

Program EVALPLOT  
(Version 2018-1)

by

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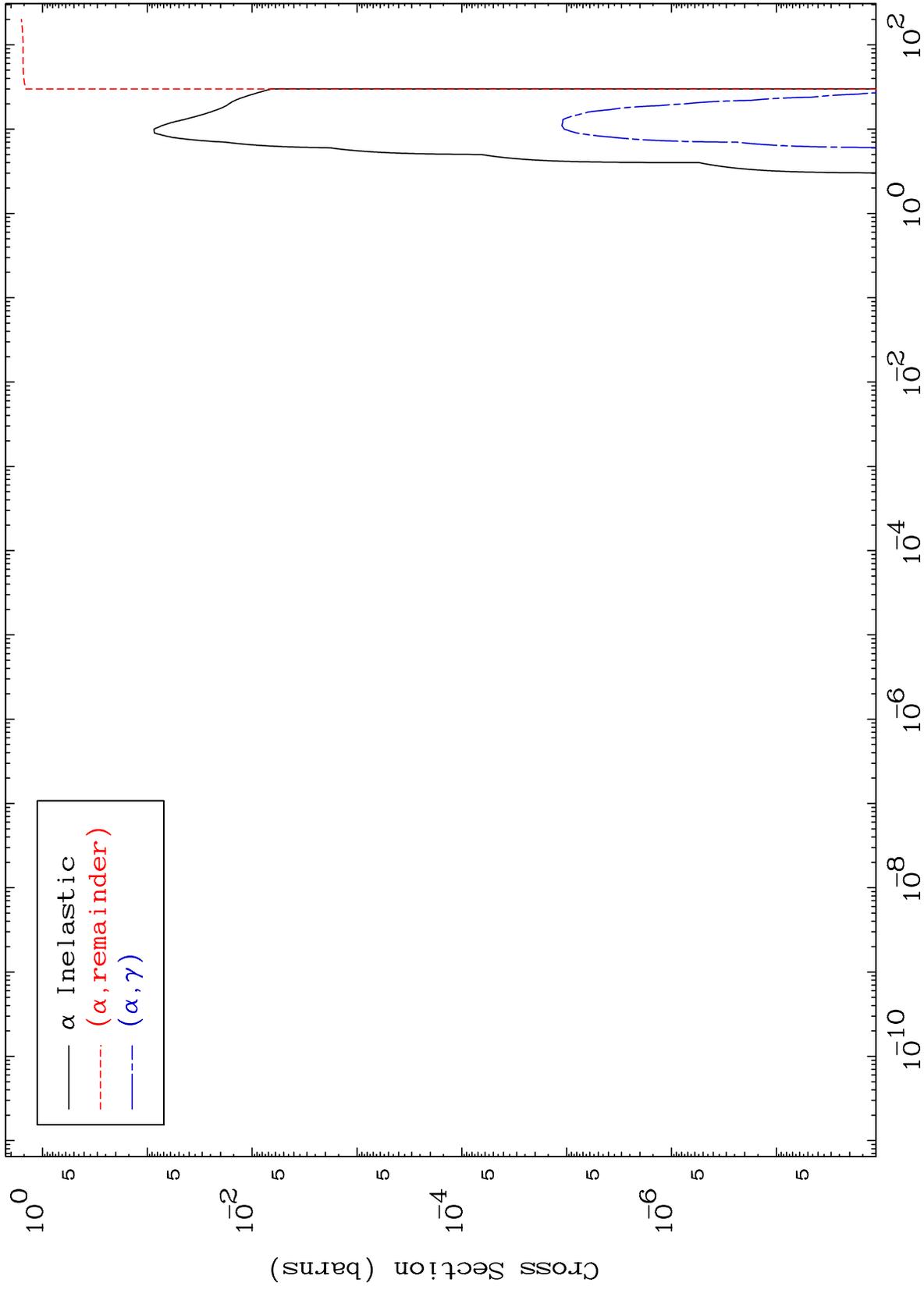
E.Mail: [redcullen1@comcast.net](mailto:redcullen1@comcast.net)  
Web: [redcullen1.net/HOMEPAGE.NEW](http://redcullen1.net/HOMEPAGE.NEW)

Press Mouse Button to Start

MAT 2537

0 Kelvin  $\alpha$  Major  
Cross Sections

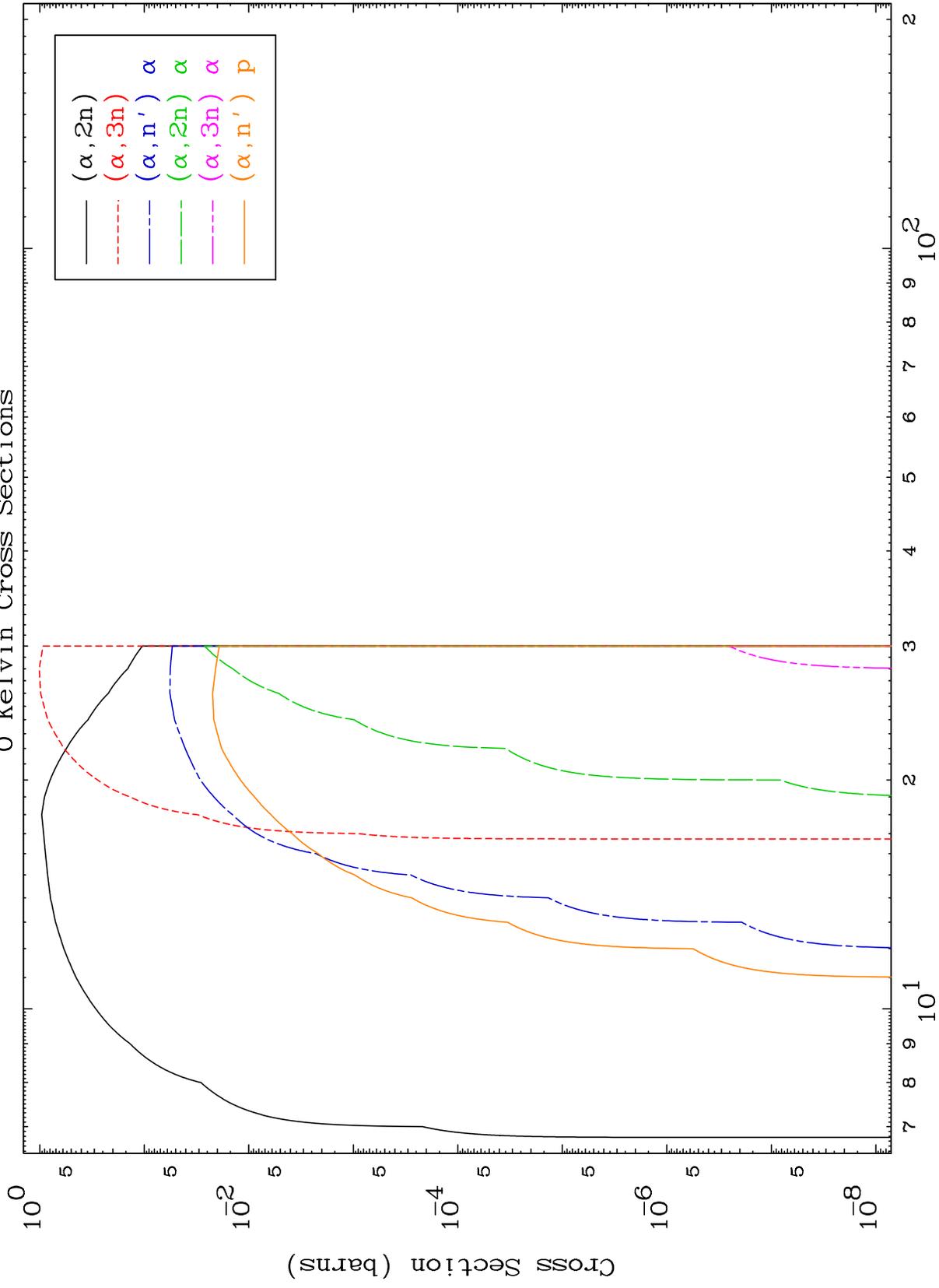
25-Mn-59



MAT 2537

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

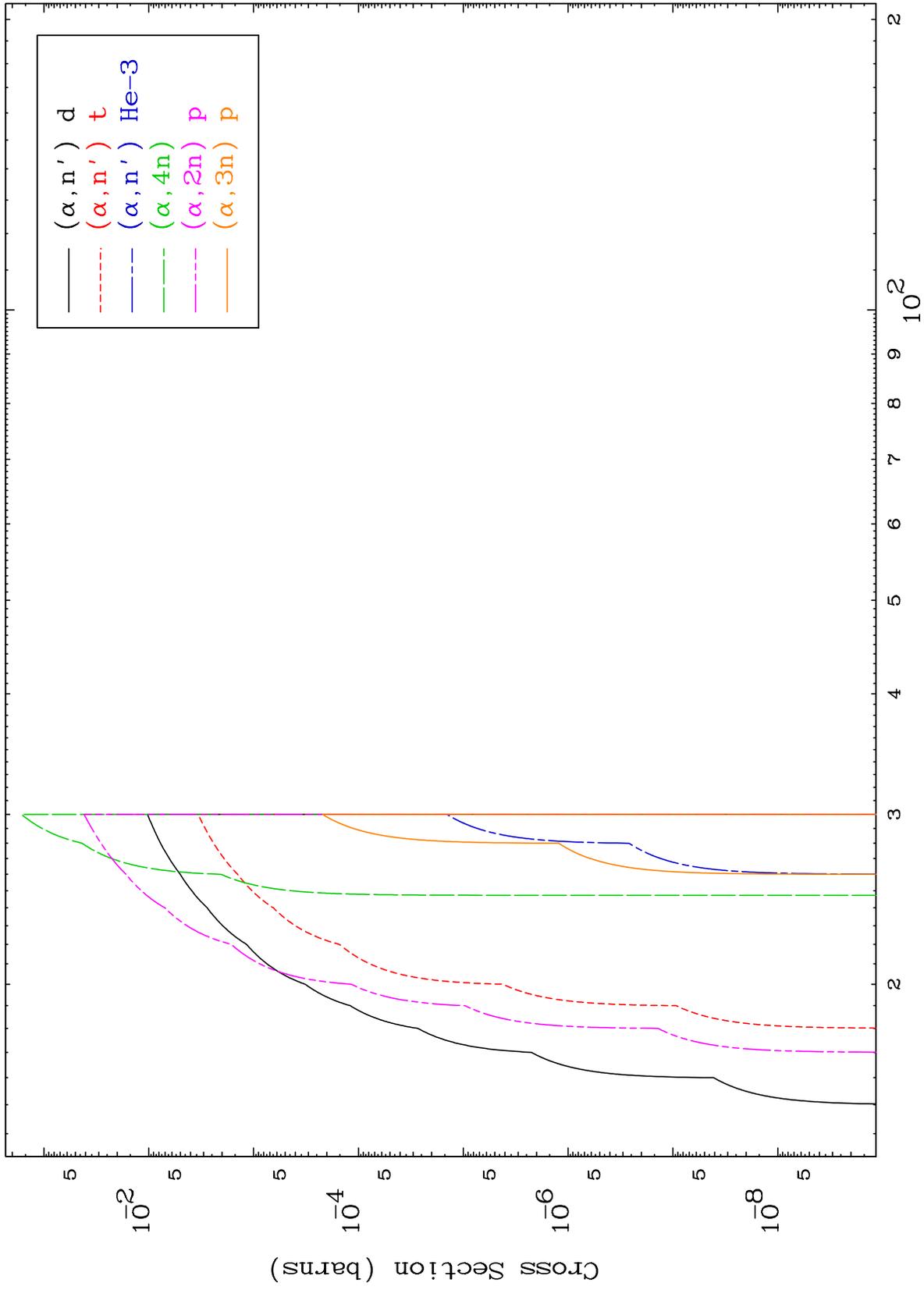
25-Mn-59

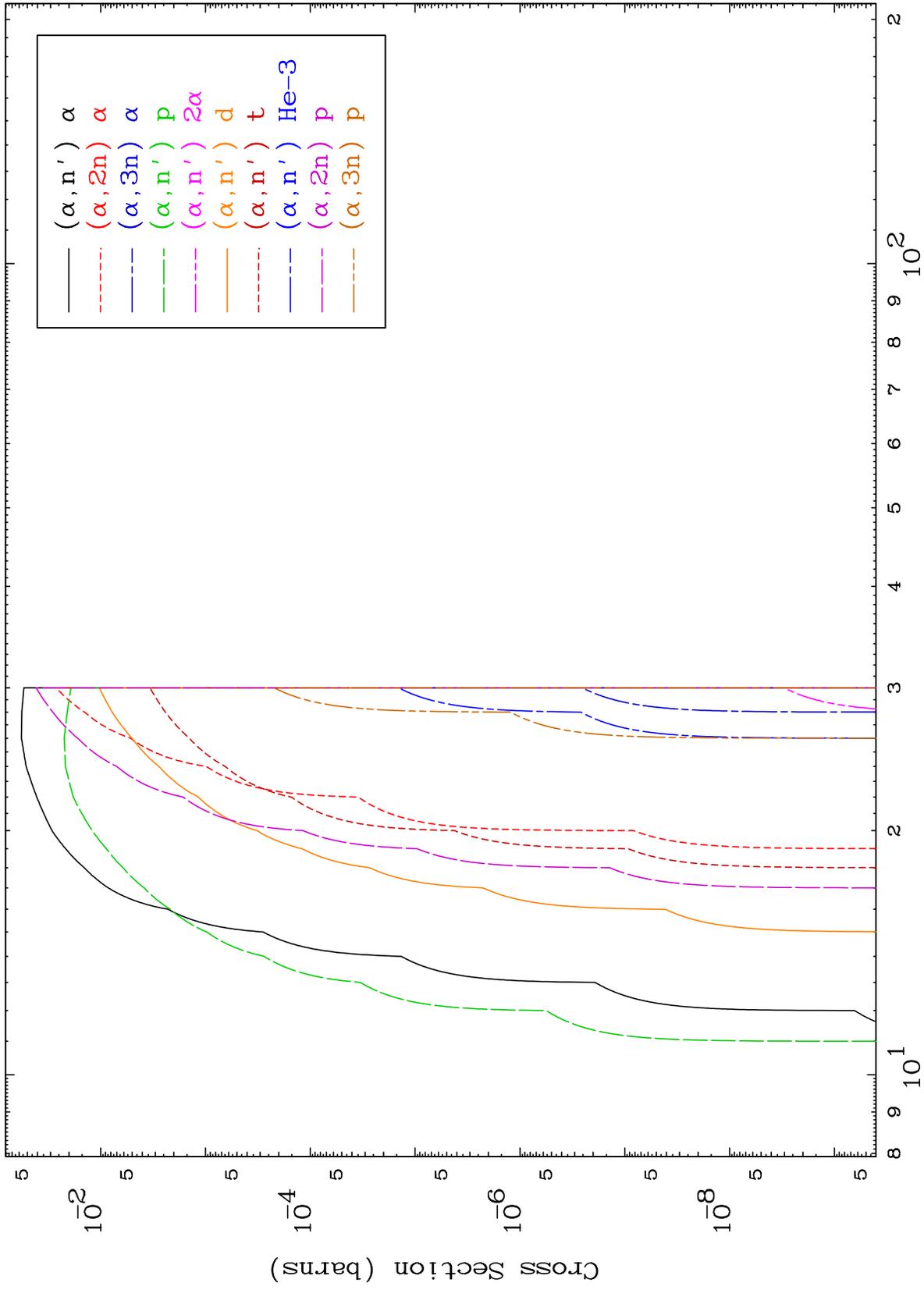


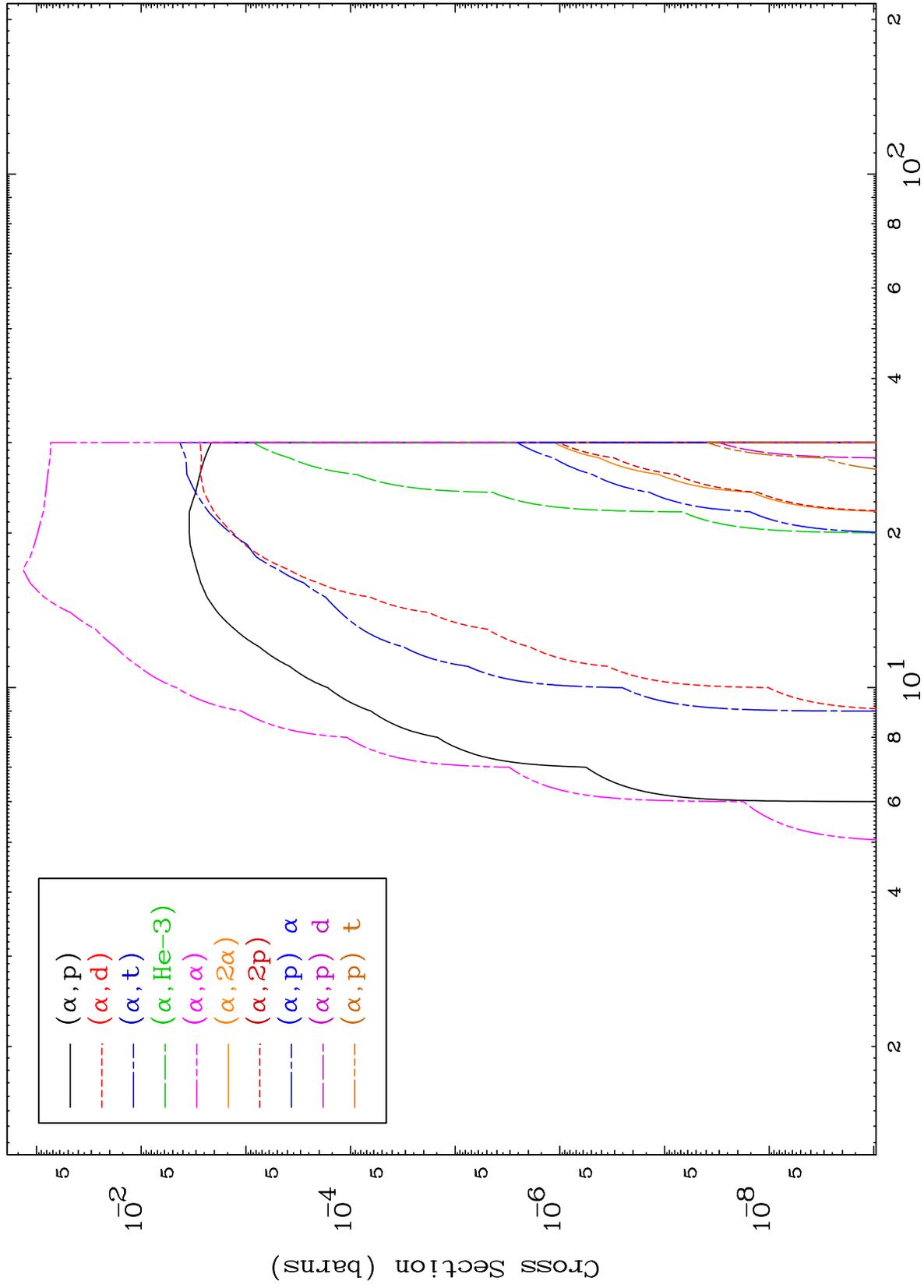
2

Incident Energy (MeV)

25-Mn-59



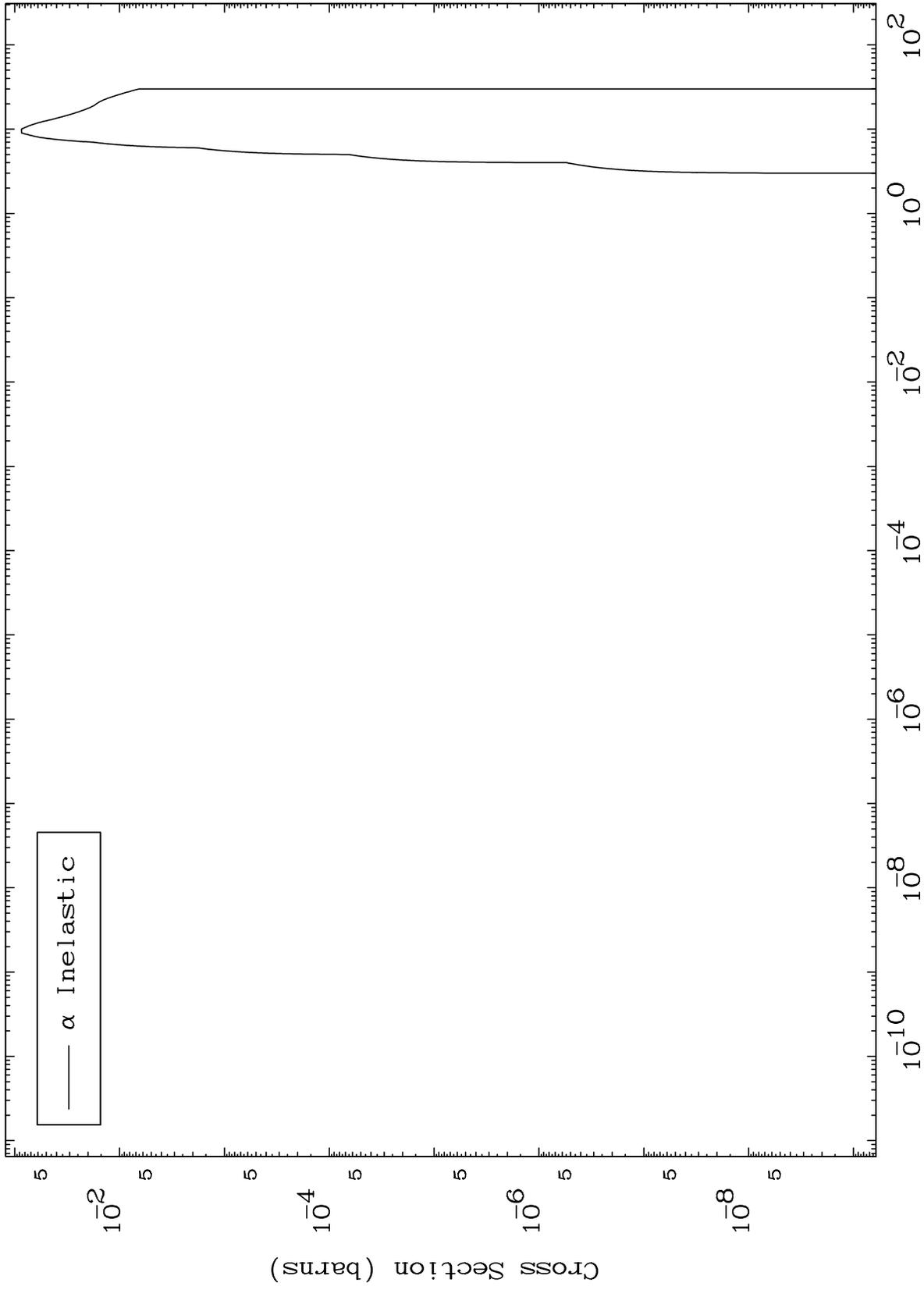




MAT 2537

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

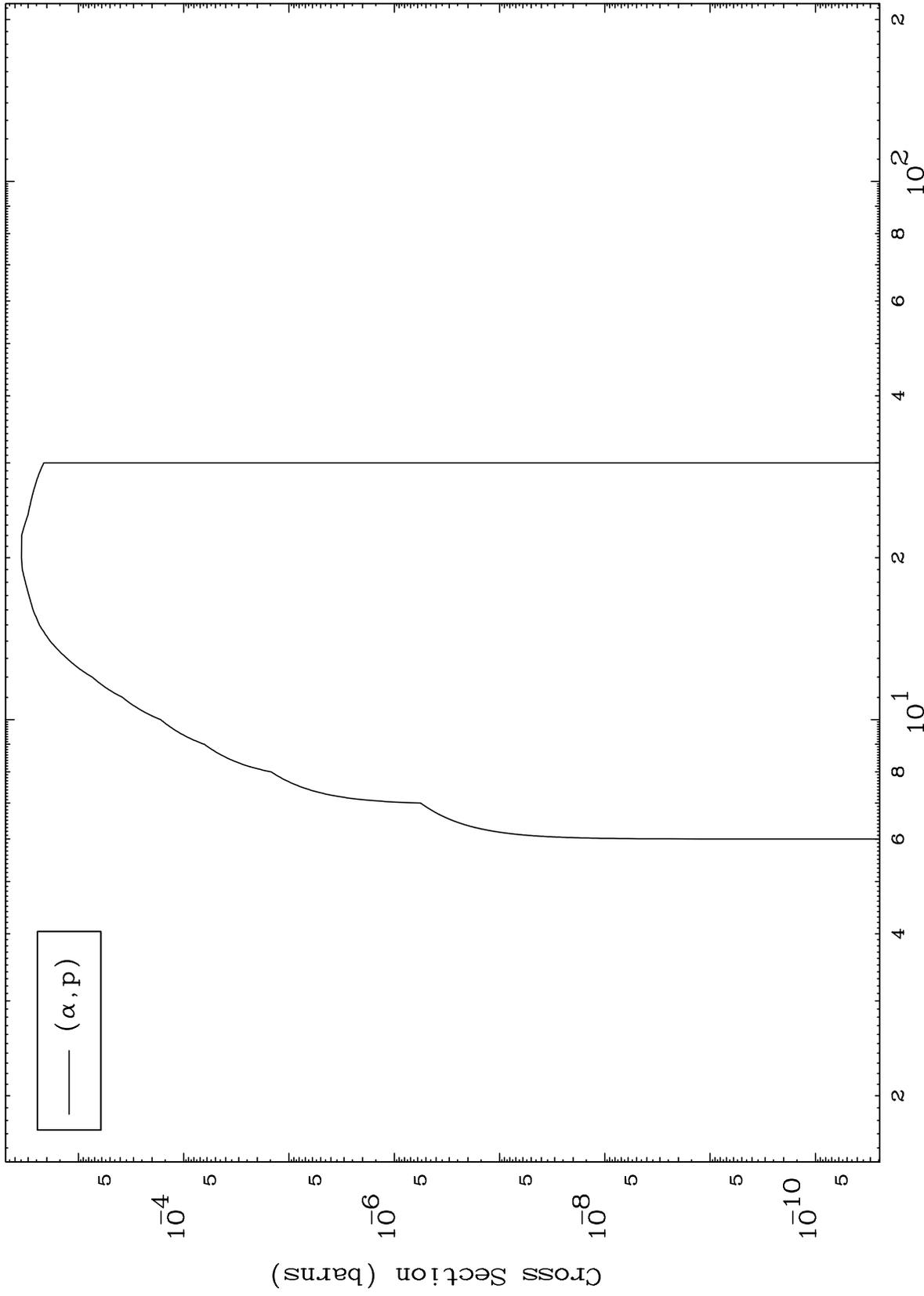
25-Mn-59



6

Incident Energy (MeV)

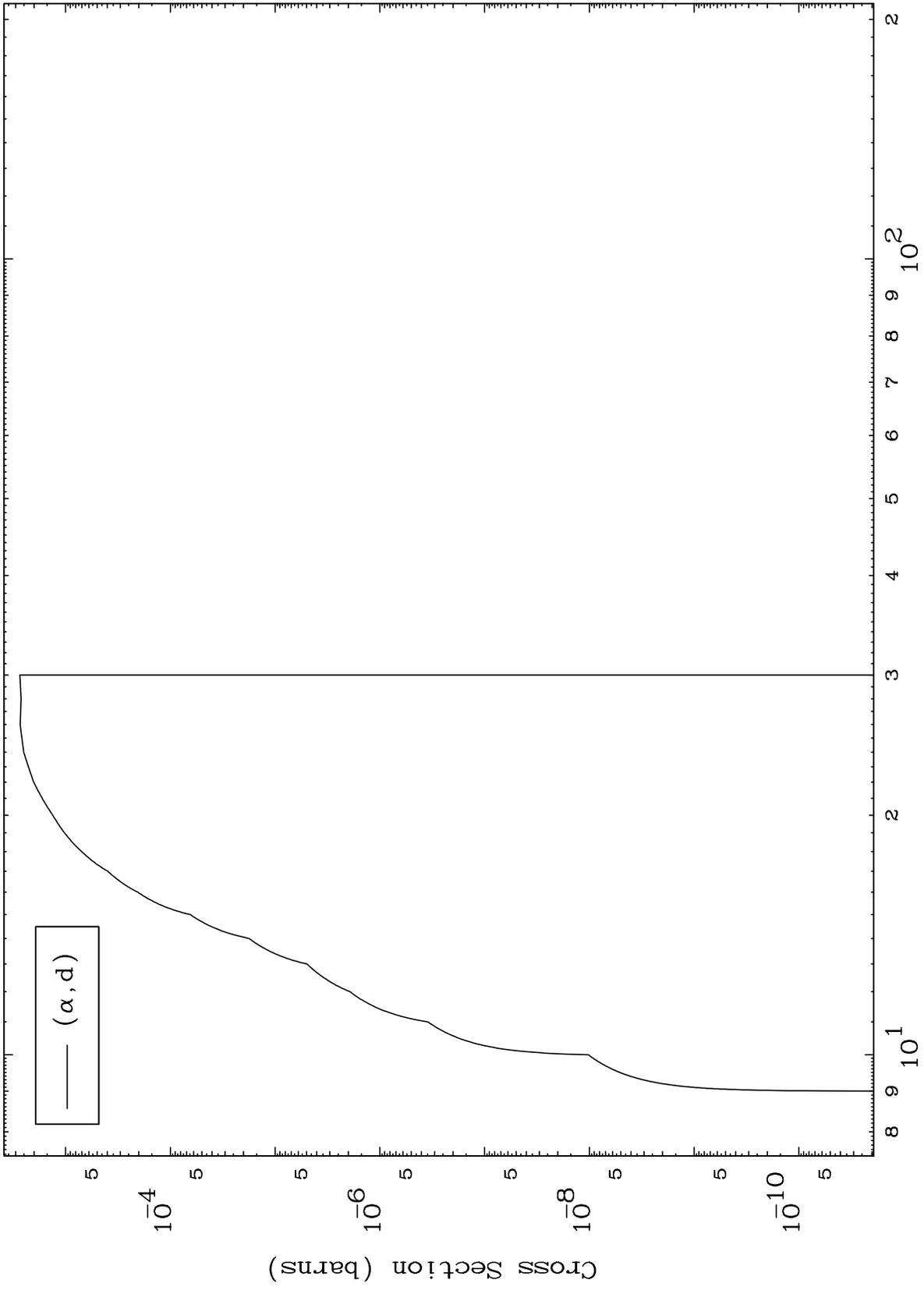
25-Mn-59



MAT 2537

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

25-Mn-59



8

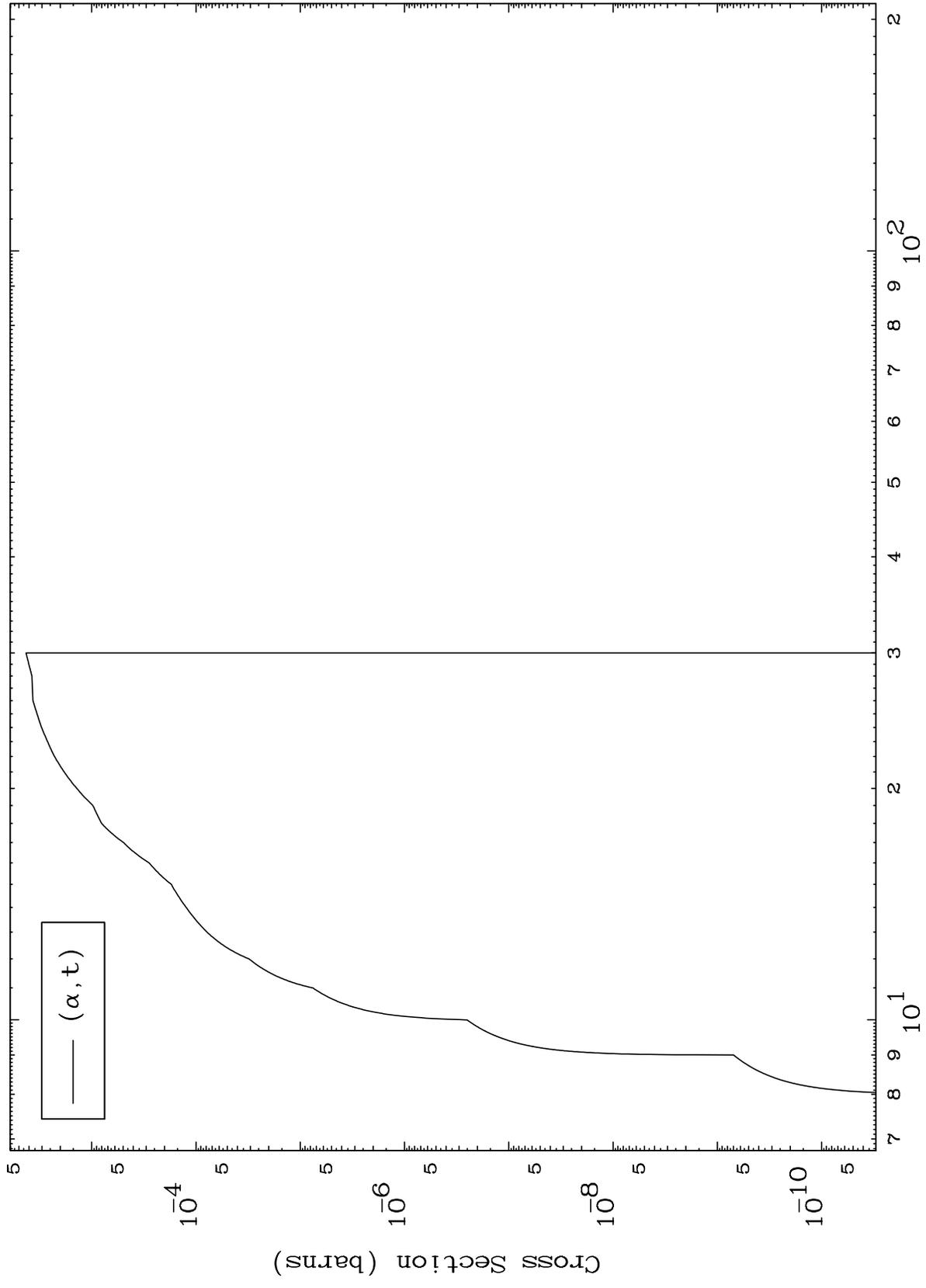
Incident Energy (MeV)

25-Mn-59

MAT 2537

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

25-Mn-59



9

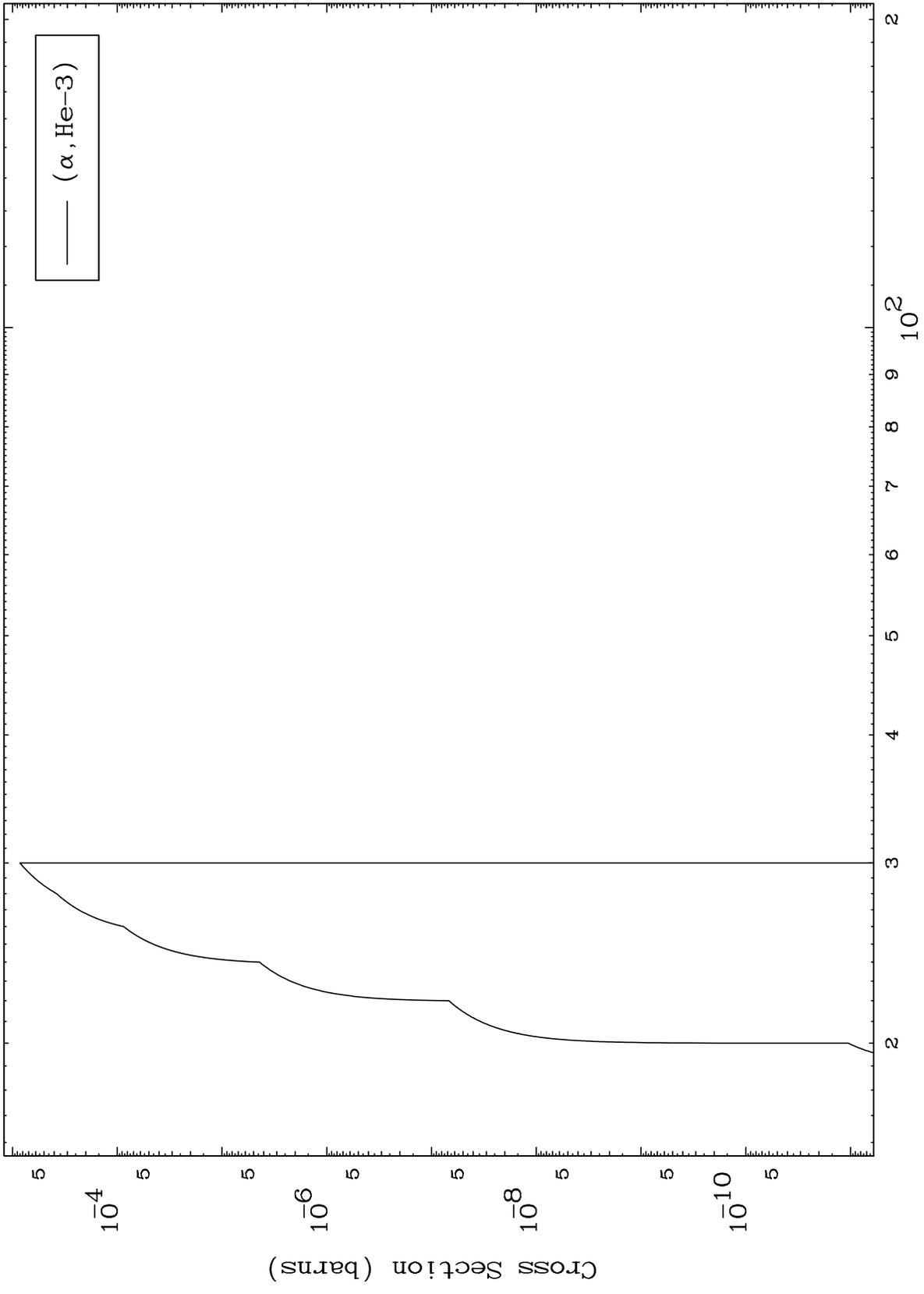
Incident Energy (MeV)

25-Mn-59

MAT 2537

( $\alpha, \text{He-3}$ ) Levels  
0 Kelvin Cross Sections

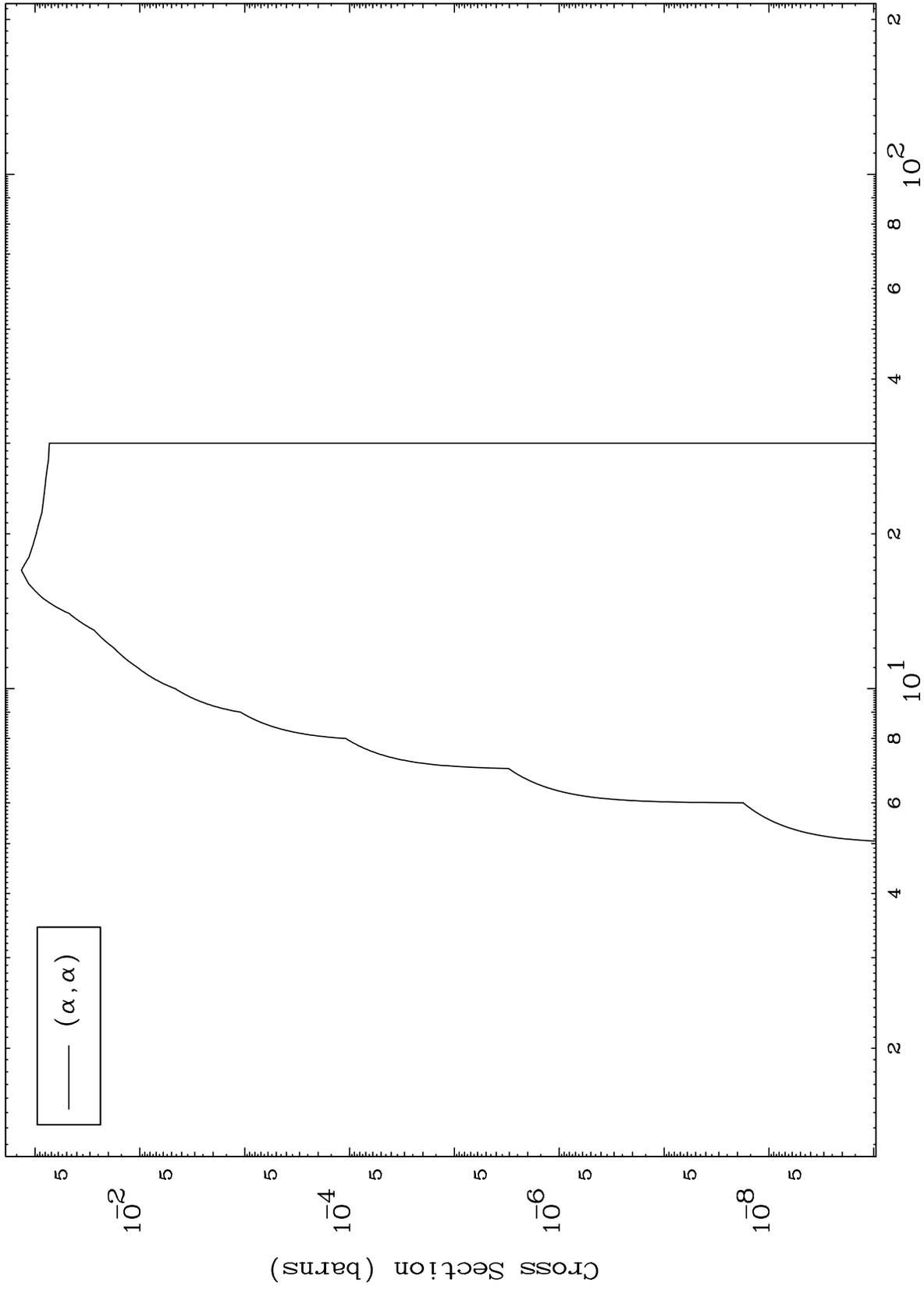
25-Mn-59



10

Incident Energy (MeV)

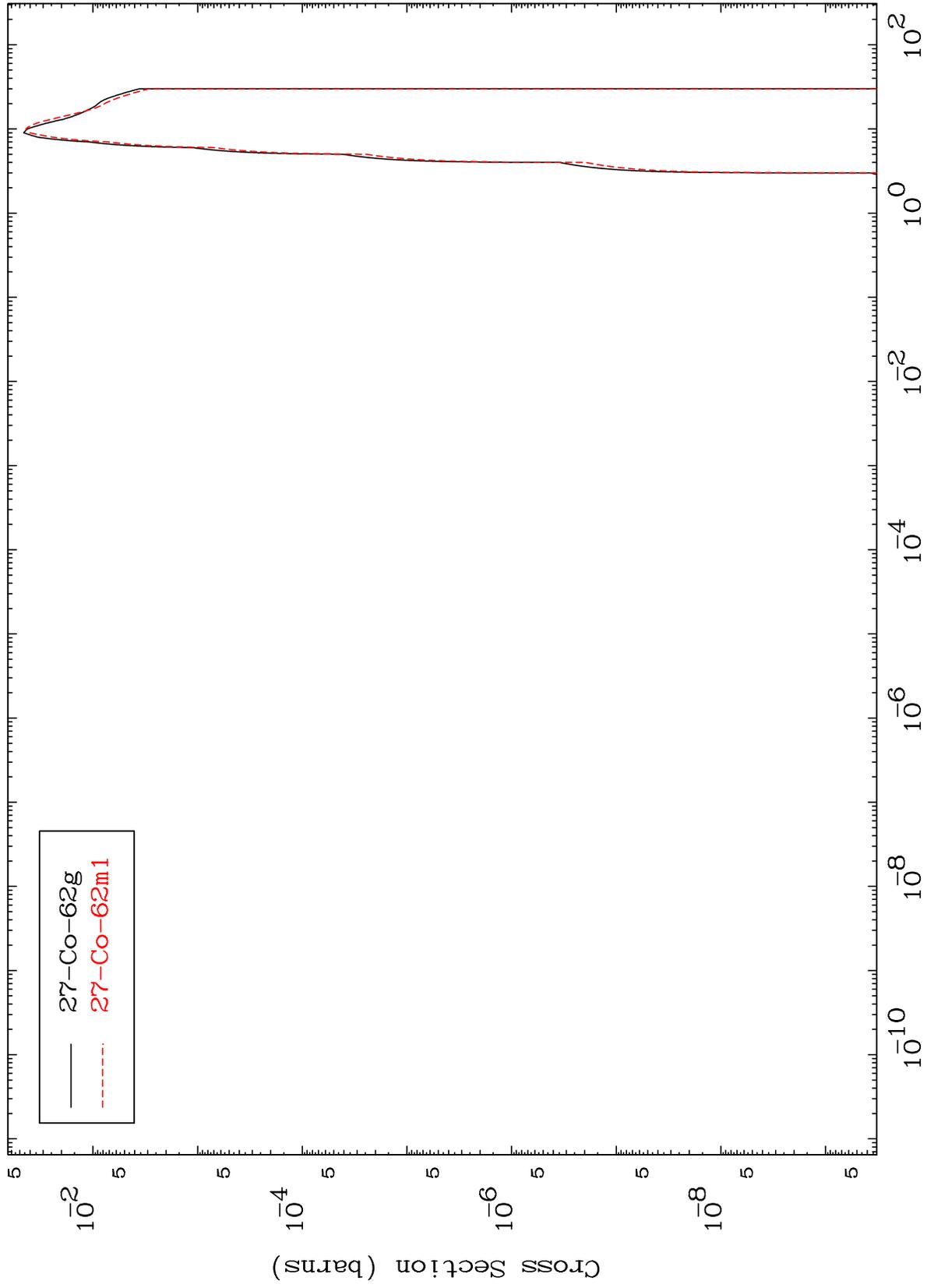
25-Mn-59



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$\alpha$  Inelastic  
Radionuclide Production Cross Section

<sup>25</sup>Mn-59



12

Incident Energy (MeV)

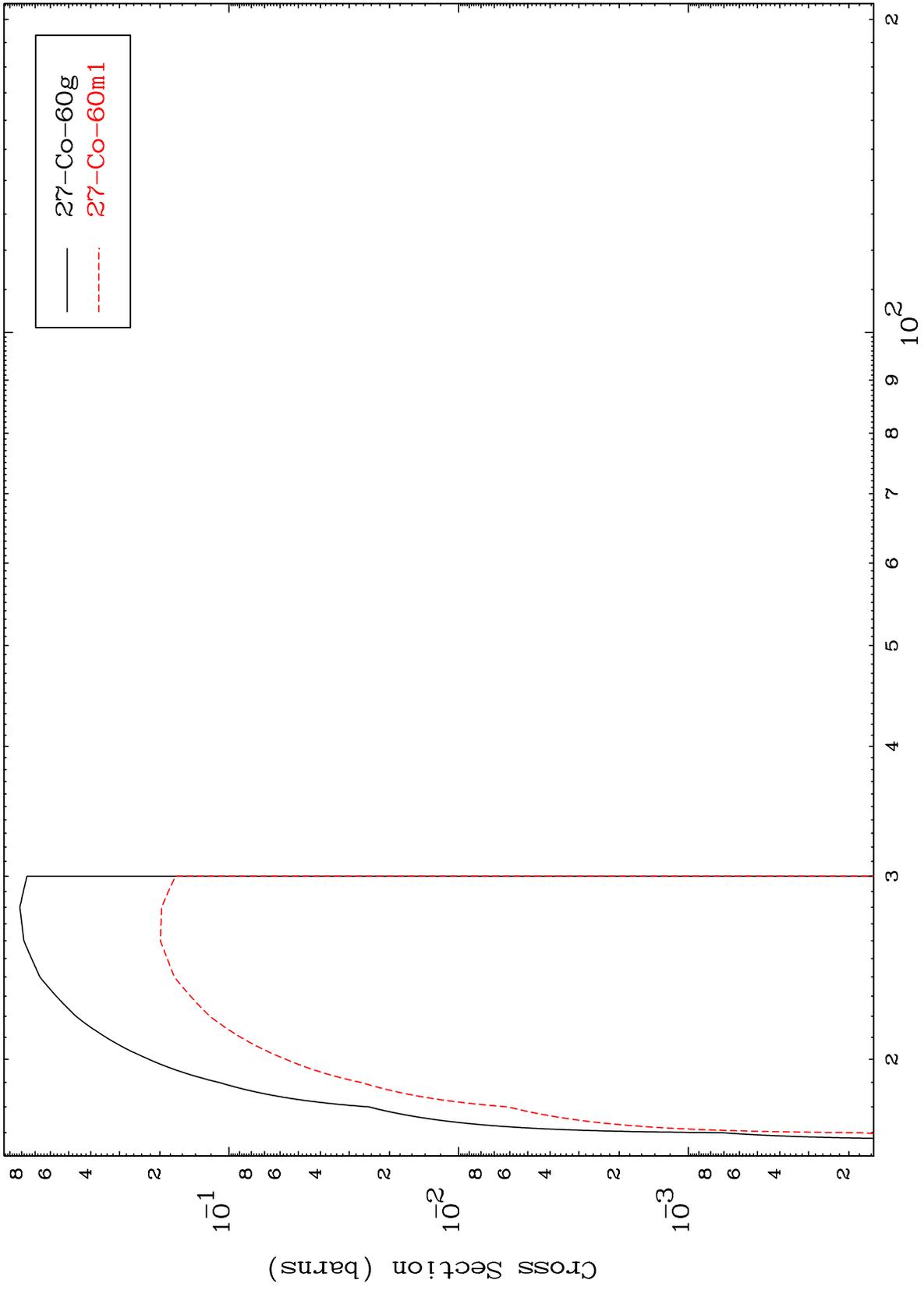
<sup>25</sup>Mn-59

MAT 2537

( $\alpha, 3n$ )

<sup>25</sup>Mn-59

Radionuclide Production Cross Section



13

Incident Energy (MeV)

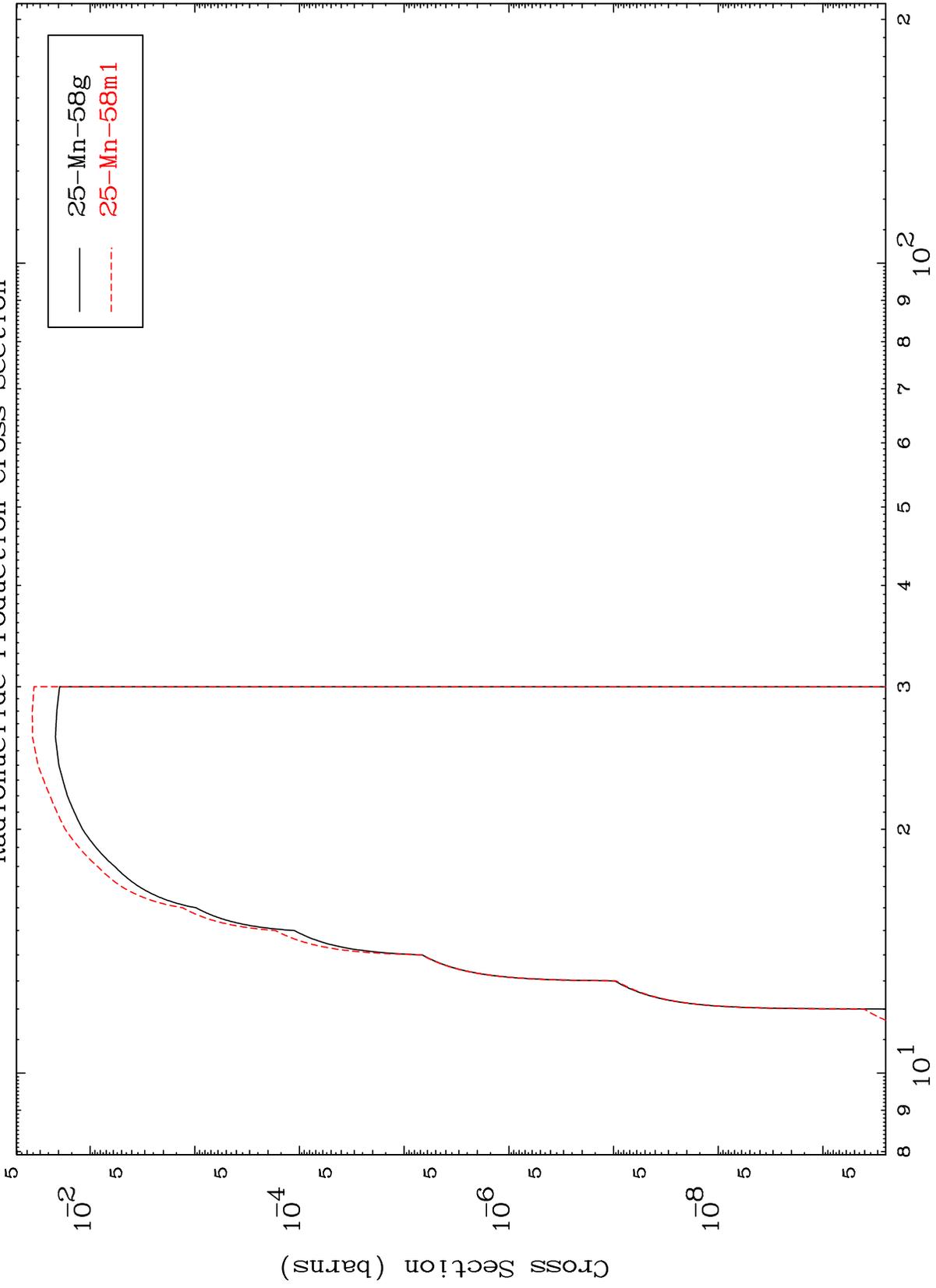
<sup>25</sup>Mn-59

MAT 2537

$(\alpha, n')$   $\alpha$

$^{25}\text{Mn}-59$

Radionuclide Production Cross Section



14

Incident Energy (MeV)

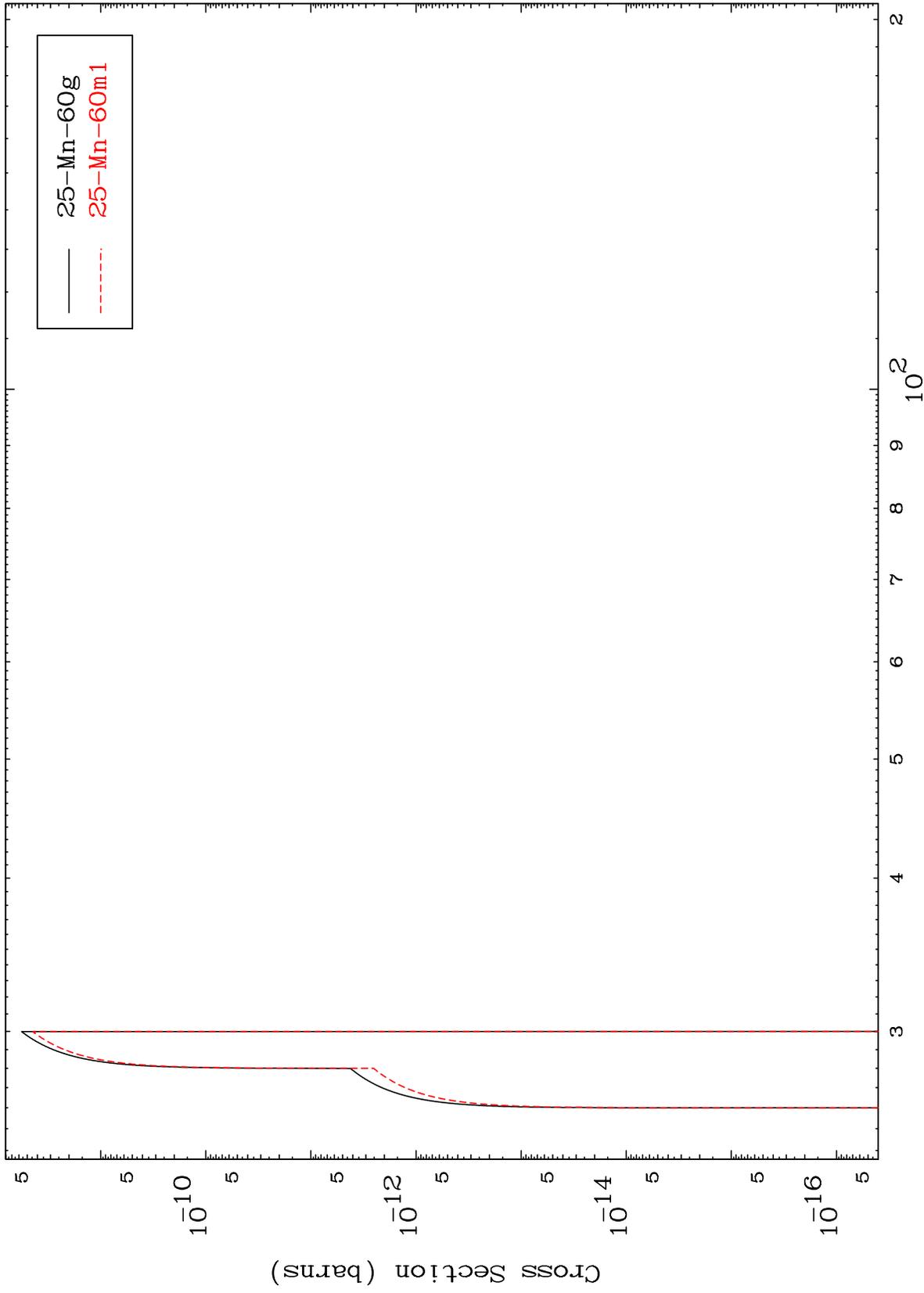
$^{25}\text{Mn}-59$

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( $\alpha, 2n$ ) p

<sup>25</sup>Mn-59

Radionuclide Production Cross Section



15

Incident Energy (MeV)

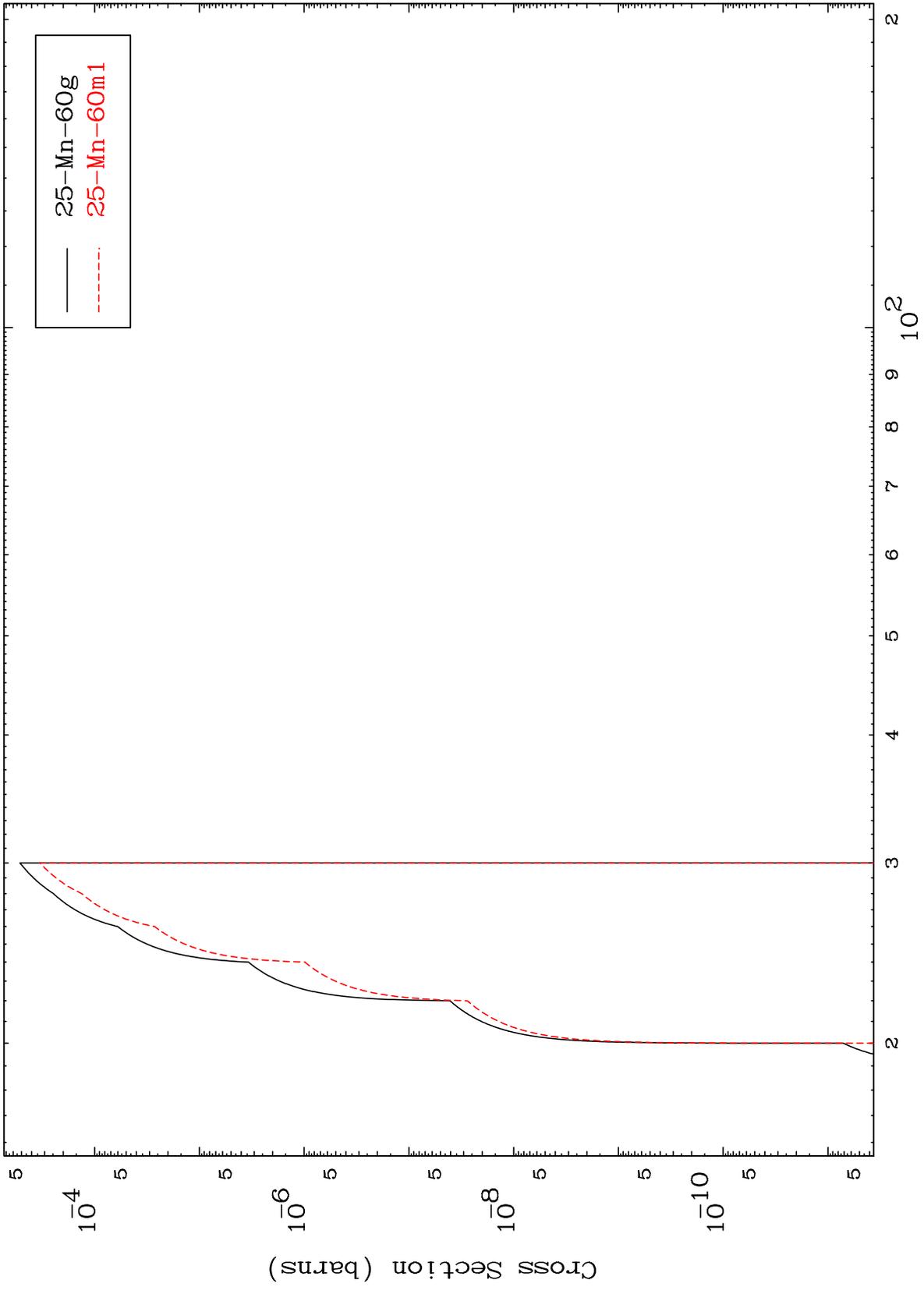
<sup>25</sup>Mn-59

MAT 2537

( $\alpha, \text{He-3}$ )

$^{25}\text{Mn-59}$

Radionuclide Production Cross Section



16

Incident Energy (MeV)

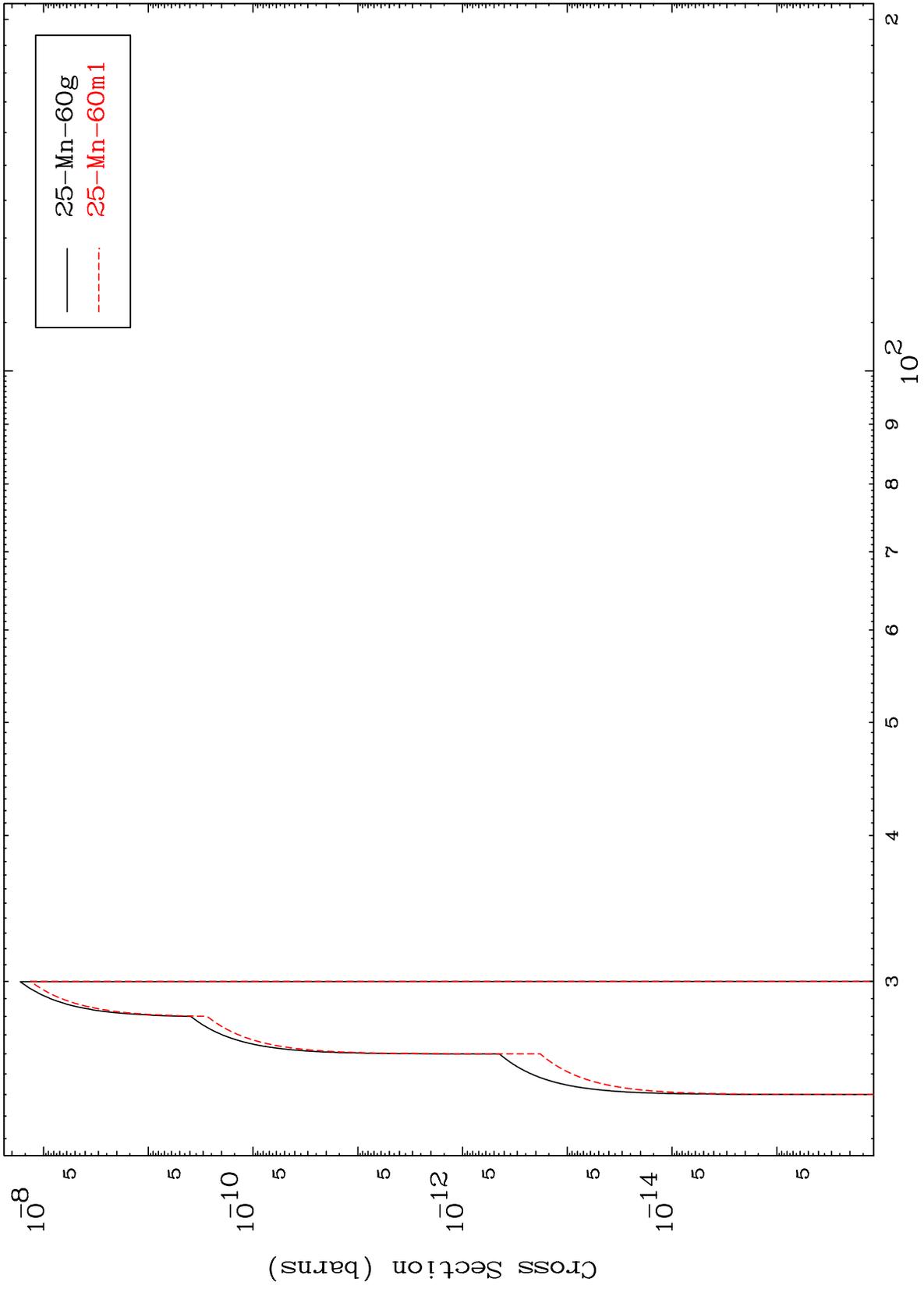
$^{25}\text{Mn-59}$

MAT 2537

( $\alpha, p$ ) d

<sup>25</sup>Mn-59

Radionuclide Production Cross Section



17

Incident Energy (MeV)

<sup>25</sup>Mn-59