

Program EVALPLOT
(Version 2018-1)

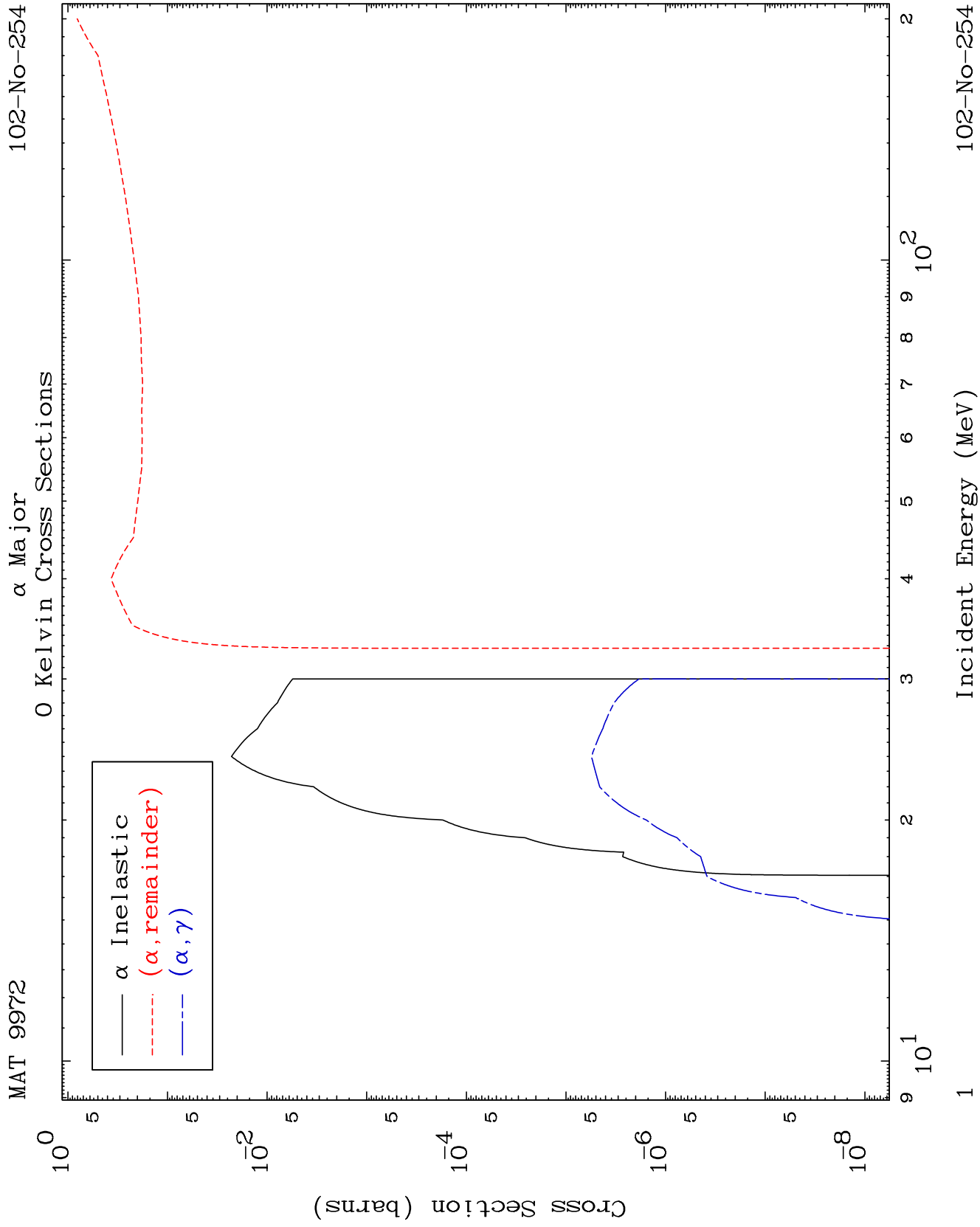
by

Dermott E. Cullen
(Present Contact Information)
Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail: redcullen1@comcast.net
Web: redcullen1.net/HOMEPAGE.NEW

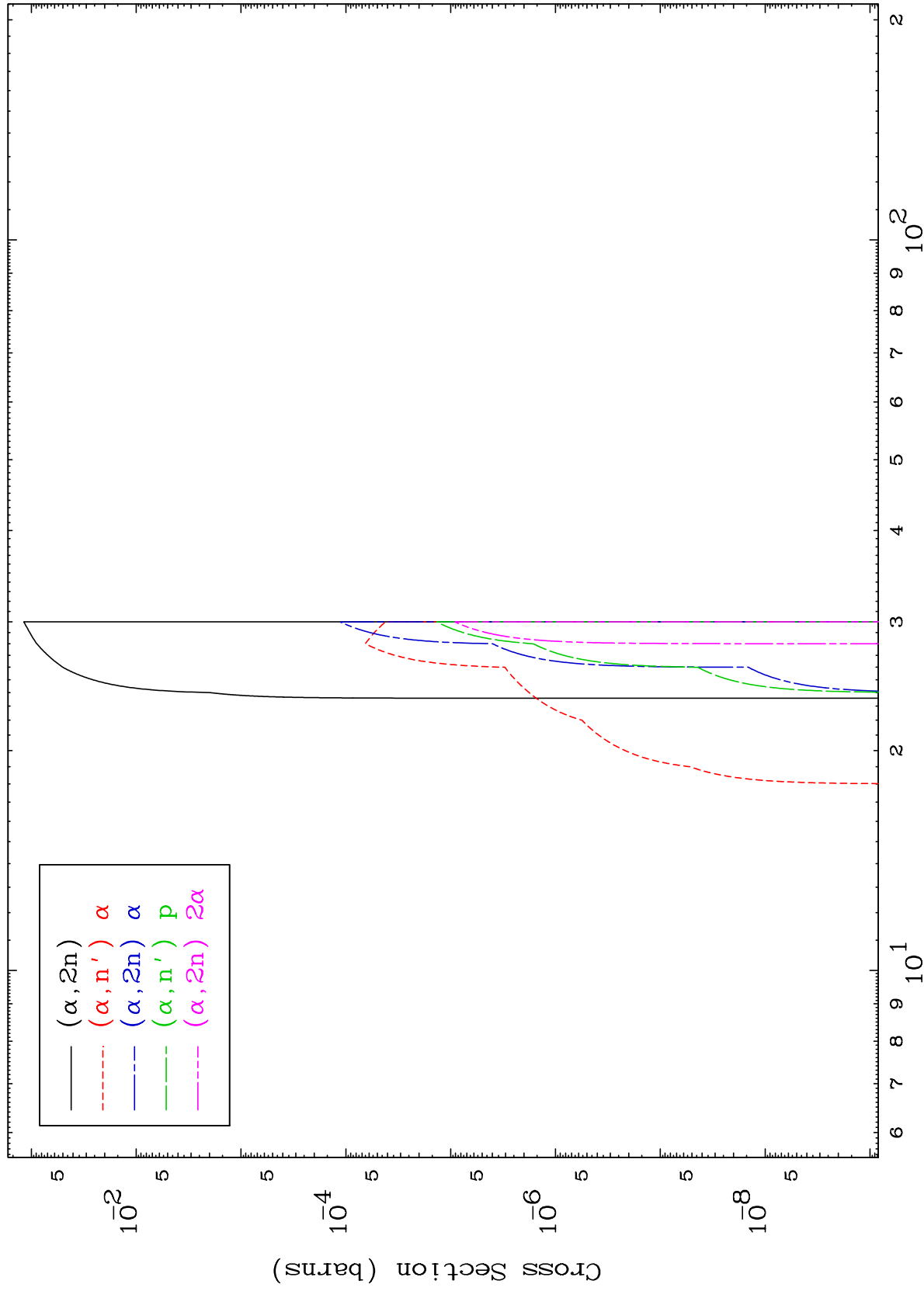
Press Mouse Button to Start



MAT 9972

α Neutron Production
0 Kelvin Cross Sections

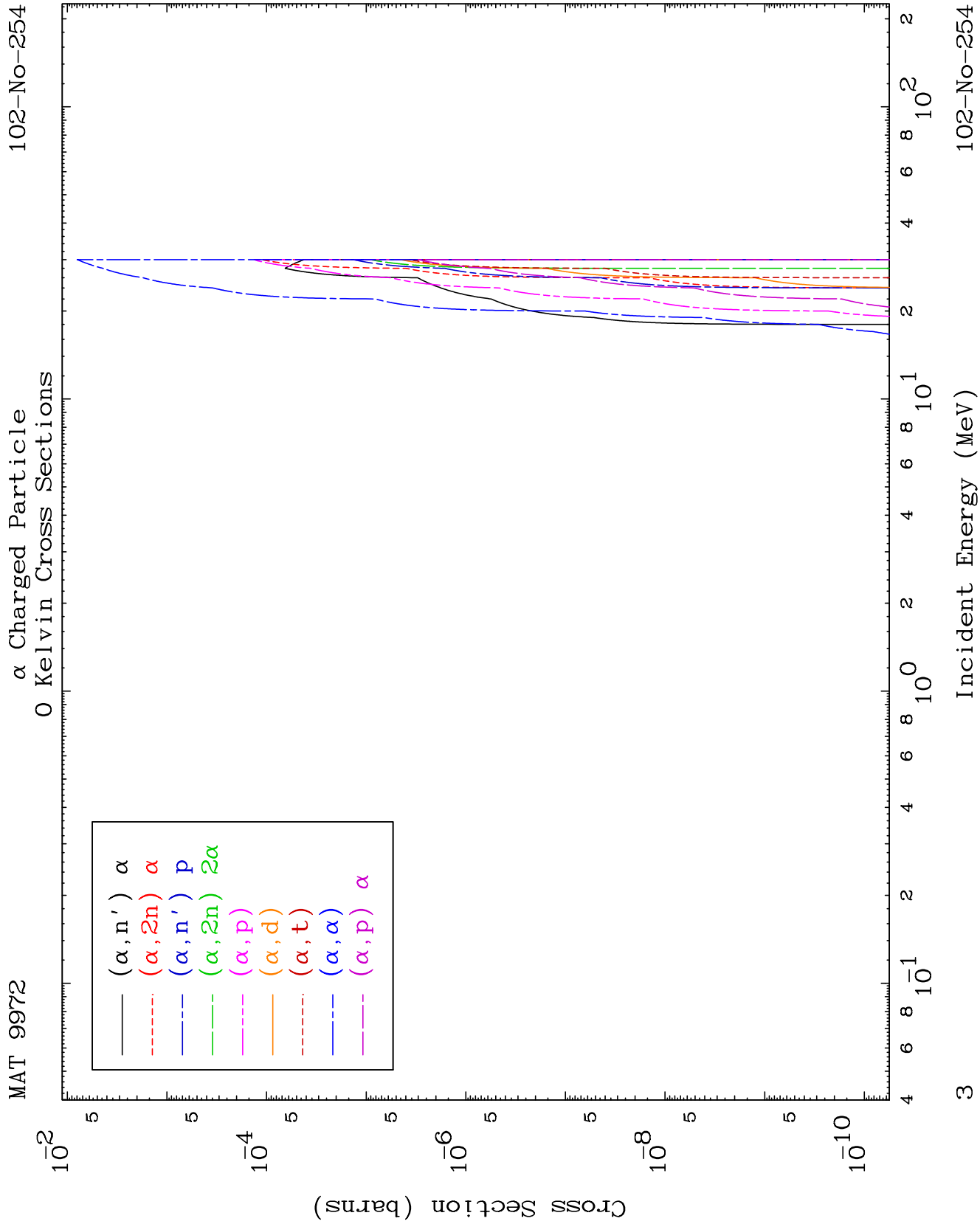
102-No-254



2

Incident Energy (MeV)

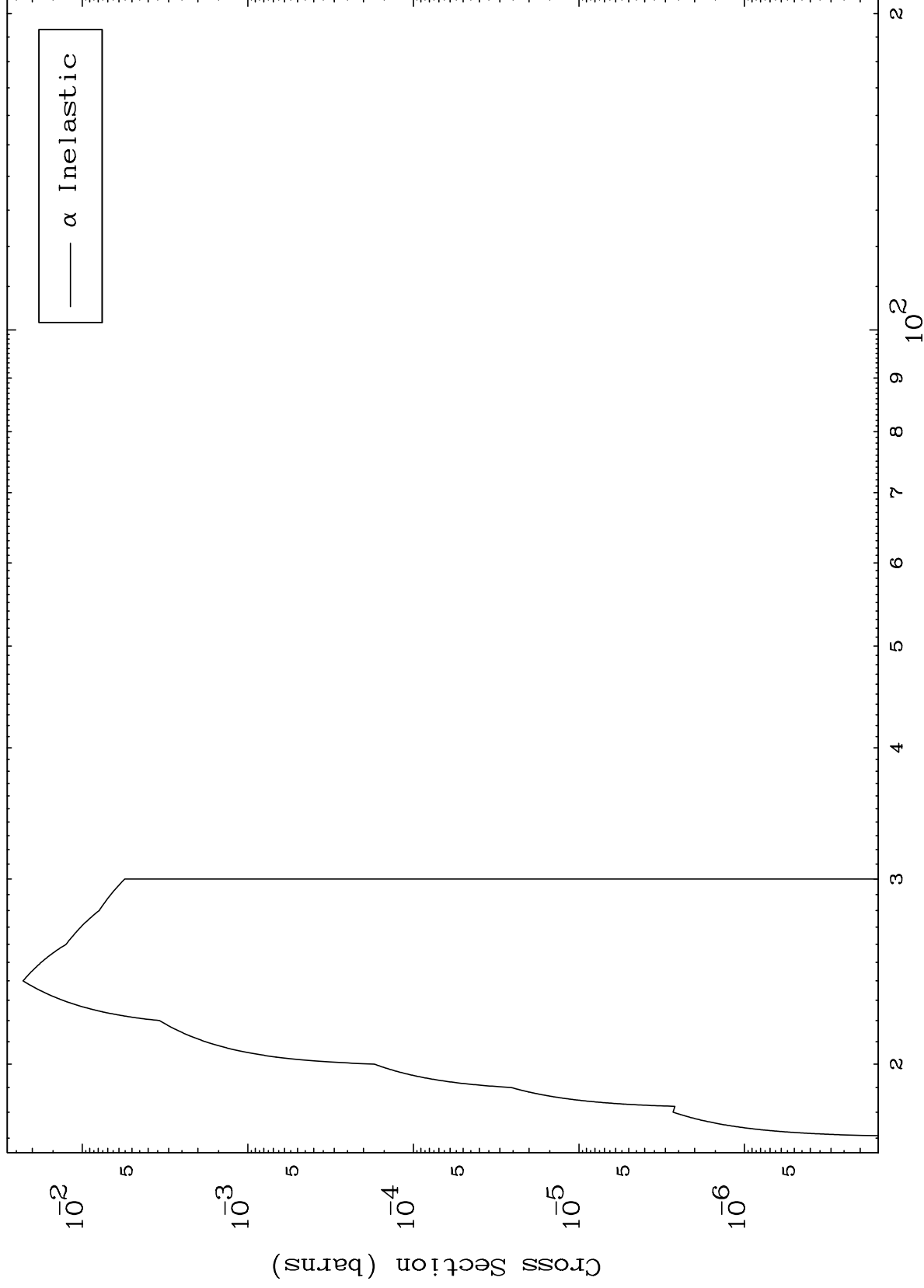
102-No-254



MAT 9972

(α, n') Level
0 Kelvin Cross Sections

102-No-254



4

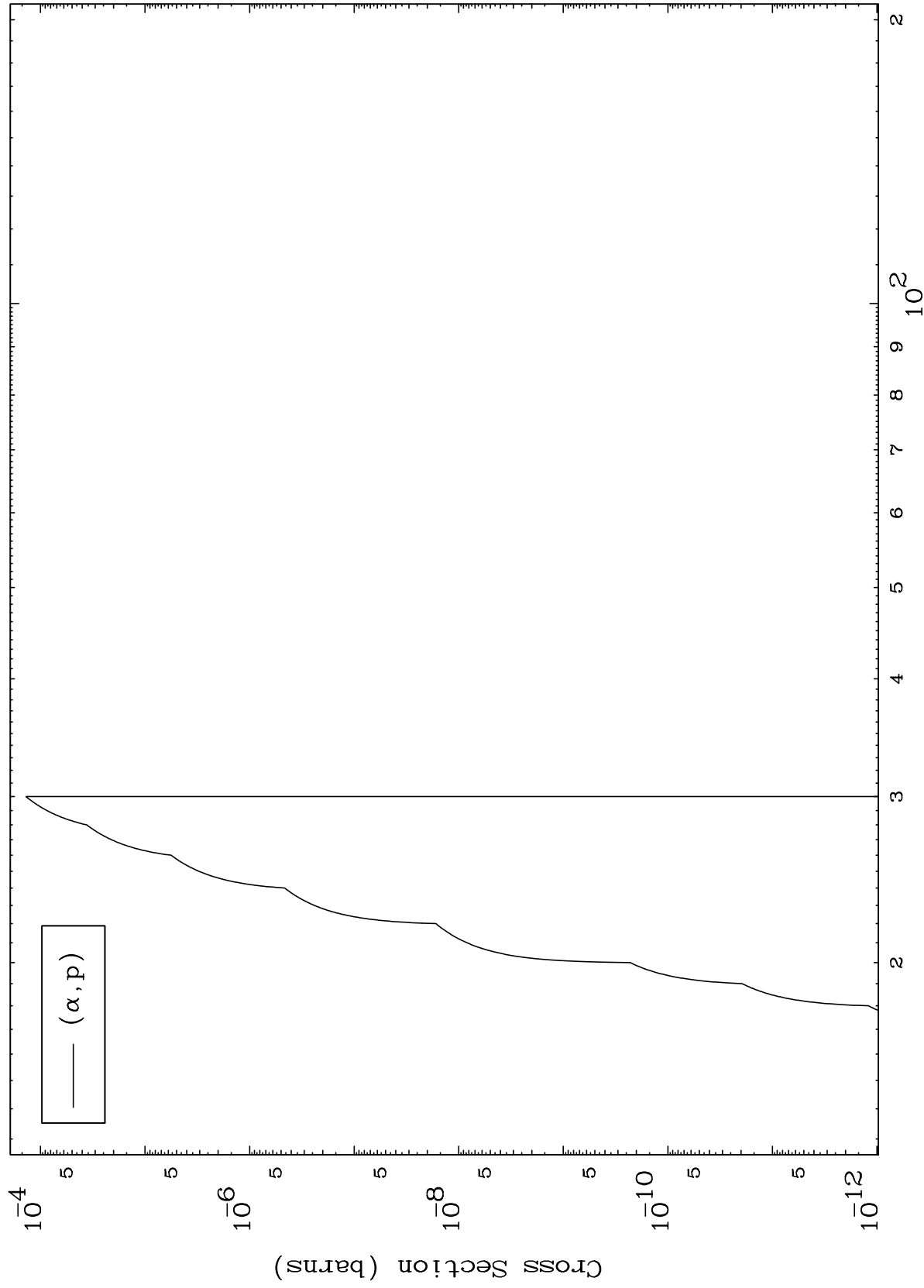
Incident Energy (MeV)

102-No-254

MAT 9972

102-No-254

(α ,p) Levels
0 Kelvin Cross Sections



5

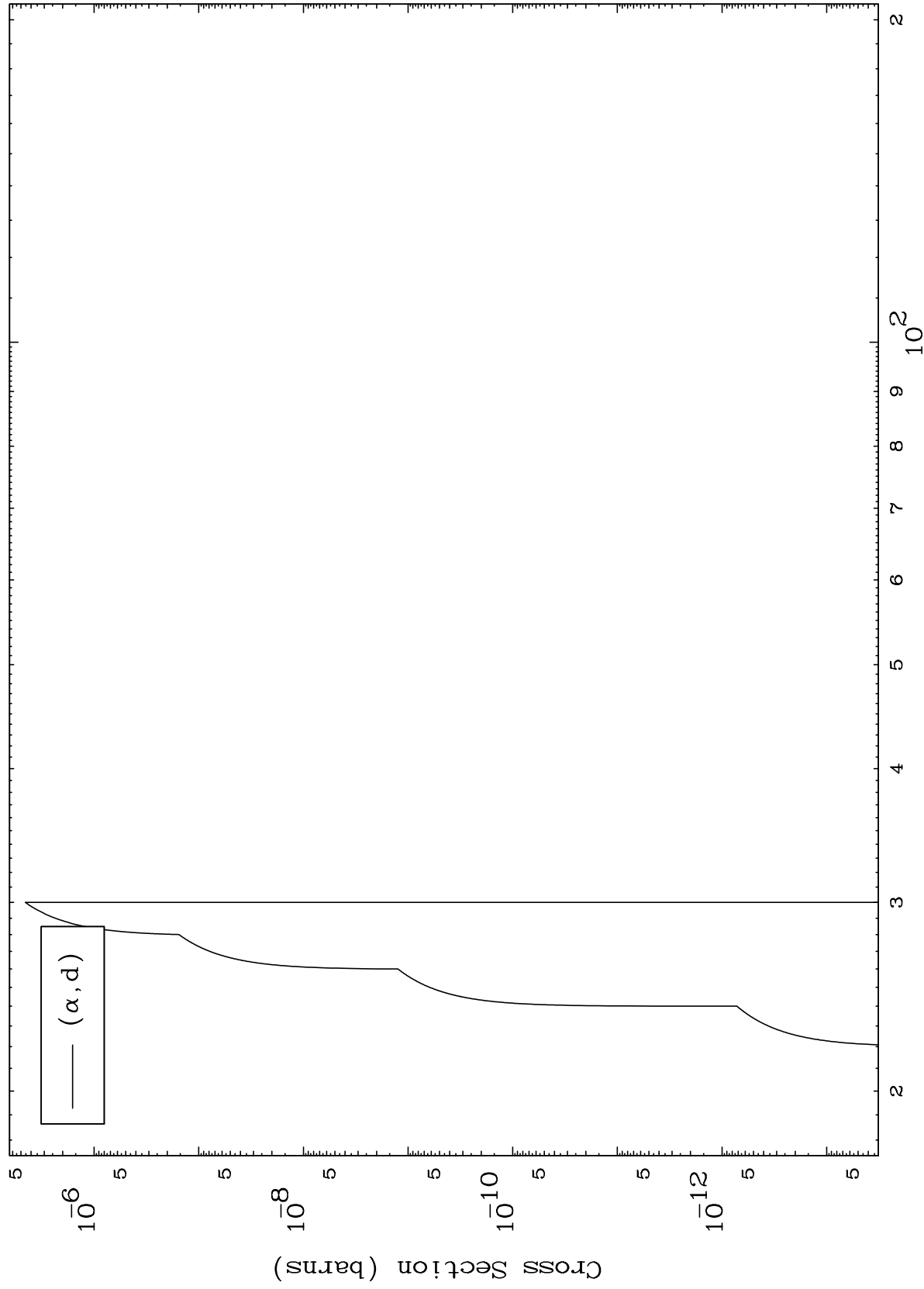
Incident Energy (MeV)

102-No-254

MAT 9972

(α ,d) Levels
0 Kelvin Cross Sections

102-No-254



6

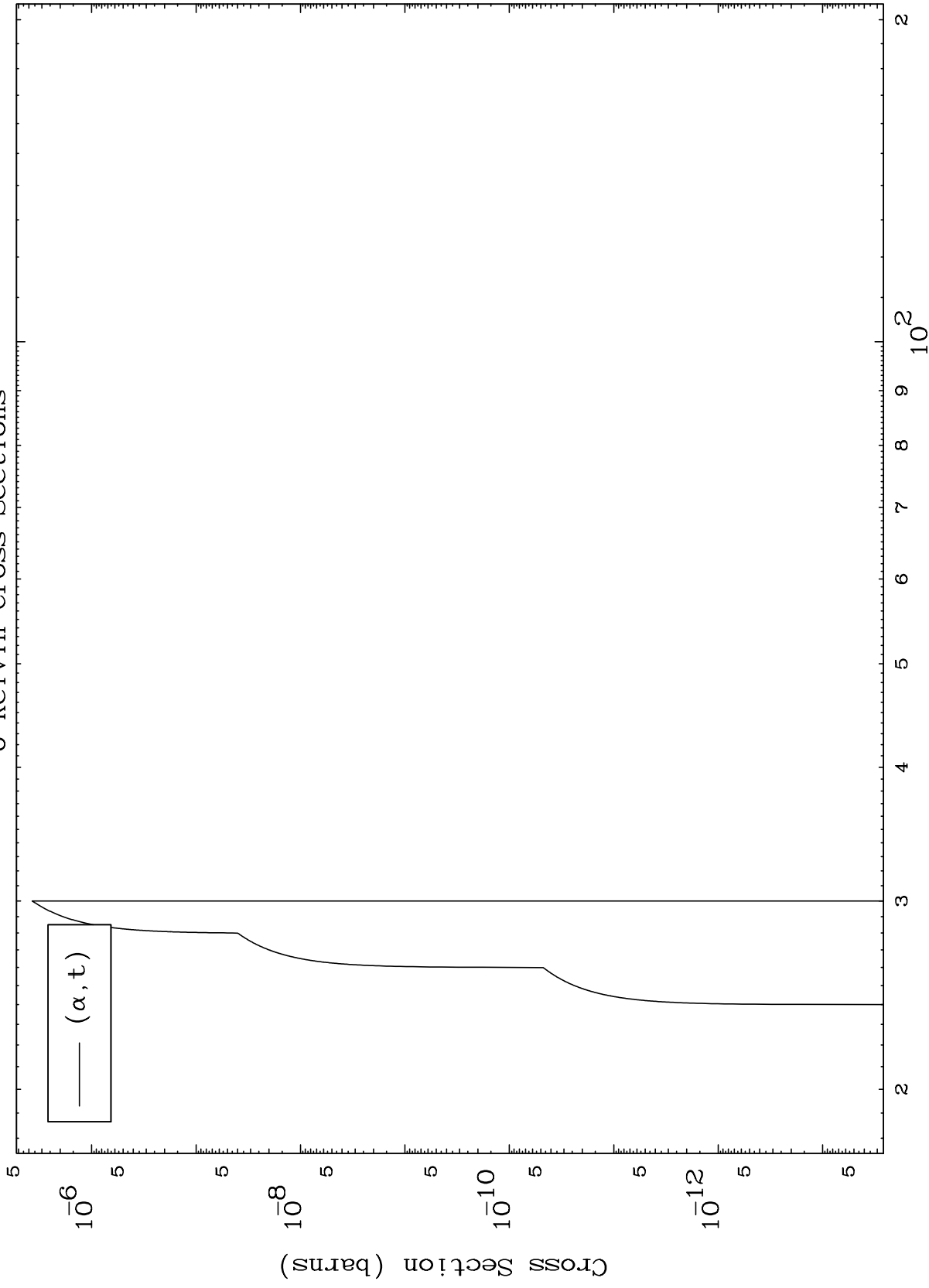
Incident Energy (MeV)

102-No-254

MAT 9972

(α ,t) Levels
0 Kelvin Cross Sections

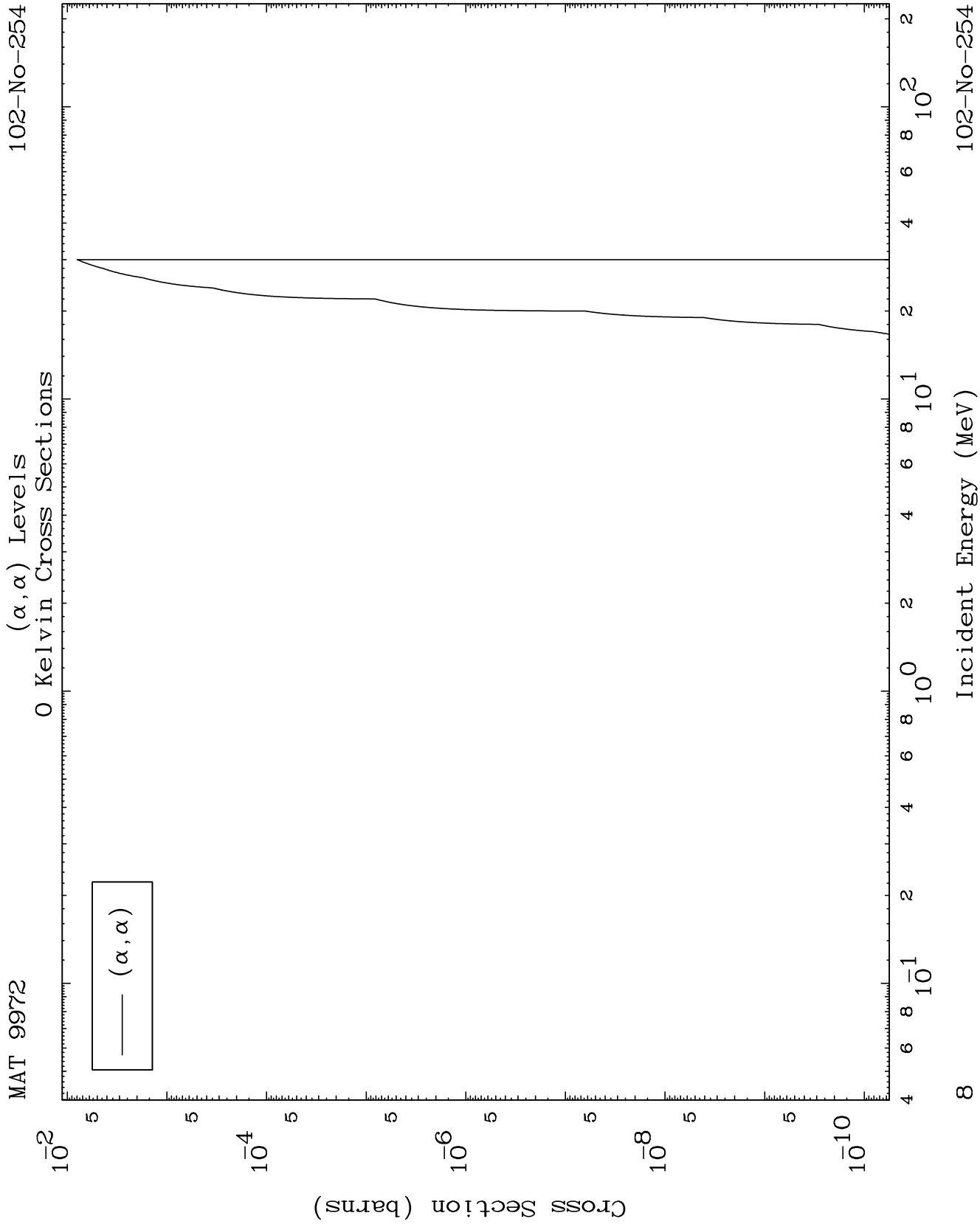
102-No-254



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Incident Energy (MeV)

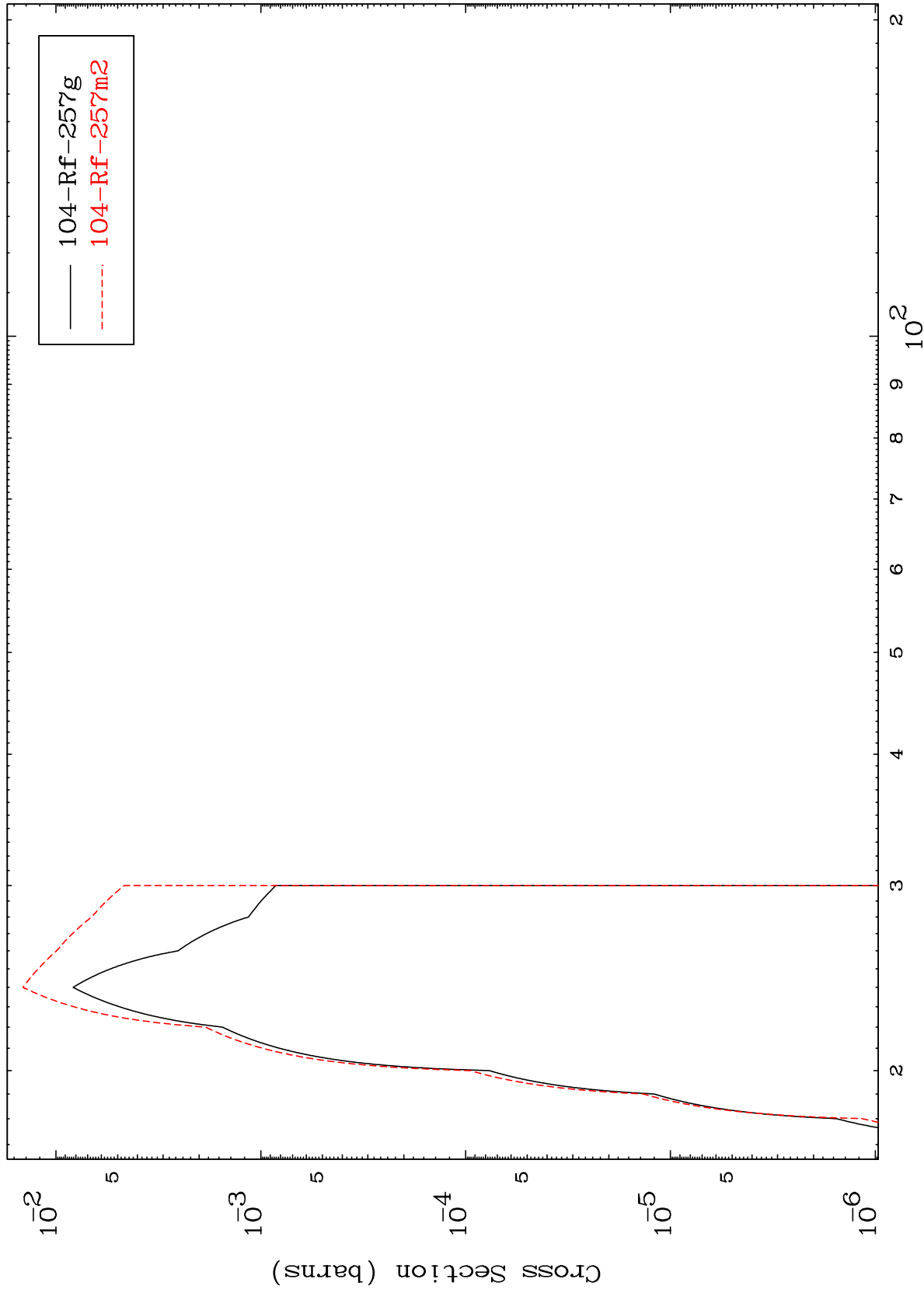
102-No-254



MAT 9972

102-No-254

α Inelastic
Radionuclide Production Cross Section



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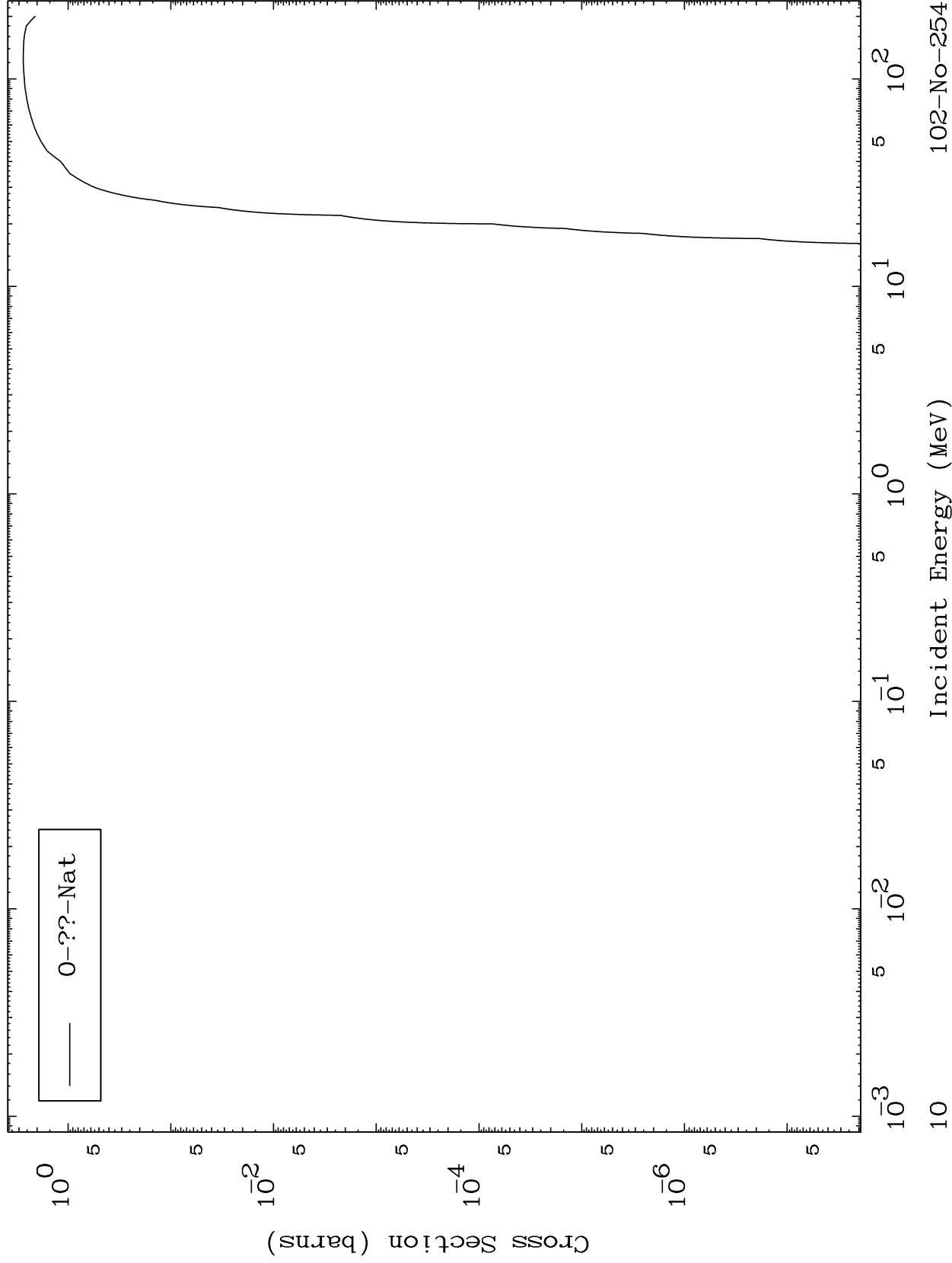
Incident Energy (MeV)

102-No-254

MAT 9972

α Fission
Radionuclide Production Cross Section

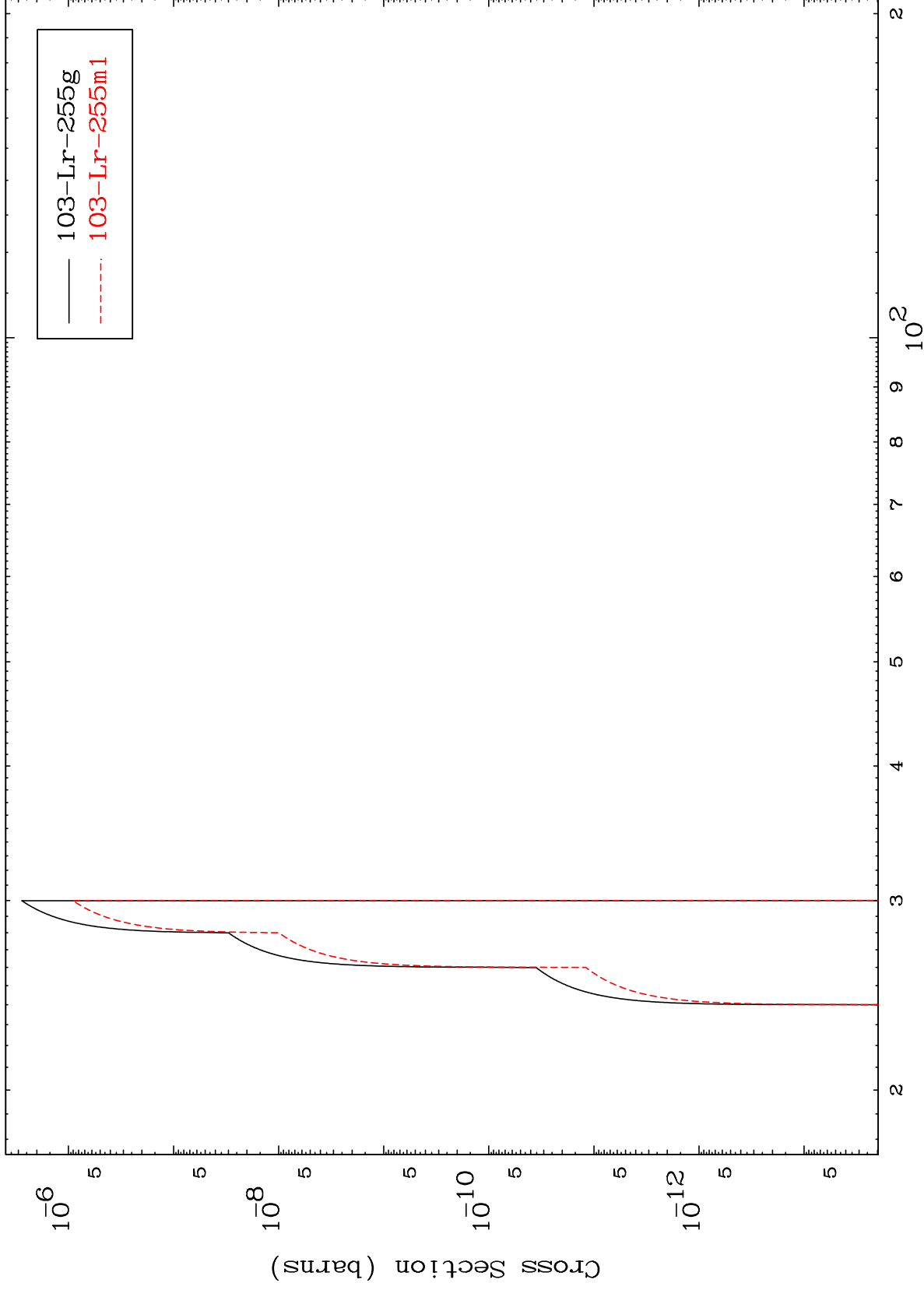
102-No-254



MAT 9972

102-No-254

(α, t)
Radionuclide Production Cross Section



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Incident Energy (MeV)

102-No-254