

Program EVALPLOT  
(Version 2018-1)

by

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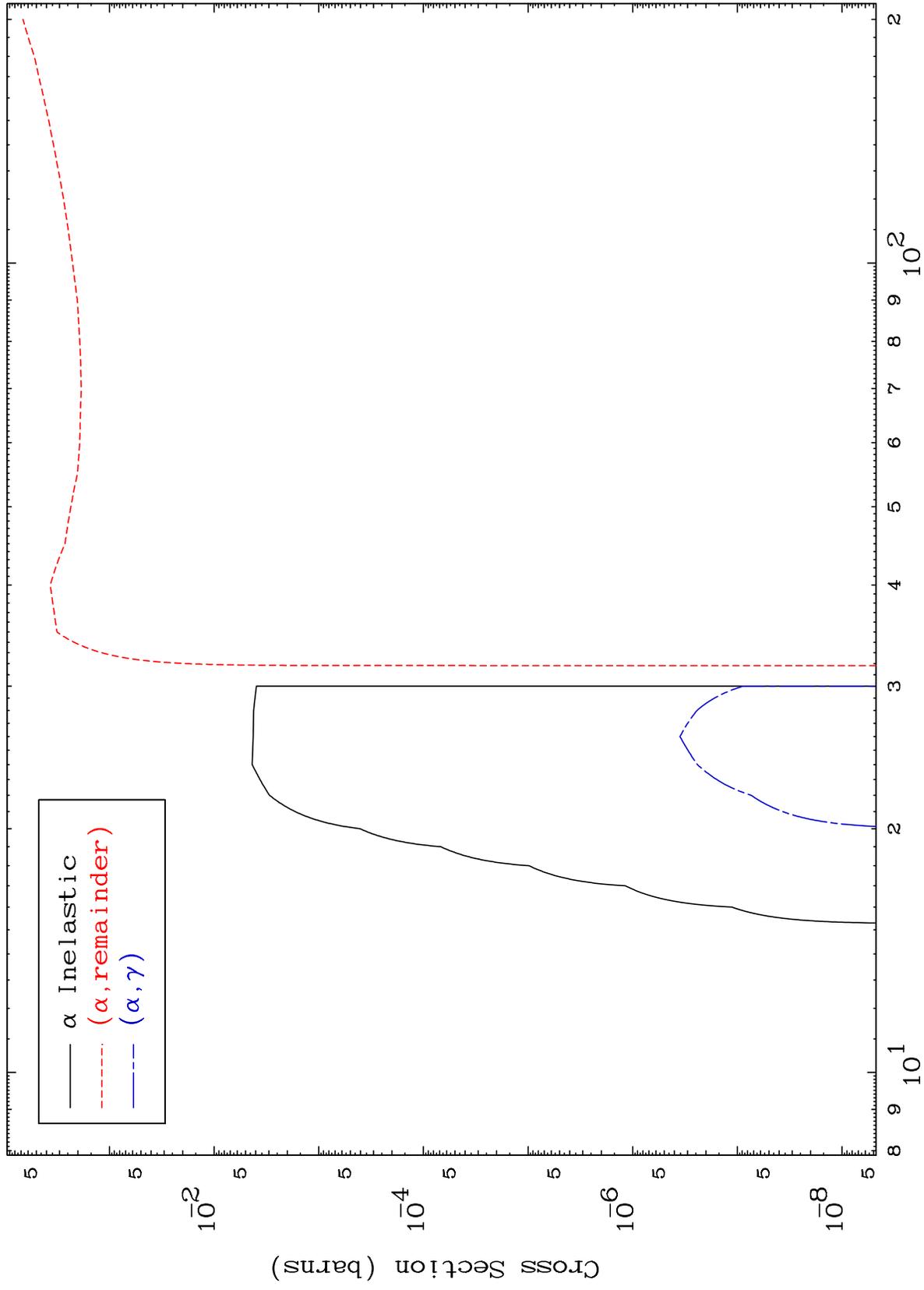
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9958

0 Kelvin  $\alpha$  Major  
Cross Sections

101-Md-257



1

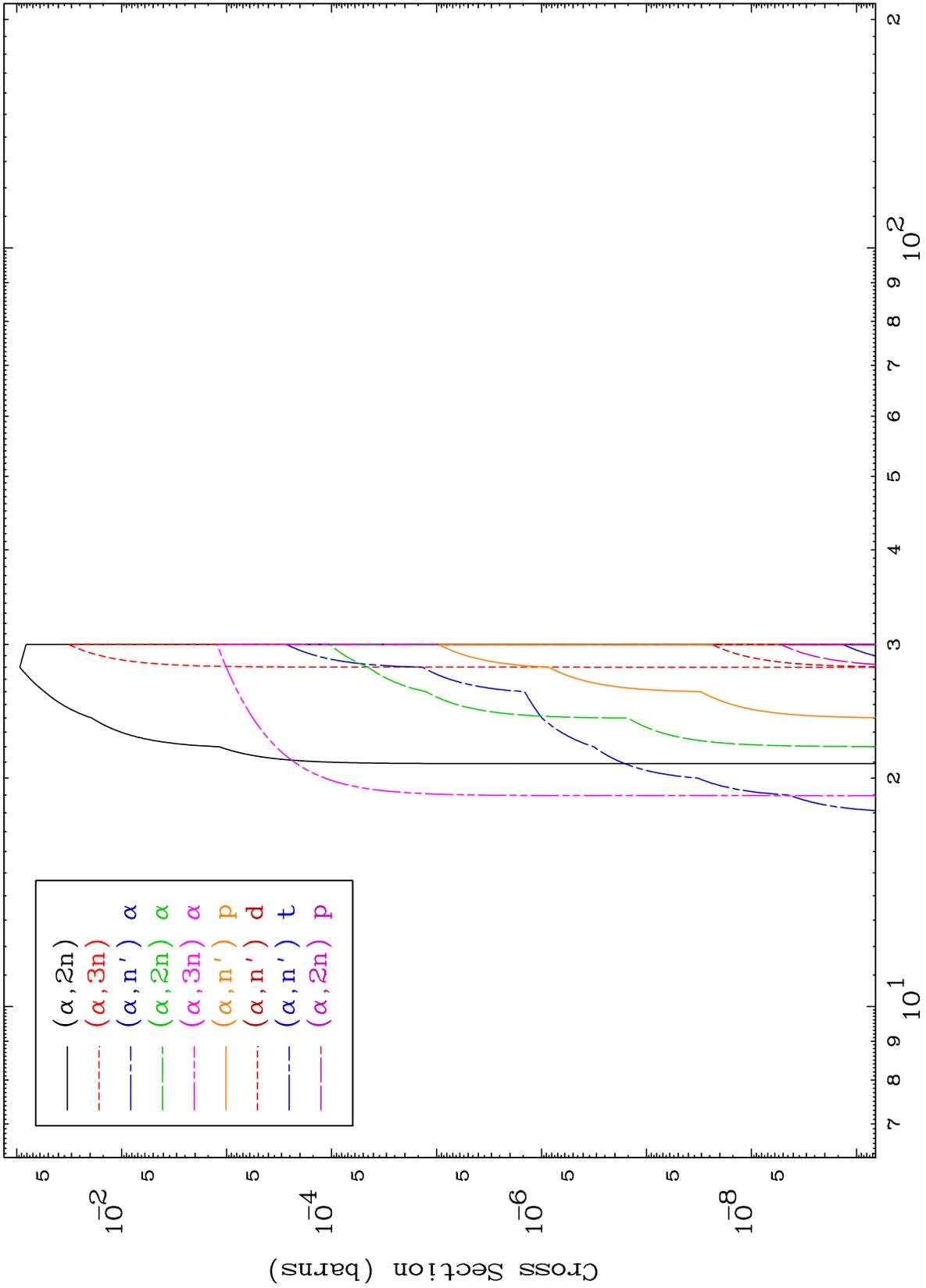
Incident Energy (MeV)

101-Md-257

MAT 9958

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

101-Md-257



2

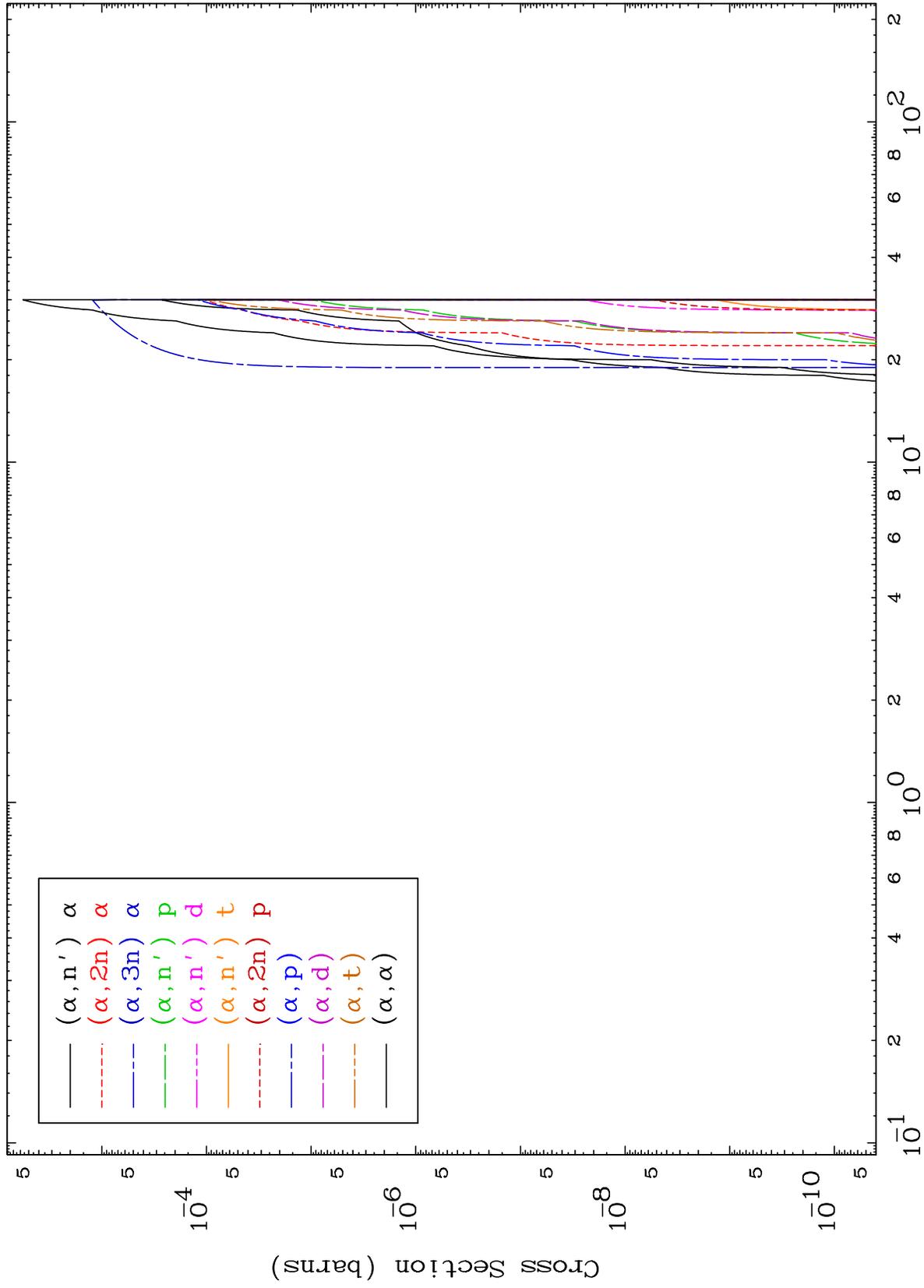
Incident Energy (MeV)

101-Md-257

MAT 9958

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

101-Md-257



Incident Energy (MeV)

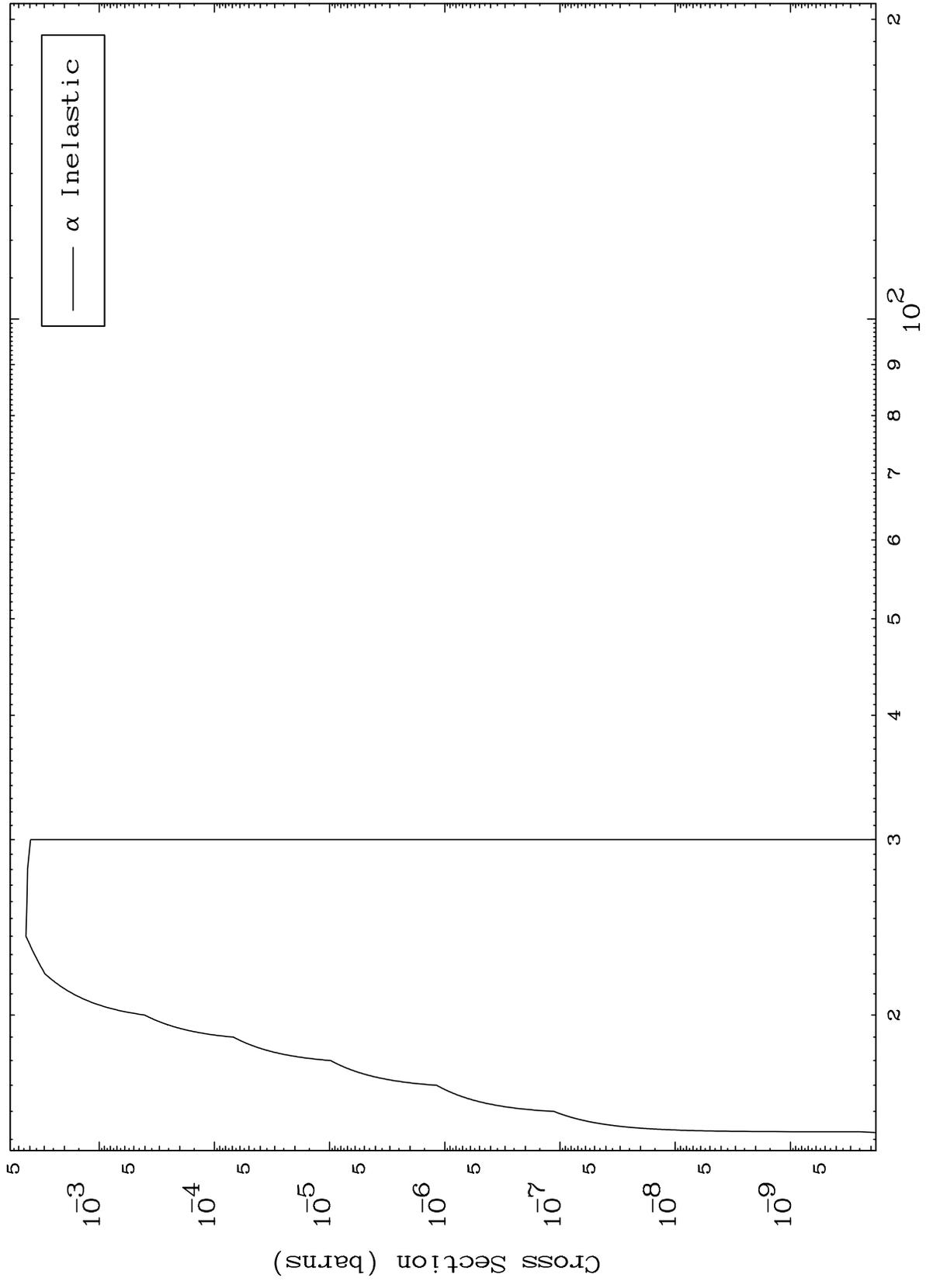
101-Md-257

3

MAT 9958

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

101-Md-257

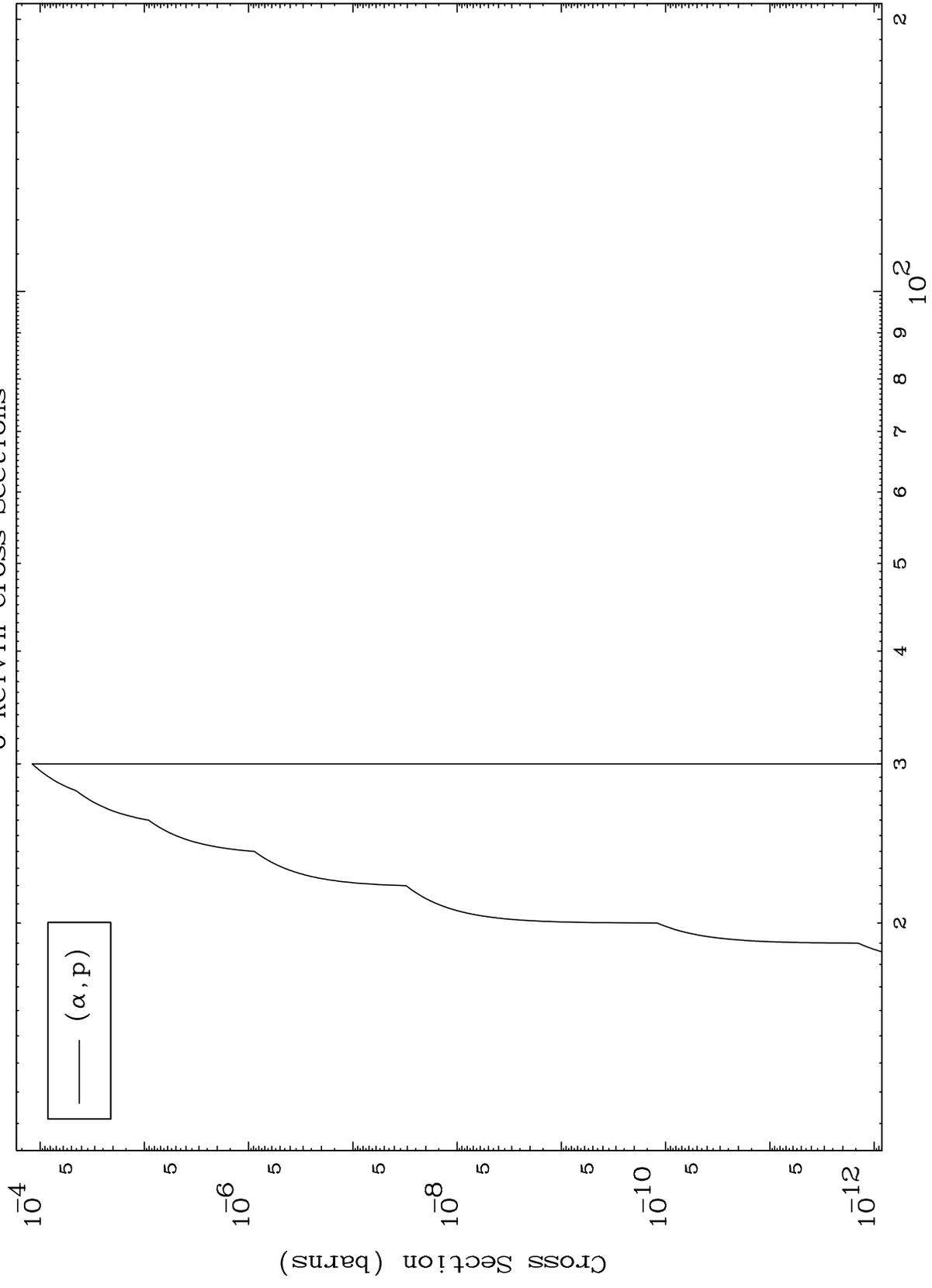


$\alpha$  Inelastic

MAT 9958

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

101-Md-257



5

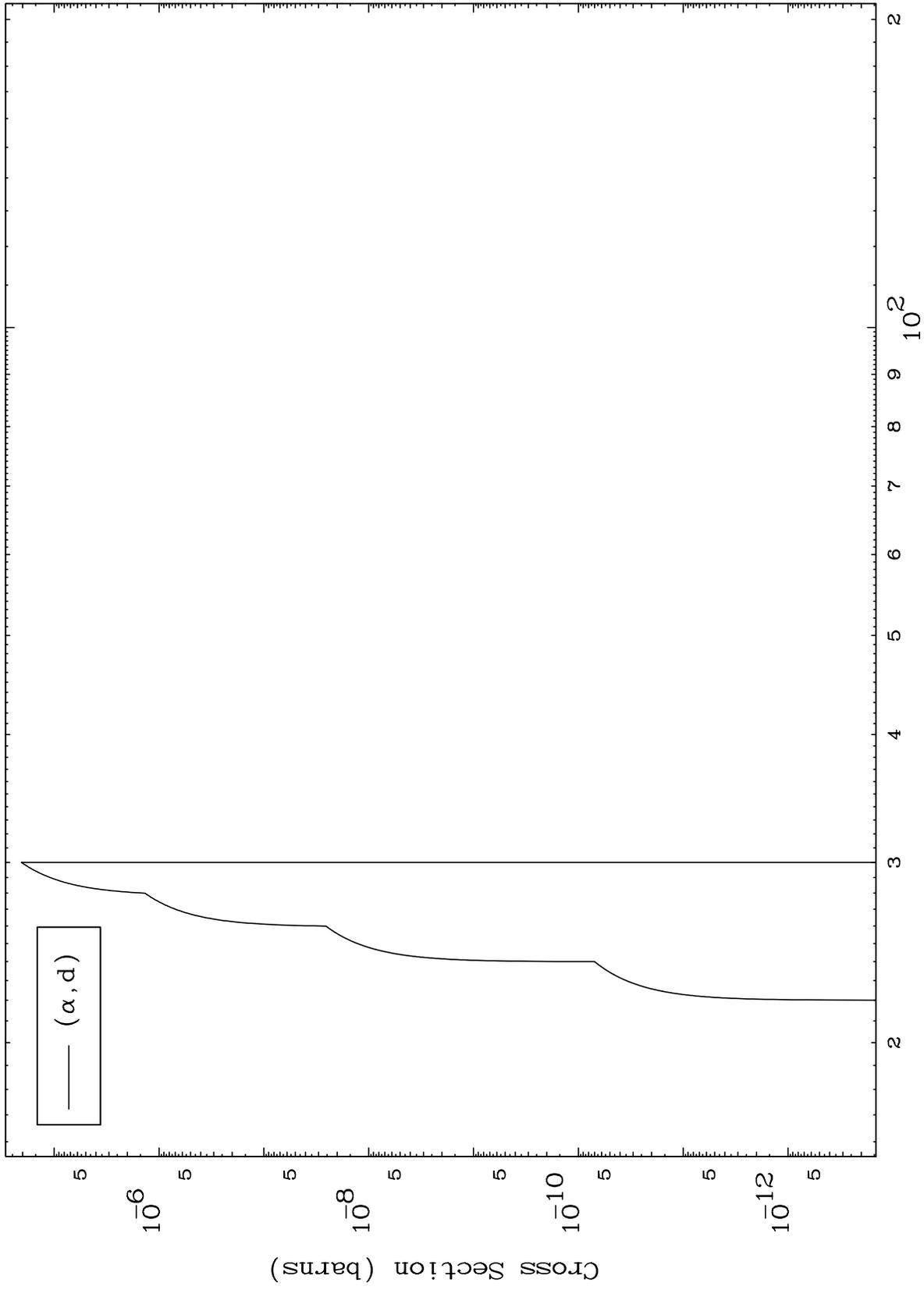
Incident Energy (MeV)

101-Md-257

MAT 9958

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

101-Md-257



6

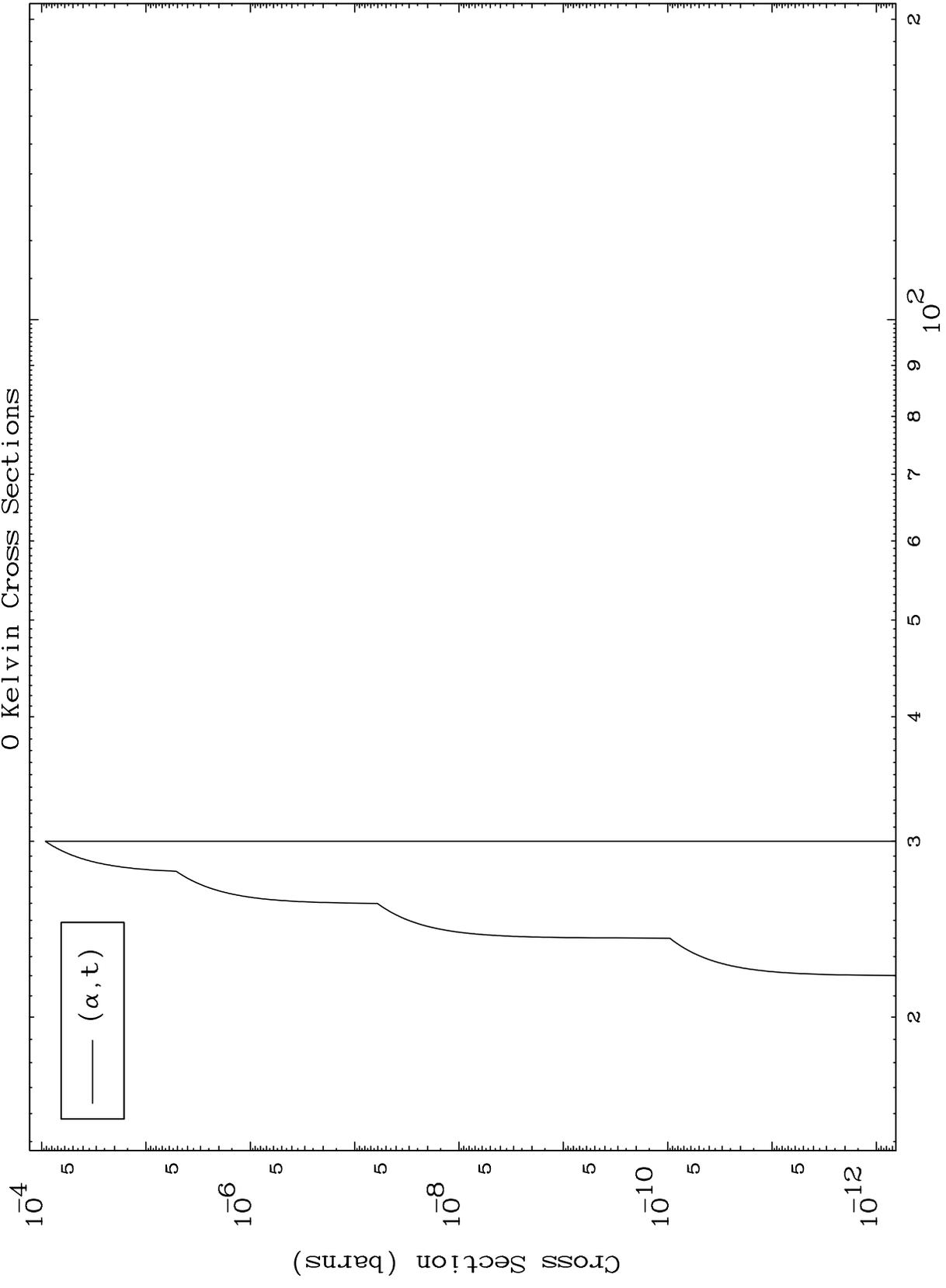
Incident Energy (MeV)

101-Md-257

MAT 9958

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

101-Md-257



7

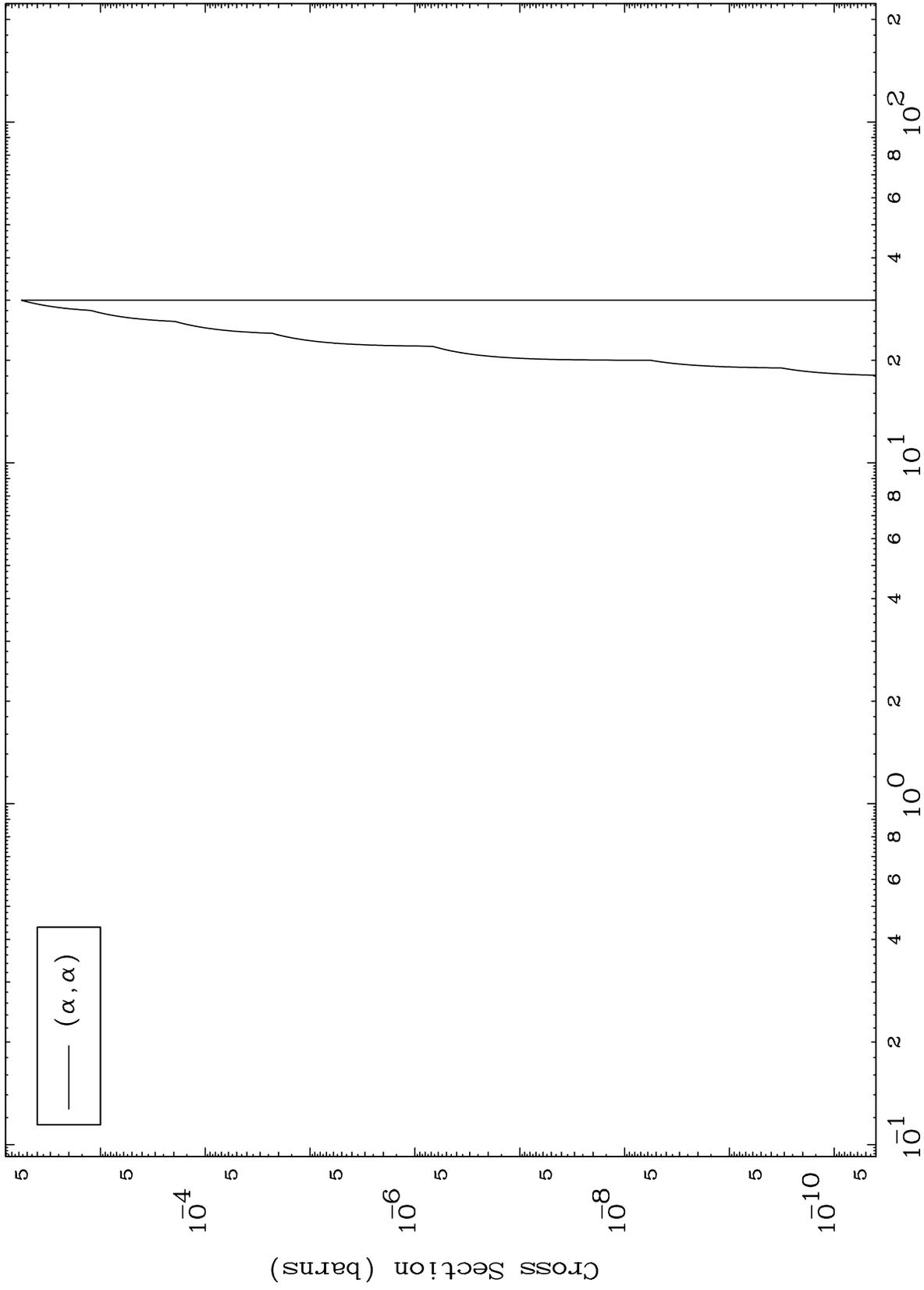
Incident Energy (MeV)

101-Md-257

MAT 9958

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

101-Md-257



8

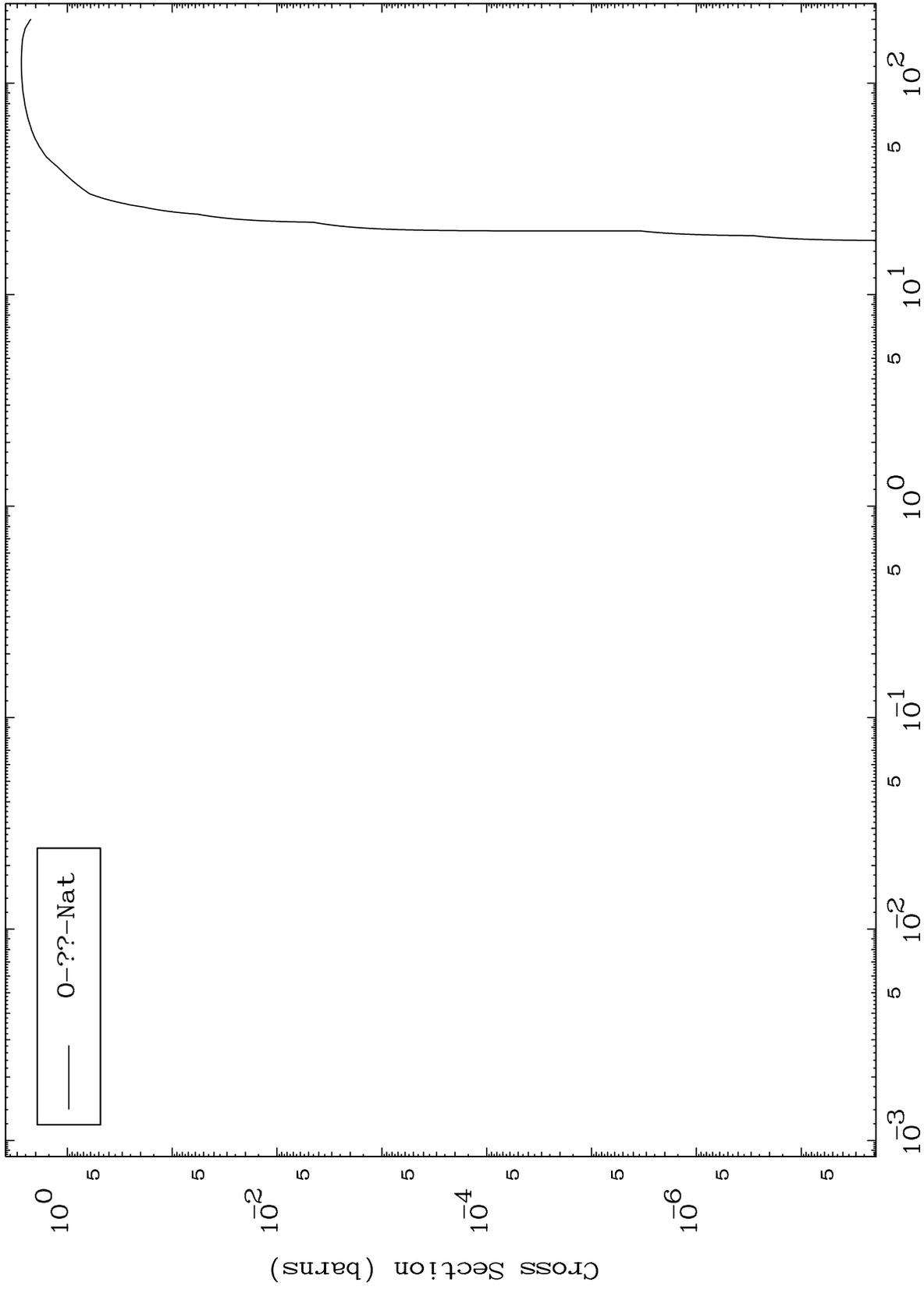
Incident Energy (MeV)

101-Md-257

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$\alpha$  Fission  
Radionuclide Production Cross Section

101-Md-257



Incident Energy (MeV)

101-Md-257

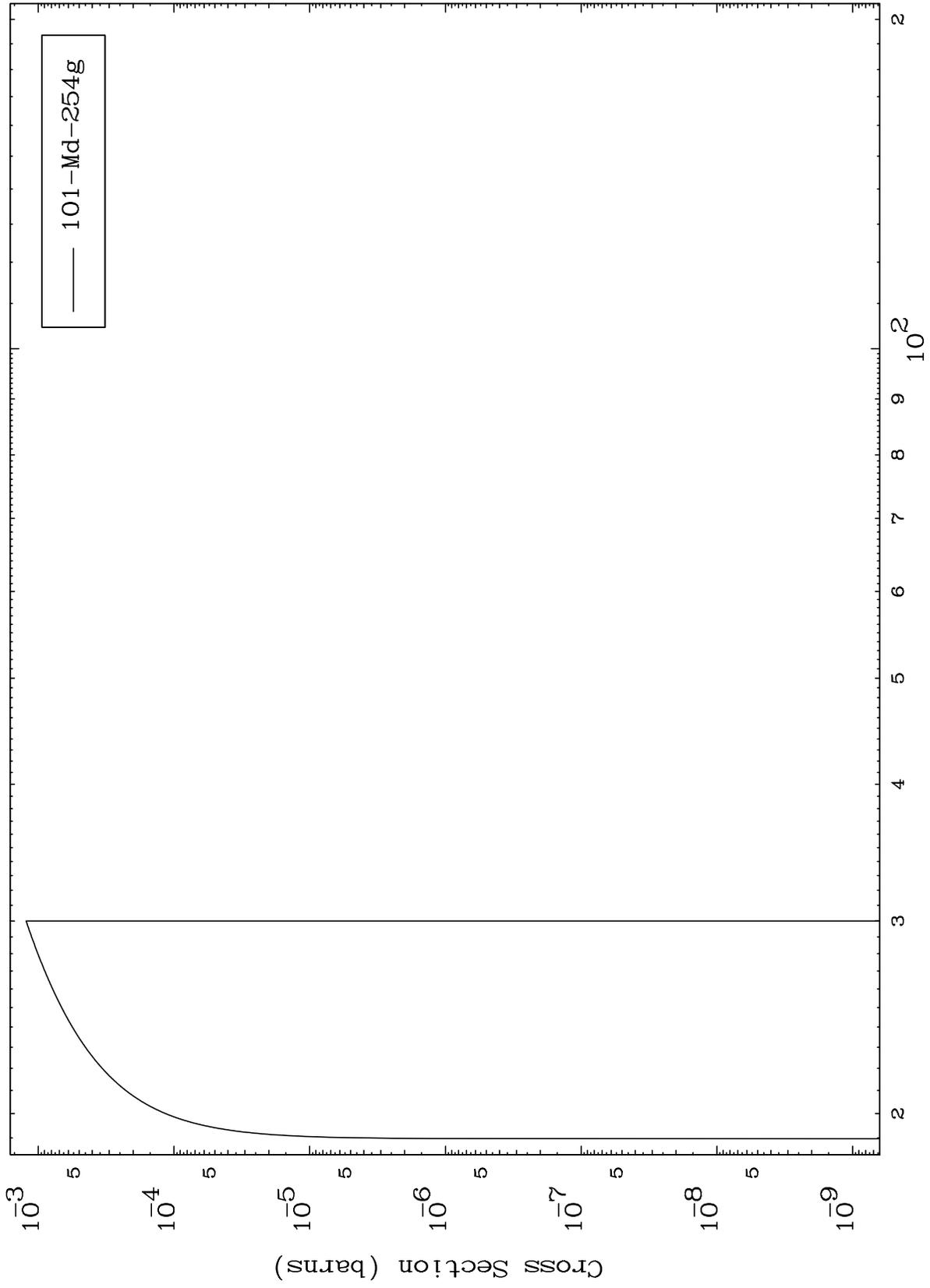
9

MAT 9958

$(\alpha, 3n) \alpha$

101-Md-257

Radionuclide Production Cross Section



10

Incident Energy (MeV)

101-Md-257