Program EVALPLOT
(Version 2018-1)
by
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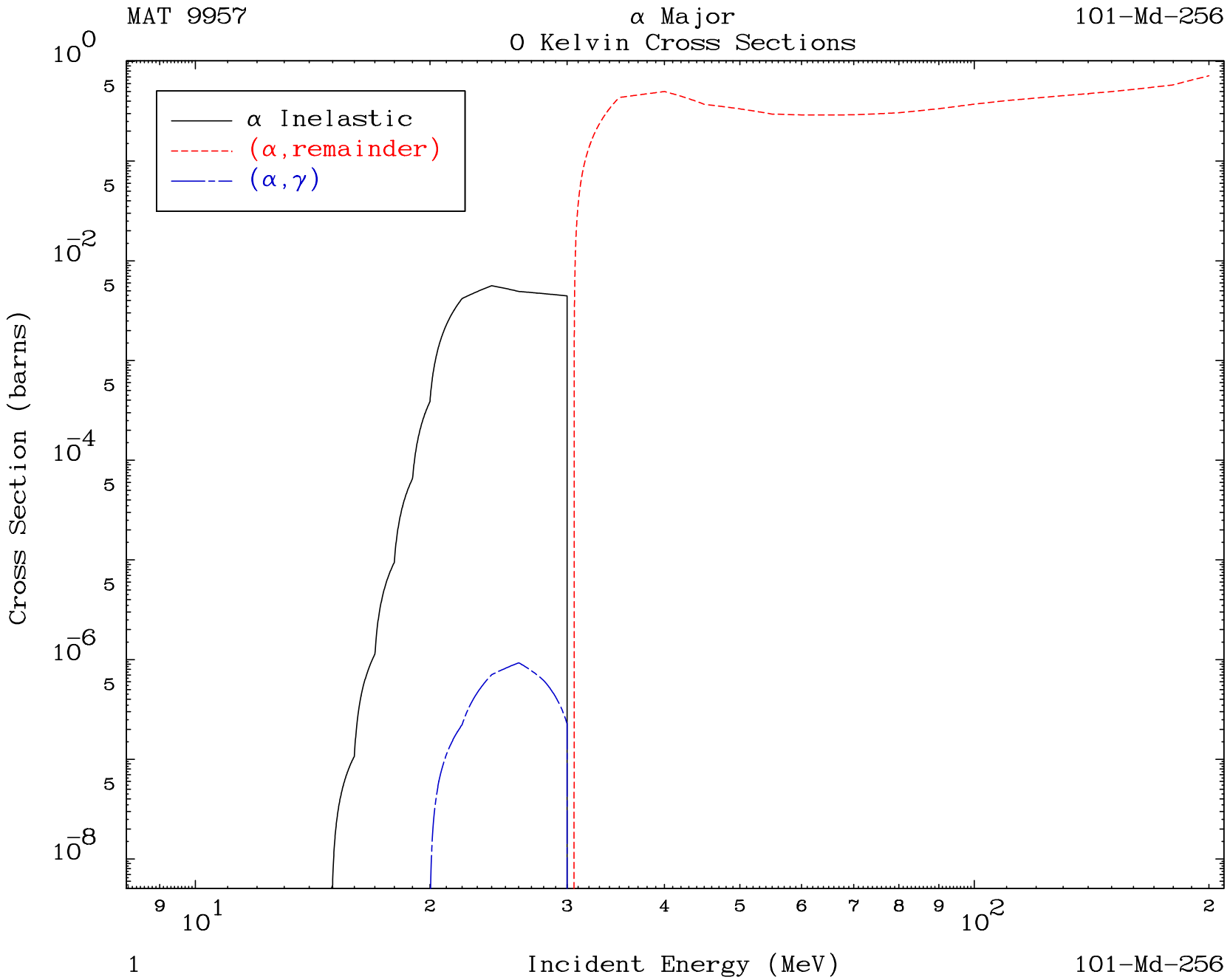
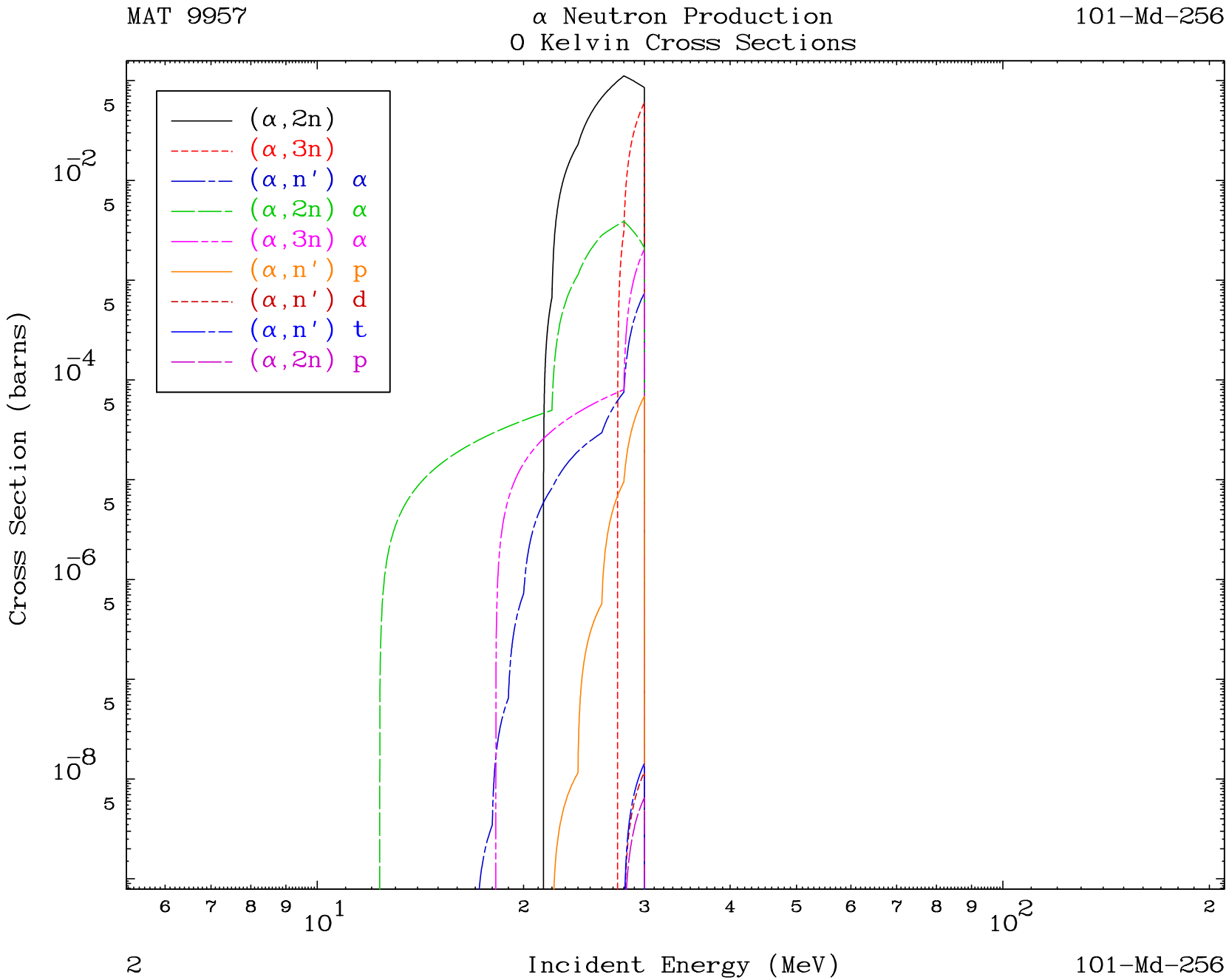
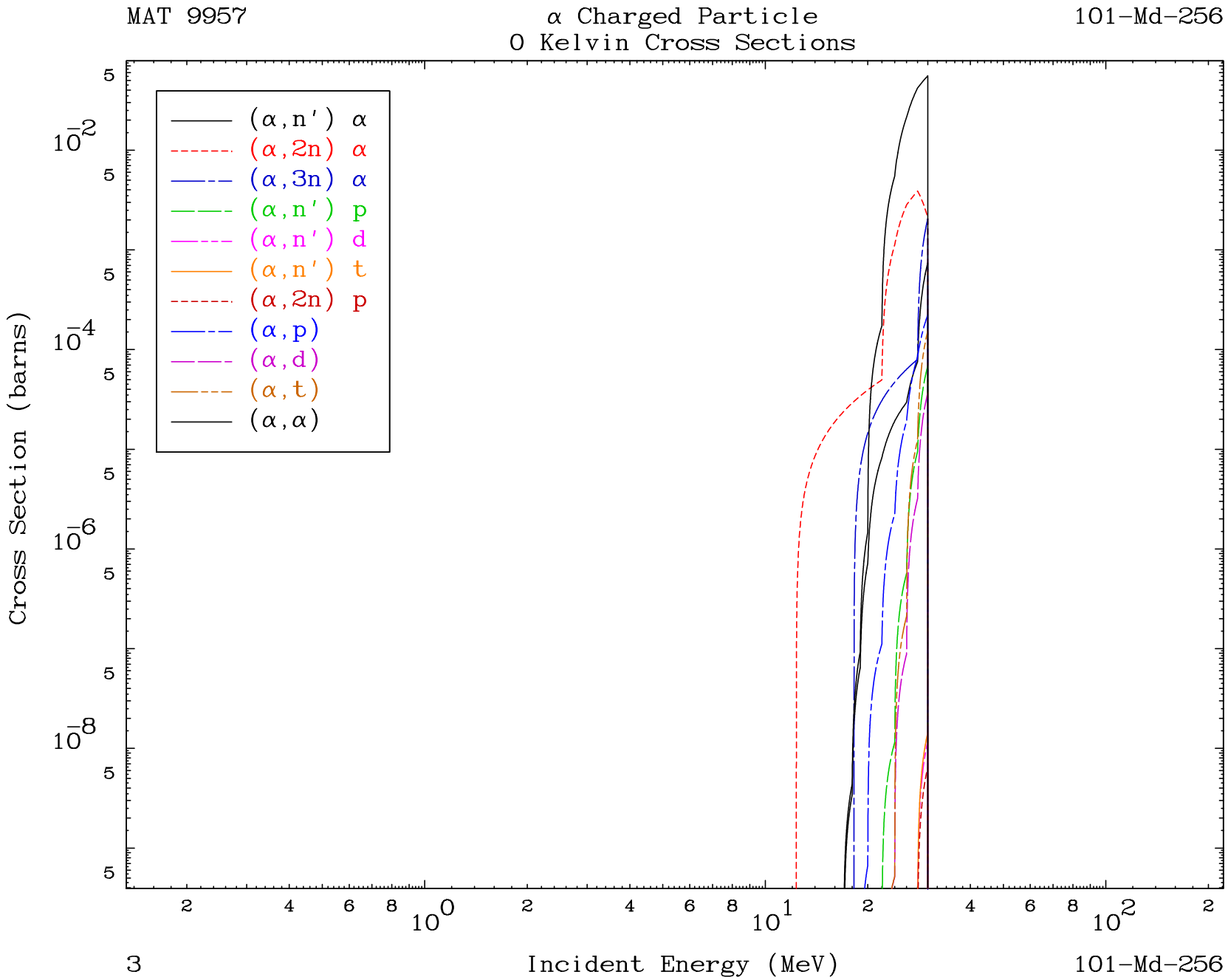
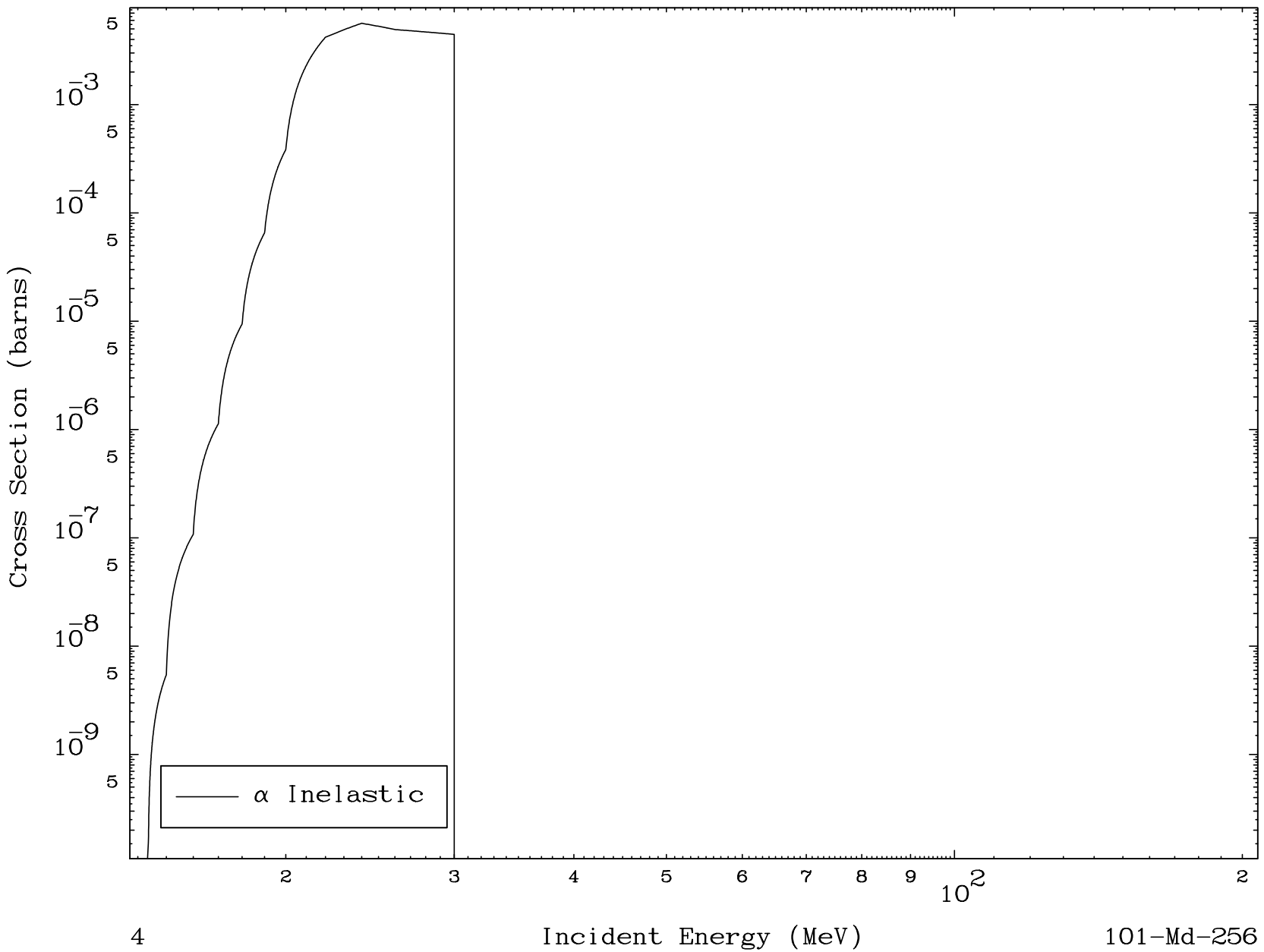
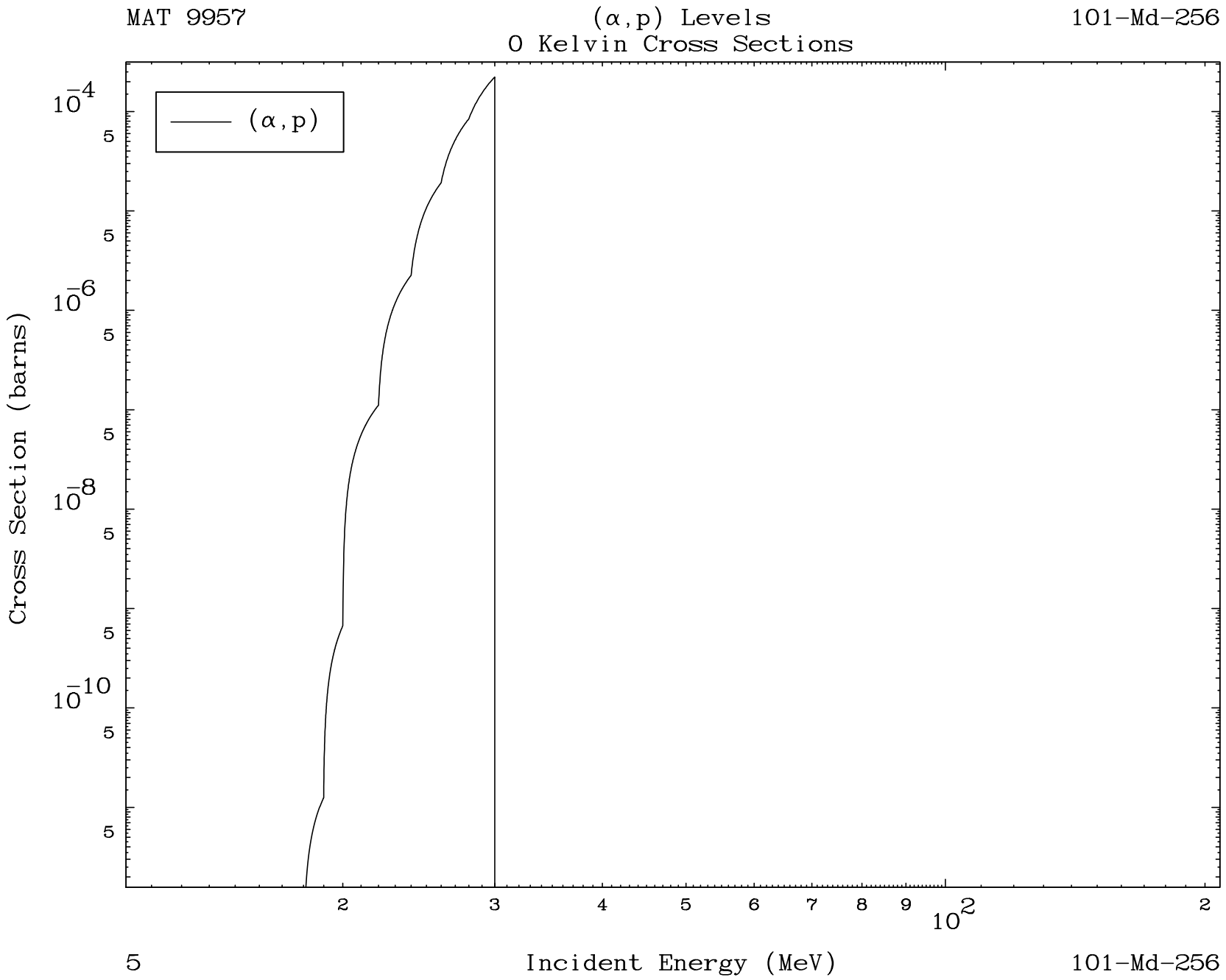
Press Mouse Button to Start

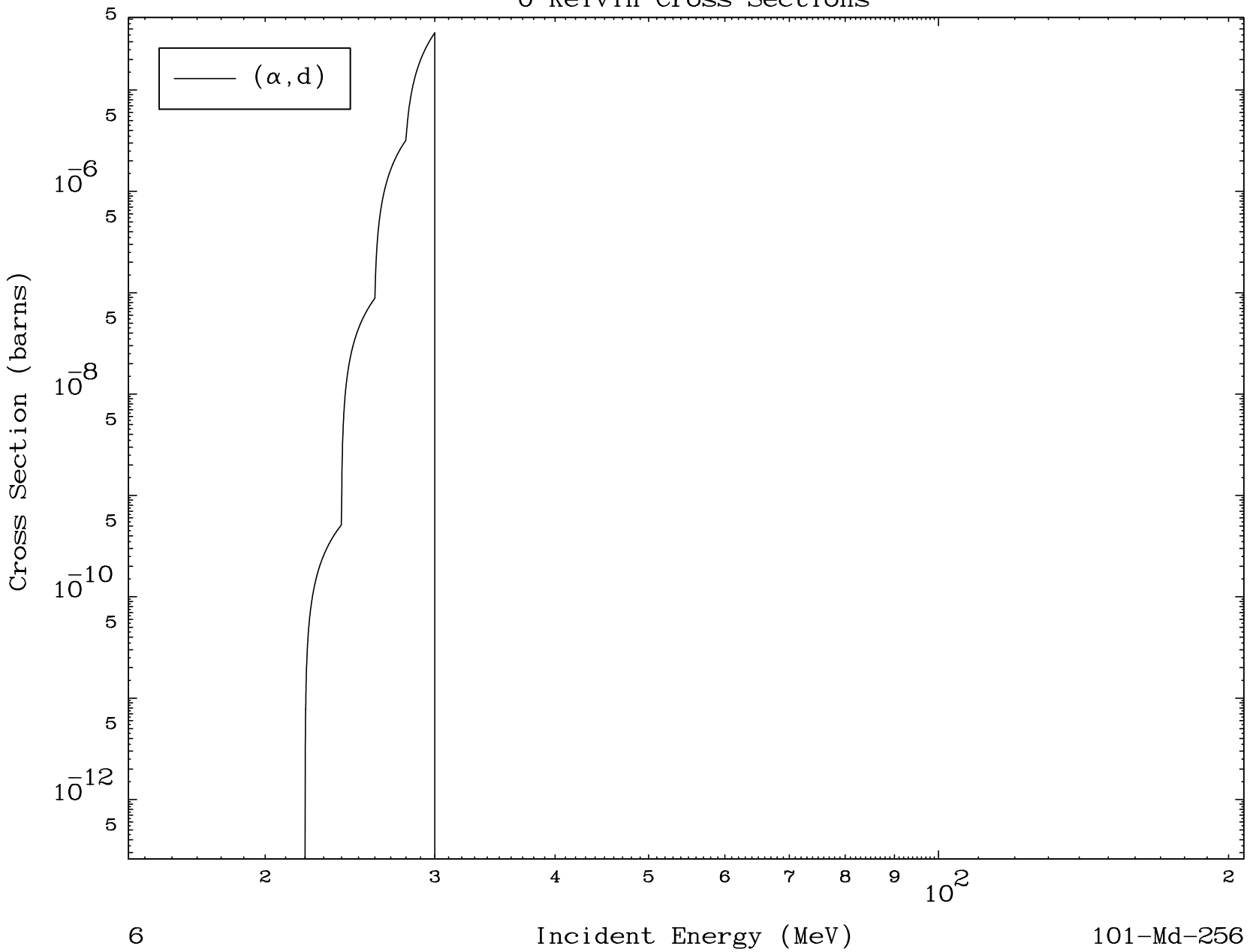

MAT 9957
α Major
101-Md-256
0 Kelvin Cross Sections
α Inelastic
(α,remainder)
(α,γ)
Cross Section (barns)
Incident Energy (MeV)
101-Md-256

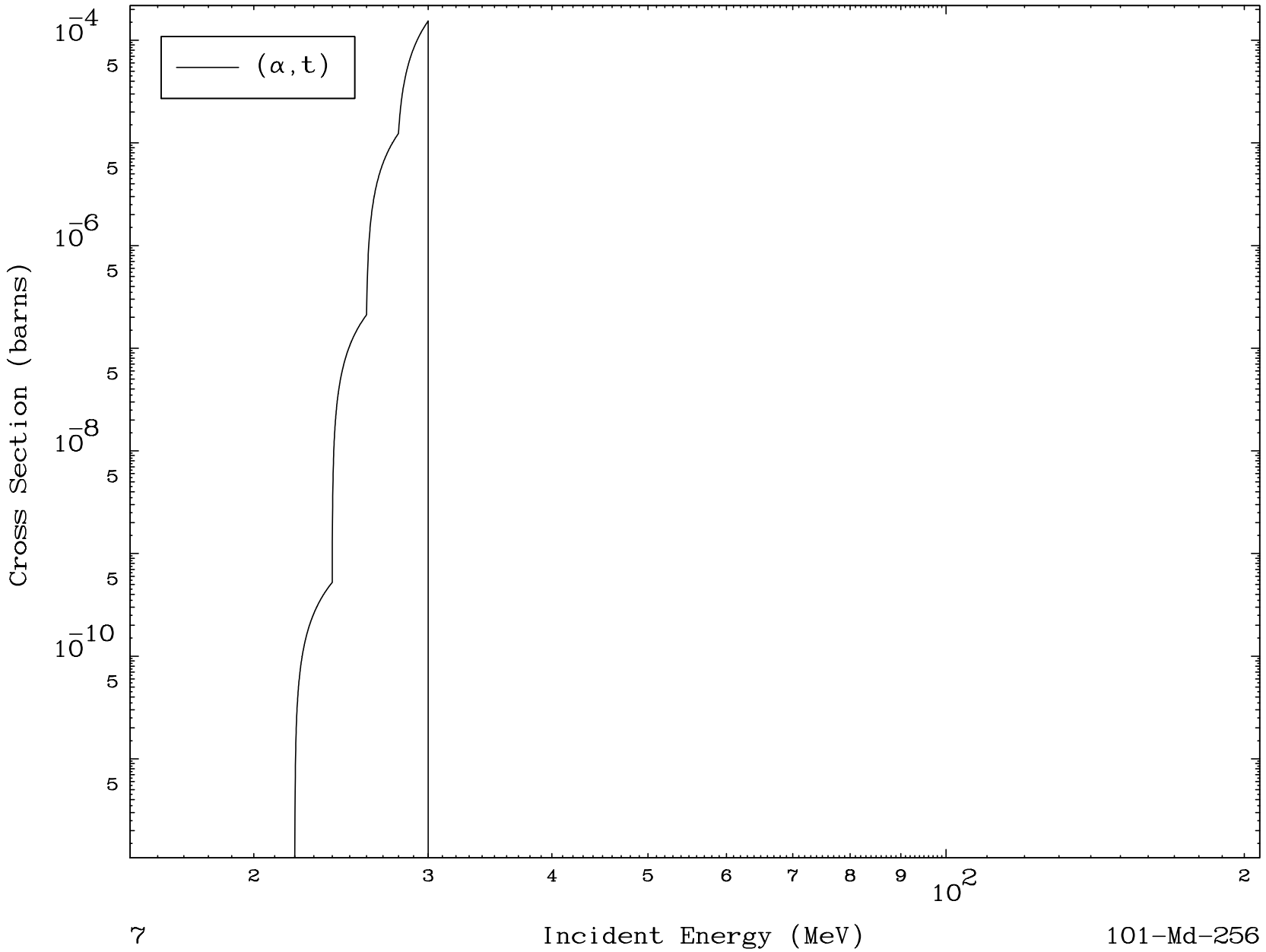

MAT 9957
α Neutron Production
0 Kelvin Cross Sections
101-Md-256
(α,2n)
(α,3n)
(α,n') α
(α,2n) α
(α,3n) α
(α,n') p
(α,n') d
(α,n') t
(α,2n) p
Cross Section (barns)
Incident Energy (MeV)
101-Md-256
2

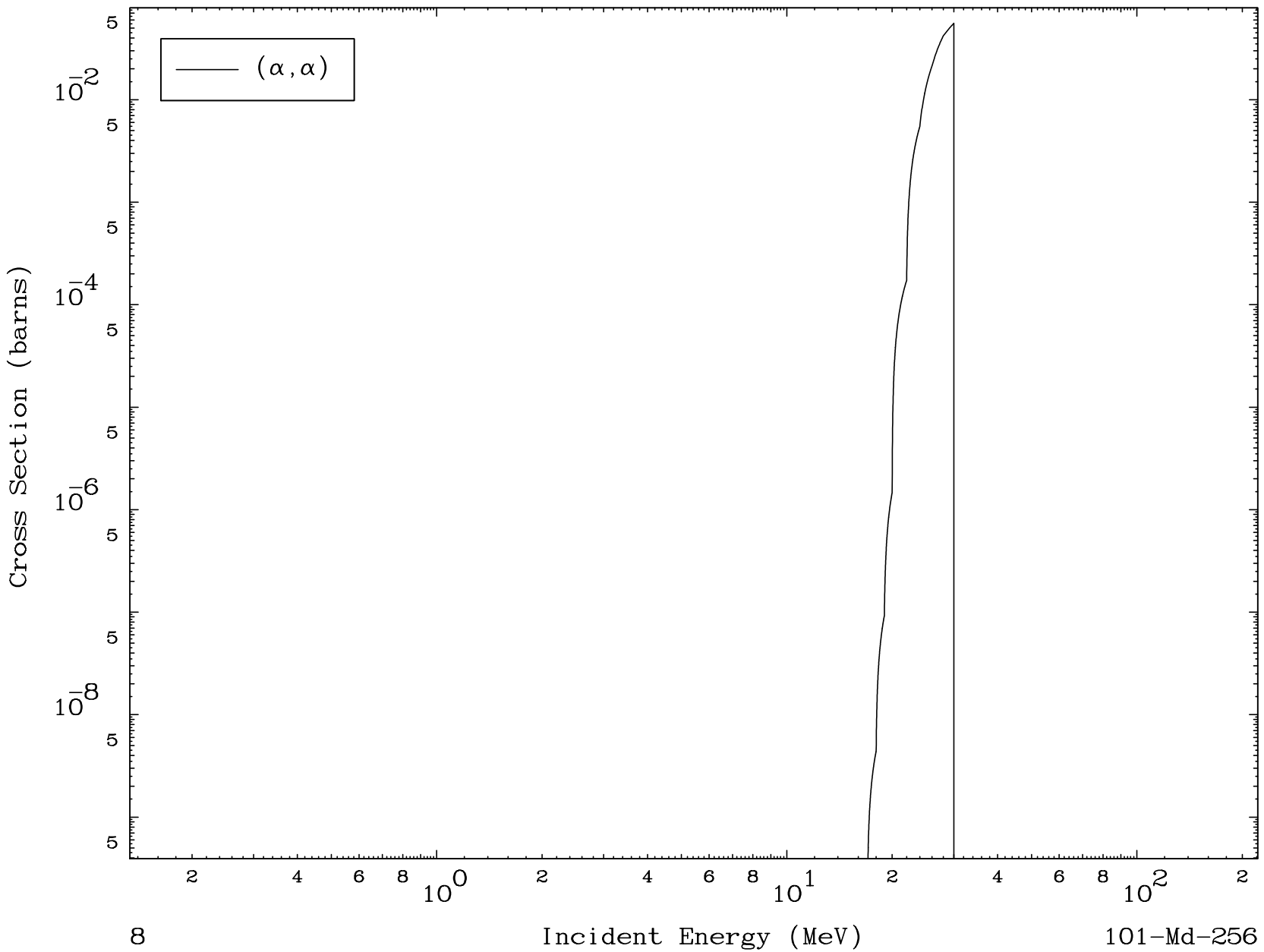

MAT 9957
α Charged Particle
0 Kelvin Cross Sections
101-Md-256
(α,n') α
(α,2n) α
(α,3n) α
(α,n') p
(α,n') d
(α,n') t
(α,2n) p
(α,p)
(α,d)
(α,t)
(α,α)
Cross Section (barns)
Incident Energy (MeV)
3
101-Md-256

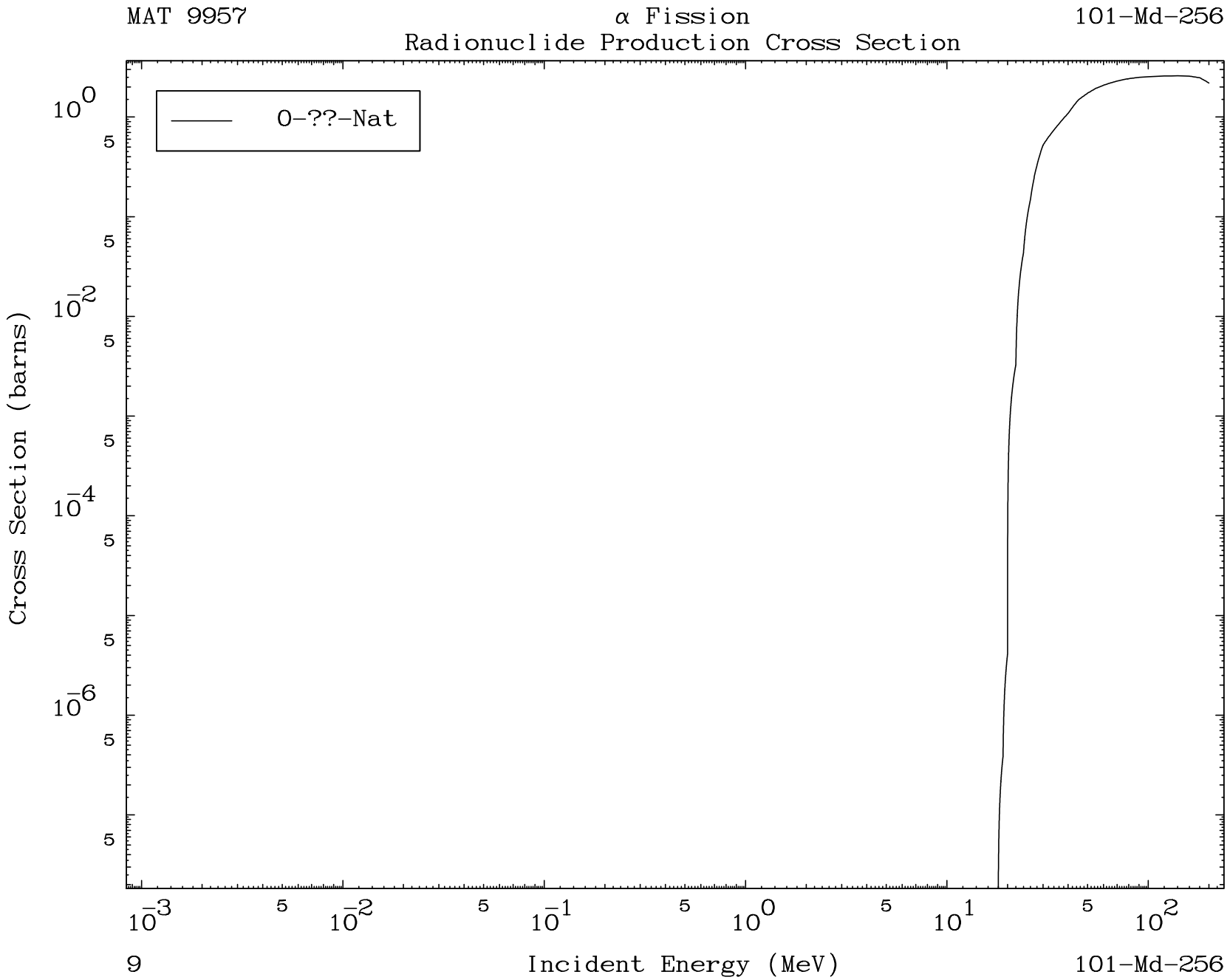

0 Kelvin Cross Sections
Cross Section (barns)
10-3
10-4
10-5
10-6
10-7
10-8
10-9
α Inelastic
2
3
4
5
6
7
8
9
10 2
2
Incident Energy (MeV)

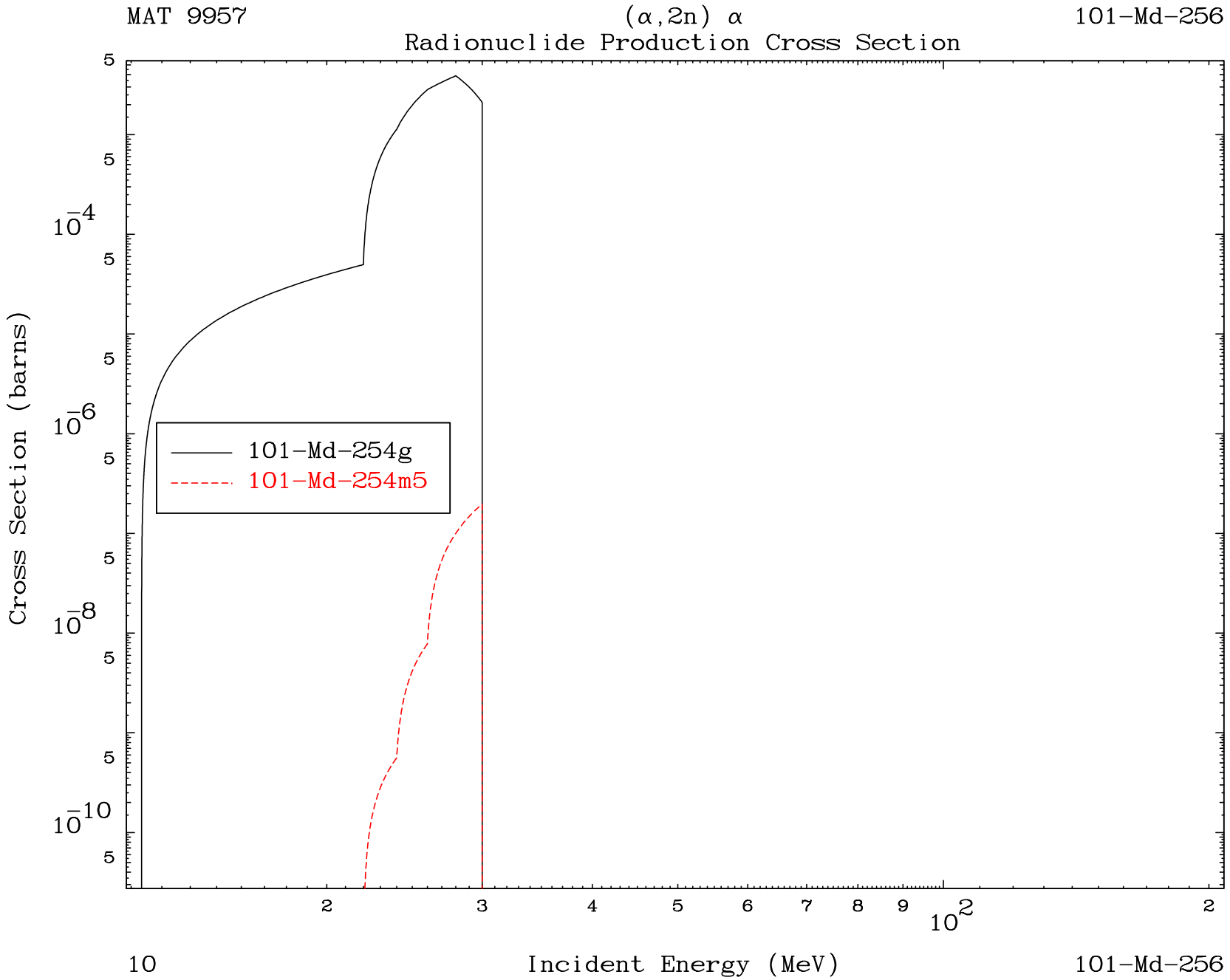

MAT 9957
(α,p) Levels
101-Md-256
0 Kelvin Cross Sections
(α,p)
Cross Section (barns)
10-4
10-6
10-8
10-10
Incident Energy (MeV)
2
3
4
5
6
7
8
9
10 2


0 Kelvin Cross Sections
(α,d)
Cross Section (barns)
10^-6
10^-8
10^-10
10^-12
5
2
3
4
5
6
7
8
9
10^2
2
Incident Energy (MeV)


0 Kelvin Cross Sections
(α,t)
Cross Section (barns)
10^-4
5
10^-6
5
10^-8
5
10^-10
5
5
2
3
4
5
6
7
8
9
10^2
2
Incident Energy (MeV)


0 Kelvin Cross Sections
(α,α)
Cross Section (barns)
10-2
10-4
10-6
10-8
100
101
102
Incident Energy (MeV)


MAT 9957
α Fission
101-Md-256
Radionuclide Production Cross Section
0-??-Nat
Cross Section (barns)
10^0
10^-2
10^-4
10^-6
10^-3
10^-2
10^-1
10^0
10^1
10^2
Incident Energy (MeV)
9
101-Md-256


MAT 9957
(α,2n) α
101-Md-256
Radionuclide Production Cross Section
Cross Section (barns)
10^-4
10^-6
10^-8
10^-10
101-Md-254g
101-Md-254m5
10
Incident Energy (MeV)
10^2
101-Md-256