

Program EVALPLOT  
(Version 2018-1)

by

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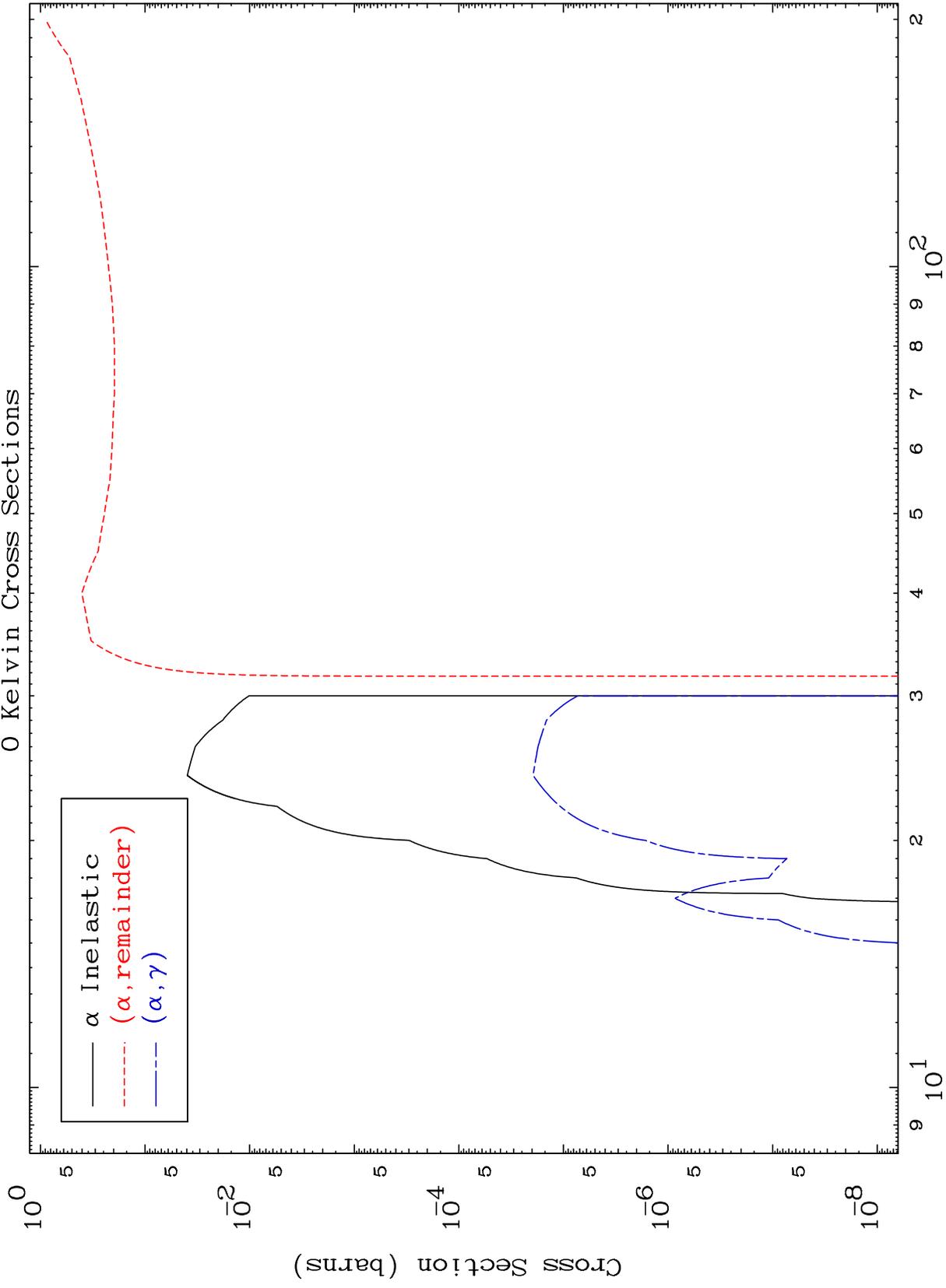
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9952

0 Kelvin  $\alpha$  Major  
Cross Sections

101-Md-251



1

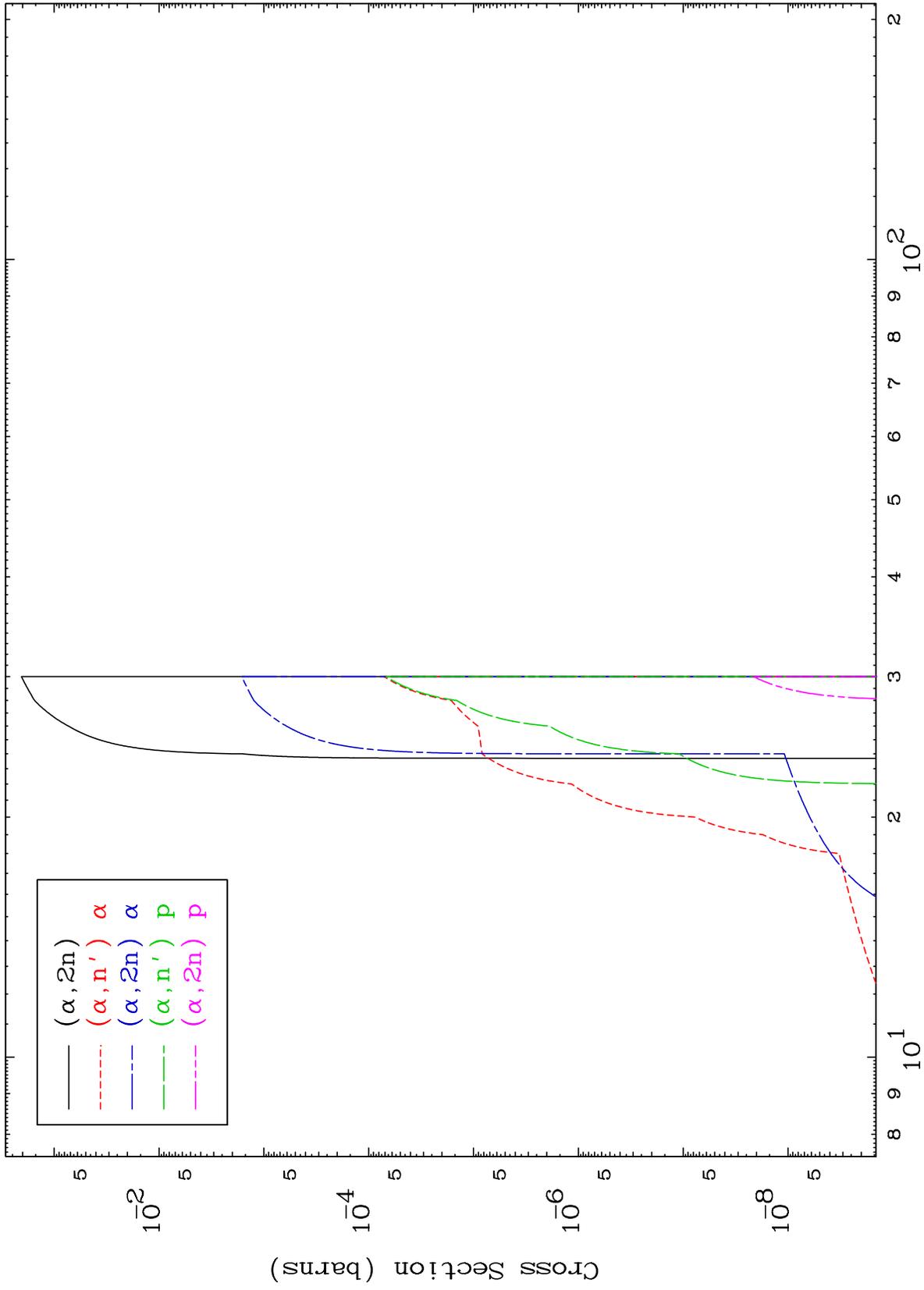
Incident Energy (MeV)

101-Md-251

MAT 9952

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

101-Md-251



2

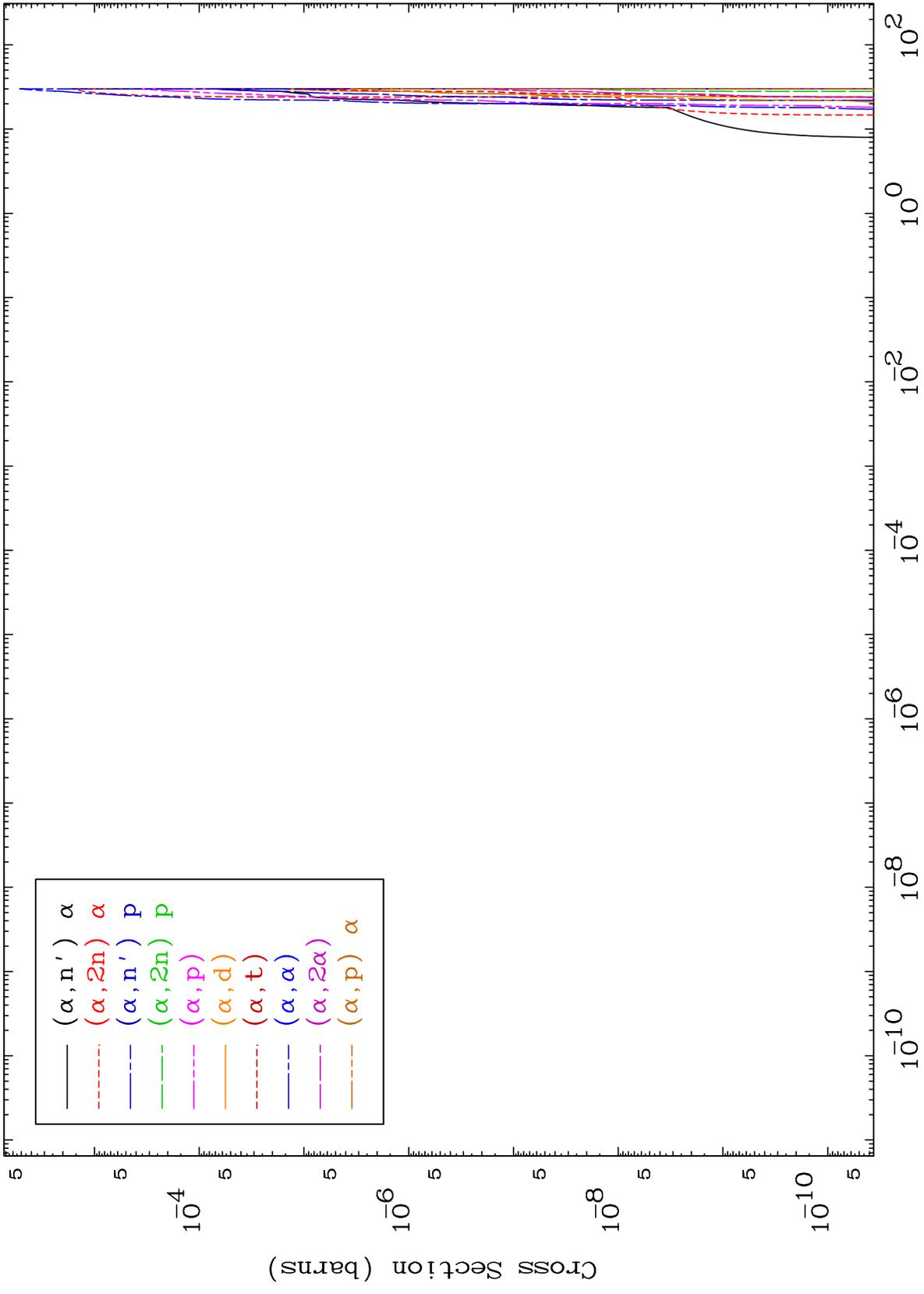
Incident Energy (MeV)

101-Md-251

MAT 9952

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

101-Md-251



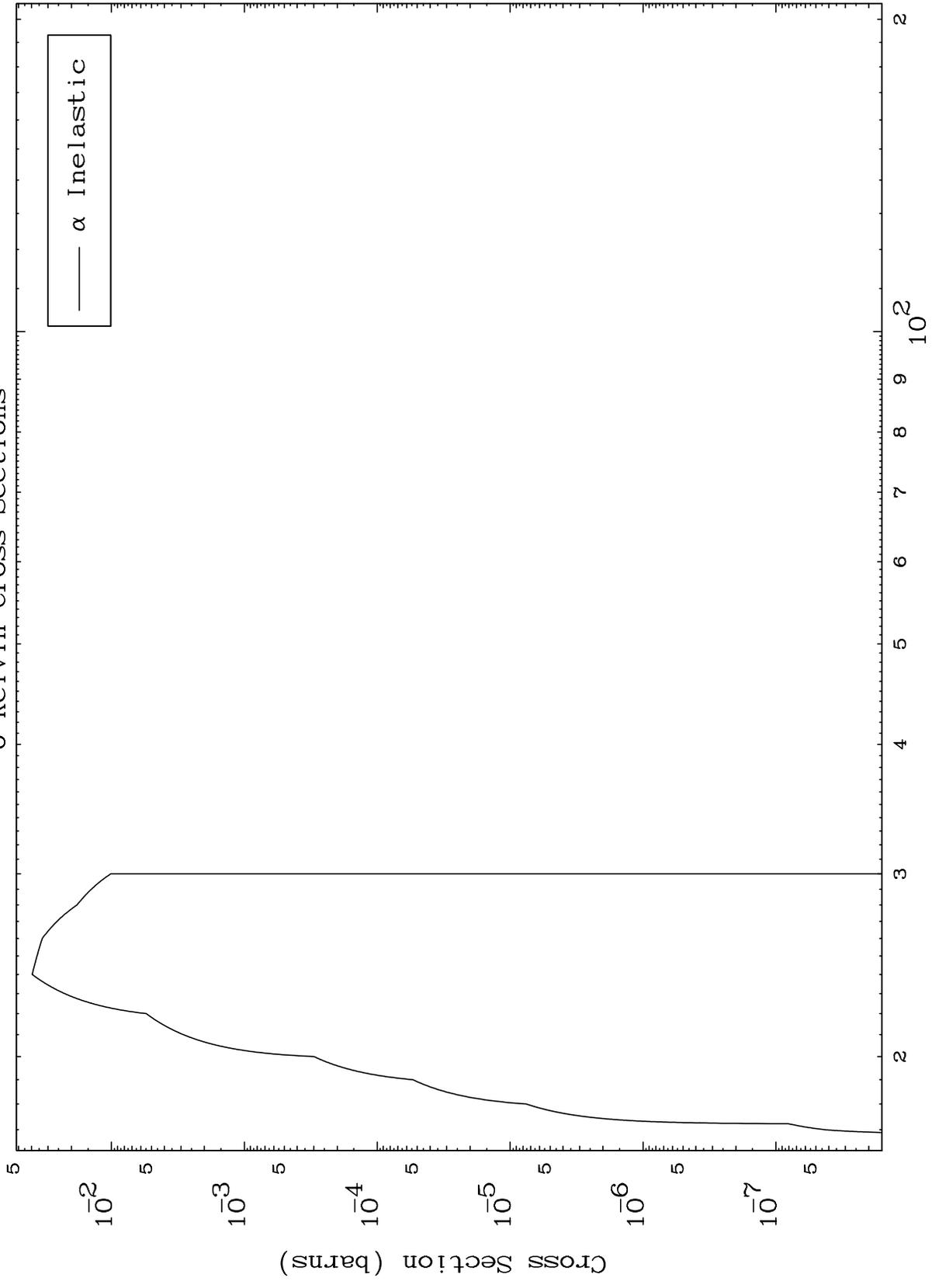
101-Md-251

Incident Energy (MeV)

MAT 9952

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

101-Md-251



4

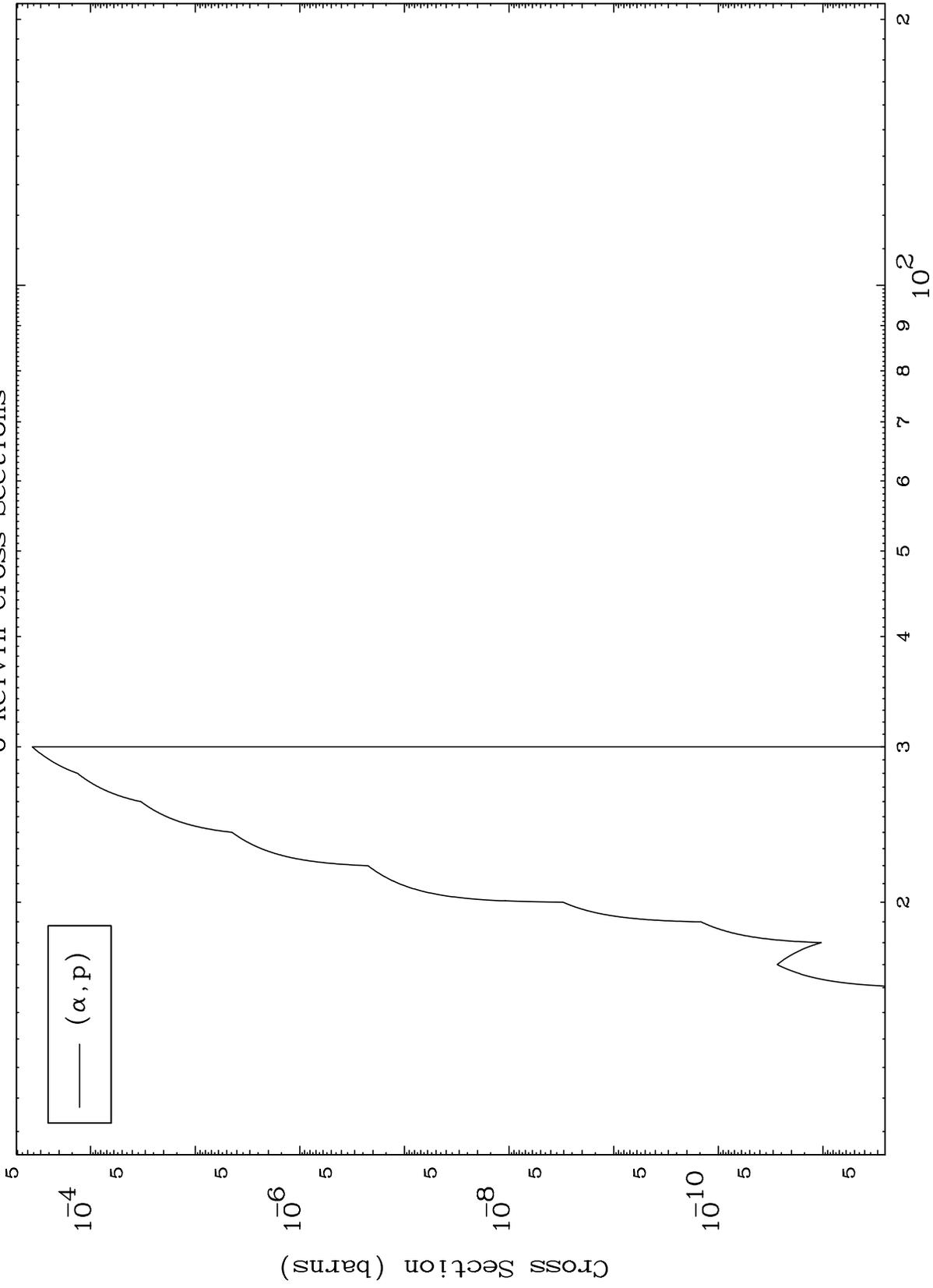
Incident Energy (MeV)

101-Md-251

MAT 9952

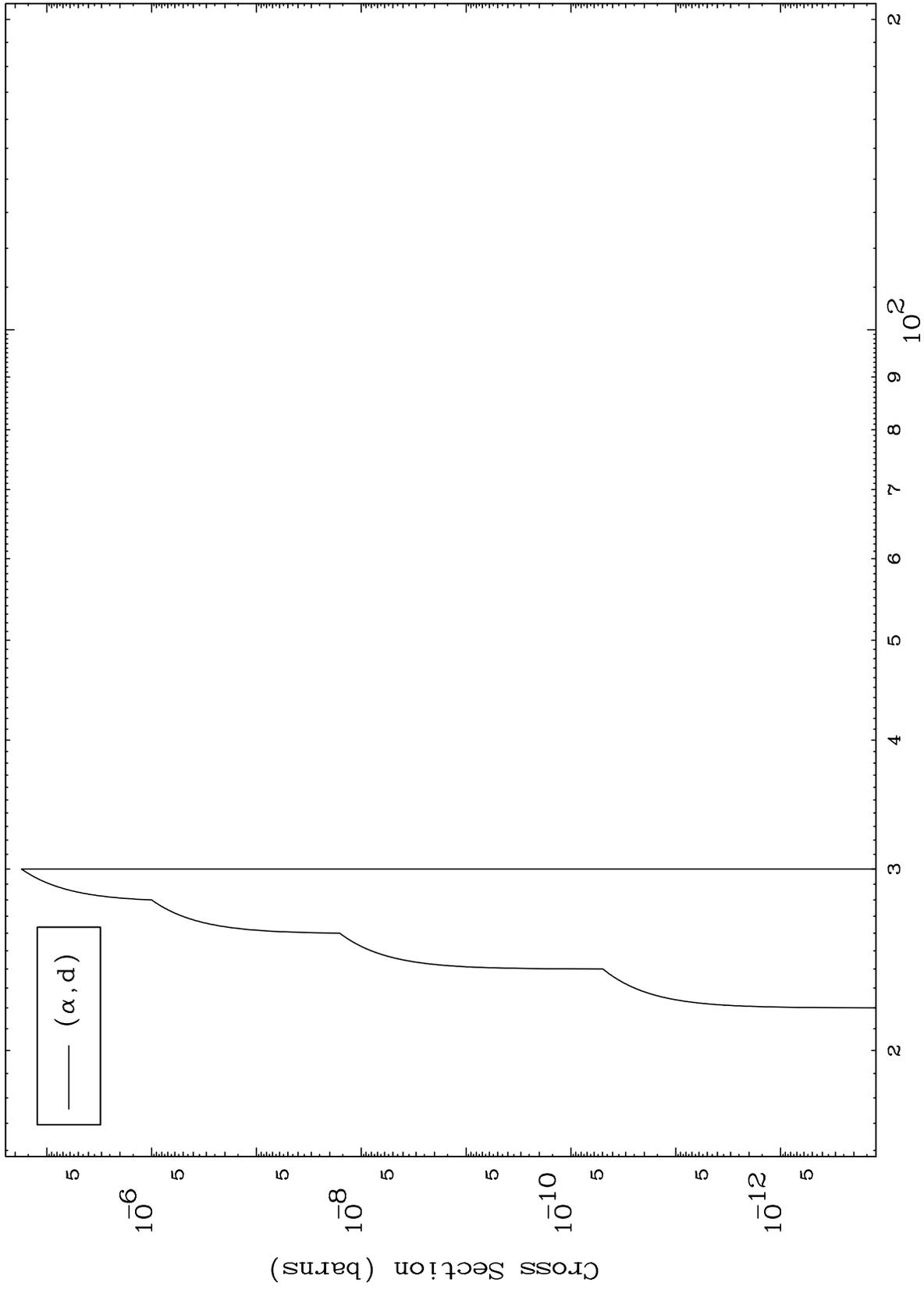
( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

101-Md-251



5

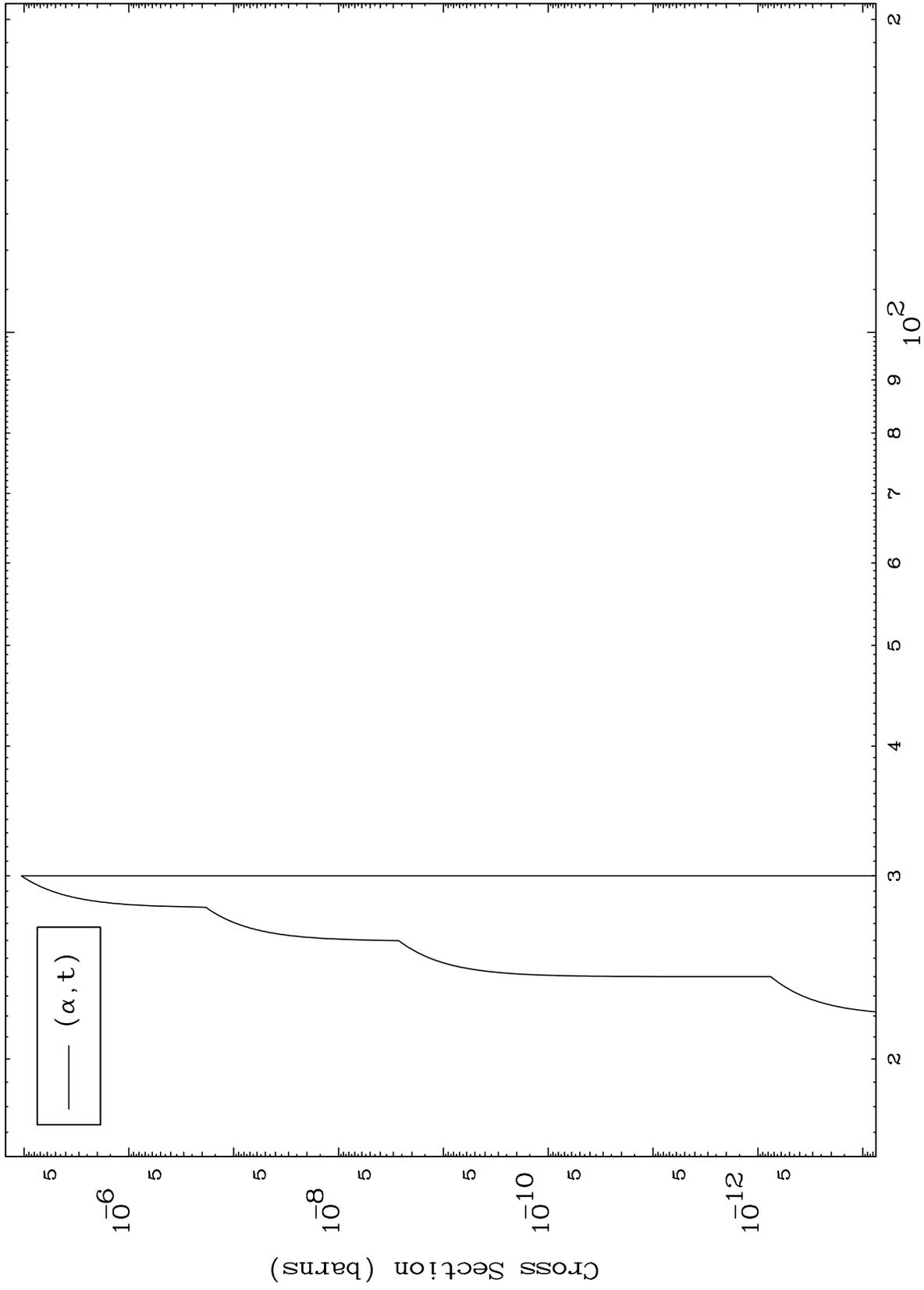
101-Md-251



MAT 9952

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

101-Md-251



7

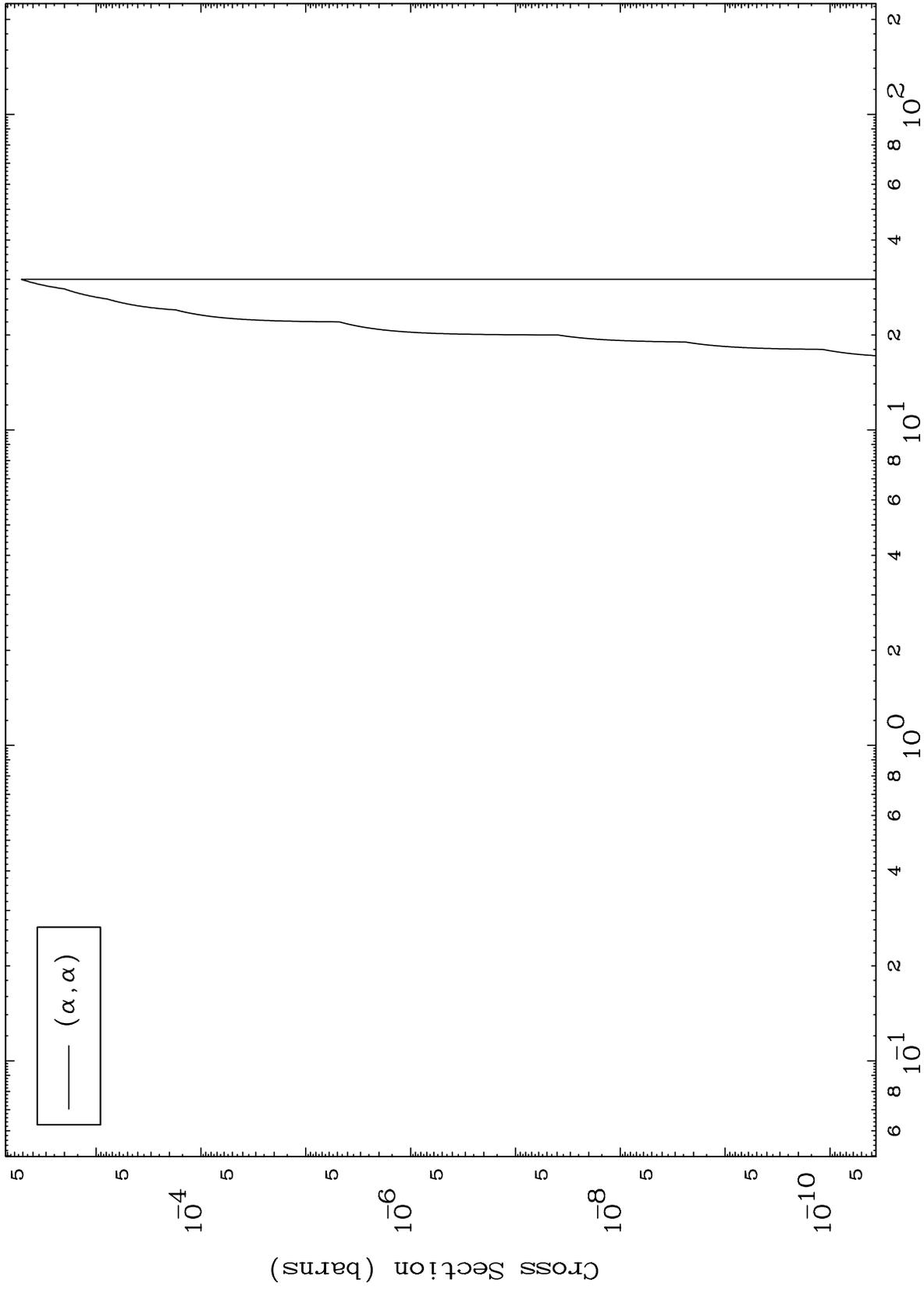
Incident Energy (MeV)

101-Md-251

MAT 9952

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

101-Md-251



8

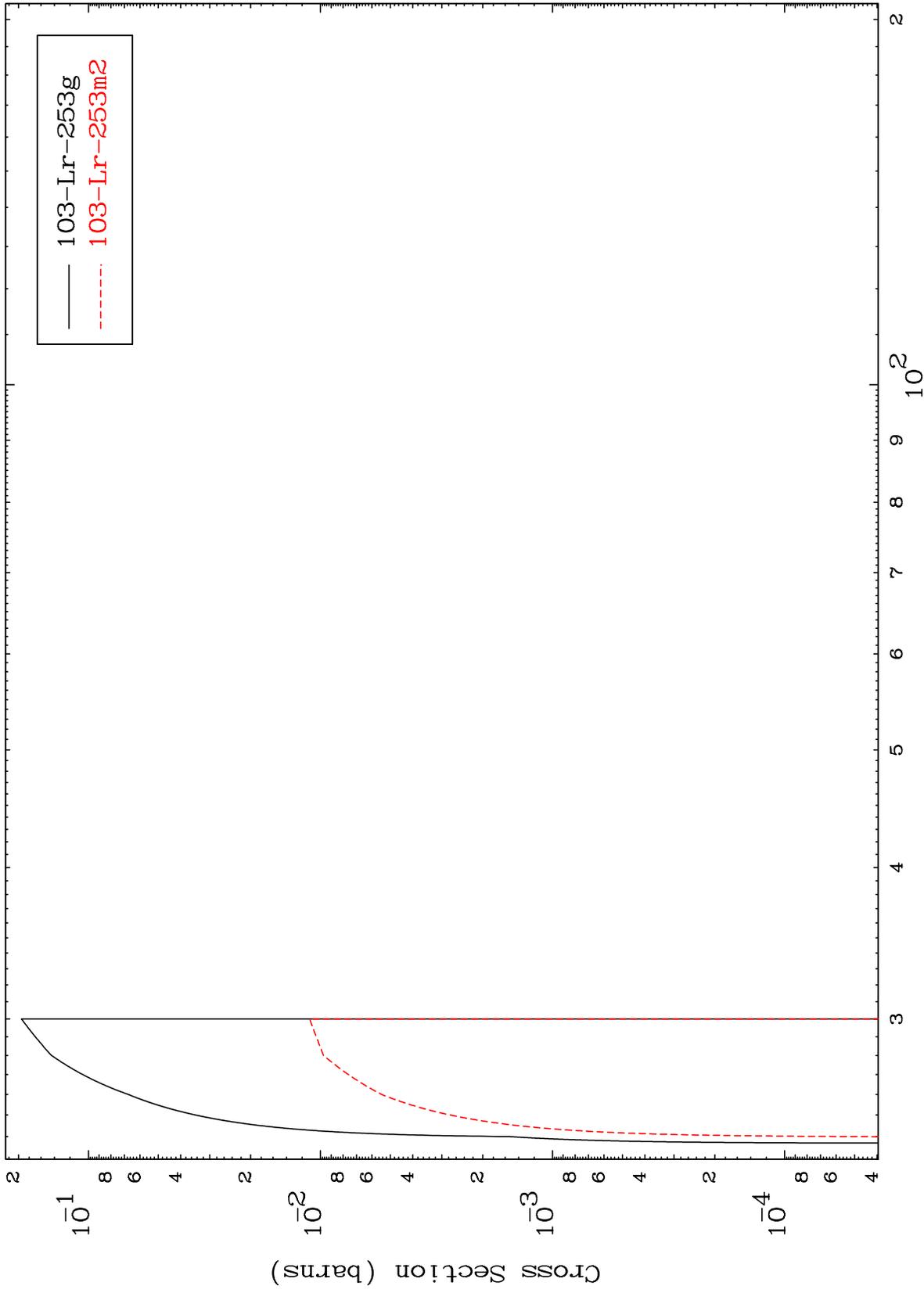
Incident Energy (MeV)

101-Md-251

MAT 9952

101-Md-251

Radionuclide Production Cross Section  
( $\alpha, 2n$ )



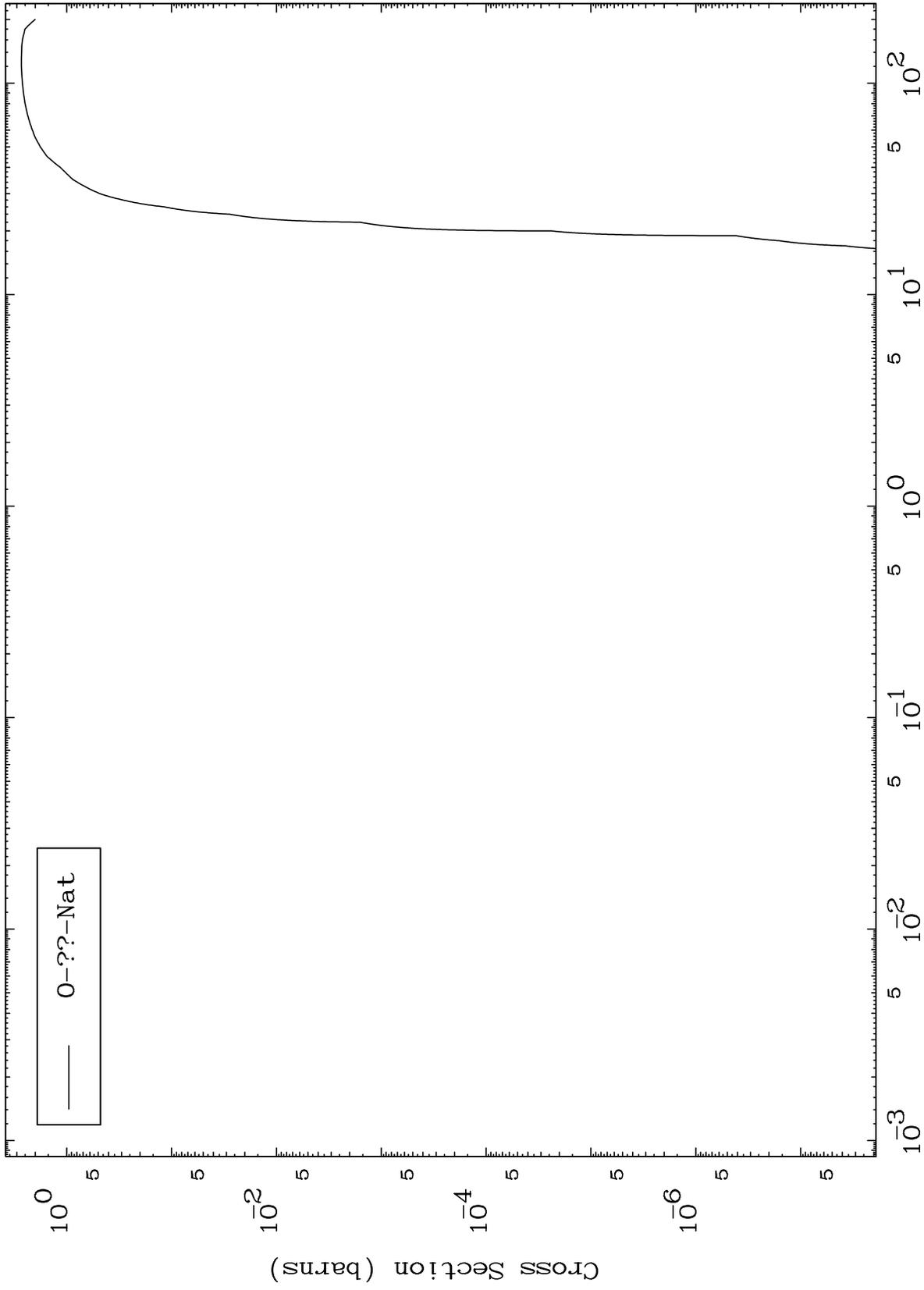
9

101-Md-251

MAT 9952

$\alpha$  Fission  
Radionuclide Production Cross Section

101-Md-251



Incident Energy (MeV)

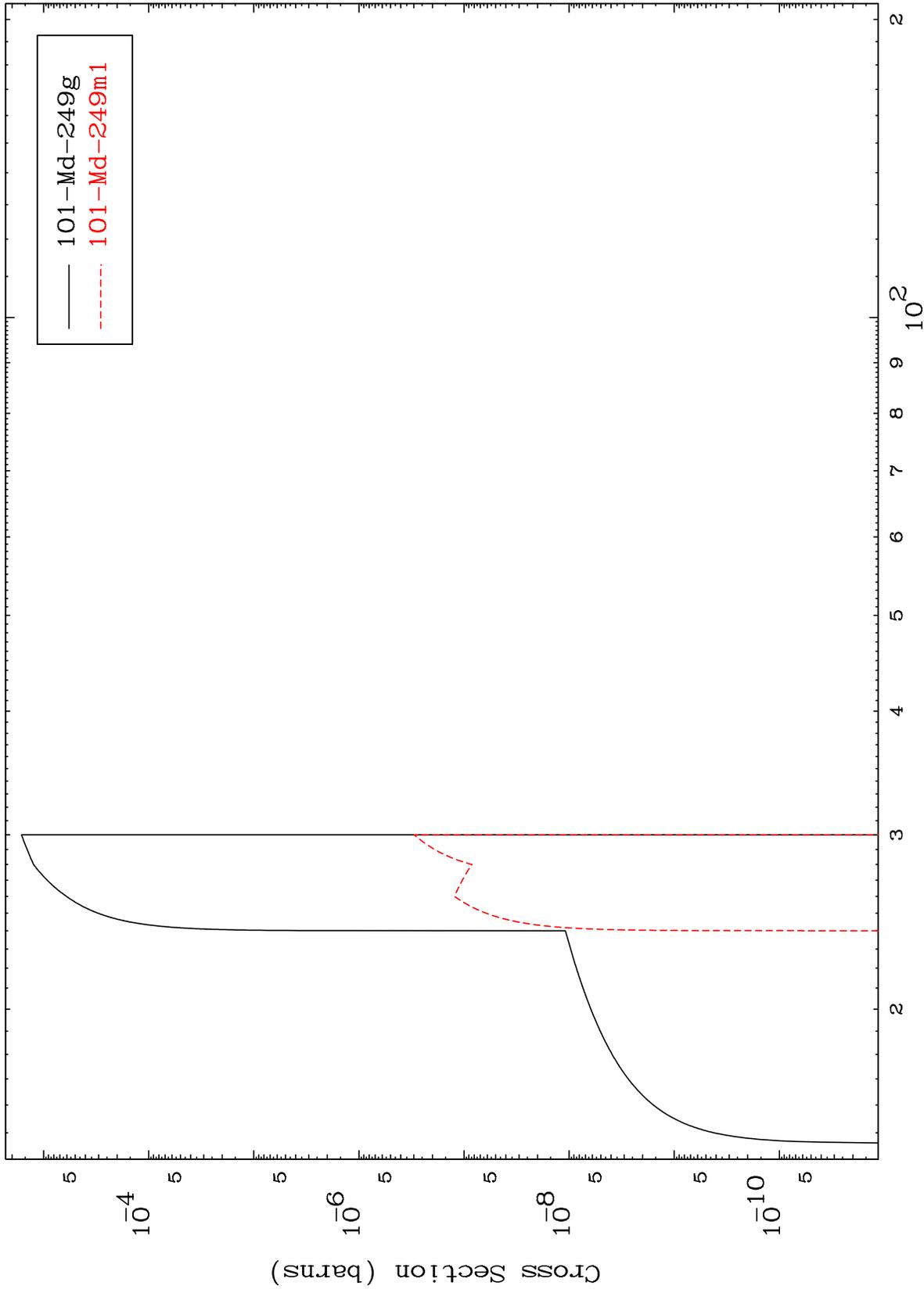
101-Md-251

MAT 9952

$(\alpha, 2n) \alpha$

101-Md-251

Radionuclide Production Cross Section



11

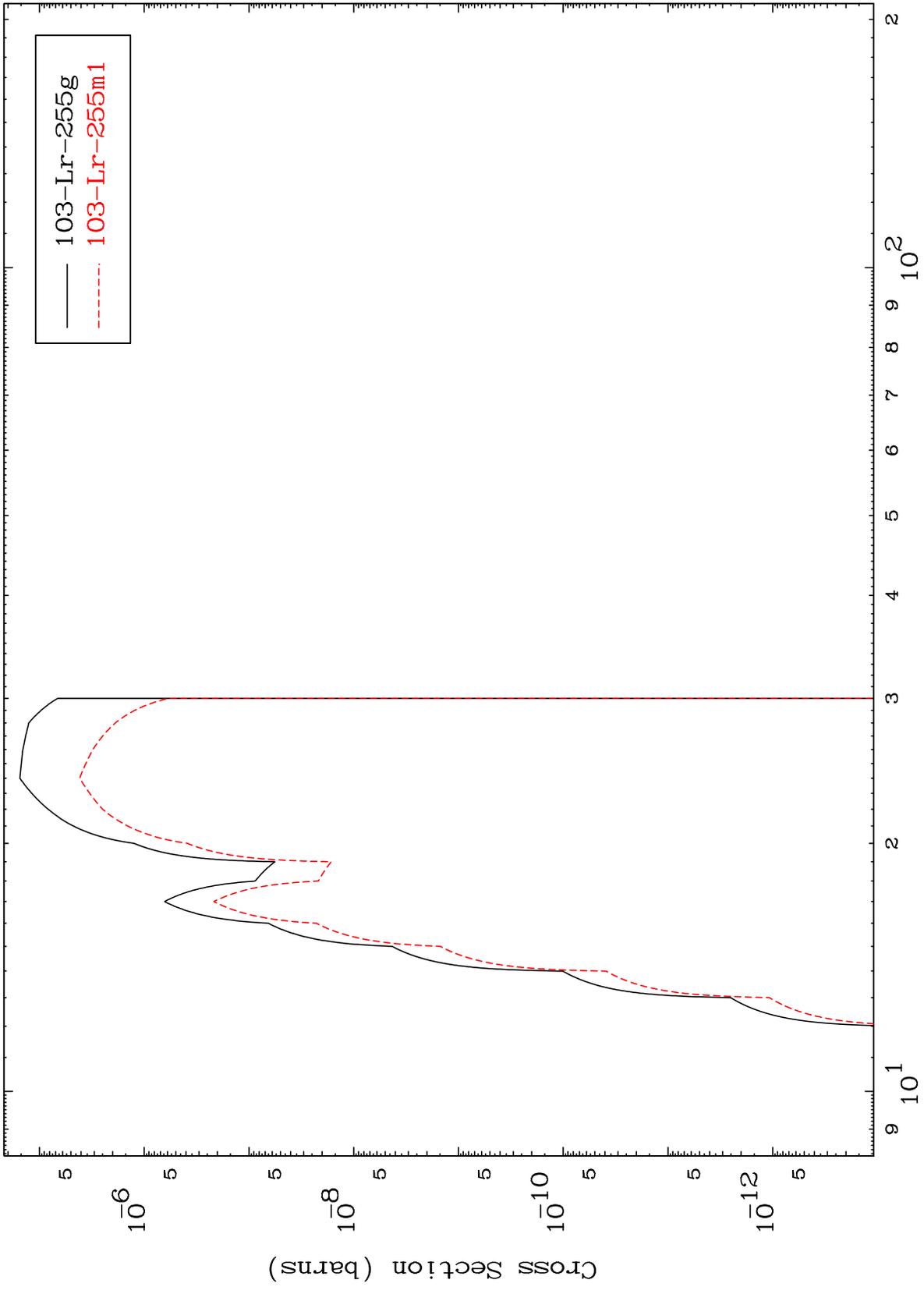
Incident Energy (MeV)

101-Md-251

MAT 9952

101-Md-251

$(\alpha, \gamma)$   
Radionuclide Production Cross Section



101-Md-251

Incident Energy (MeV)

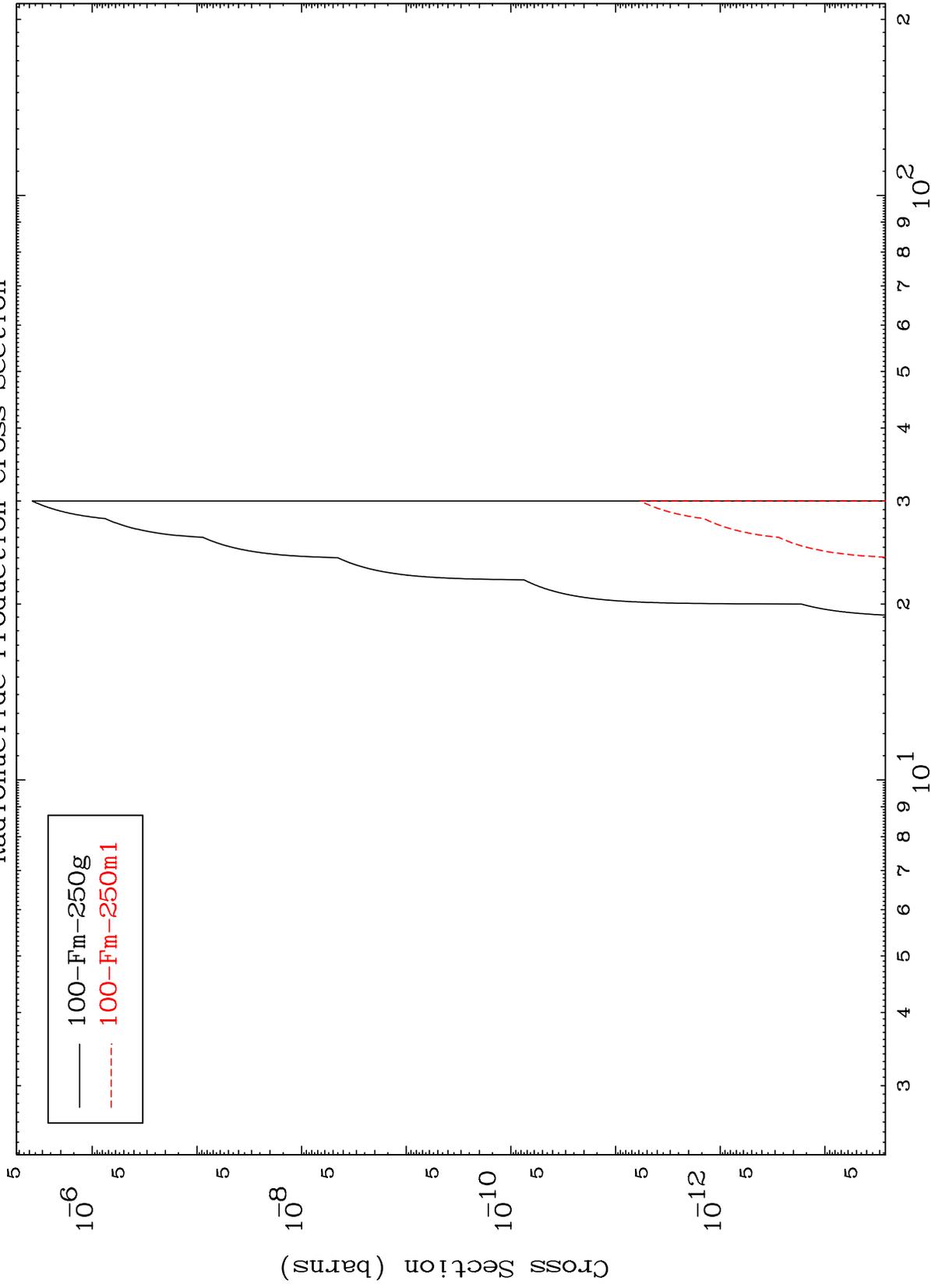
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MAT 9952

( $\alpha, p$ )  $\alpha$

101-Md-251

Radionuclide Production Cross Section



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Incident Energy (MeV)

101-Md-251