

Program EVALPLOT
(Version 2018-1)

by

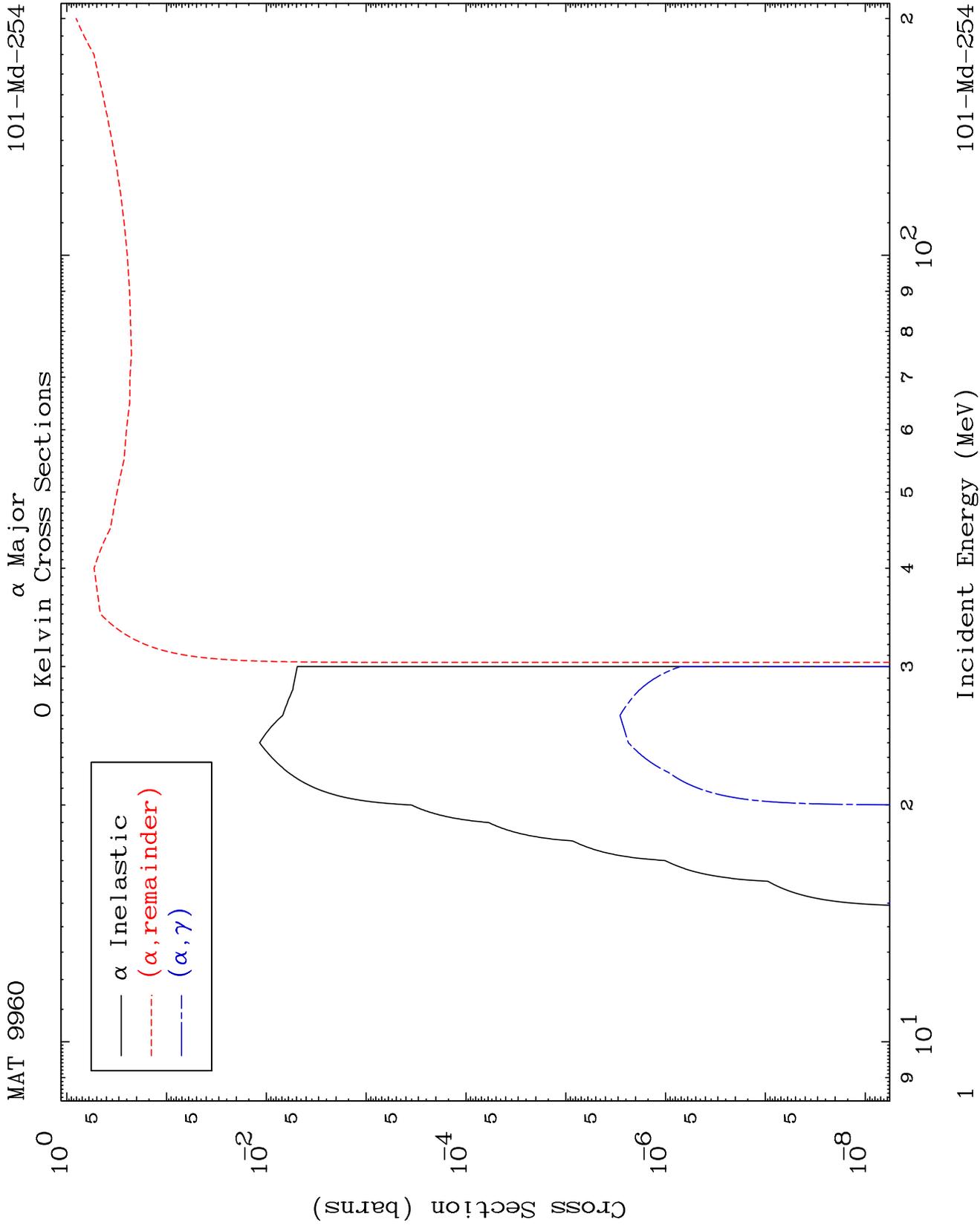
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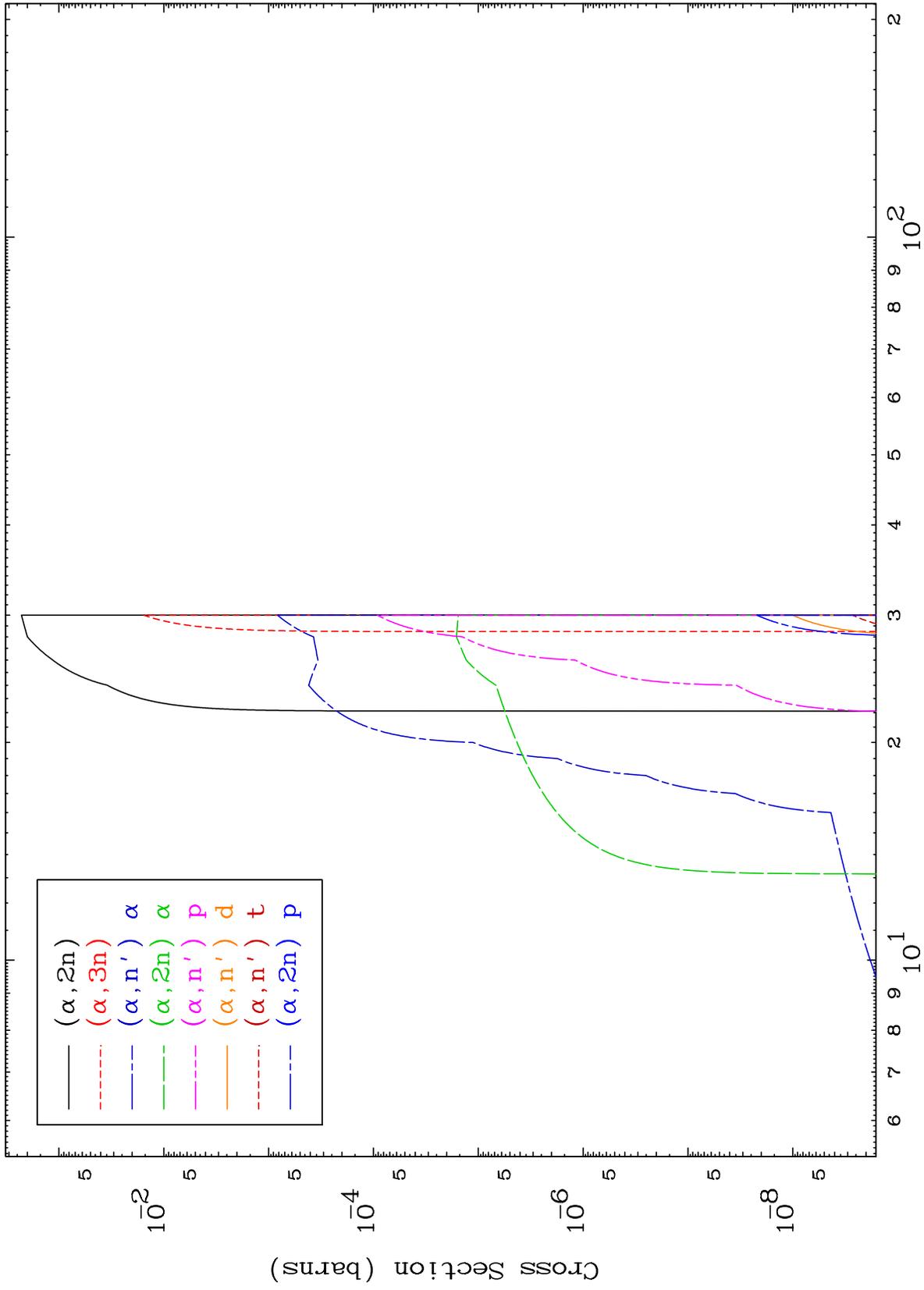
Press Mouse Button to Start



MAT 9960

α Neutron Production
0 Kelvin Cross Sections

101-Md-254



2

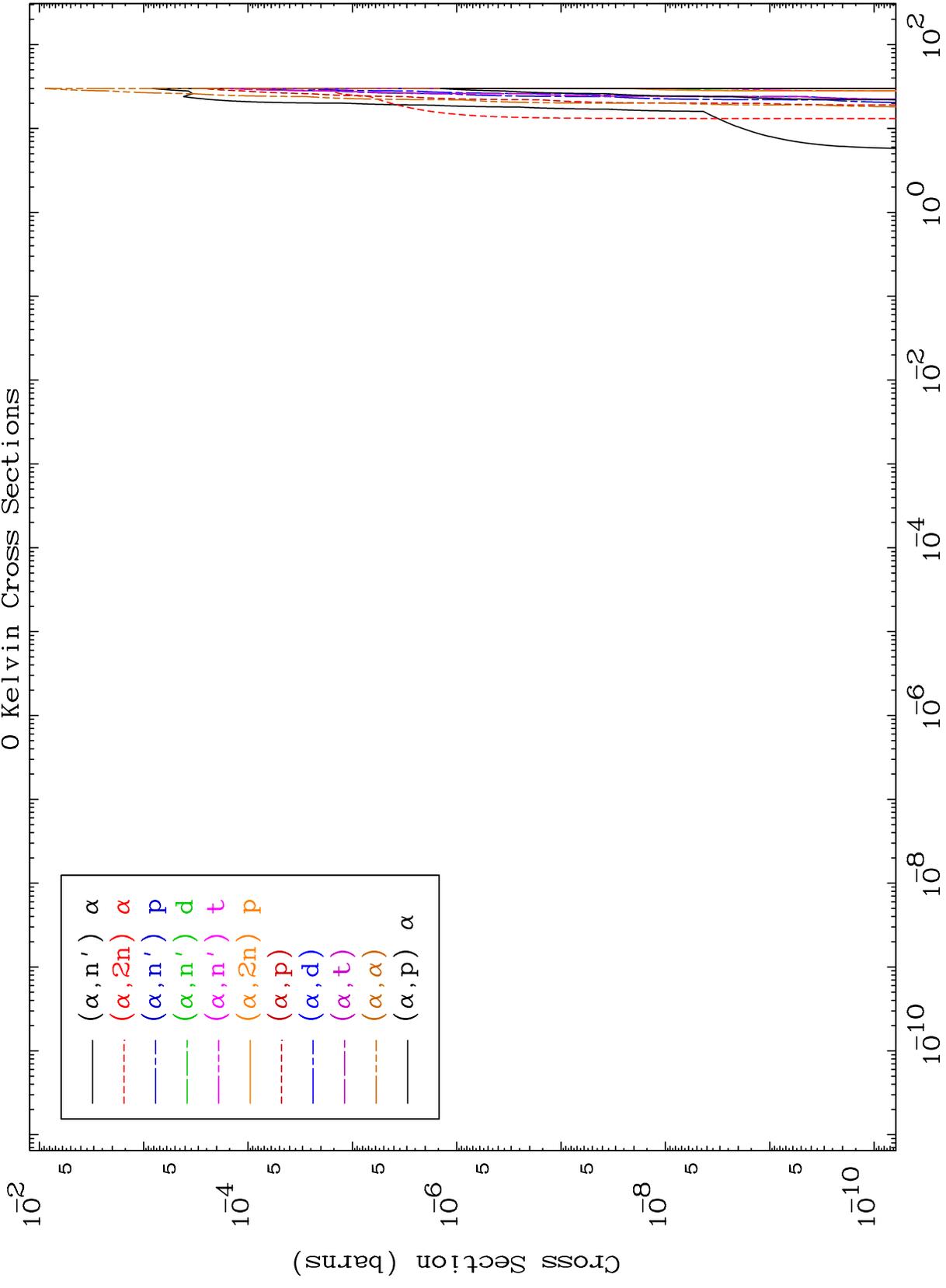
Incident Energy (MeV)

101-Md-254

MAT 9960

α Charged Particle
0 Kelvin Cross Sections

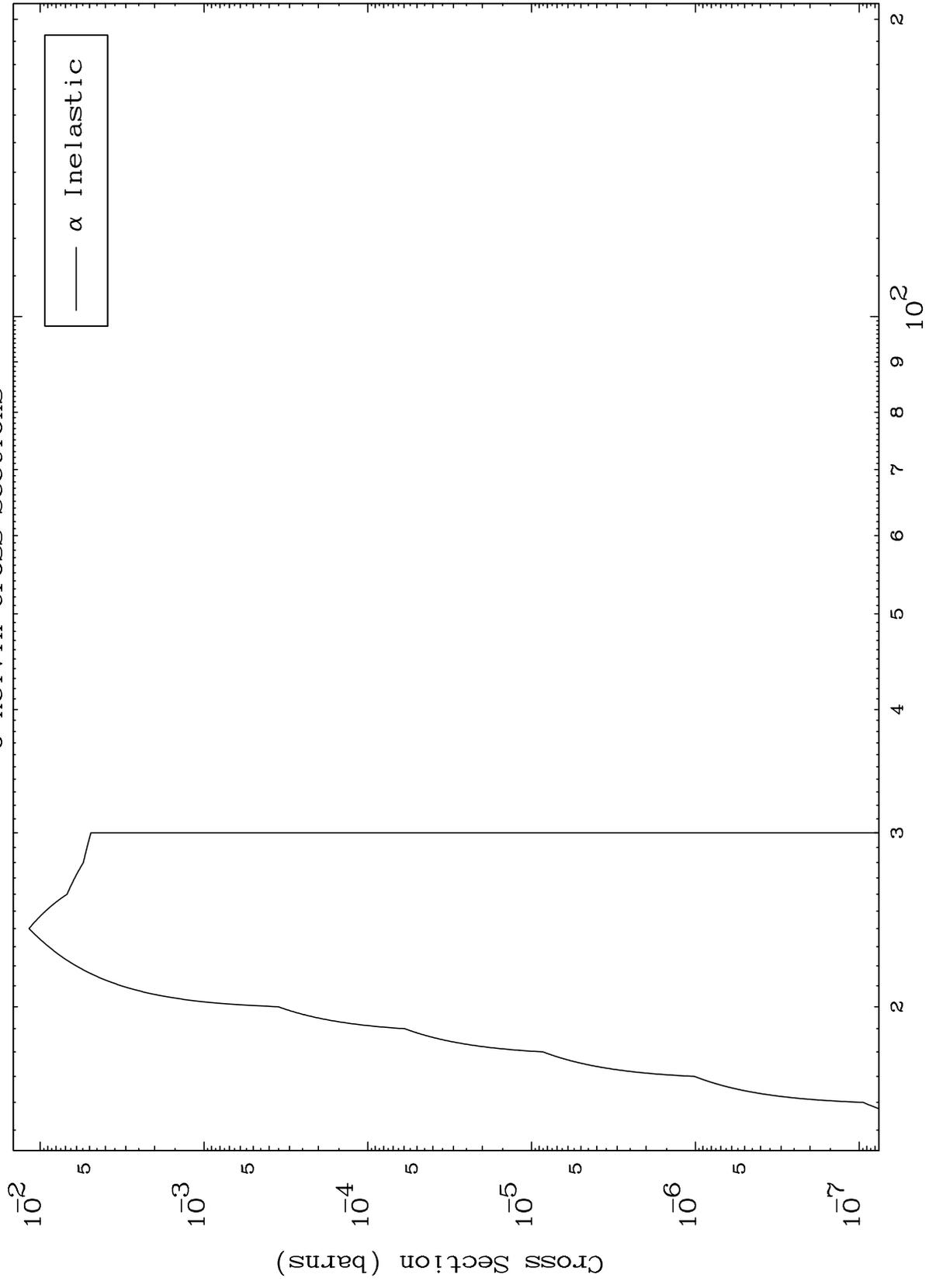
101-Md-254



MAT 9960

(α, n') Level
0 Kelvin Cross Sections

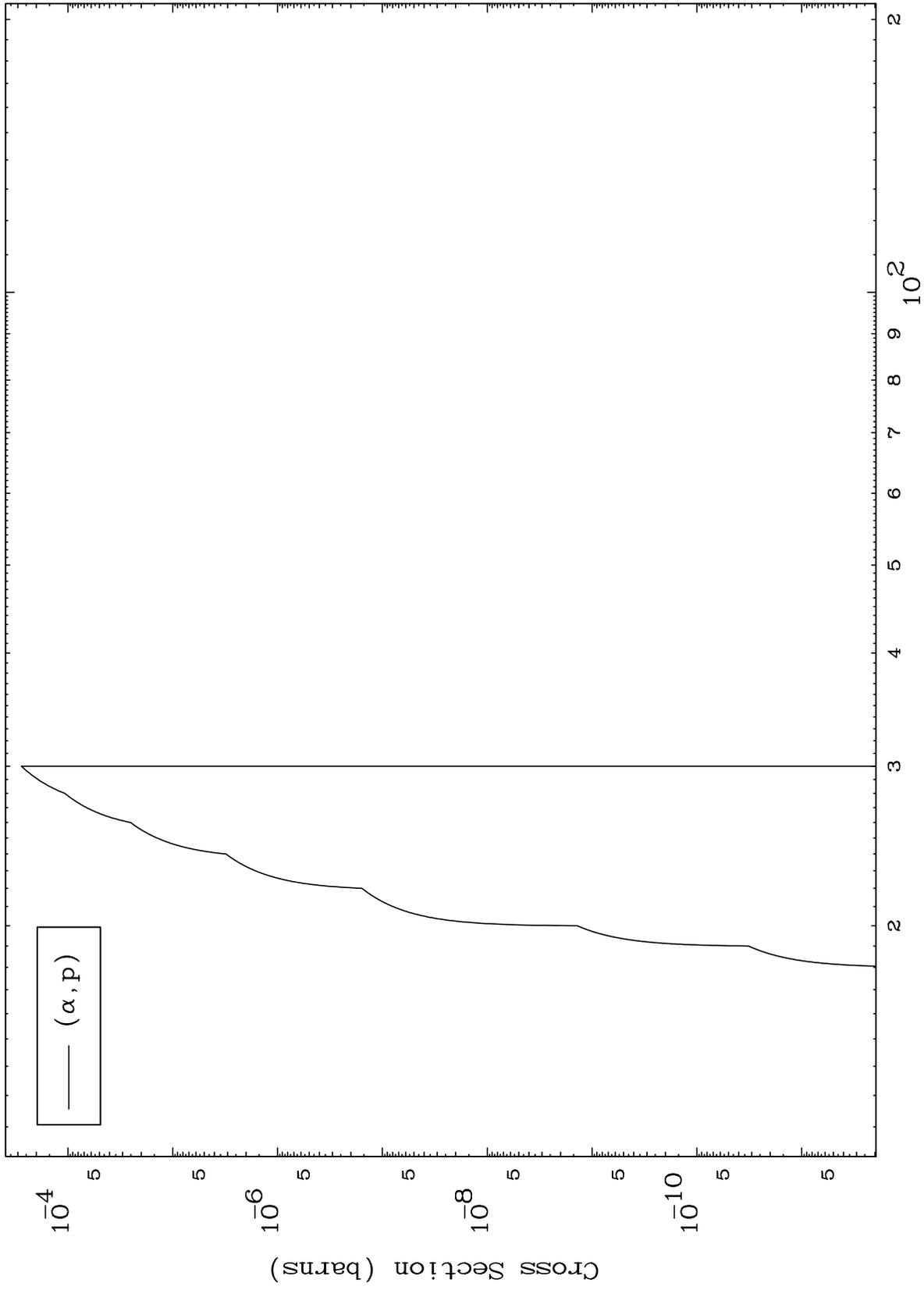
101-Md-254



4

Incident Energy (MeV)

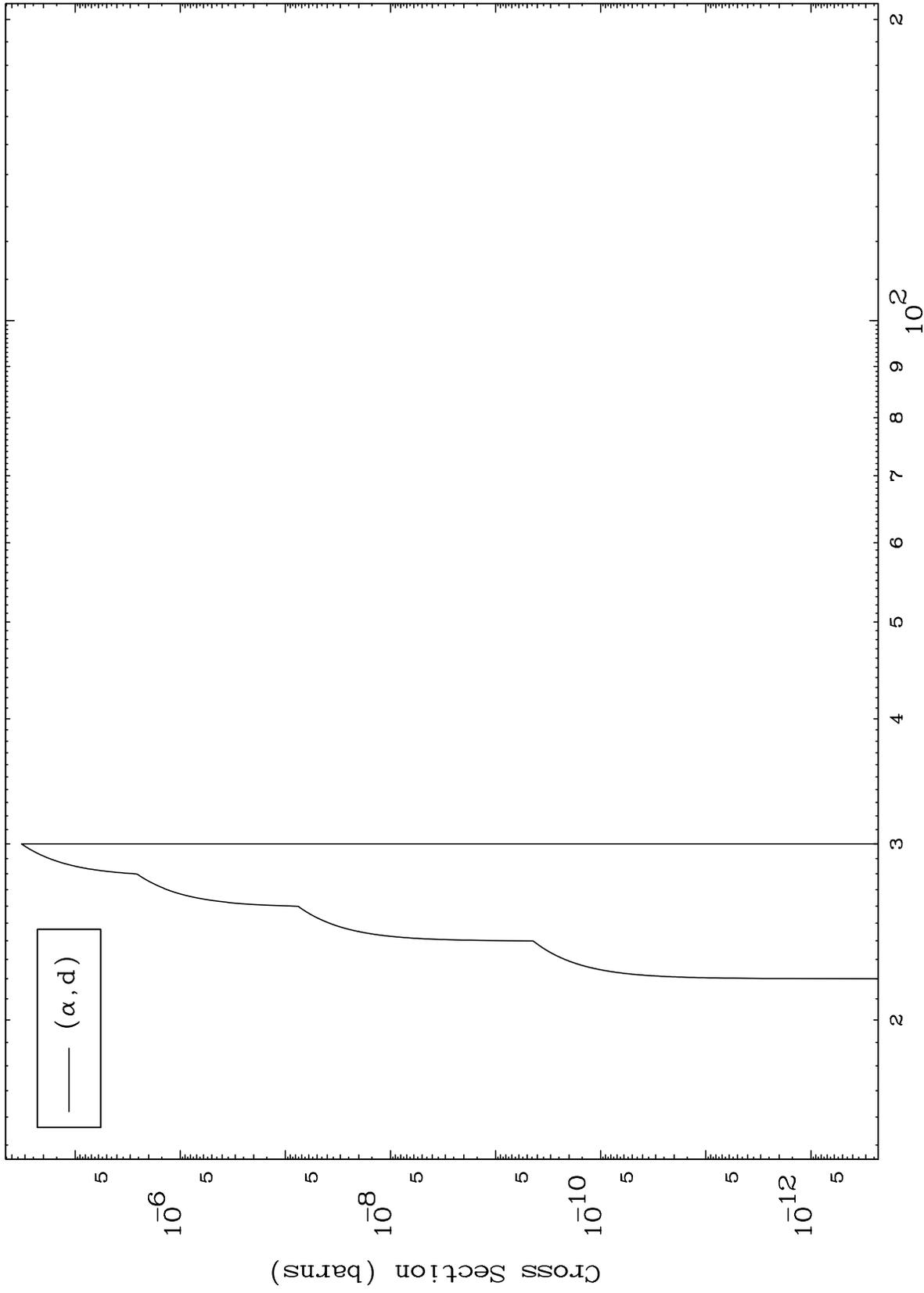
101-Md-254



MAT 9960

(α, d) Levels
0 Kelvin Cross Sections

101-Md-254



6

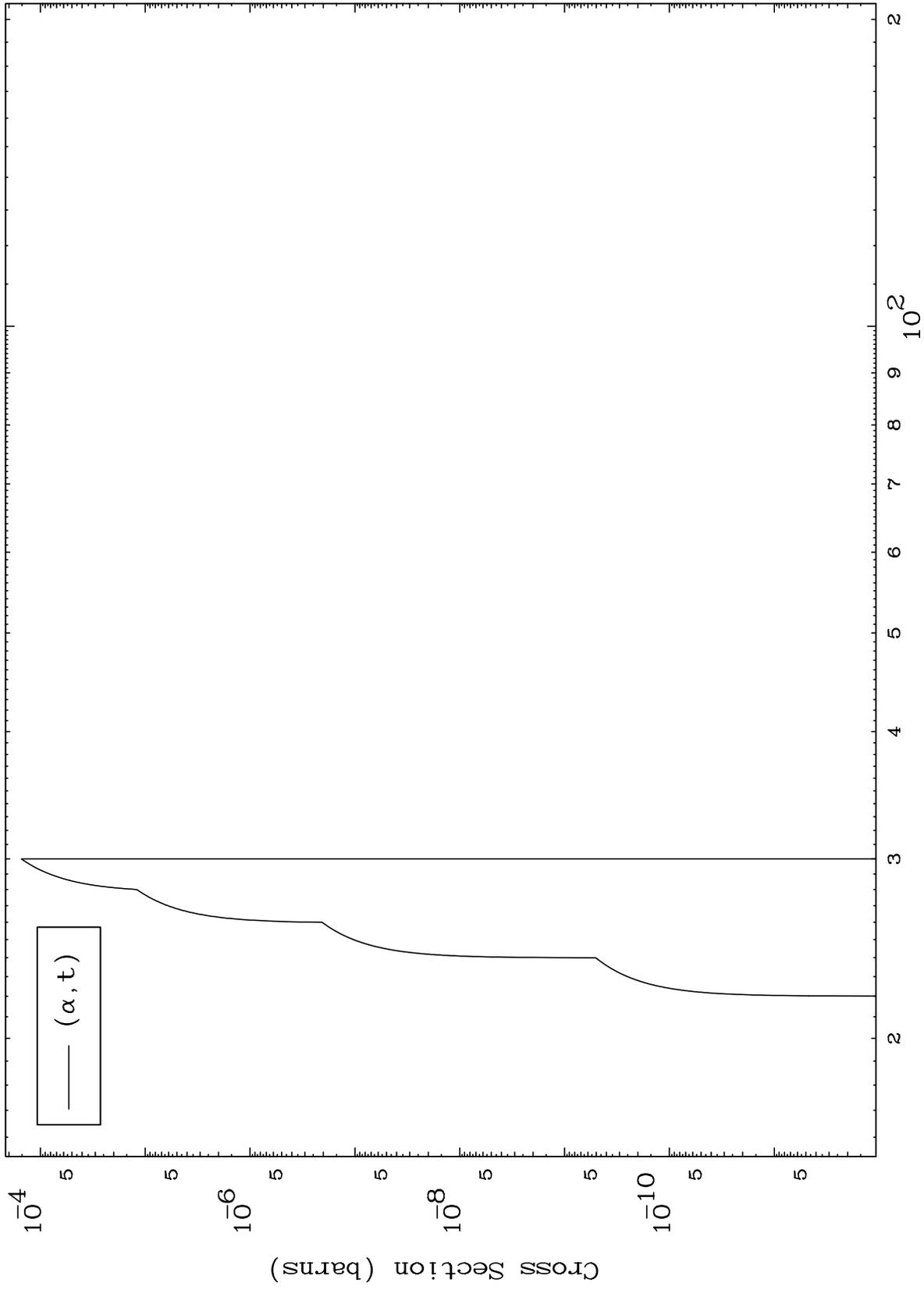
Incident Energy (MeV)

101-Md-254

MAT 9960

(α, t) Levels
0 Kelvin Cross Sections

101-Md-254



7

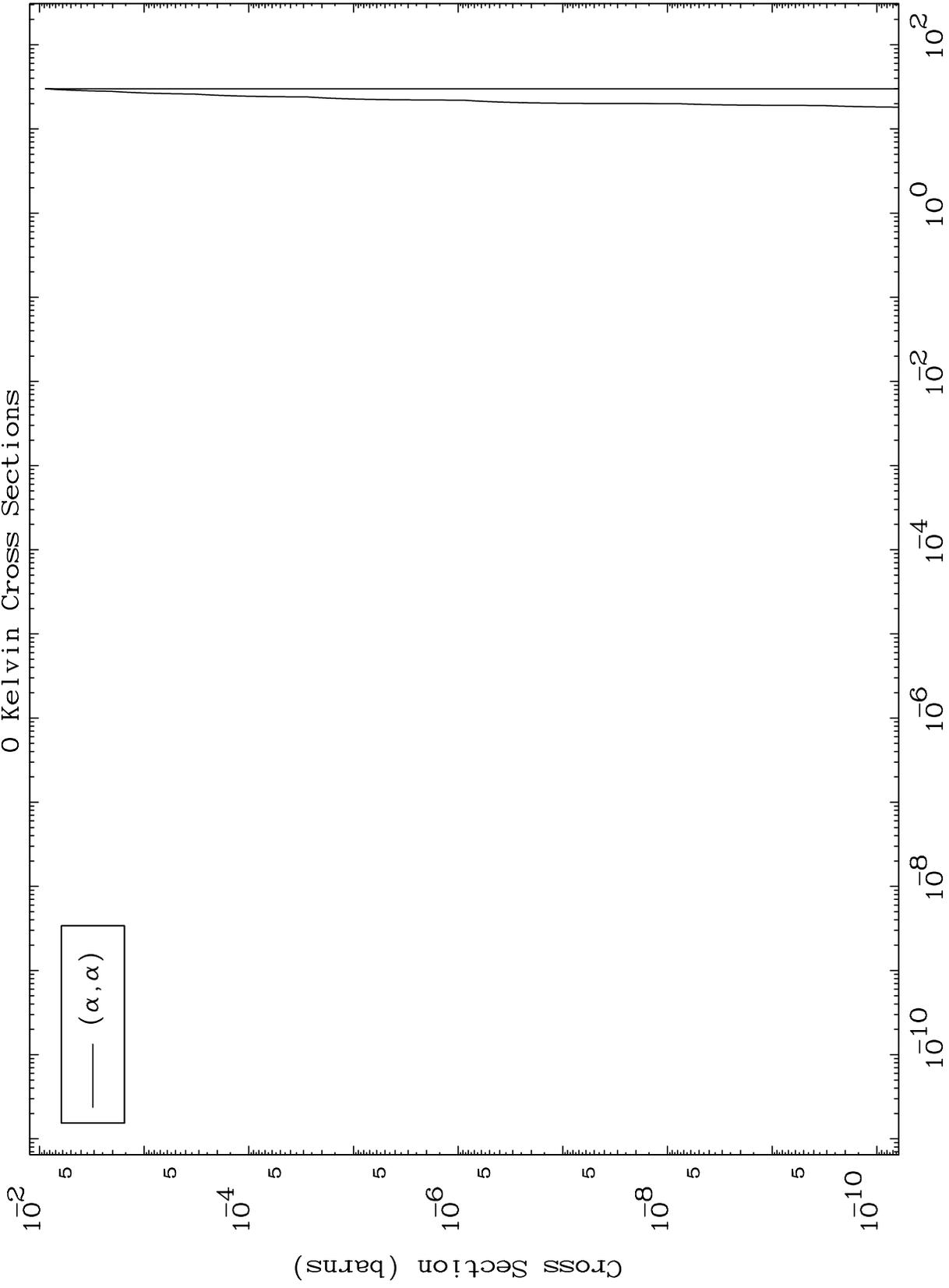
Incident Energy (MeV)

101-Md-254

MAT 9960

(α, α) Levels
0 Kelvin Cross Sections

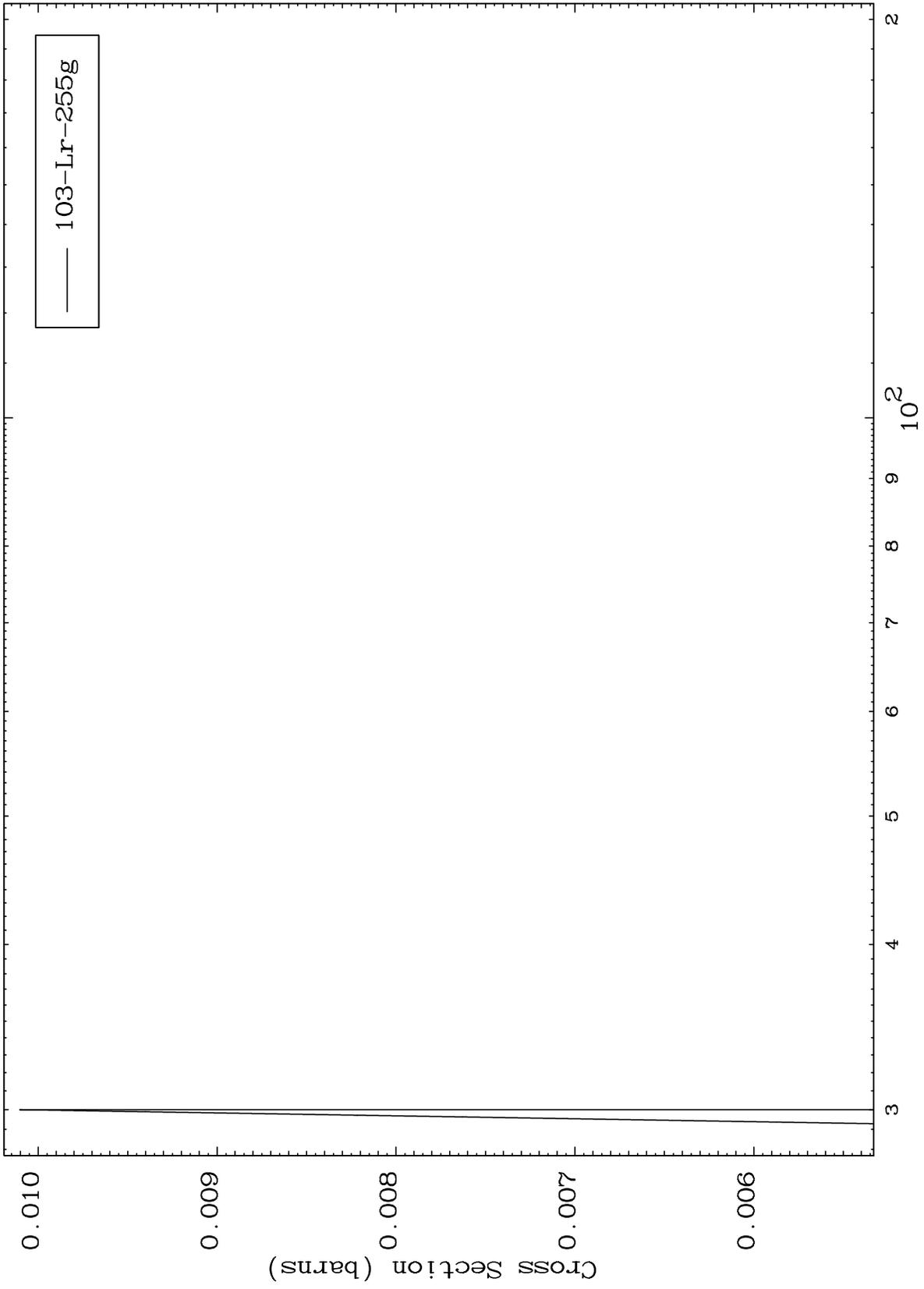
101-Md-254



MAT 9960

101-Md-254

($\alpha, 3n$)
Radionuclide Production Cross Section



— 103-Lr-255g

9

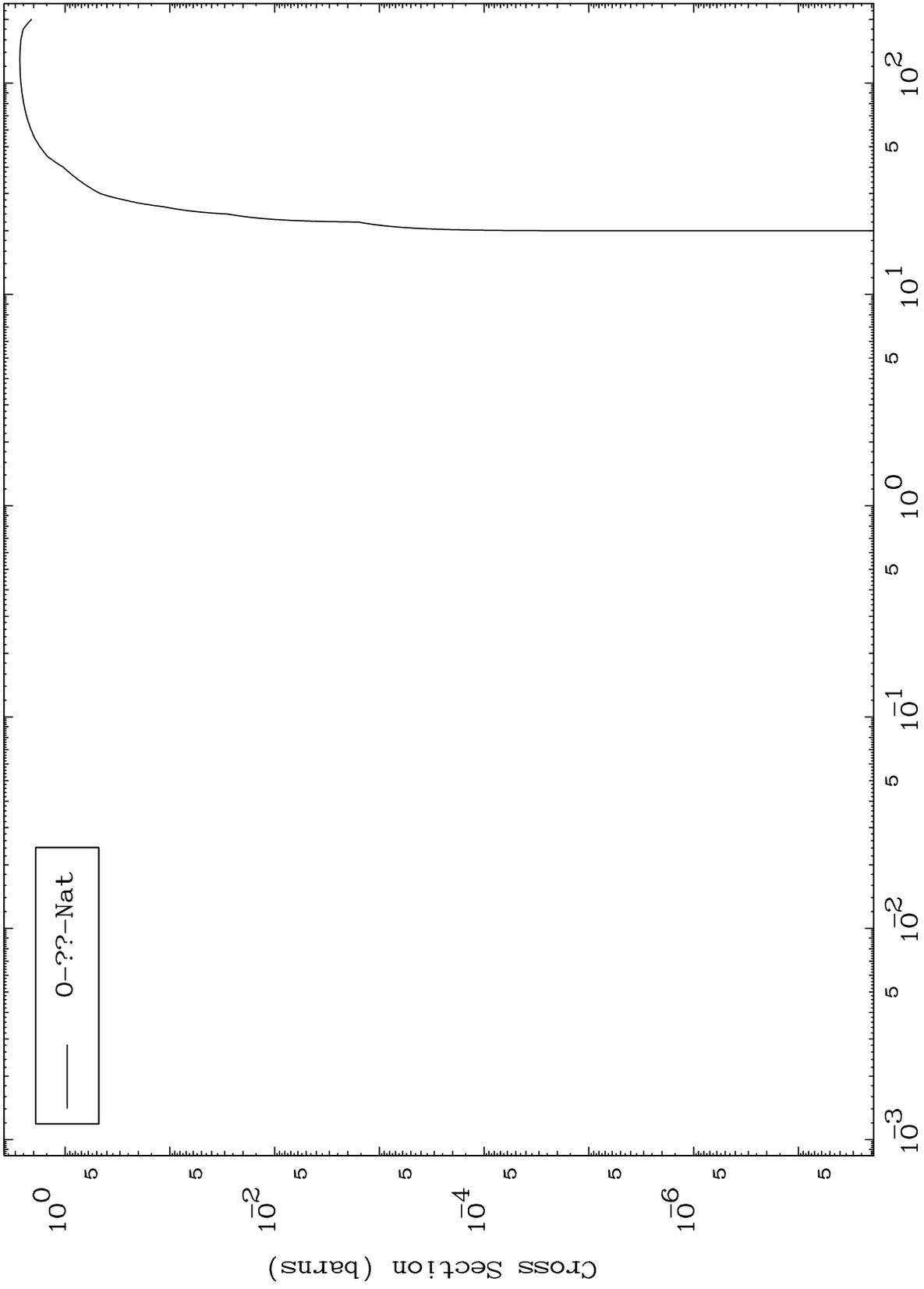
101-Md-254

Incident Energy (MeV)

MAT 9960

α Fission
Radionuclide Production Cross Section

101-Md-254



Incident Energy (MeV)

101-Md-254

