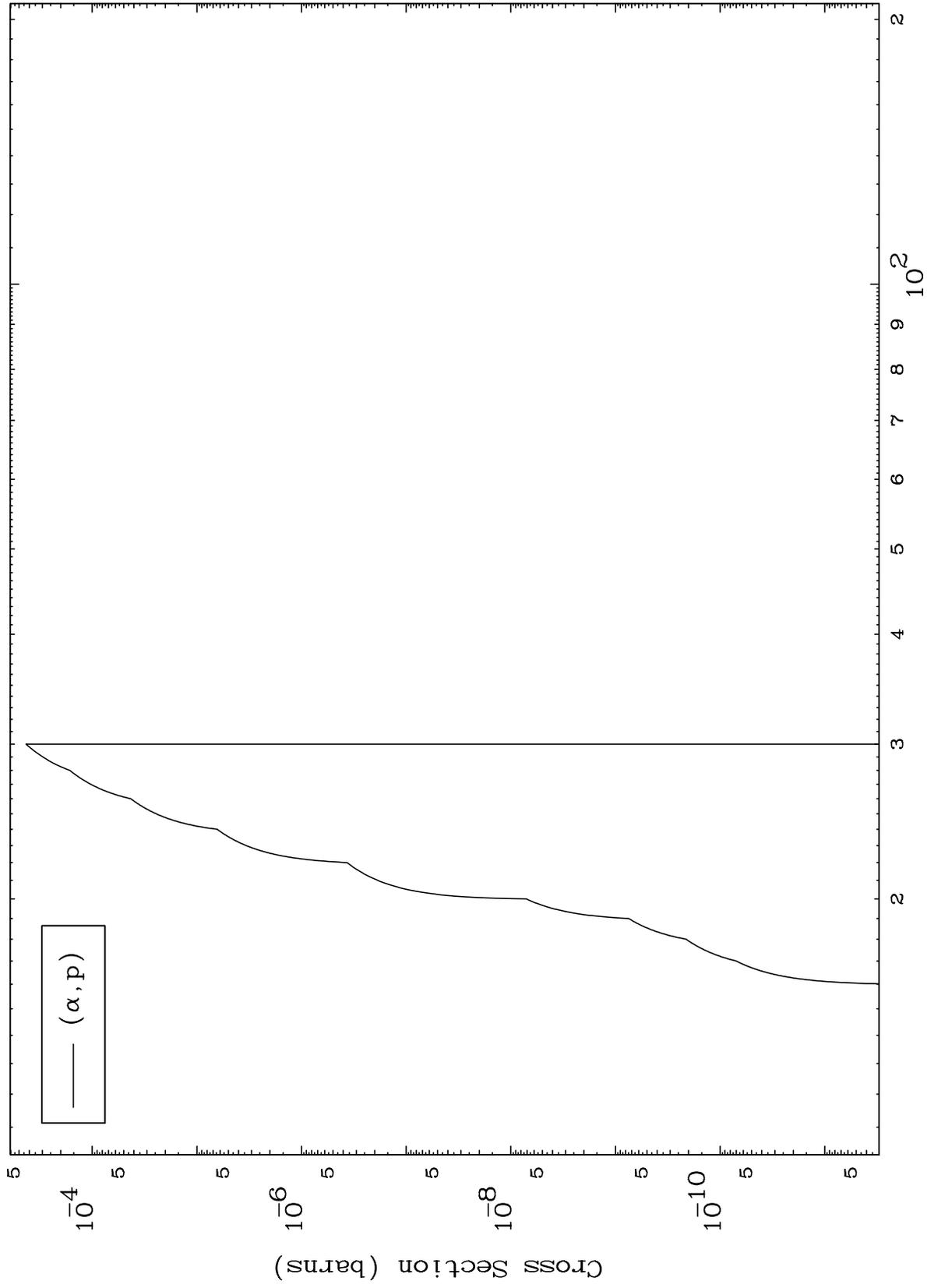
Incident Energy (MeV)

101-Md-249

MAT 9950

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

101-Md-249



5

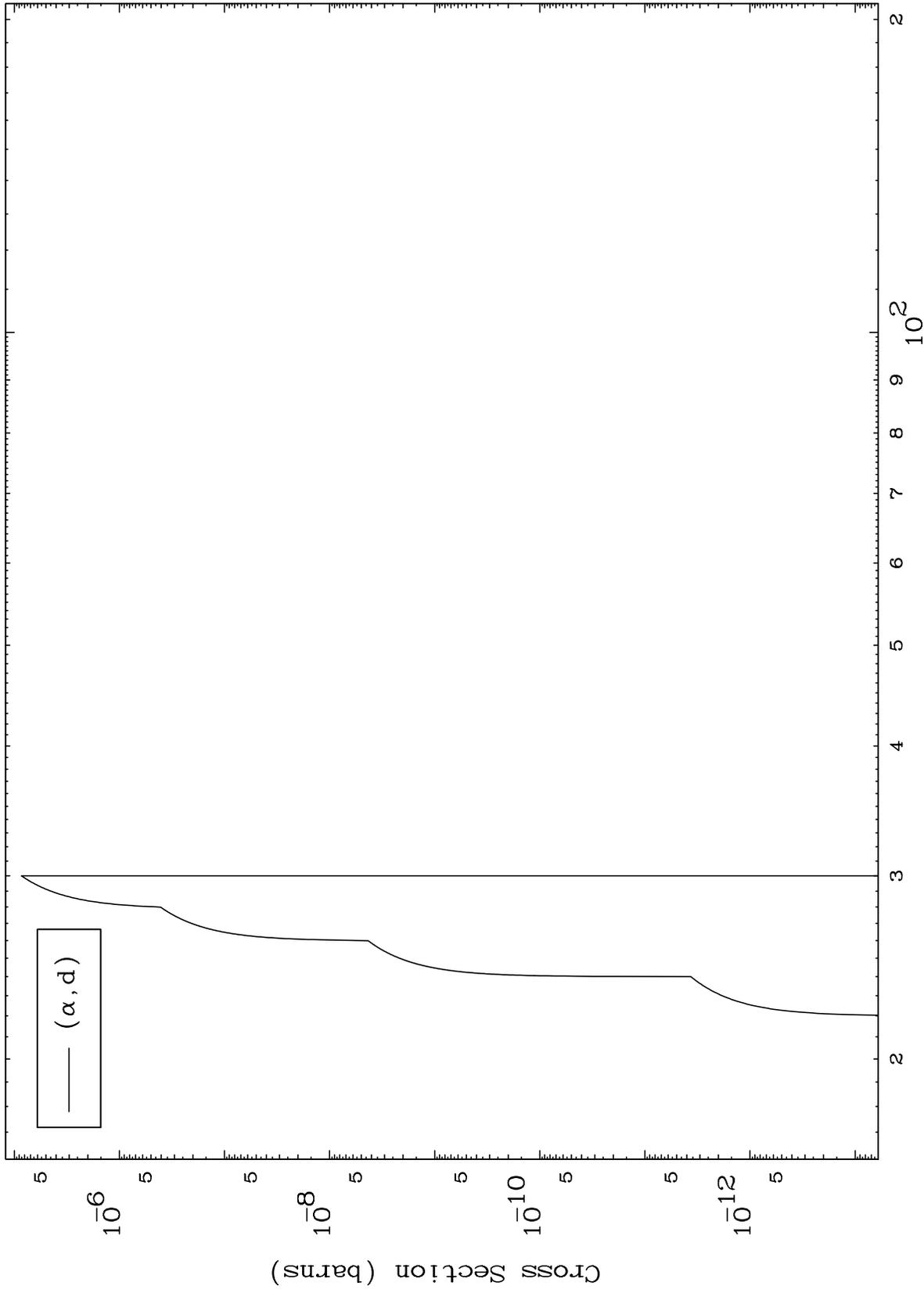
Incident Energy (MeV)

101-Md-249

MAT 9950

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

101-Md-249



6

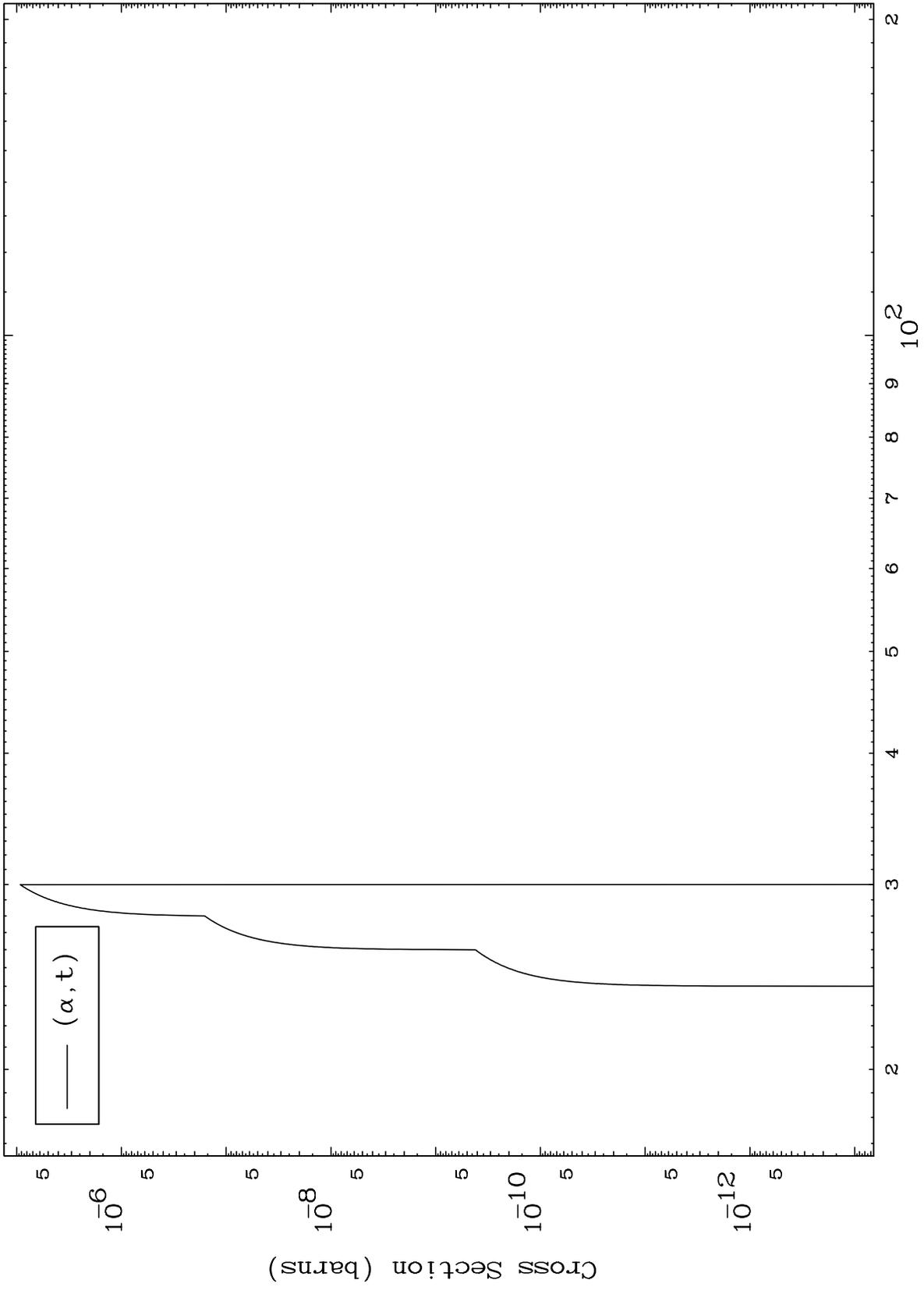
Incident Energy (MeV)

101-Md-249

MAT 9950

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

101-Md-249



7

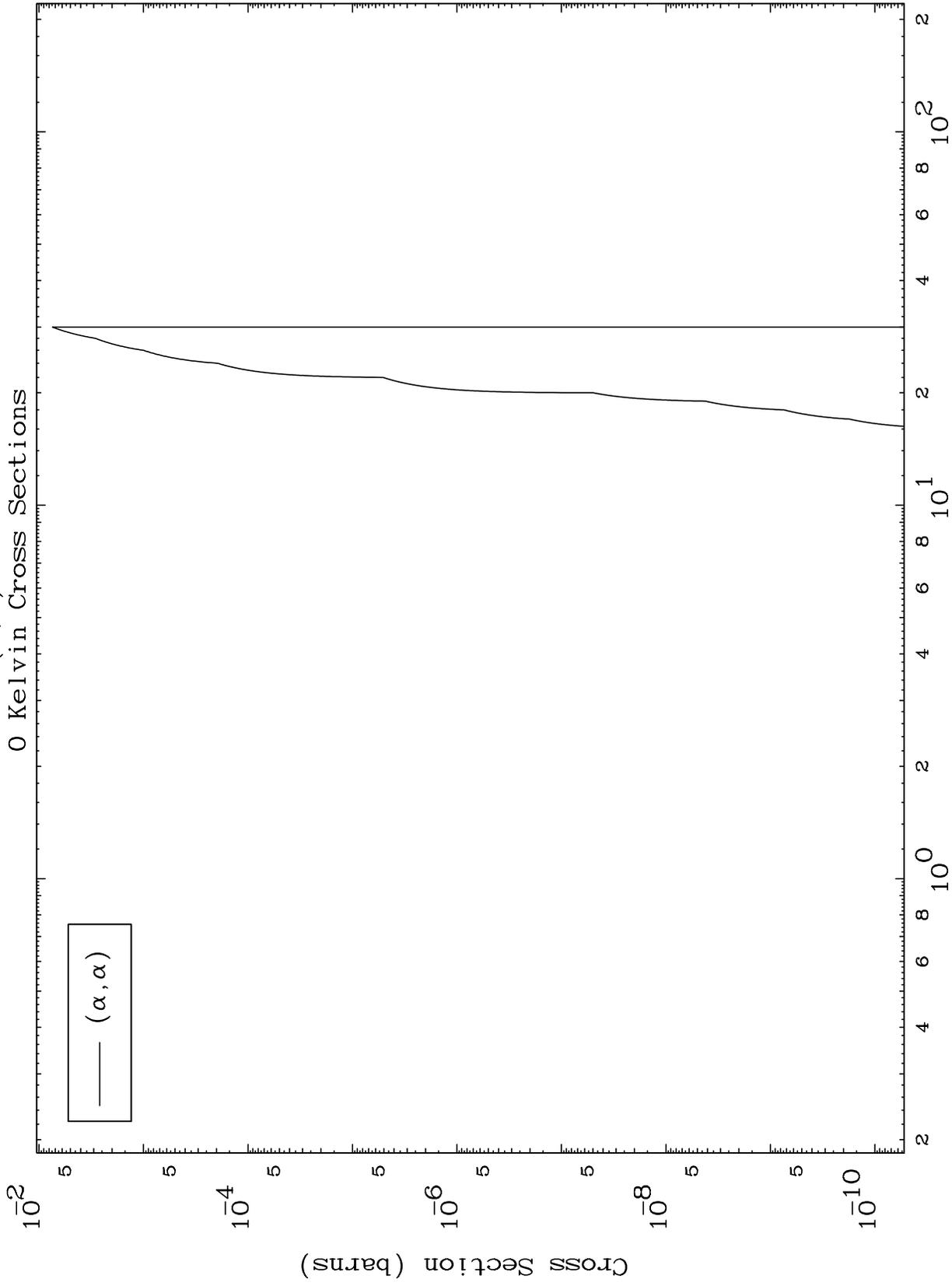
Incident Energy (MeV)

101-Md-249

MAT 9950

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

101-Md-249



8

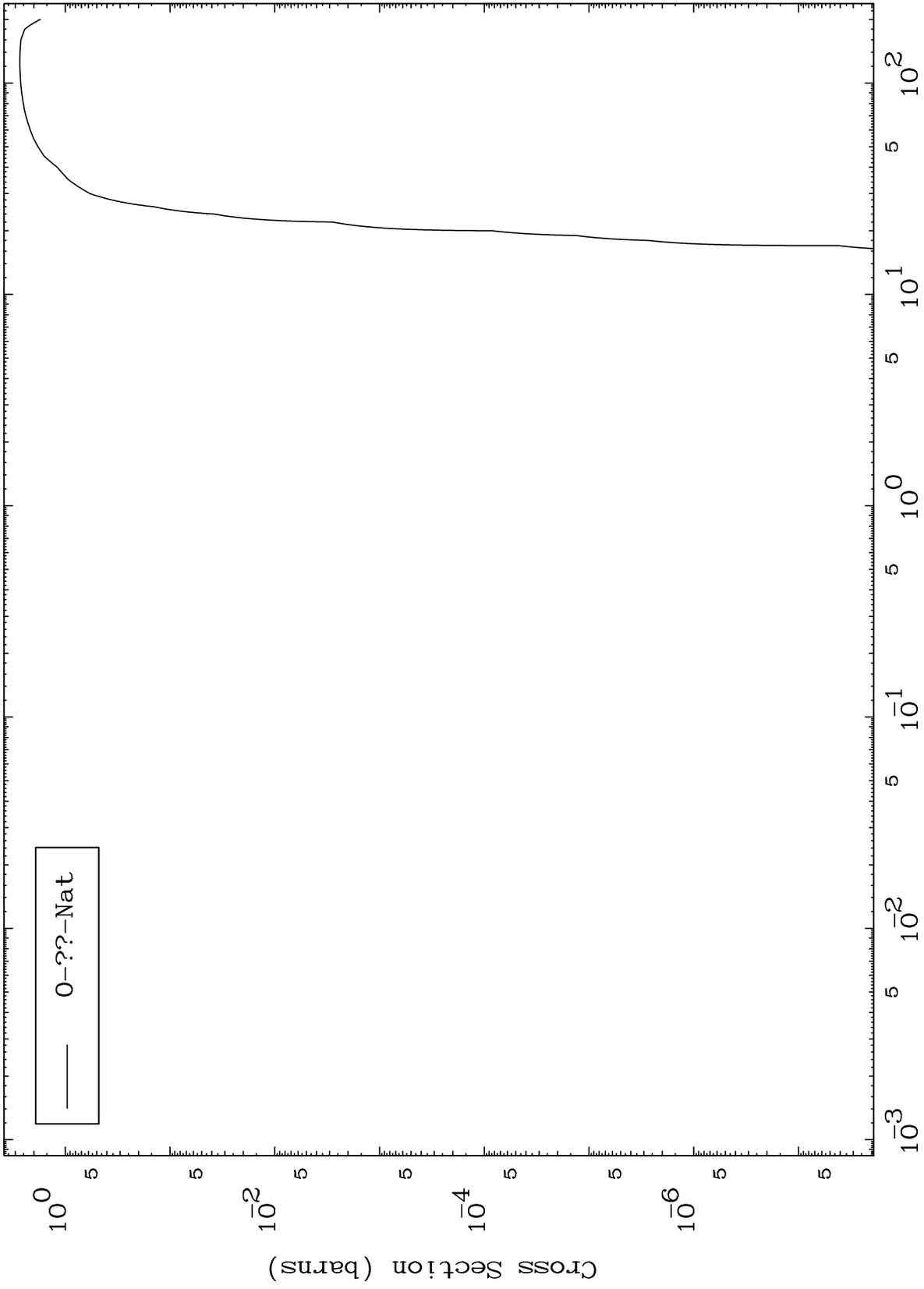
Incident Energy (MeV)

101-Md-249

MAT 9950

$\alpha$  Fission  
Radionuclide Production Cross Section

101-Md-249



Incident Energy (MeV)

101-Md-249

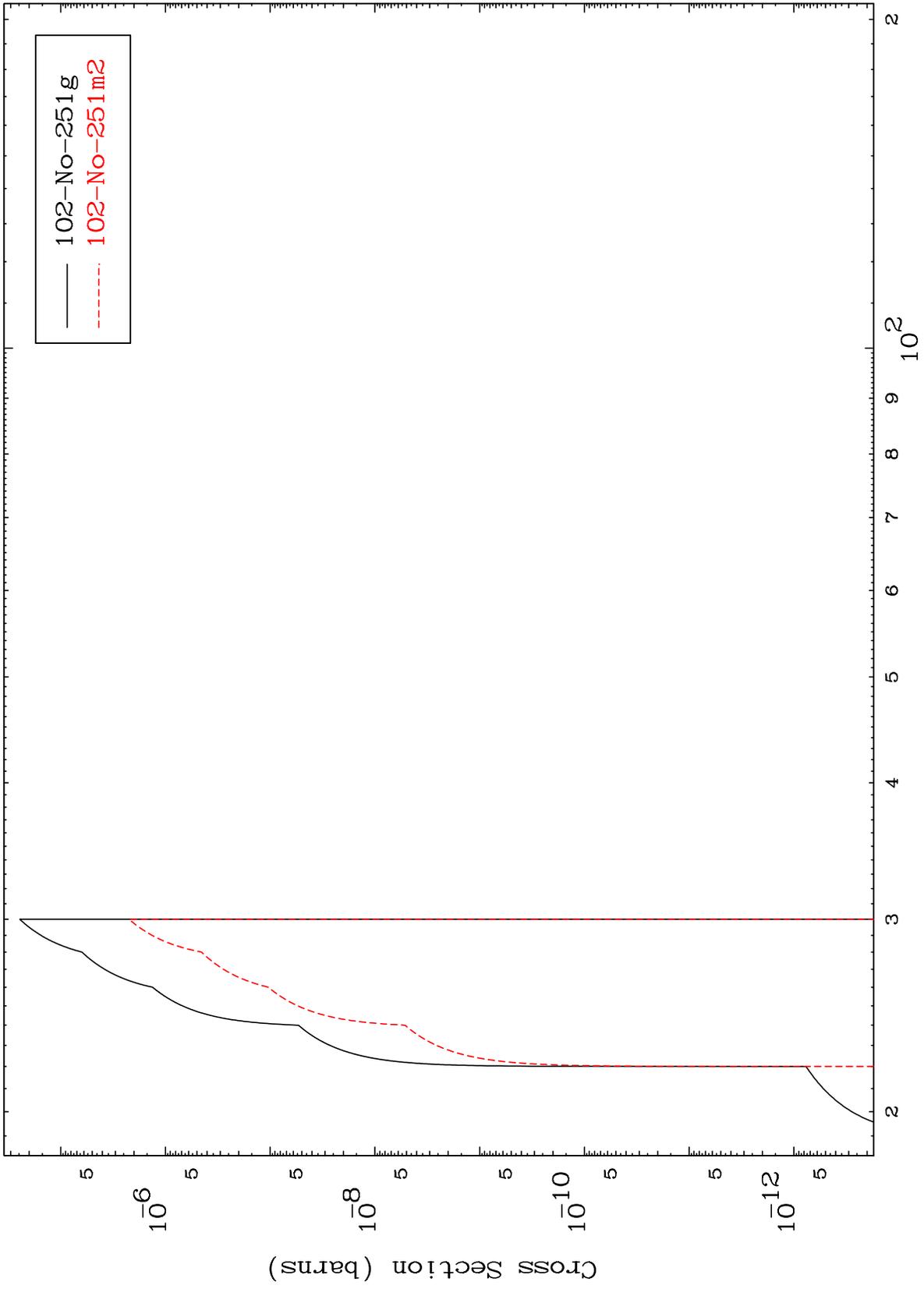
9

MAT 9950

$(\alpha, n')$  p

101-Md-249

Radionuclide Production Cross Section



10

Incident Energy (MeV)

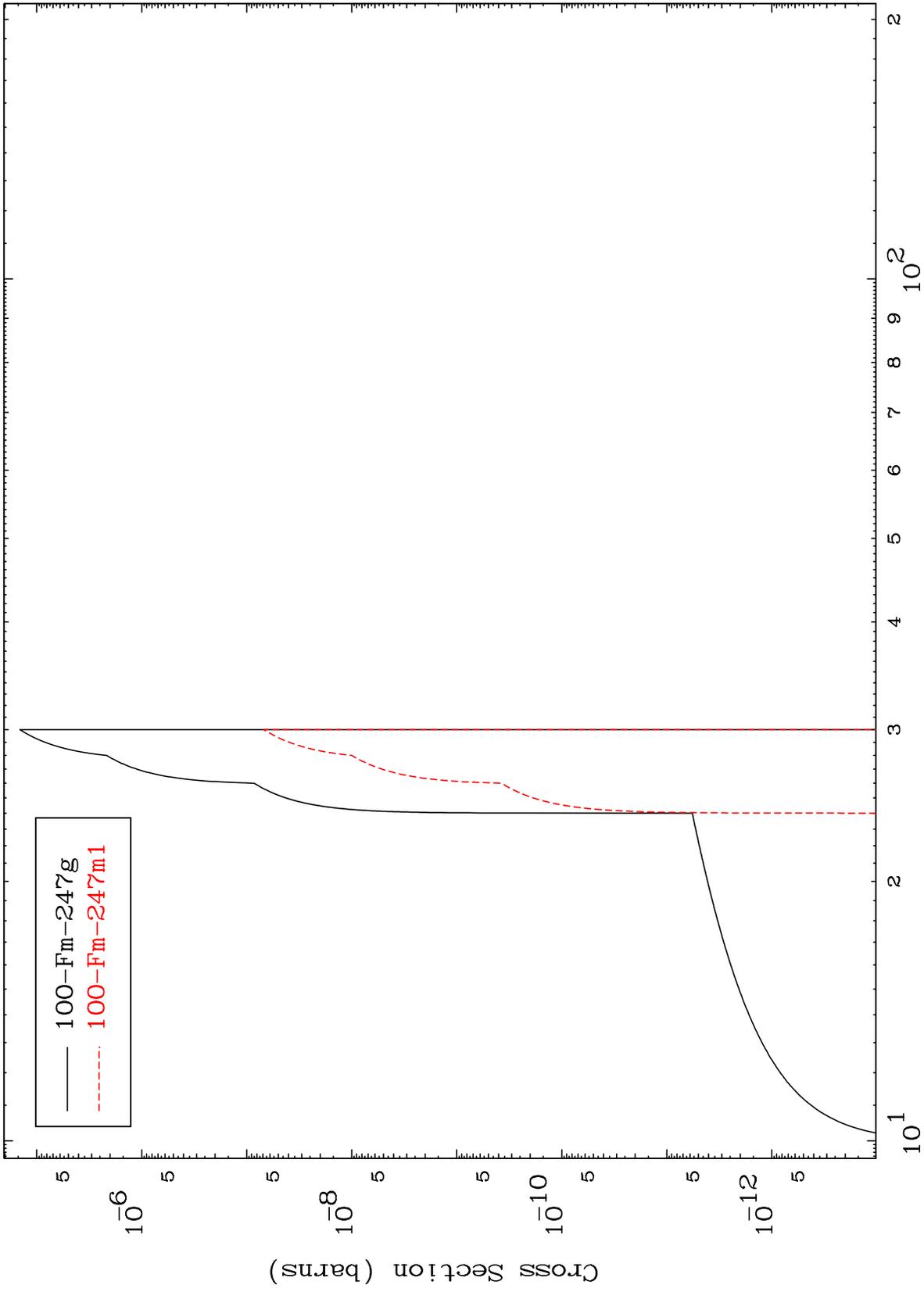
101-Md-249

MAT 9950

$(\alpha, n')$  p  $\alpha$

101-Md-249

Radionuclide Production Cross Section



11

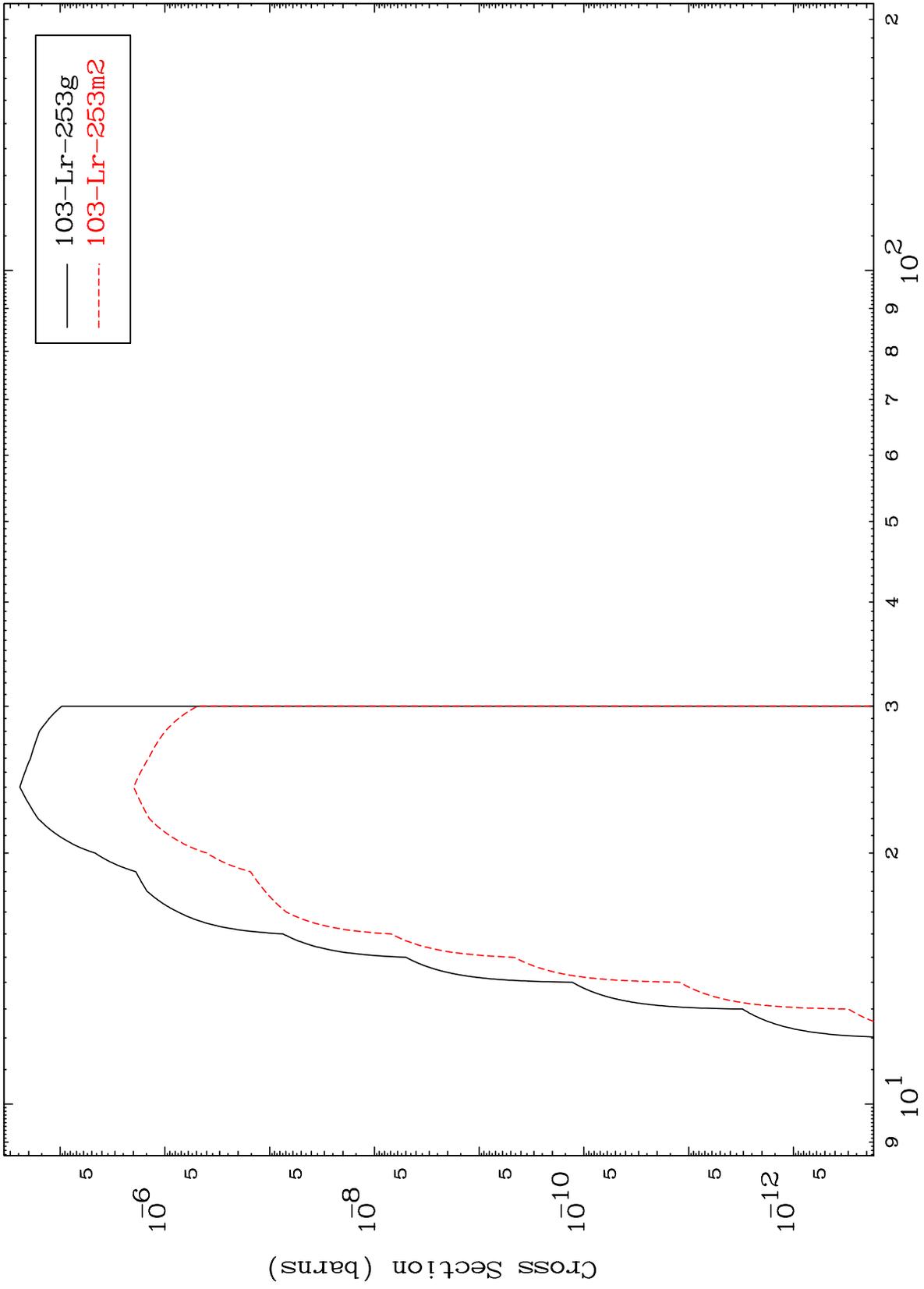
Incident Energy (MeV)

101-Md-249

MAT 9950

101-Md-249

$(\alpha, \gamma)$   
Radionuclide Production Cross Section



12

Incident Energy (MeV)

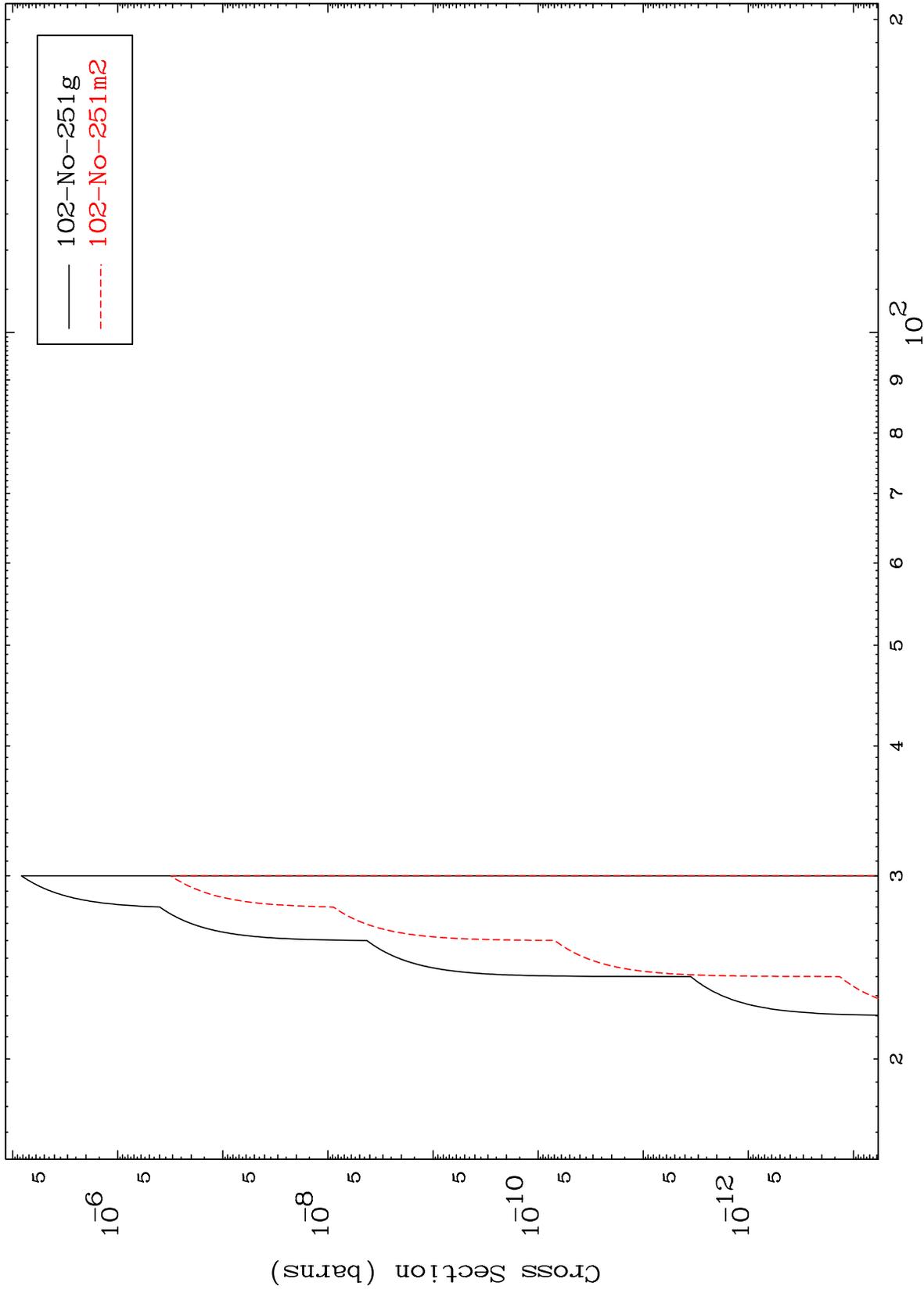
101-Md-249

MAT 9950

( $\alpha, d$ )

101-Md-249

Radionuclide Production Cross Section



13

Incident Energy (MeV)

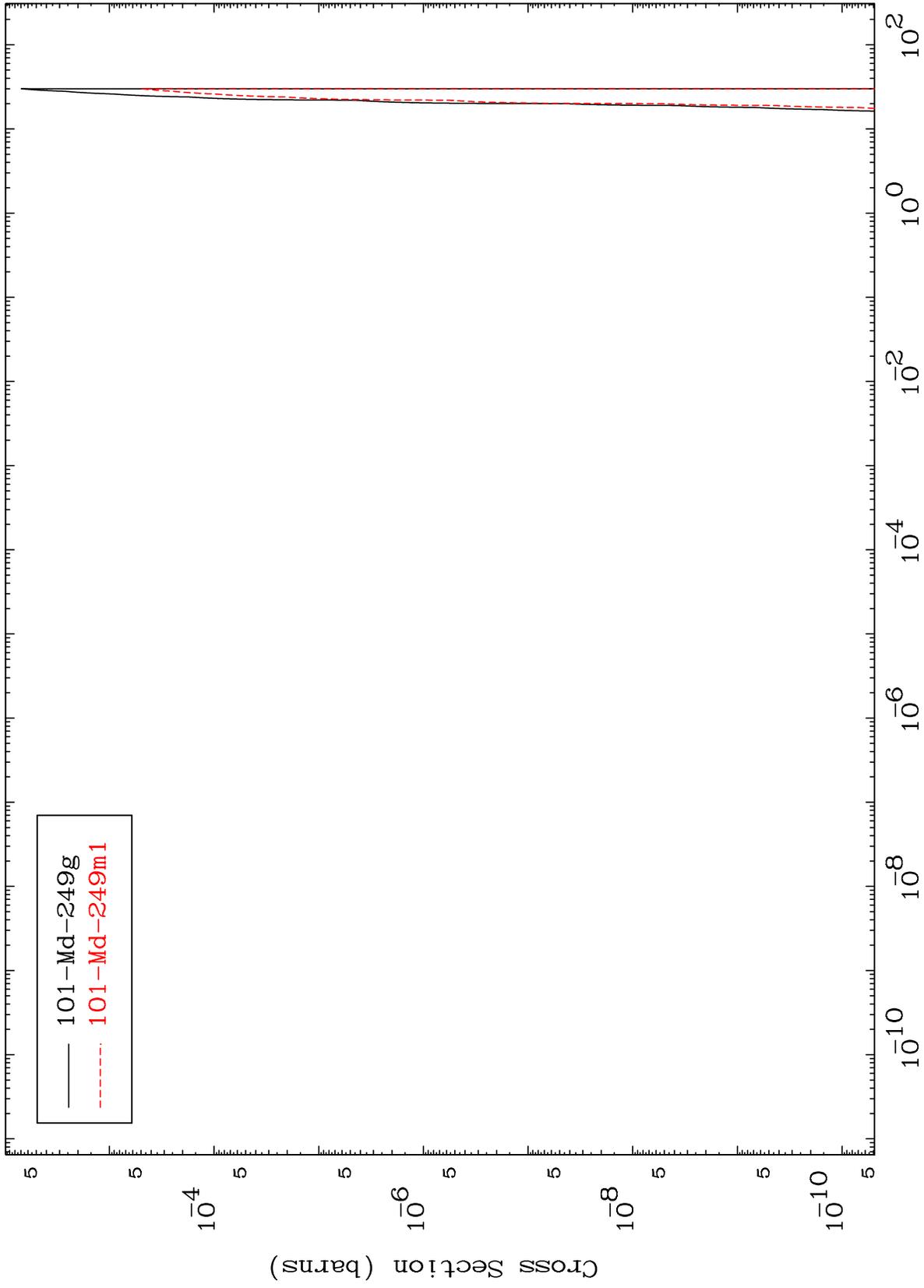
101-Md-249

MAT 9950

( $\alpha, \alpha$ )

101-Md-249

Radionuclide Production Cross Section

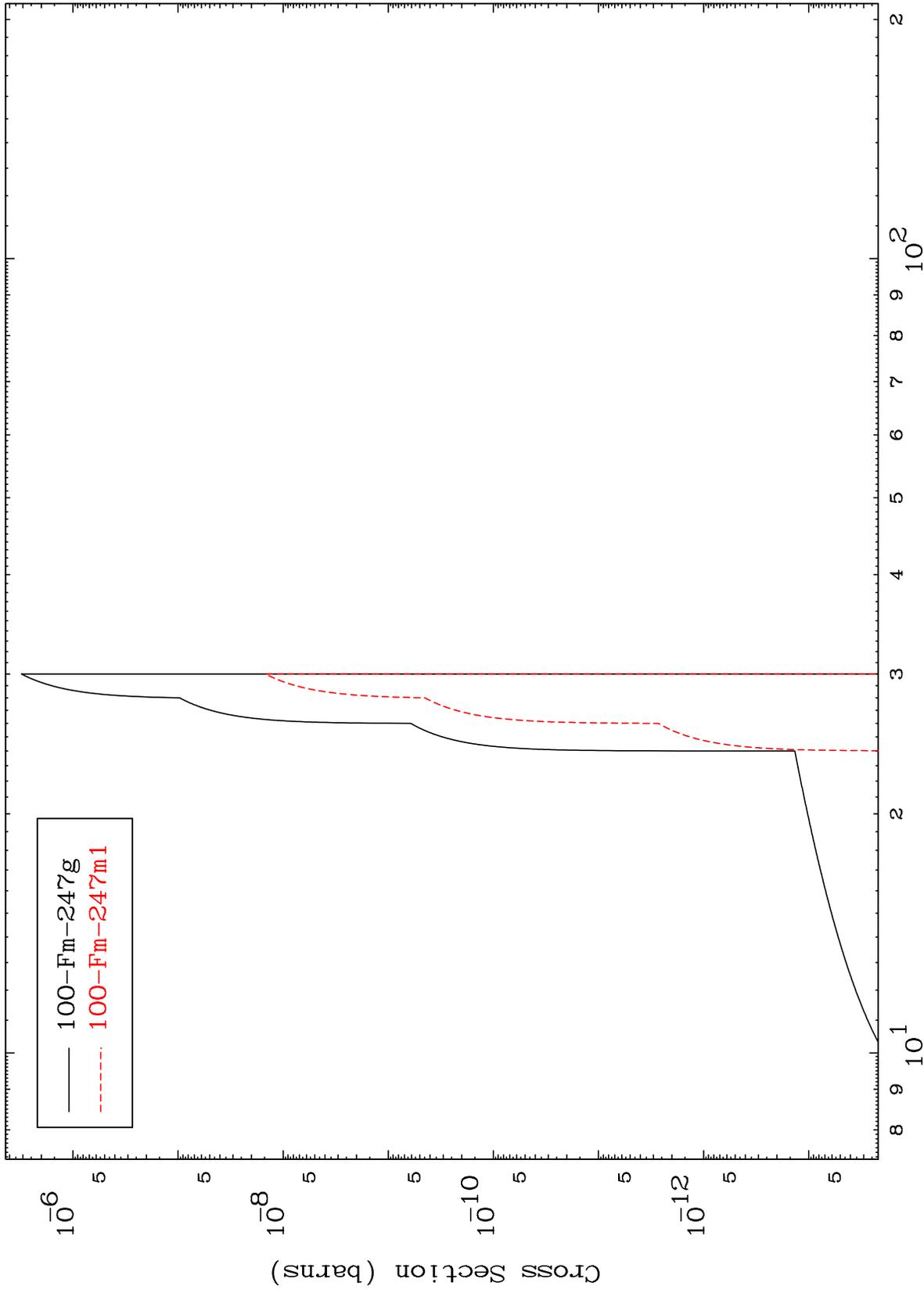


MAT 9950

( $\alpha, d$ )  $\alpha$

101-Md-249

Radionuclide Production Cross Section



15

Incident Energy (MeV)

101-Md-249