

Program EVALPLOT  
(Version 2018-1)

by

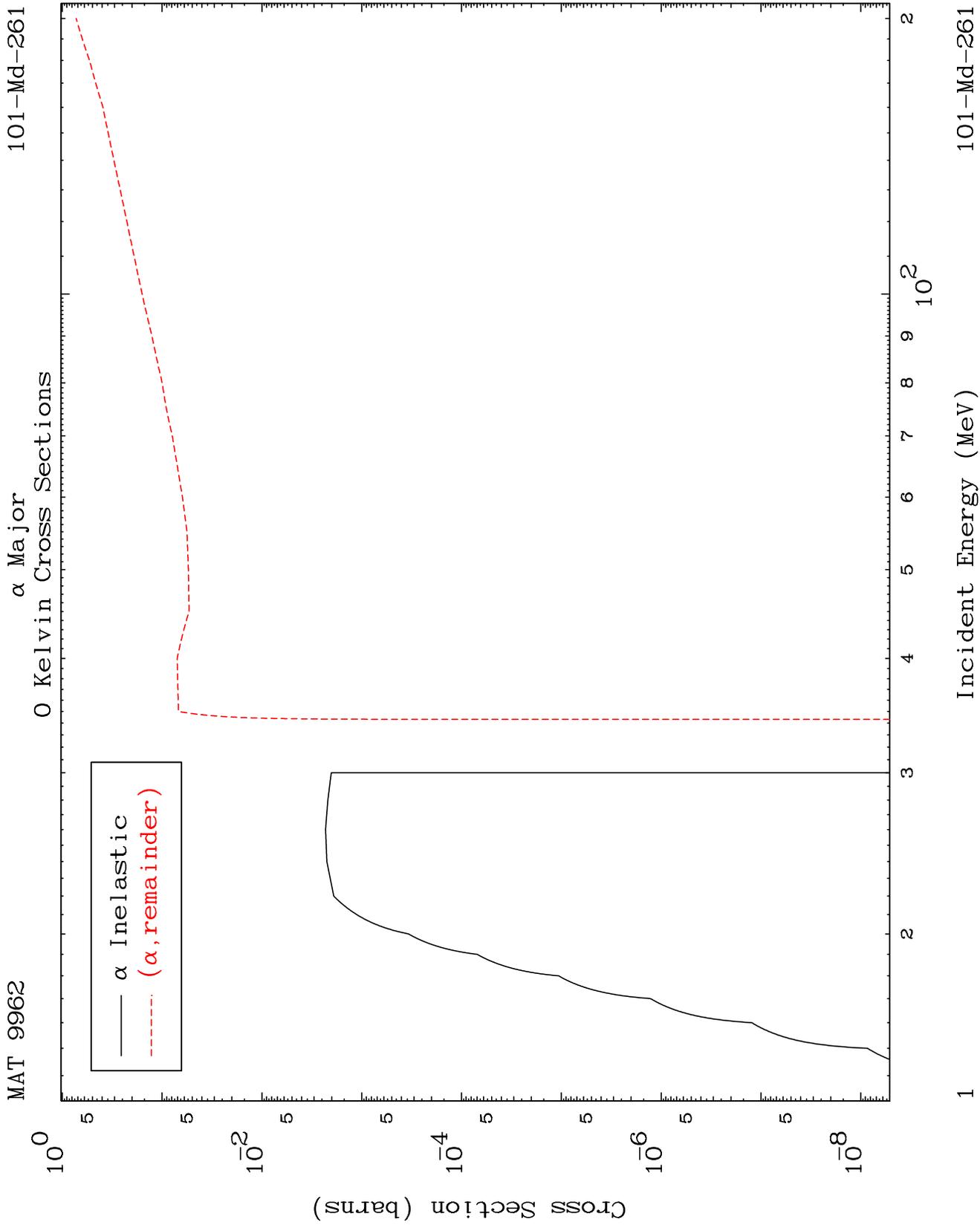
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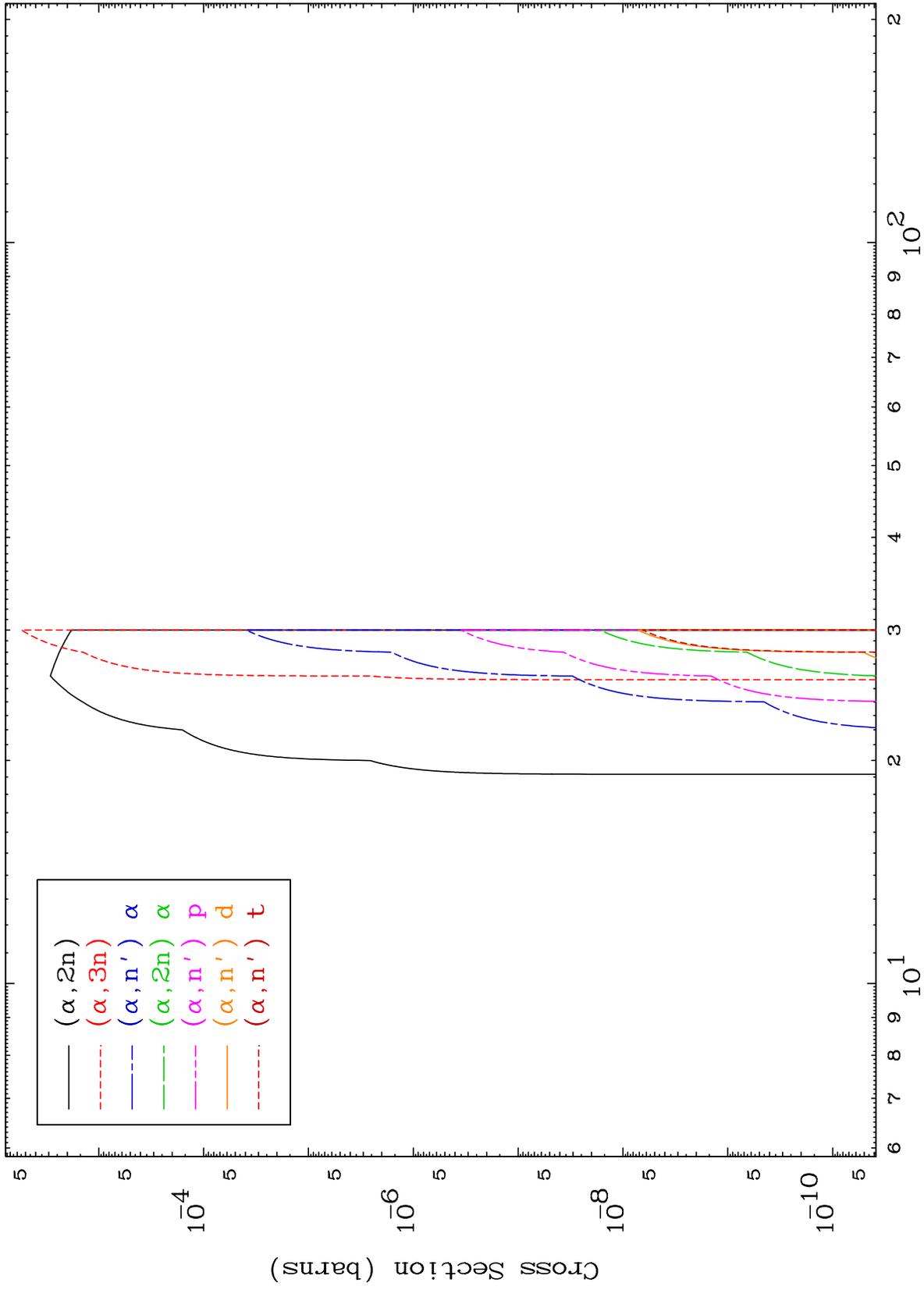
Press Mouse Button to Start



MAT 9962

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

101-Md-261



2

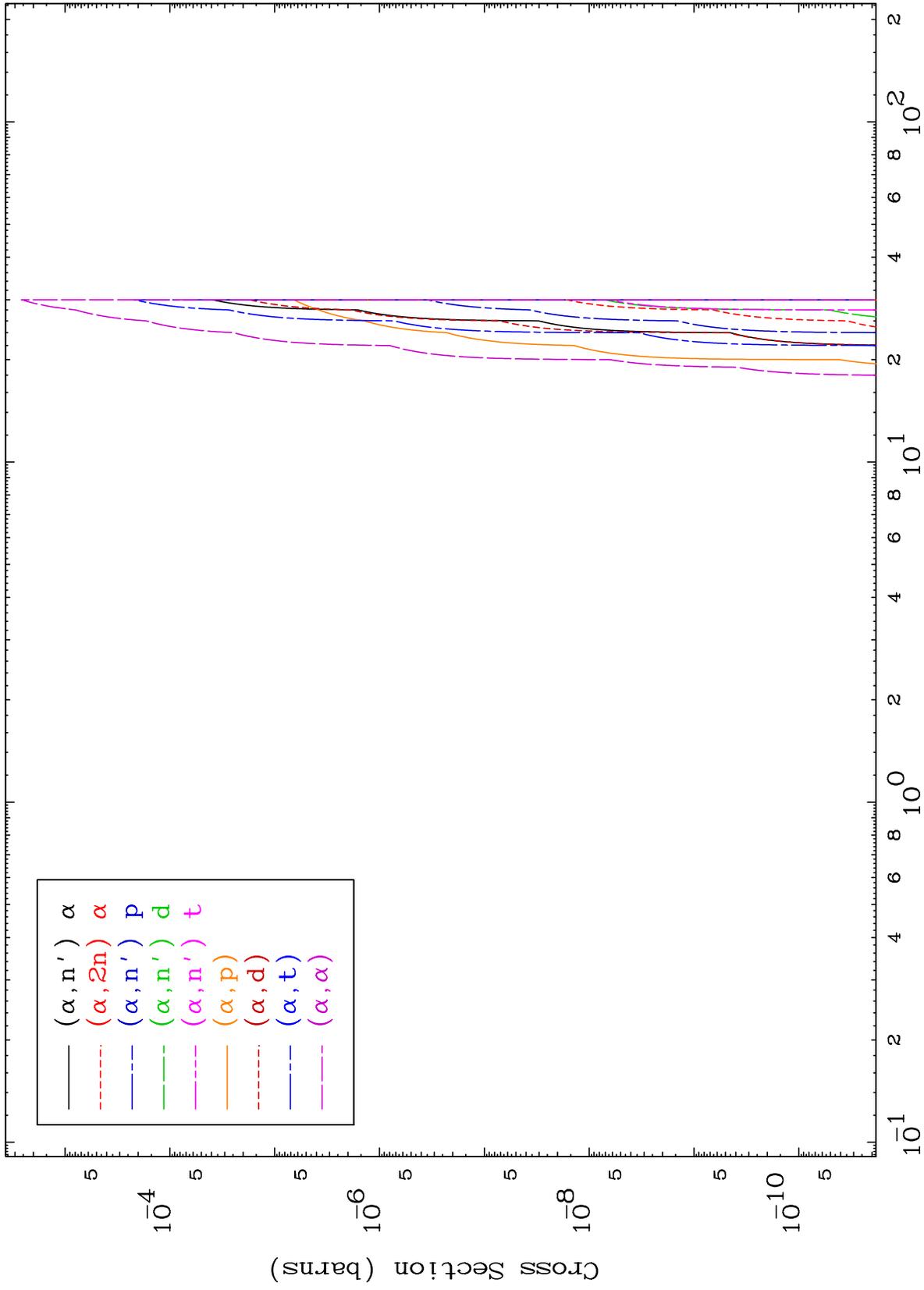
Incident Energy (MeV)

101-Md-261

MAT 9962

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

101-Md-261



3

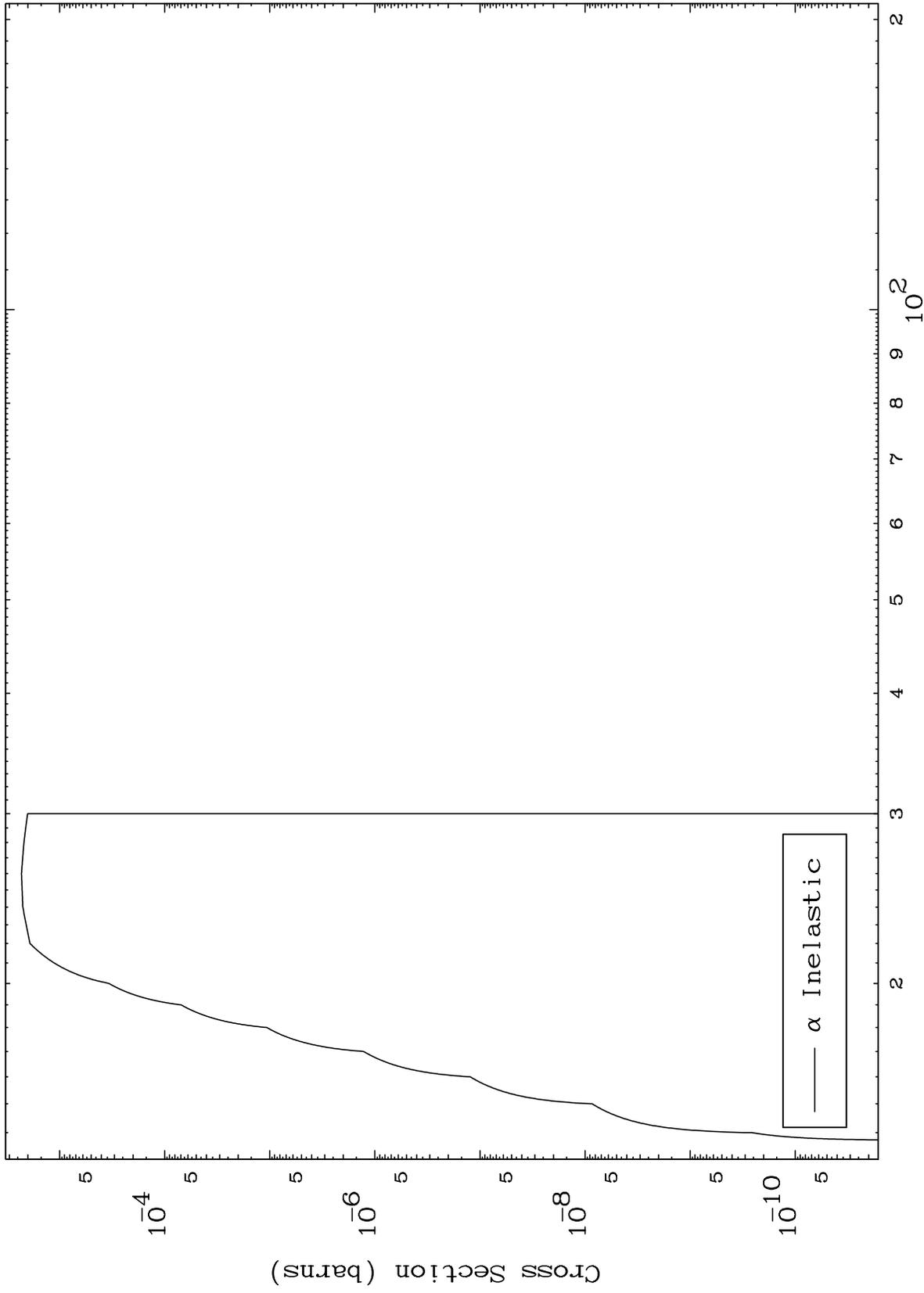
Incident Energy (MeV)

101-Md-261

MAT 9962

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

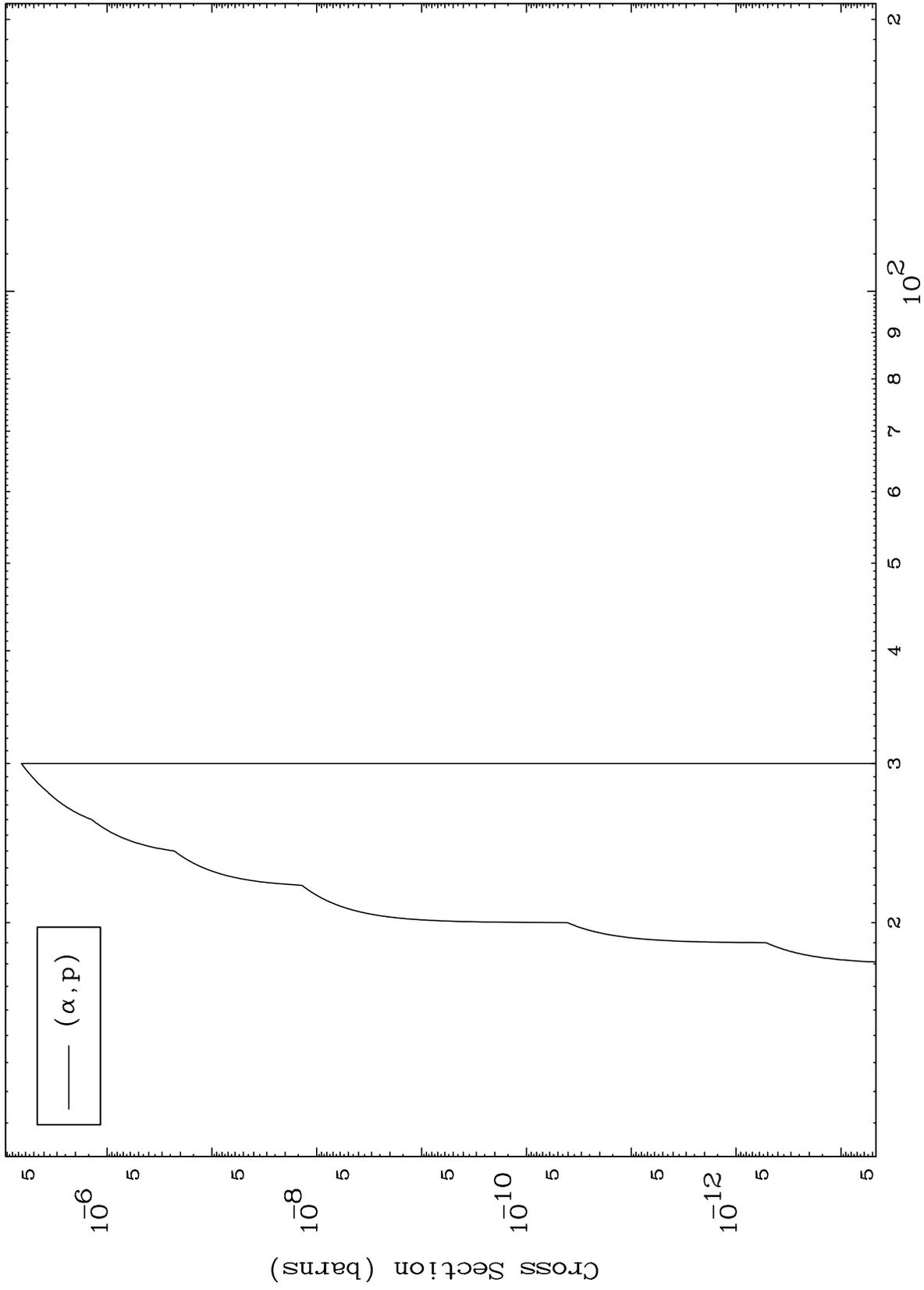
101-Md-261



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Incident Energy (MeV)

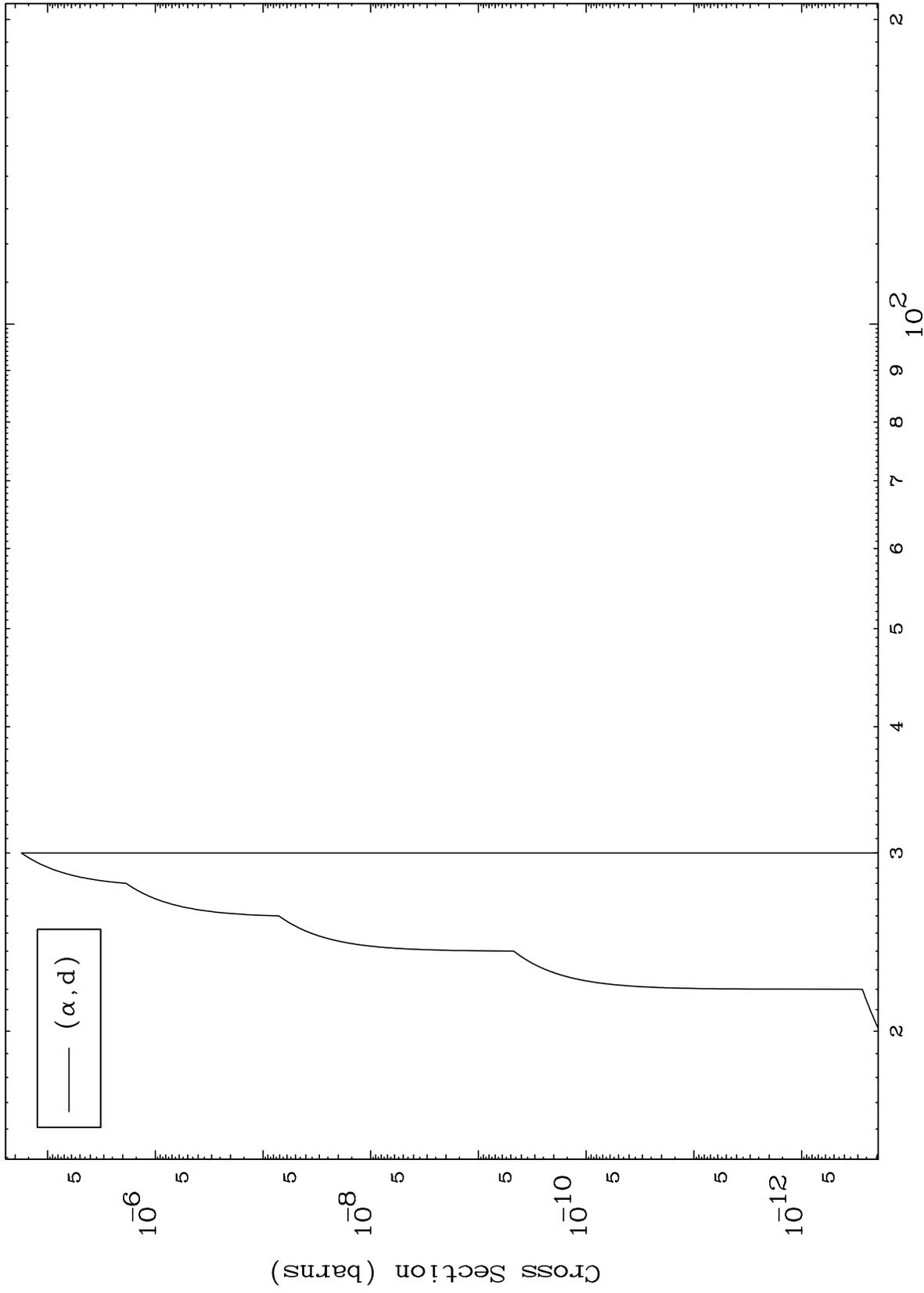
101-Md-261



MAT 9962

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

101-Md-261



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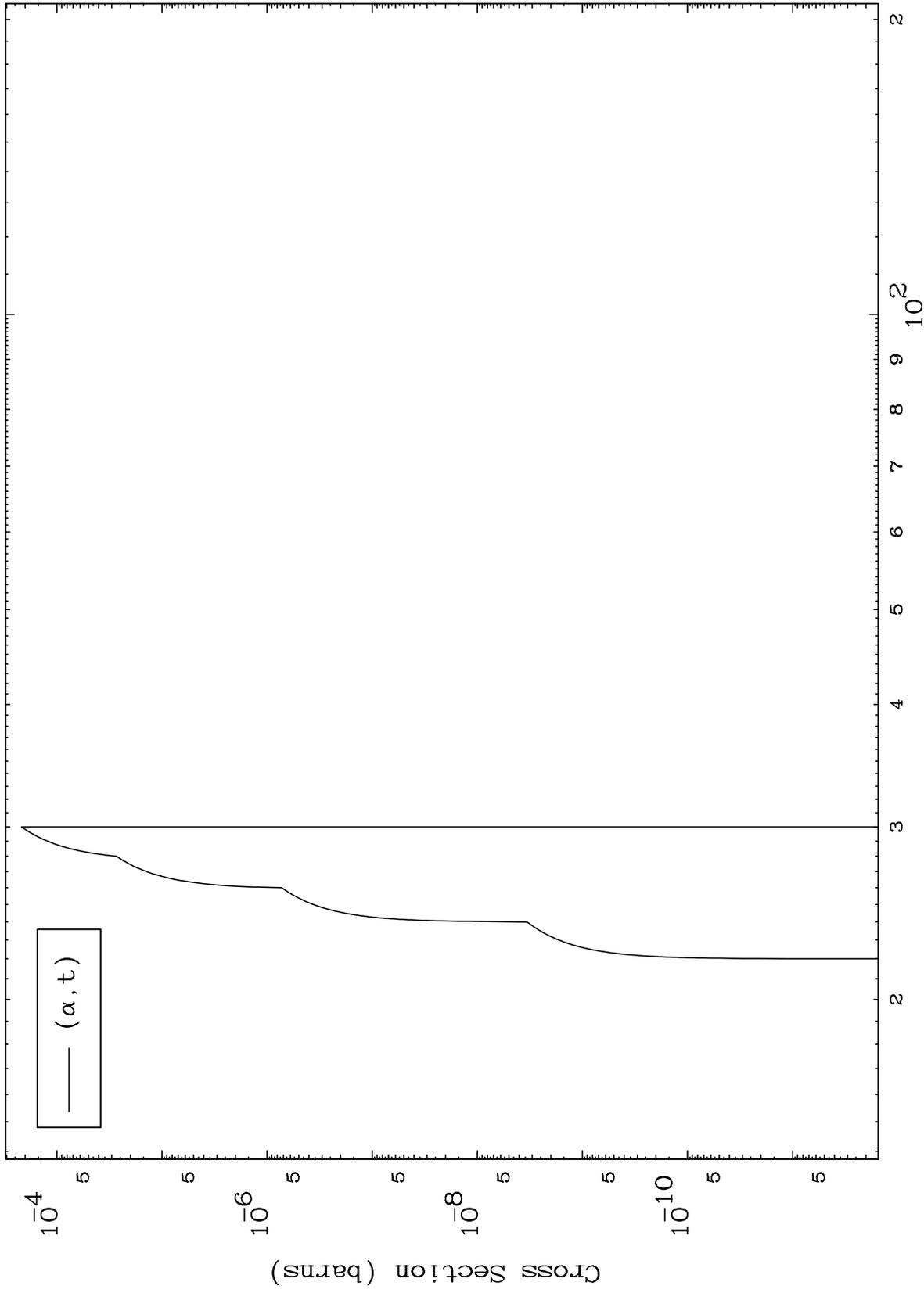
Incident Energy (MeV)

101-Md-261

MAT 9962

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

101-Md-261



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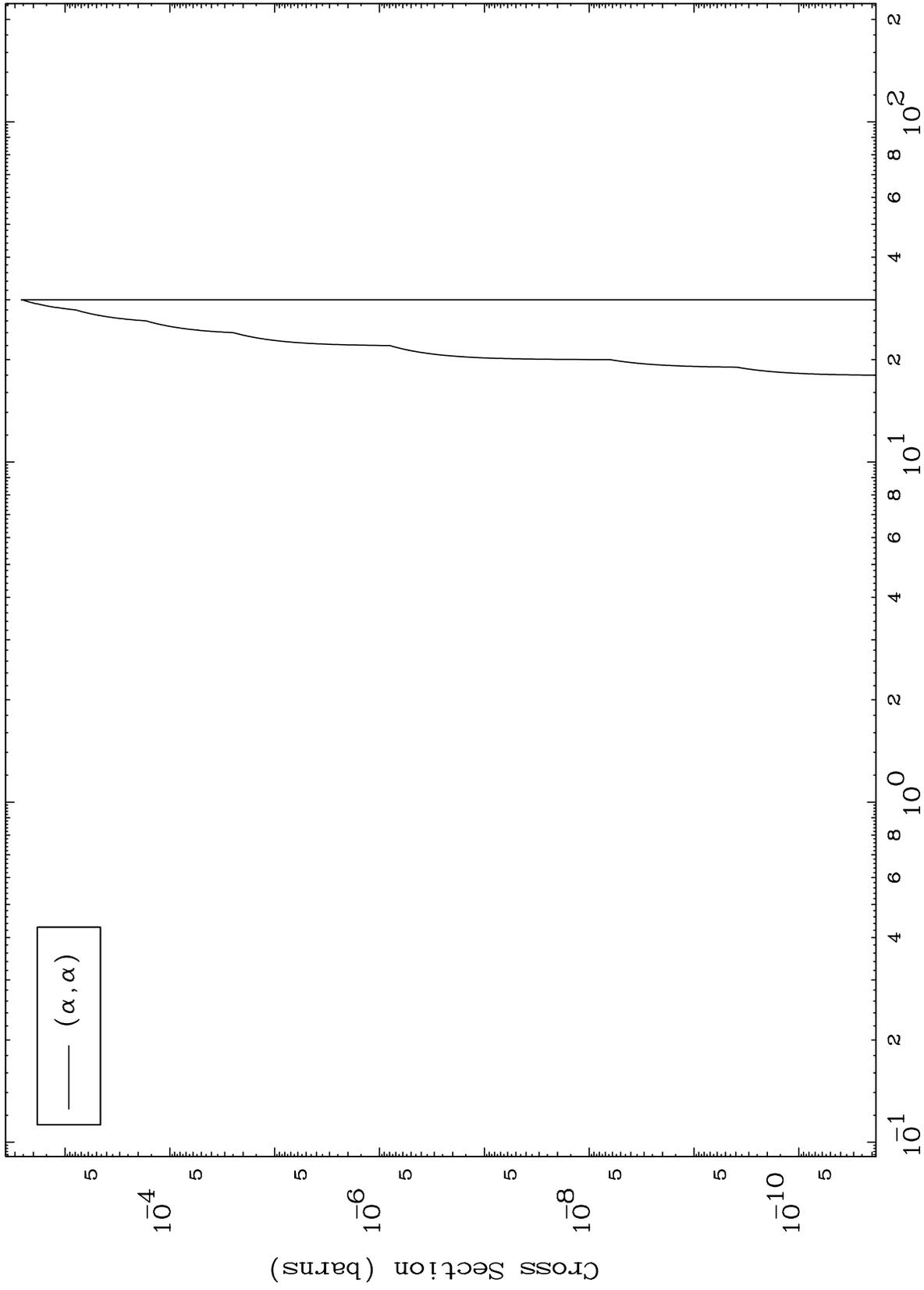
Incident Energy (MeV)

101-Md-261

MAT 9962

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

101-Md-261



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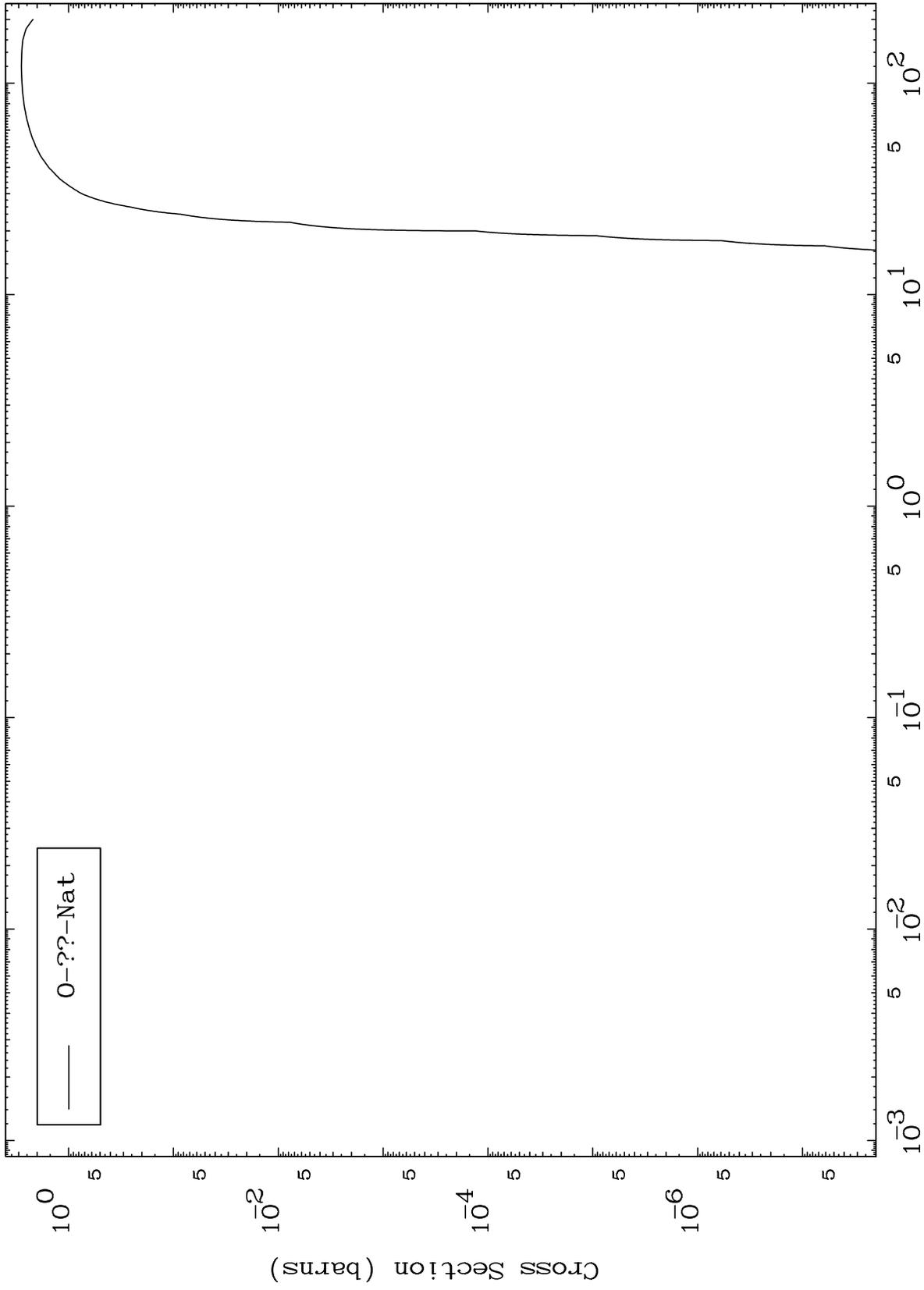
Incident Energy (MeV)

101-Md-261

MAT 9962

$\alpha$  Fission  
Radionuclide Production Cross Section

101-Md-261



Incident Energy (MeV)

101-Md-261

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