

Program EVALPLOT  
(Version 2018-1)

by

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(Present Contact Information)

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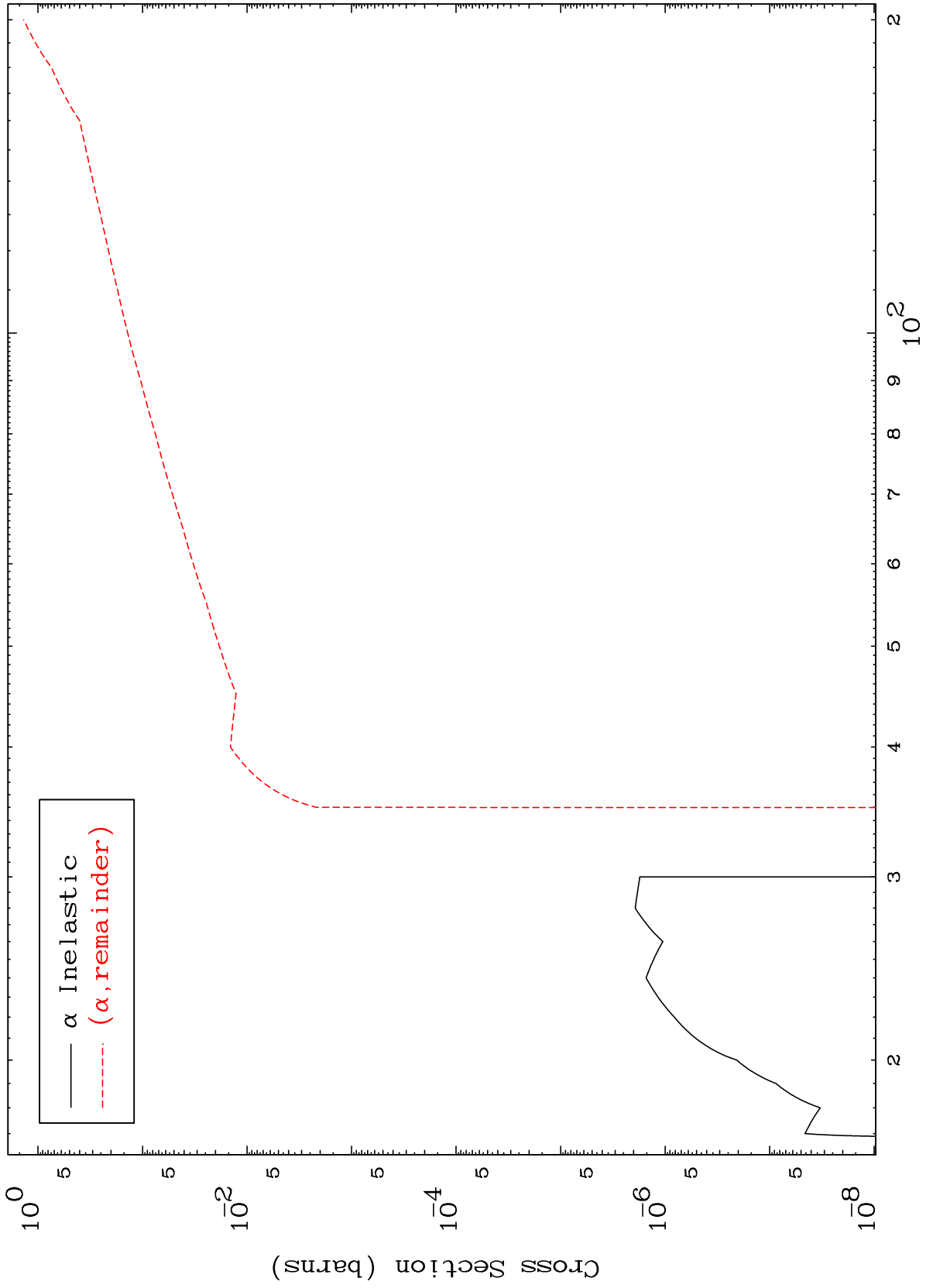
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 977

0 Kelvin  $\alpha$  Major  
Cross Sections

109-Mt-277



1

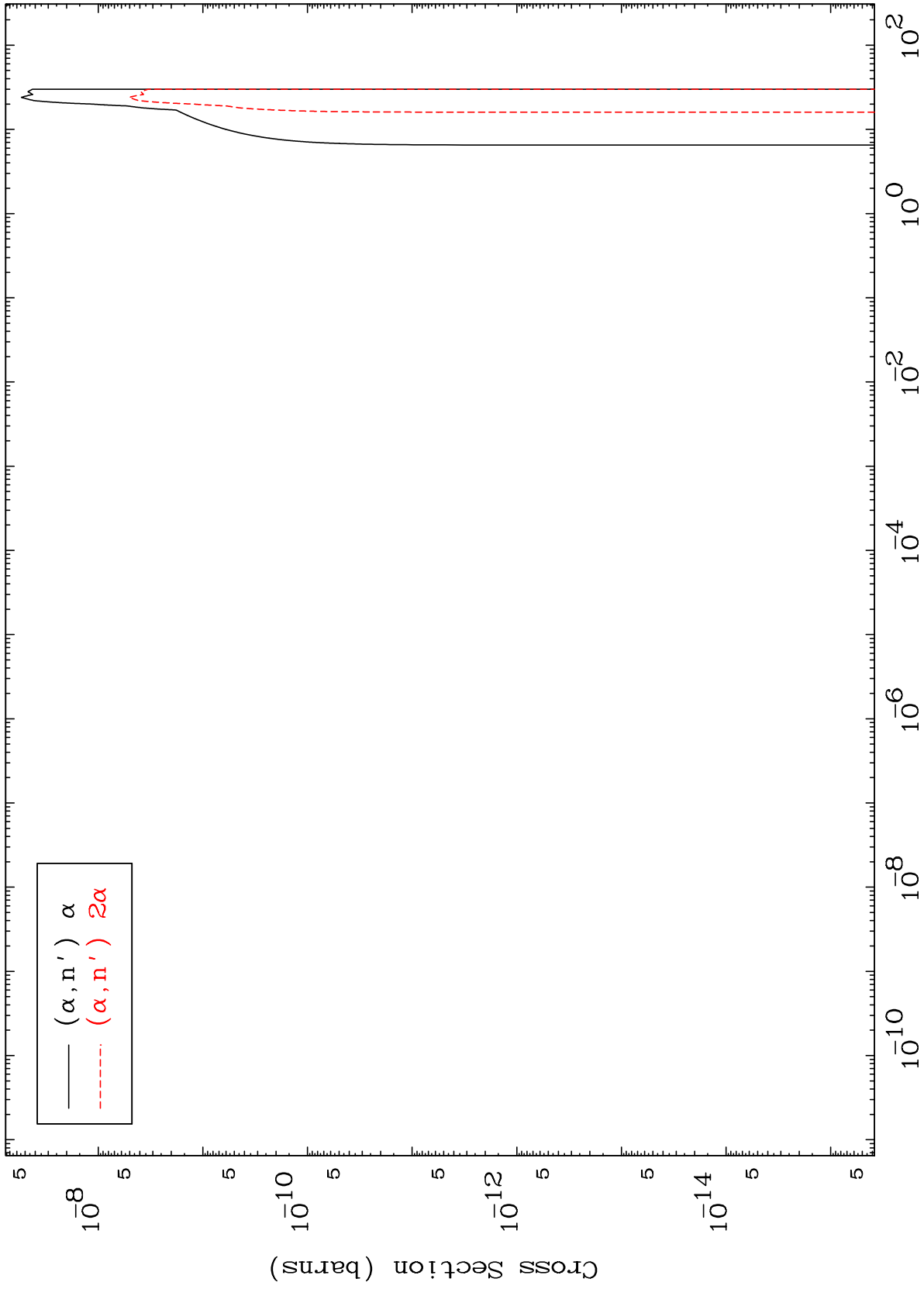
Incident Energy (MeV)

109-Mt-277

MAT 977

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

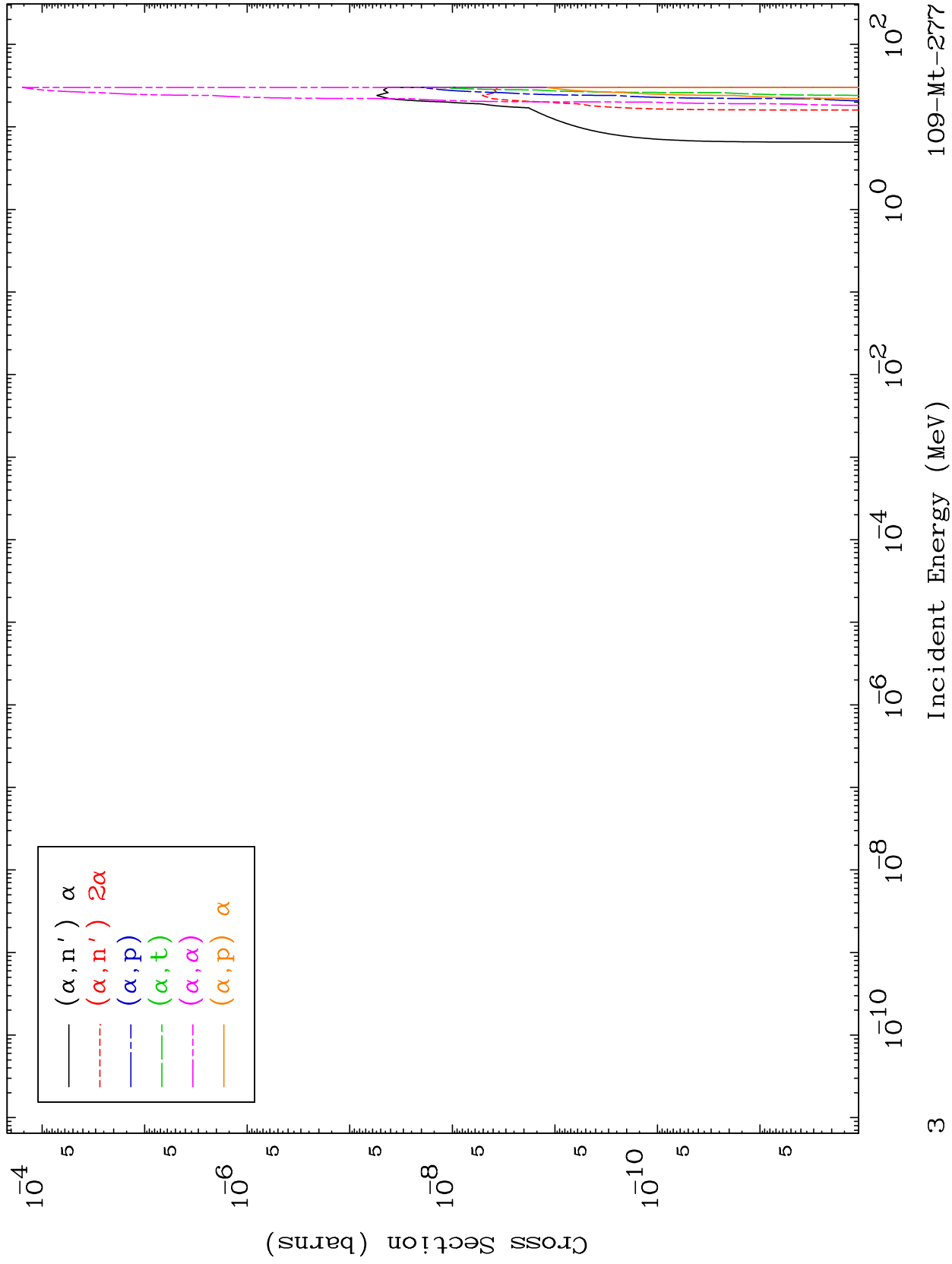
109-Mt-277



2

Incident Energy (MeV)

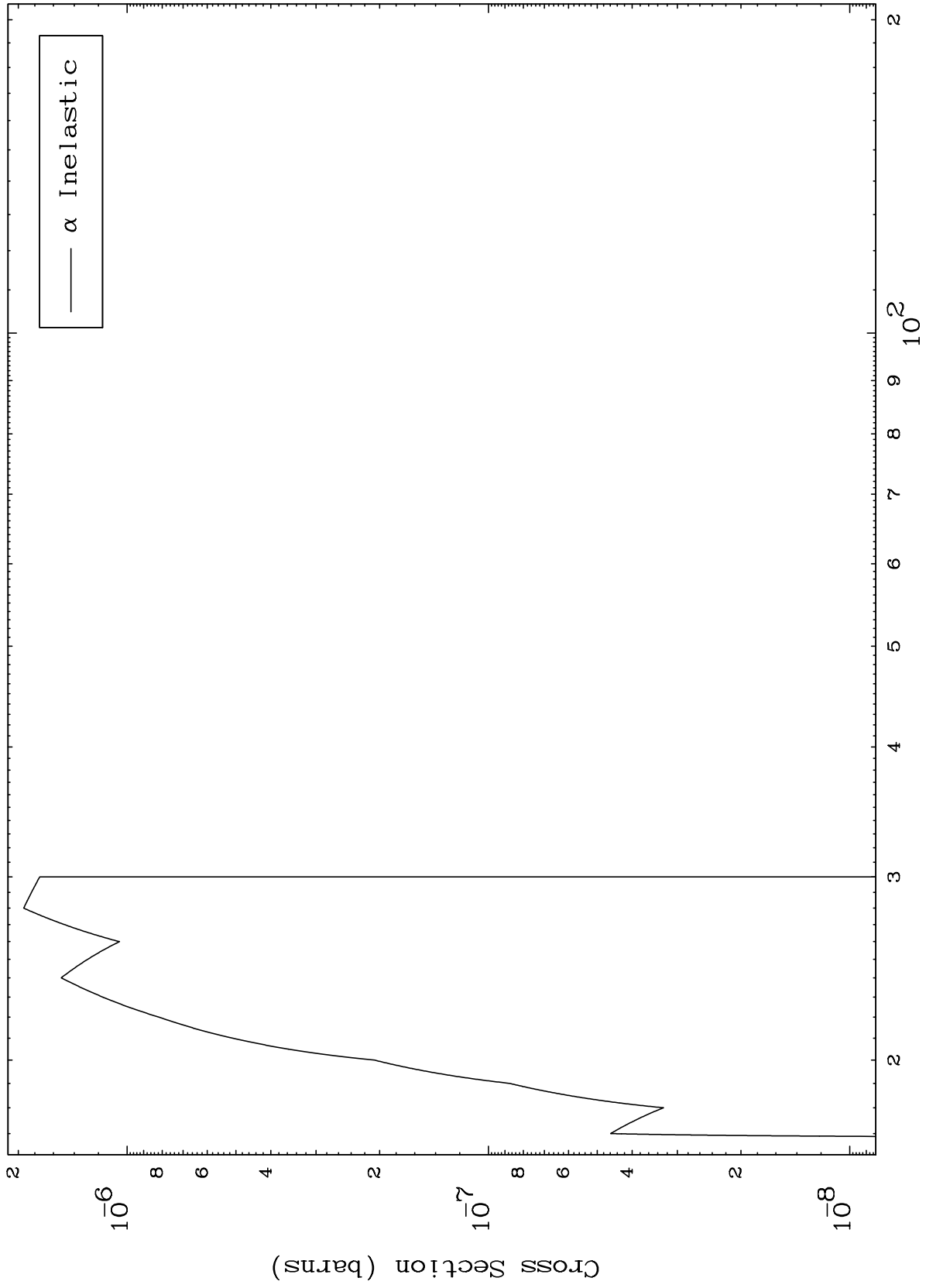
109-Mt-277



MAT 977

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

109-Mt-277



4

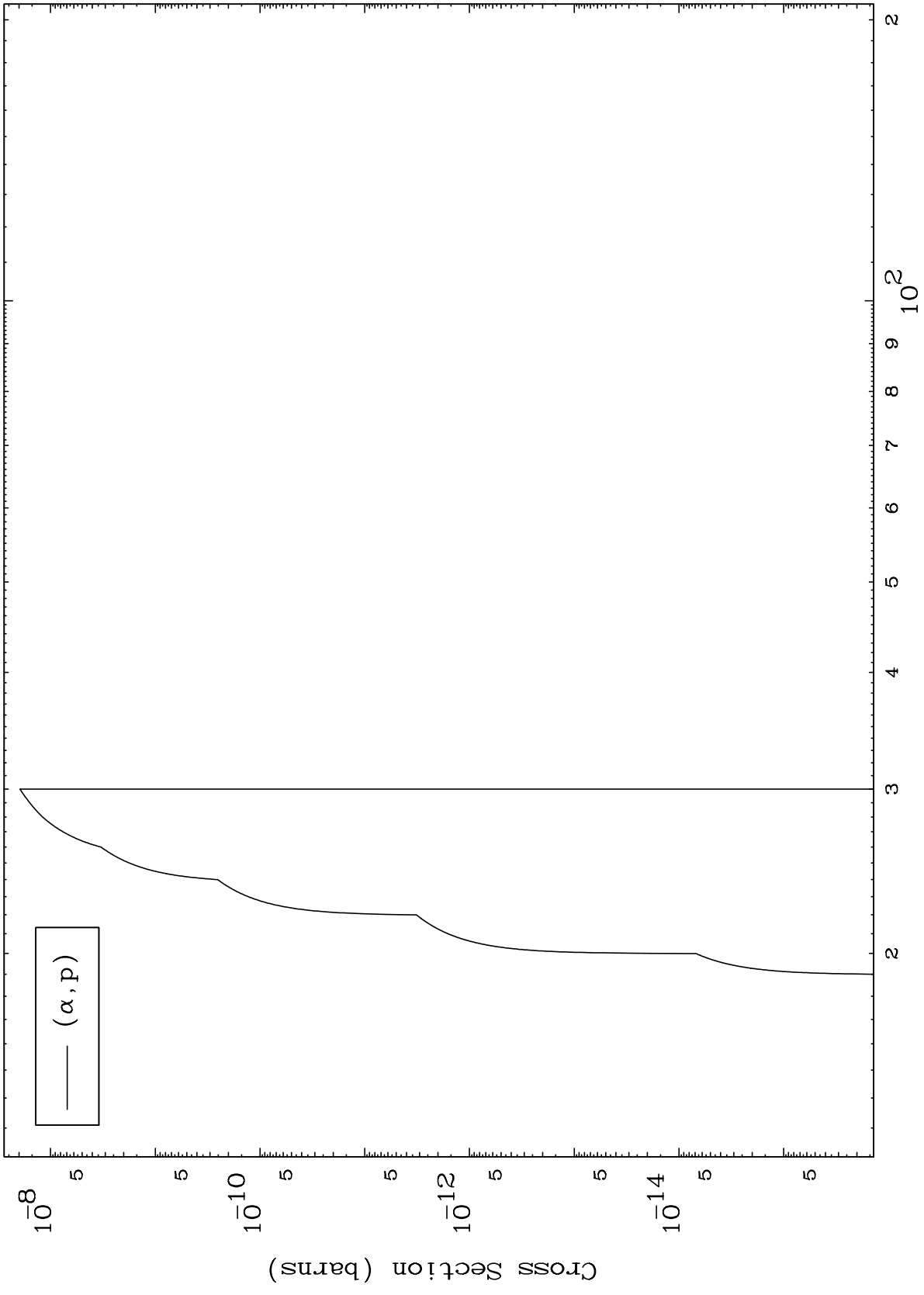
Incident Energy (MeV)

109-Mt-277

MAT 977

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

109-Mt-277



5

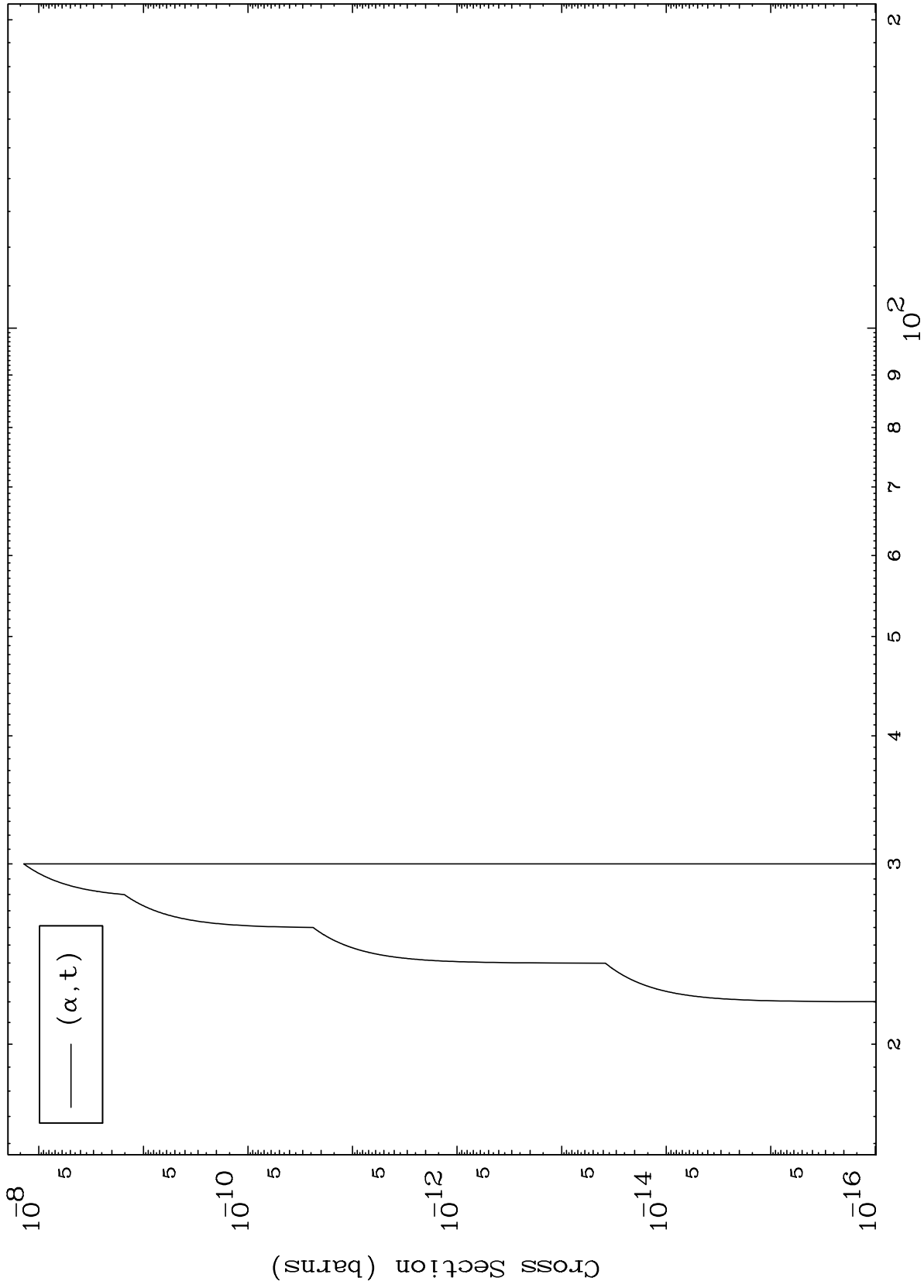
Incident Energy (MeV)

109-Mt-277

MAT 977

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

109-Mt-277



6

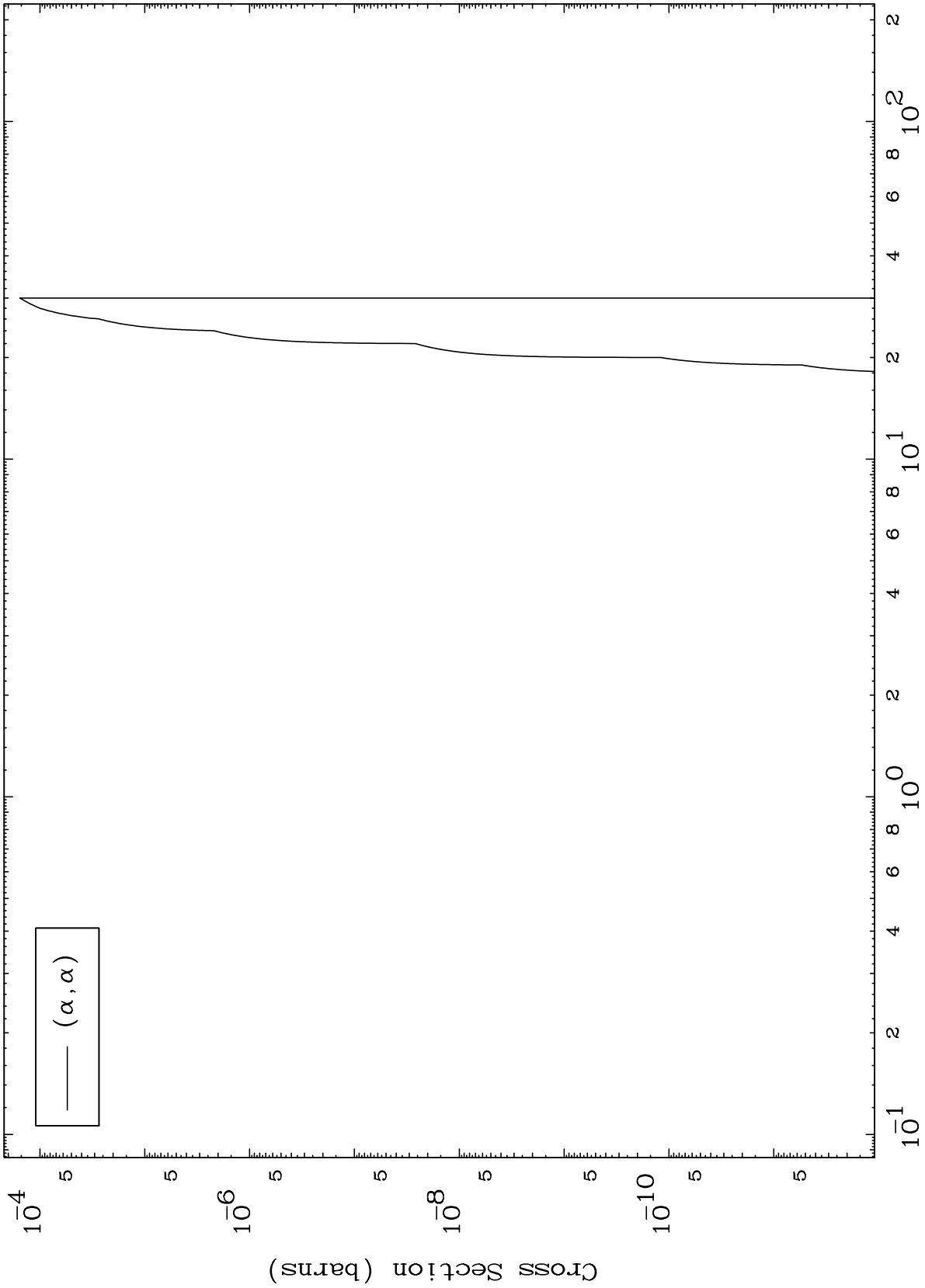
Incident Energy (MeV)

109-Mt-277

MAT 977

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

109-Mt-277



109-Mt-277

Incident Energy (MeV)

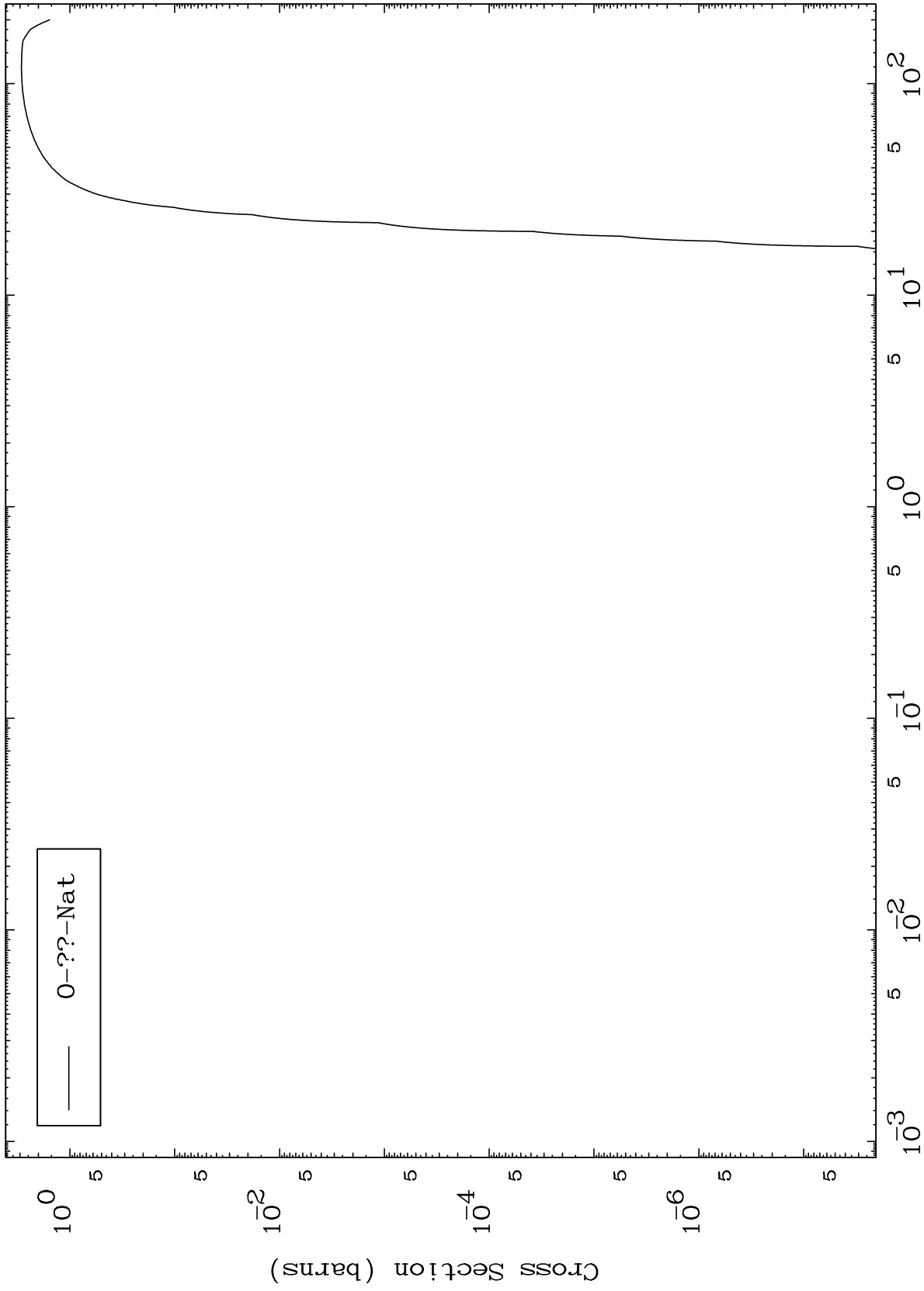
7



MAT 977

109-Mt-277

$\alpha$  Fission  
Radionuclide Production Cross Section



Incident Energy (MeV)

109-Mt-277

8