

Program EVALPLOT
(Version 2018-1)

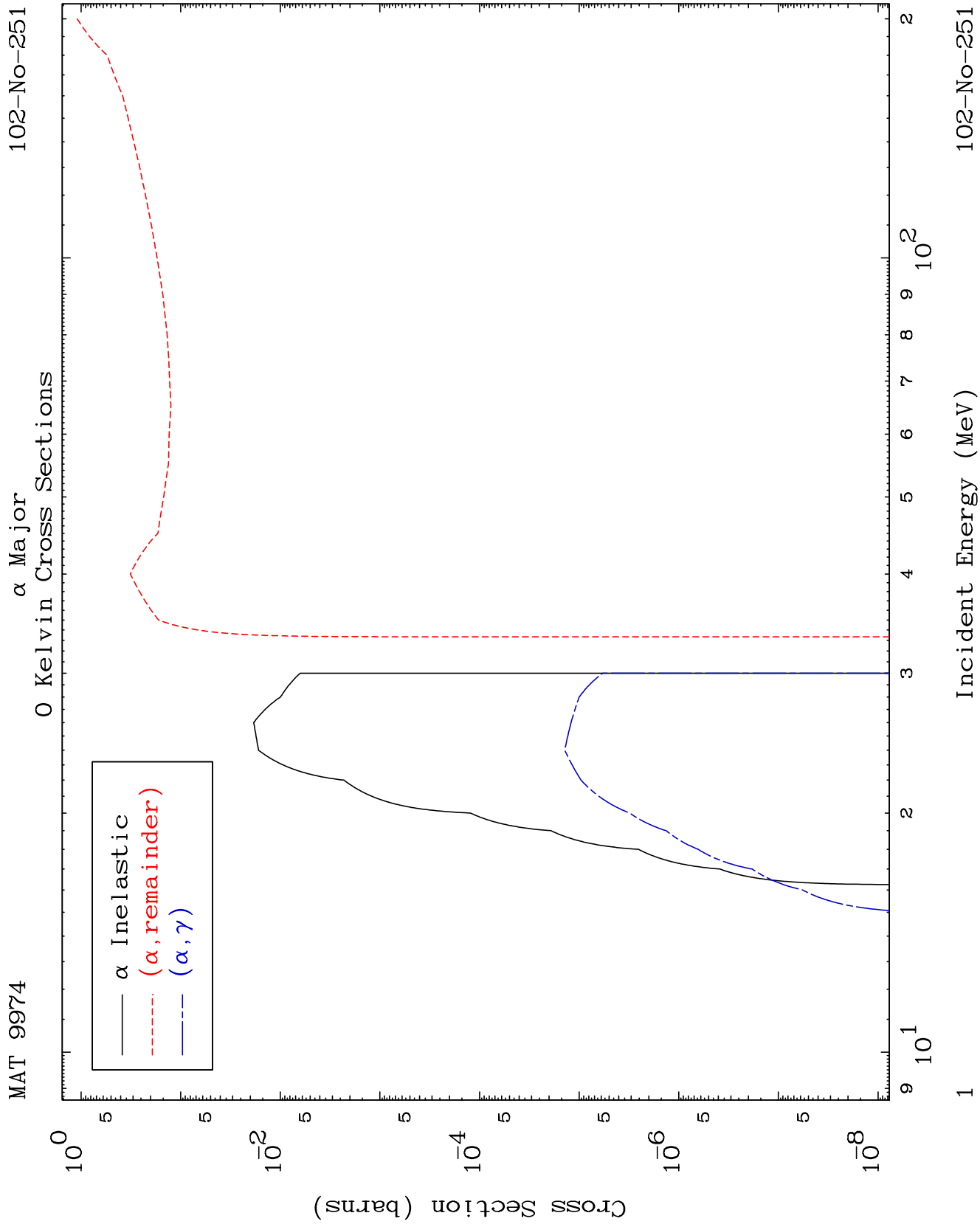
by

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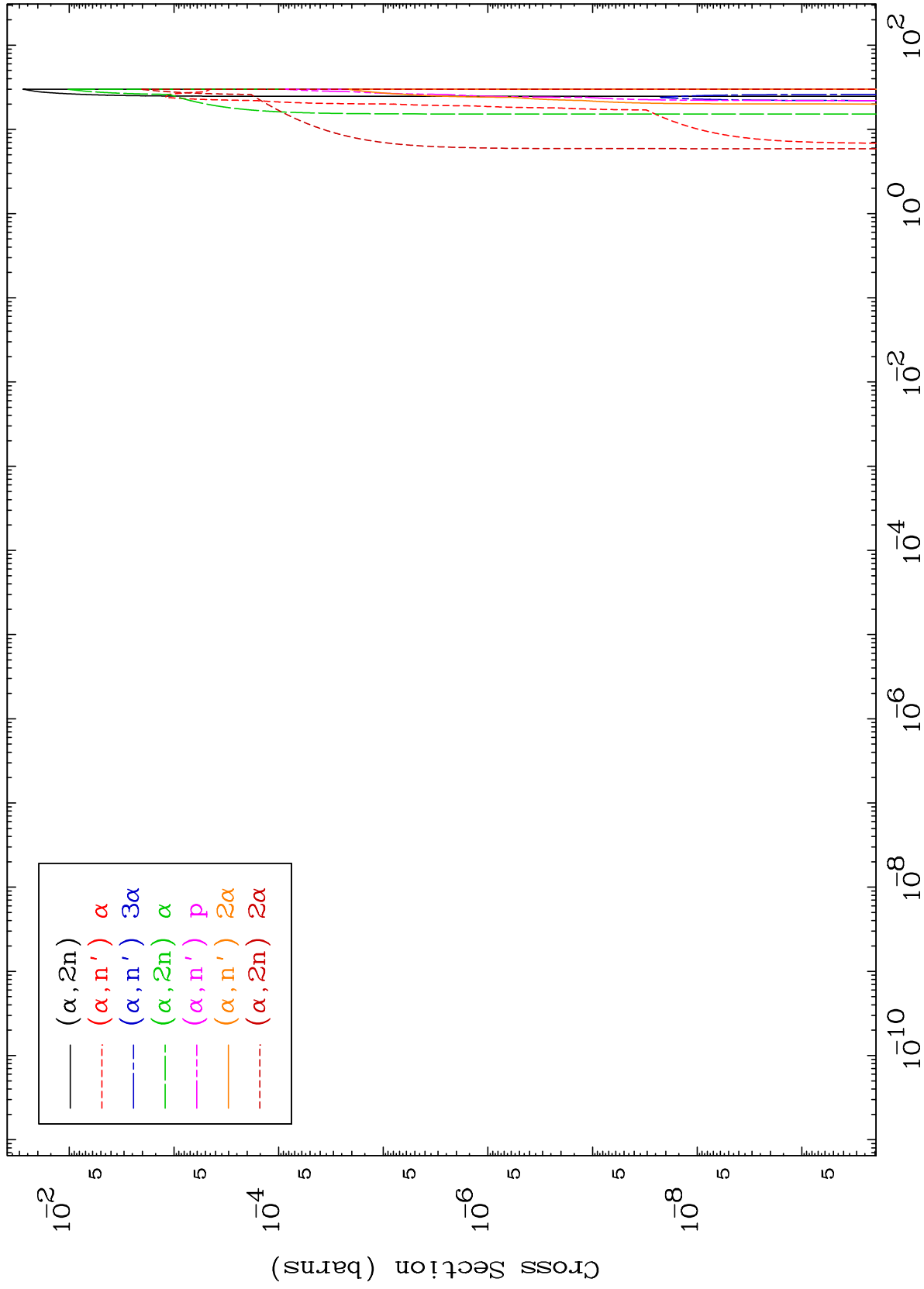
Press Mouse Button to Start



MAT 9974

α Neutron Production
0 Kelvin Cross Sections

102-No-251



2

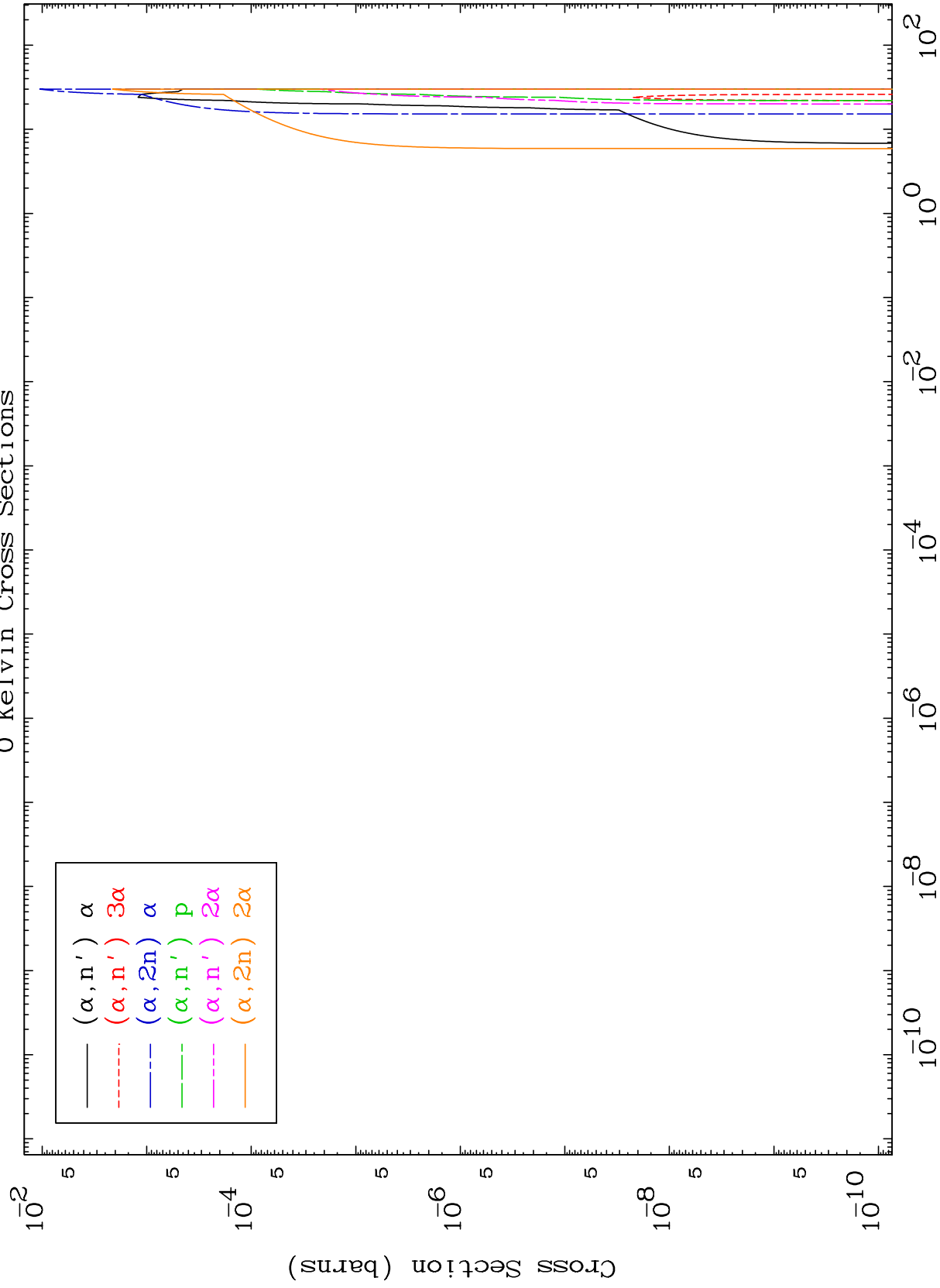
Incident Energy (MeV)

102-No-251

MAT 9974

α Charged Particle
0 Kelvin Cross Sections

102-No-251



3

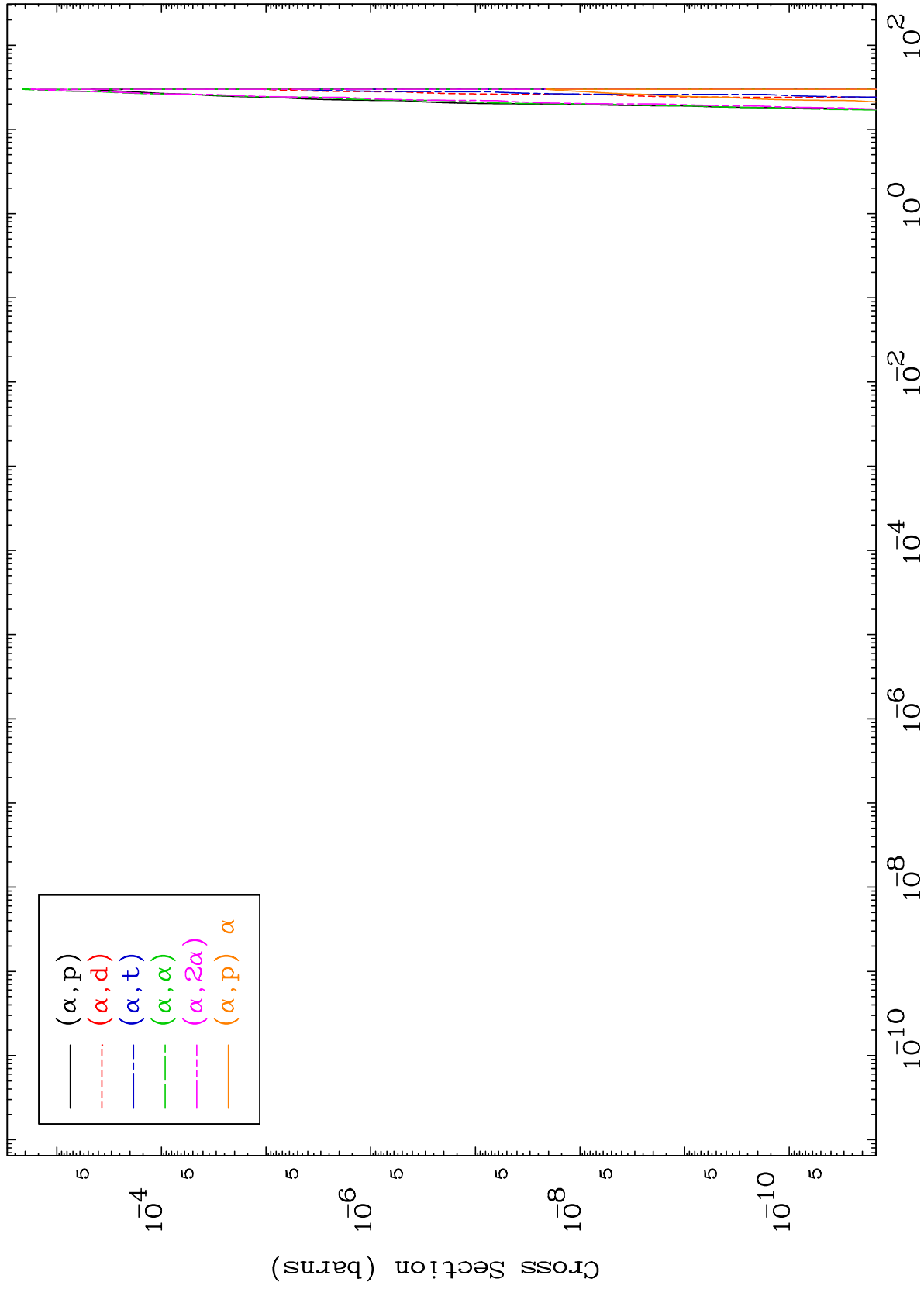
Incident Energy (MeV)

102-No-251

MAT 9974

α Charged Particle
0 Kelvin Cross Sections

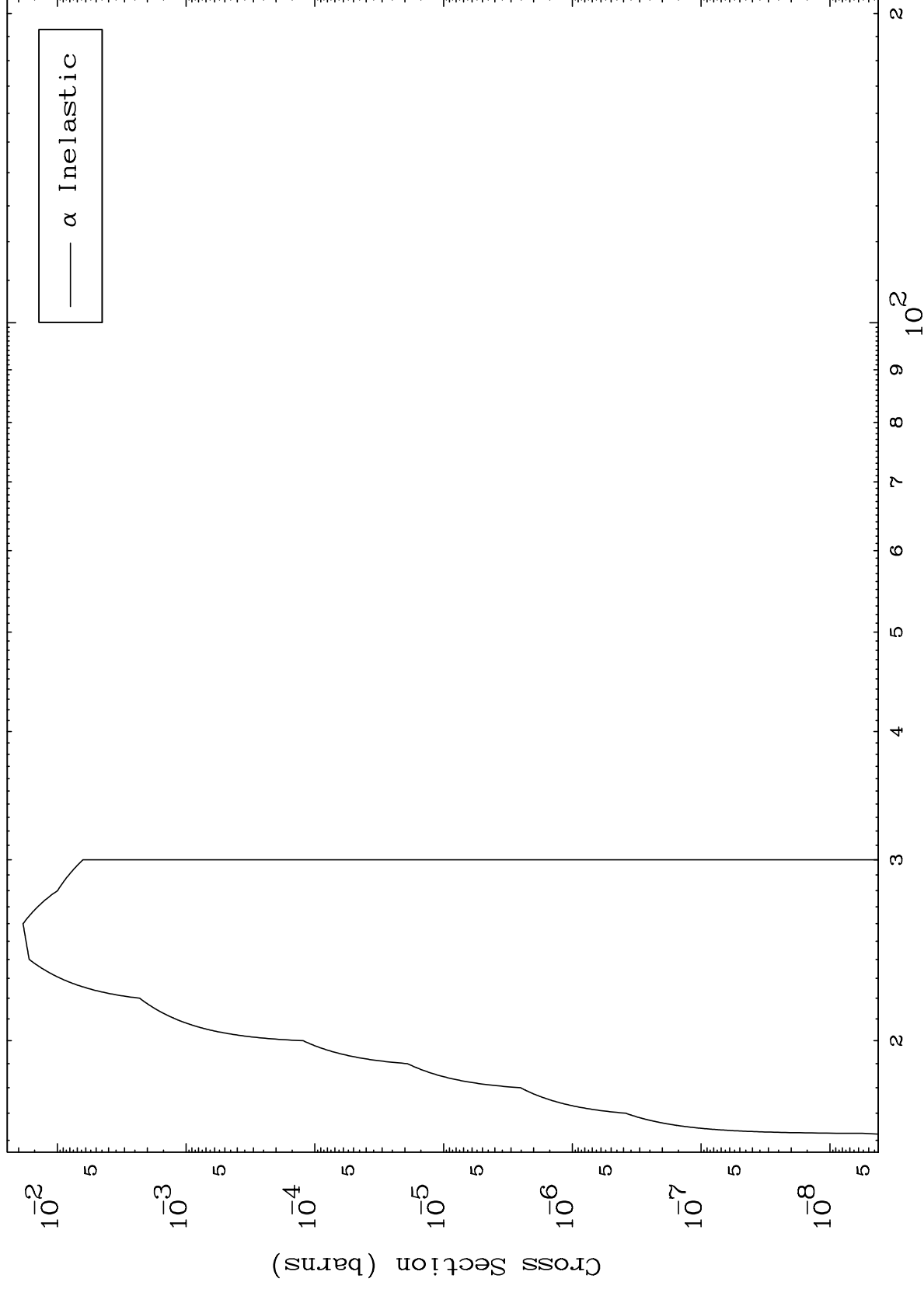
102-No-251



MAT 9974

(α, n') Level
0 Kelvin Cross Sections

102-No-251



5

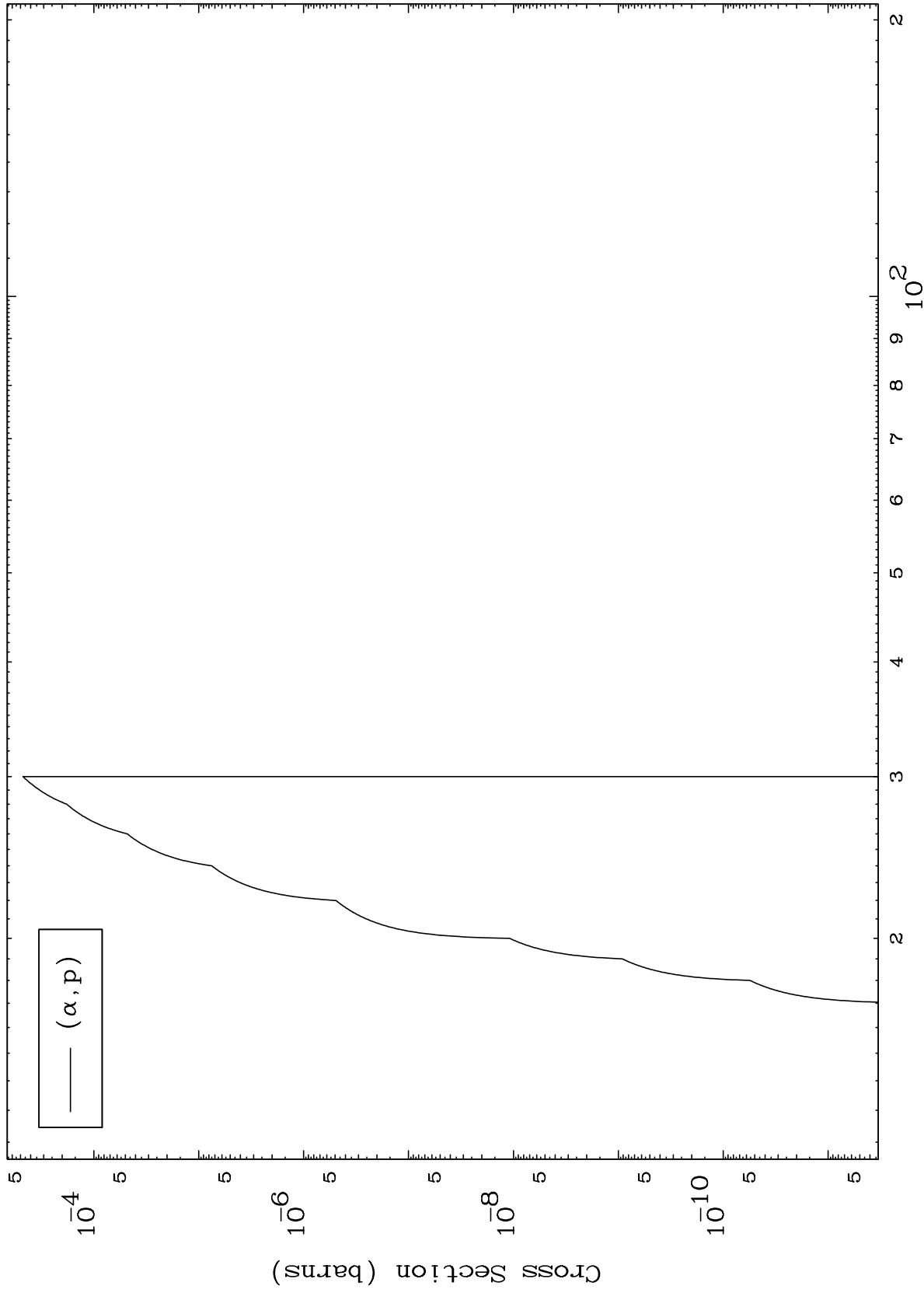
Incident Energy (MeV)

102-No-251

MAT 9974

(α ,p) Levels
0 Kelvin Cross Sections

102-No-251



6

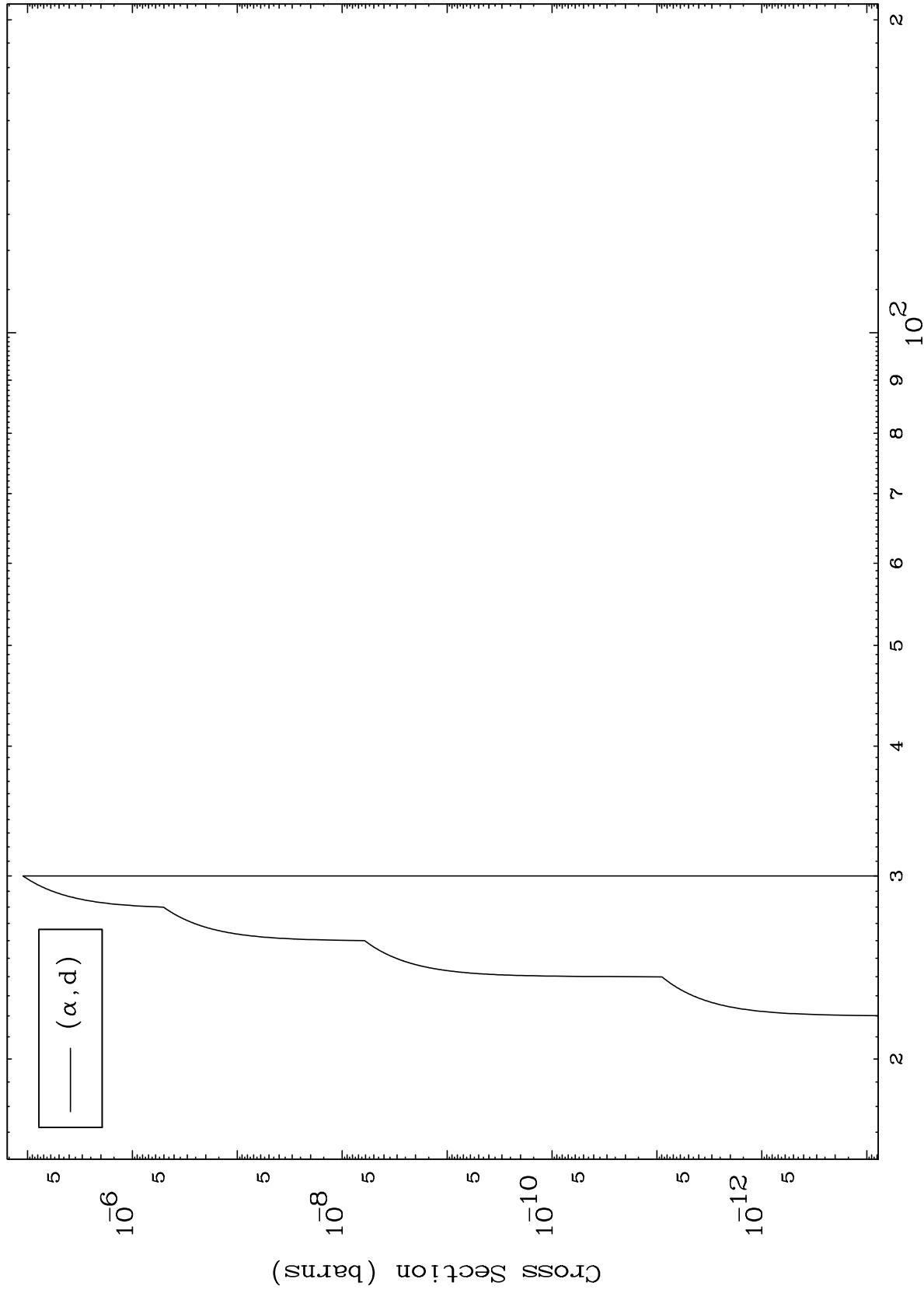
Incident Energy (MeV)

102-No-251

MAT 9974

(α ,d) Levels
0 Kelvin Cross Sections

102-No-251



7

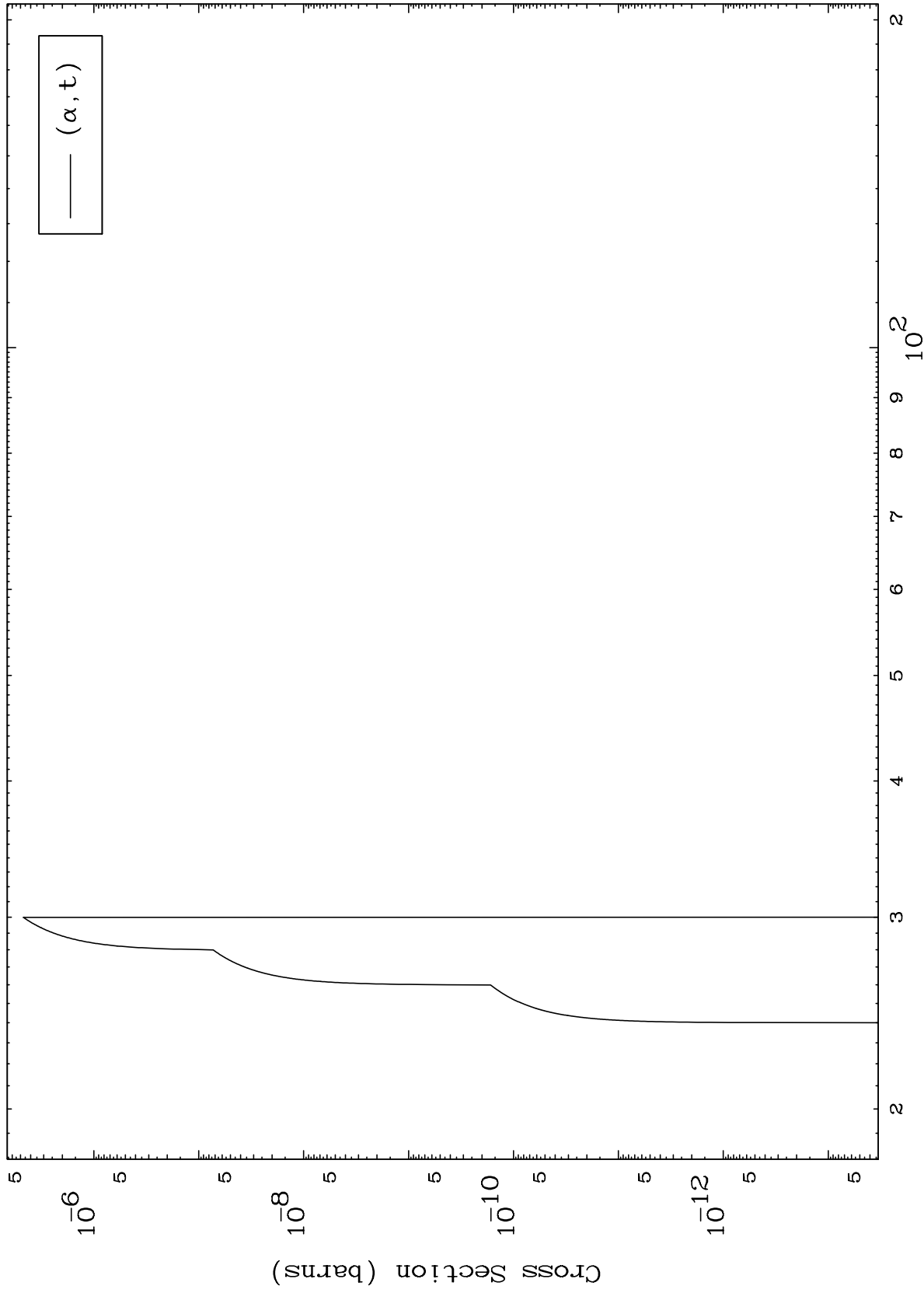
Incident Energy (MeV)

102-No-251

MAT 9974

102-No-251

(α, t) Levels
0 Kelvin Cross Sections



8

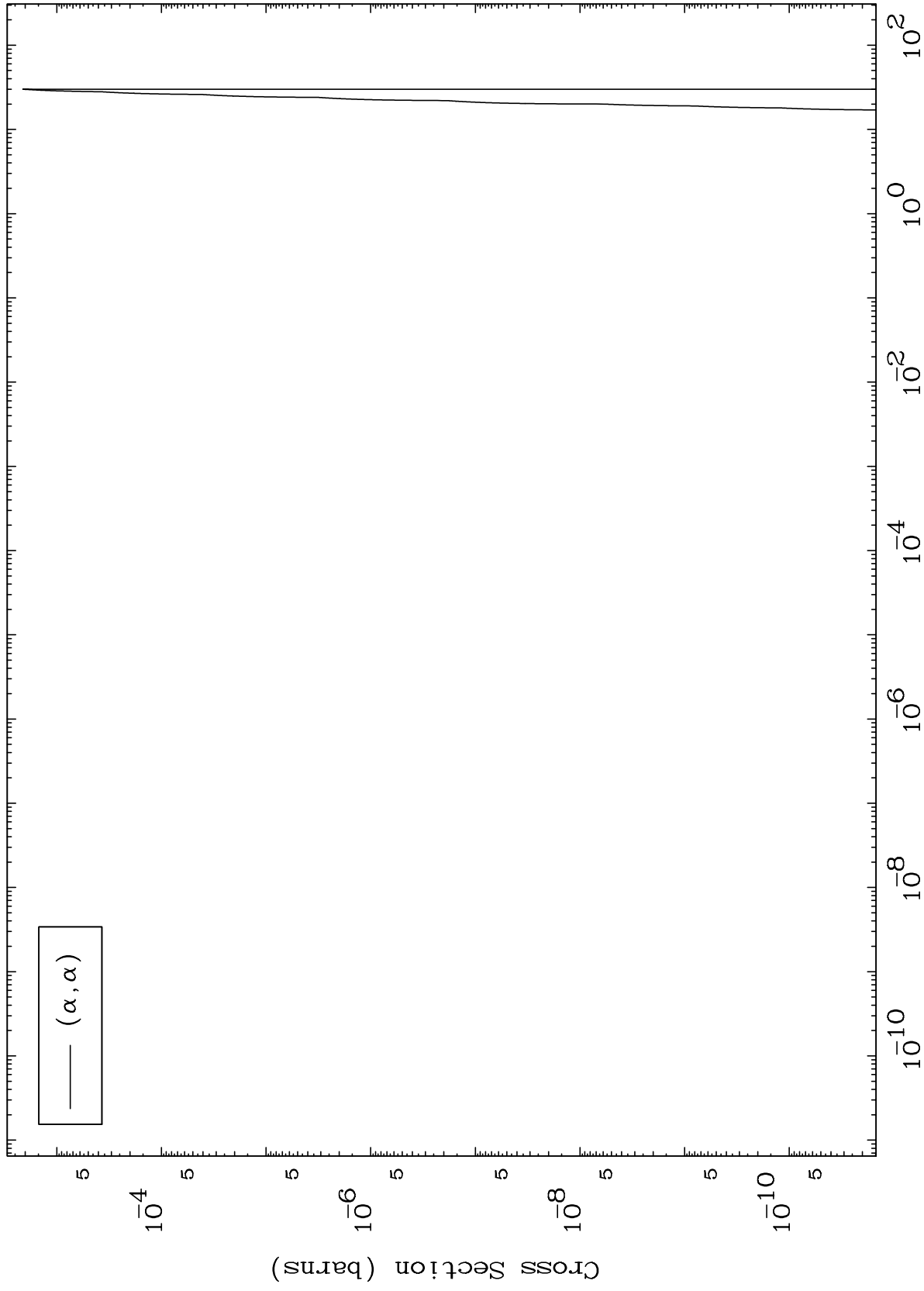
Incident Energy (MeV)

102-No-251

MAT 9974

(α, α) Levels
0 Kelvin Cross Sections

102-No-251

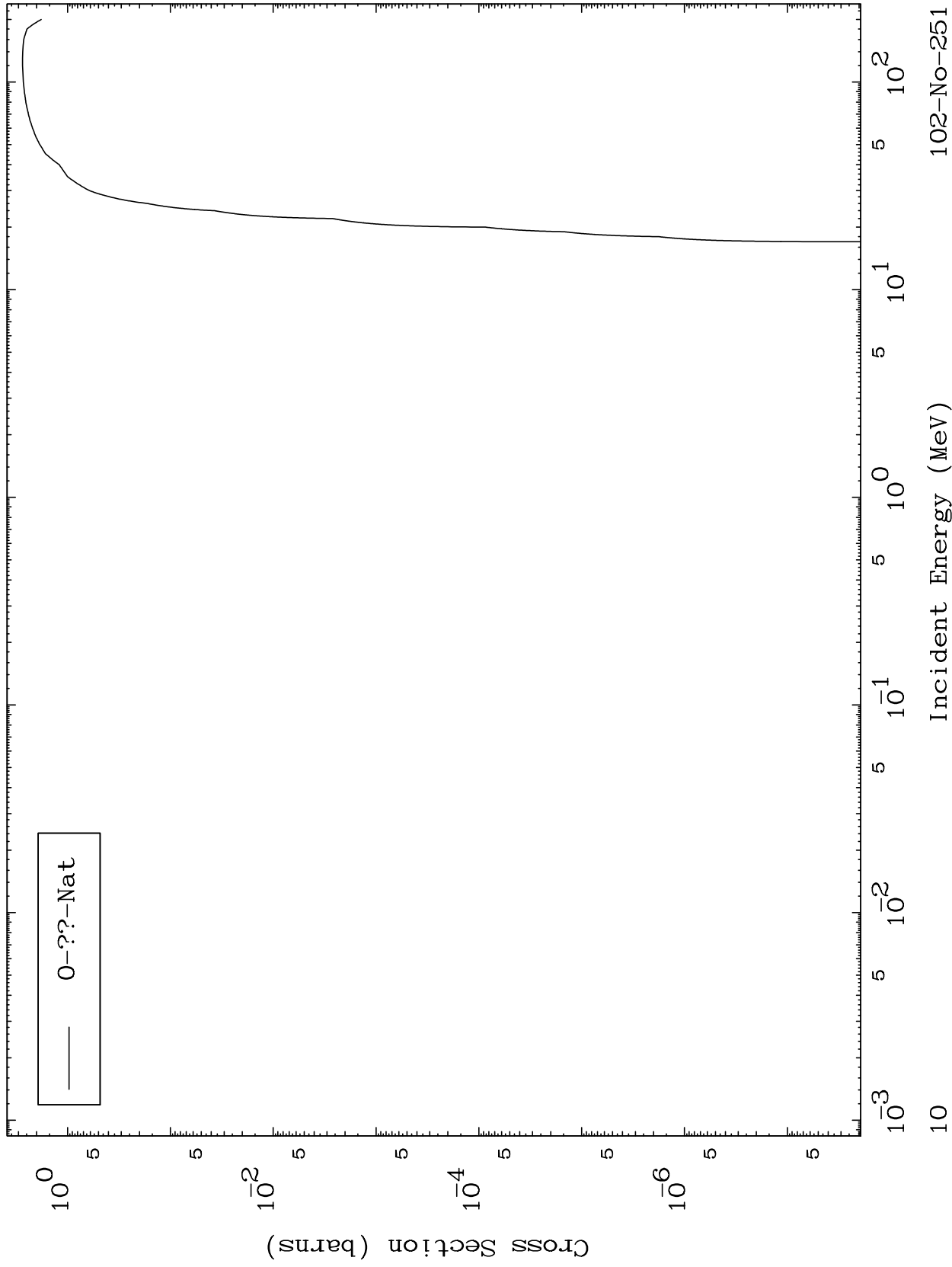


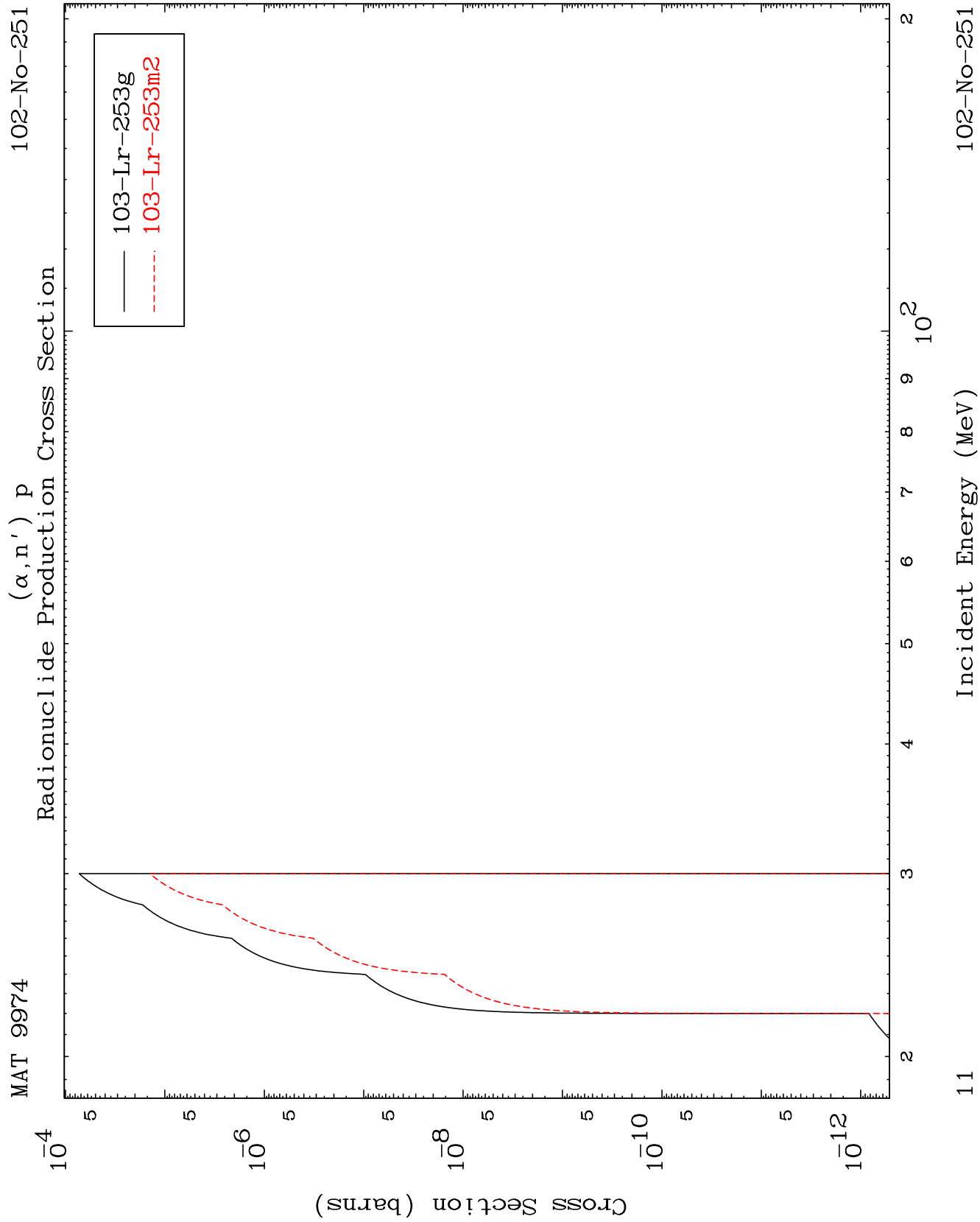
MAT 9974

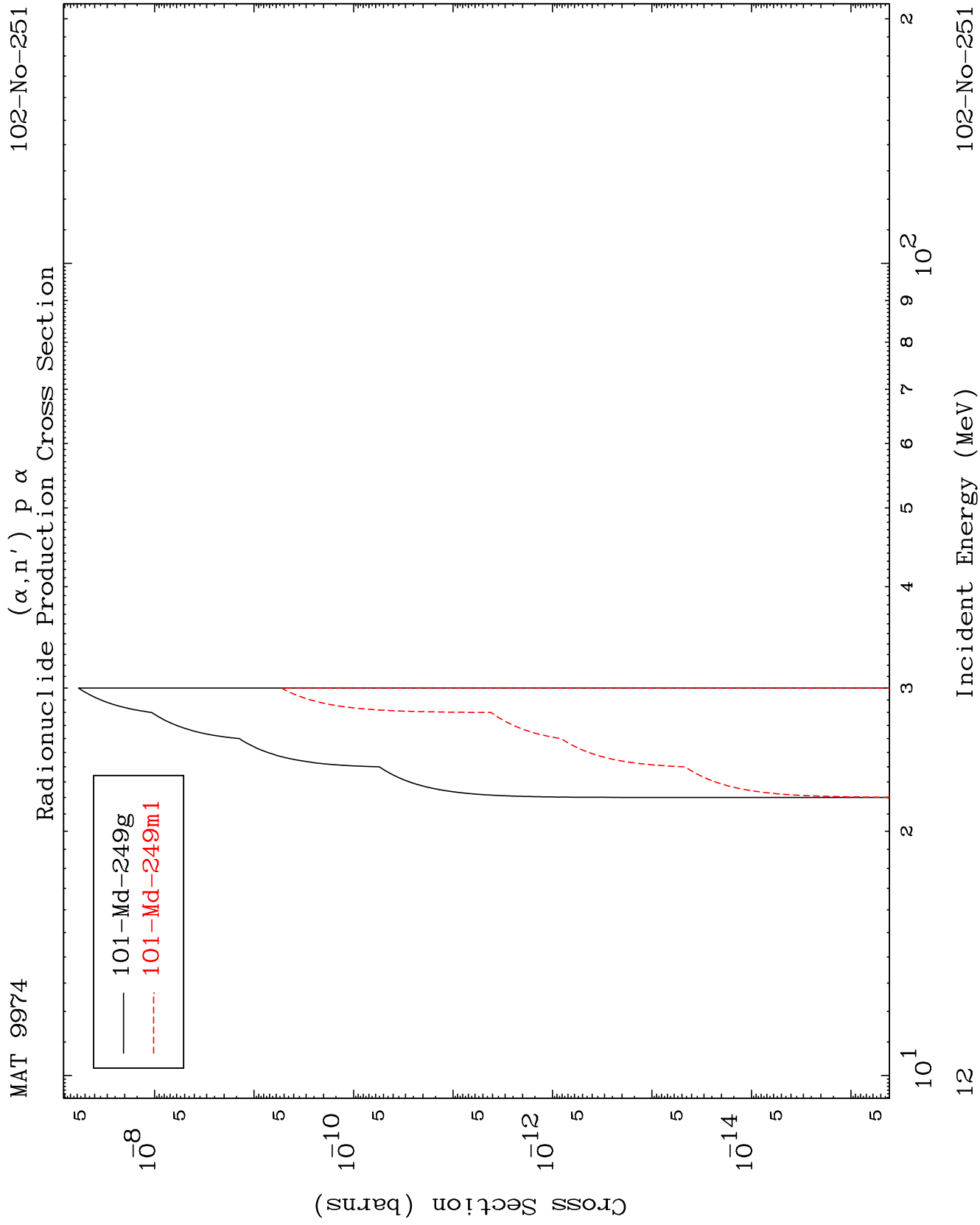
α Fission

102-No-251

Radionuclide Production Cross Section



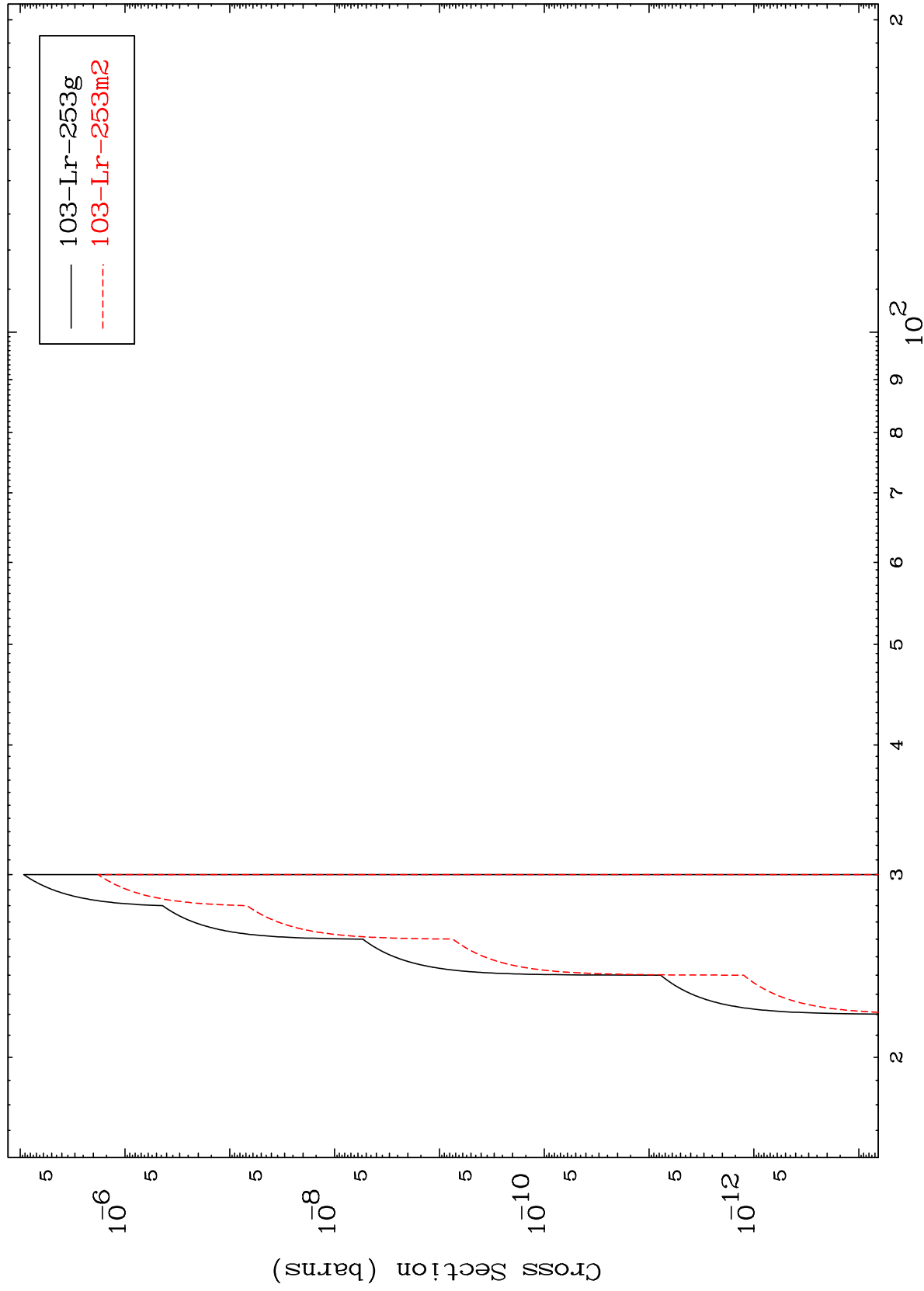




MAT 9974

102-No-251

(α, d)
Radionuclide Production Cross Section



13

Incident Energy (MeV)

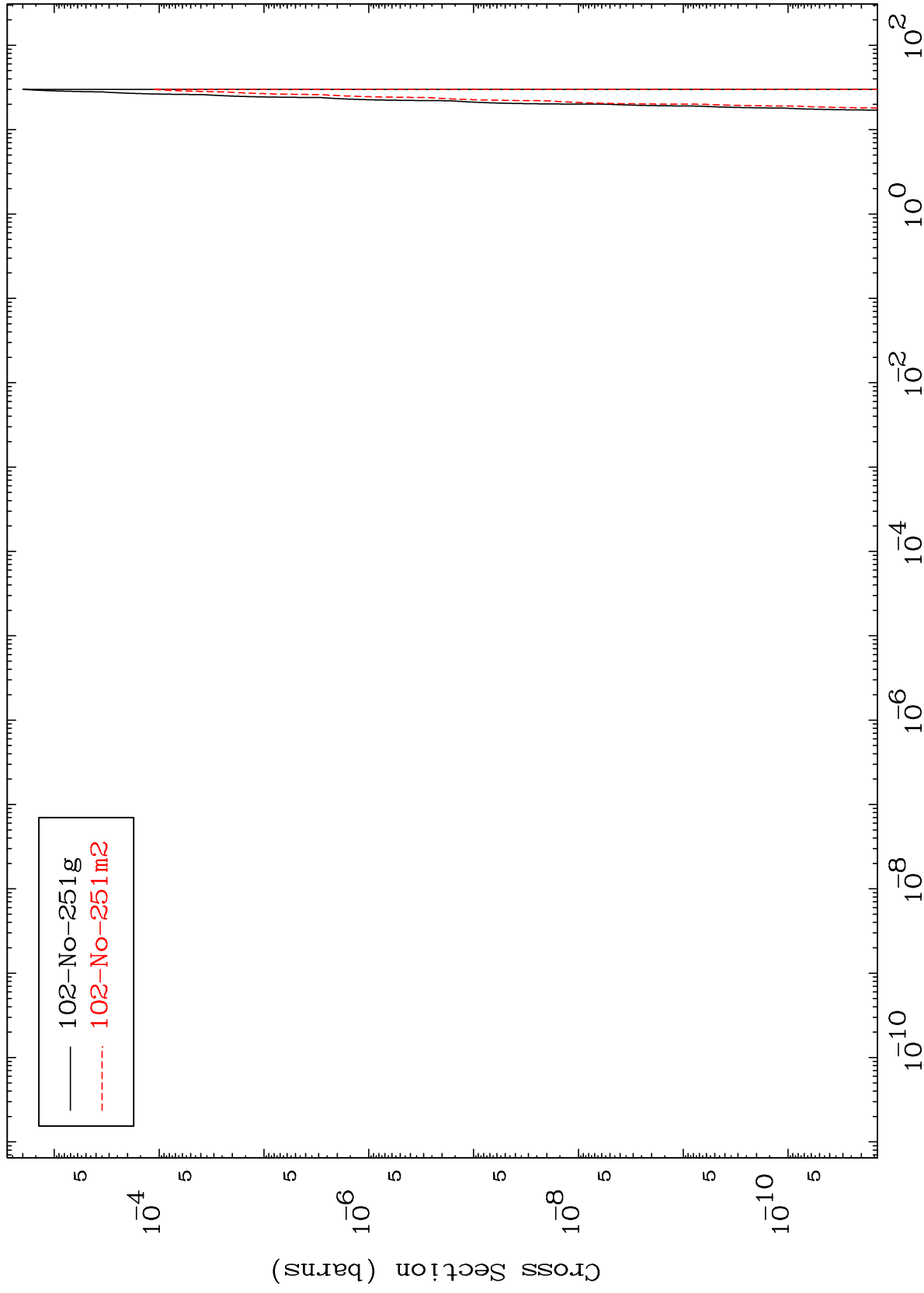
102-No-251

MAT 9974

(α, α)

102-No-251

Radionuclide Production Cross Section



14

Incident Energy (MeV)

102-No-251

