

Program EVALPLOT  
(Version 2018-1)

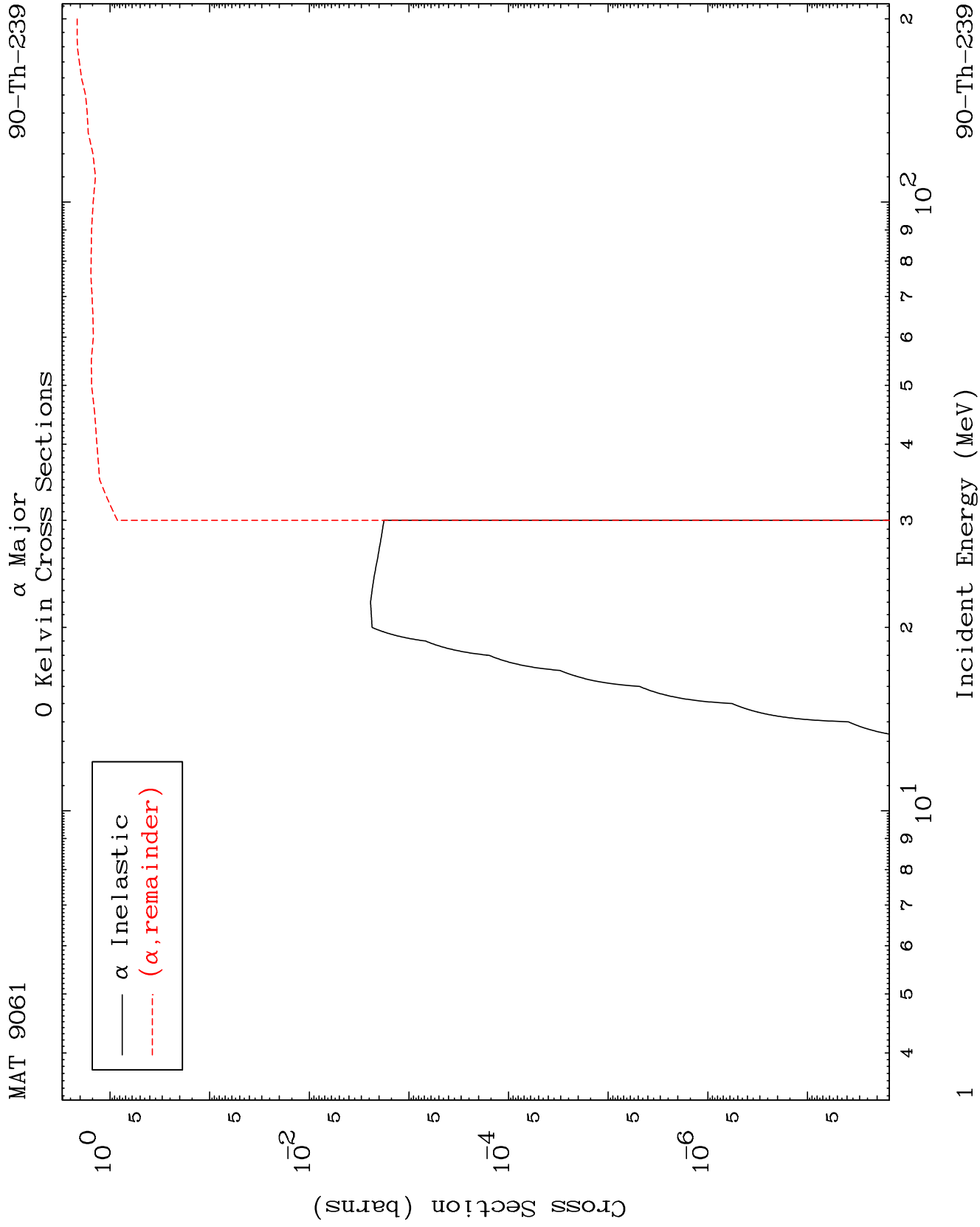
by

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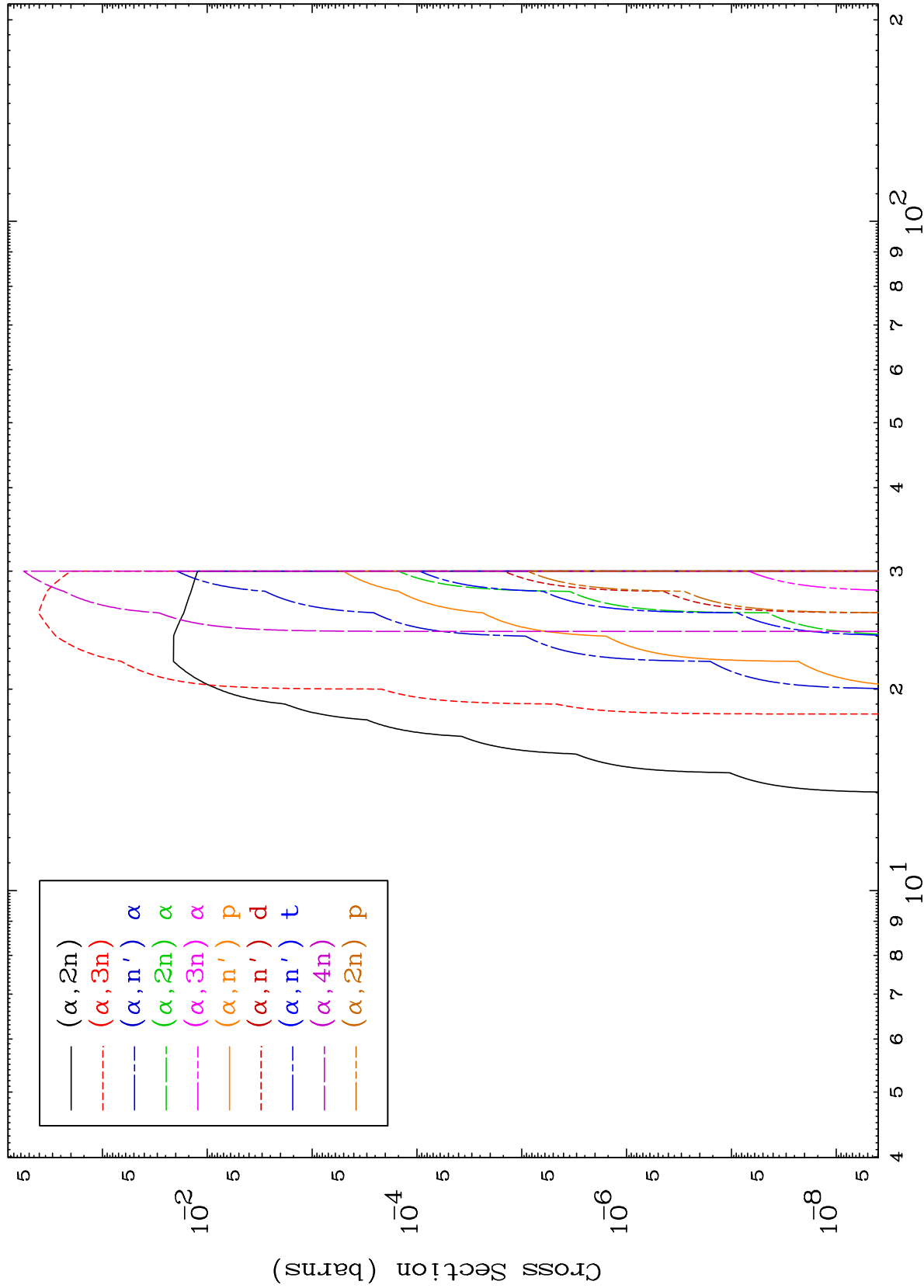
Press Mouse Button to Start



MAT 9061

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

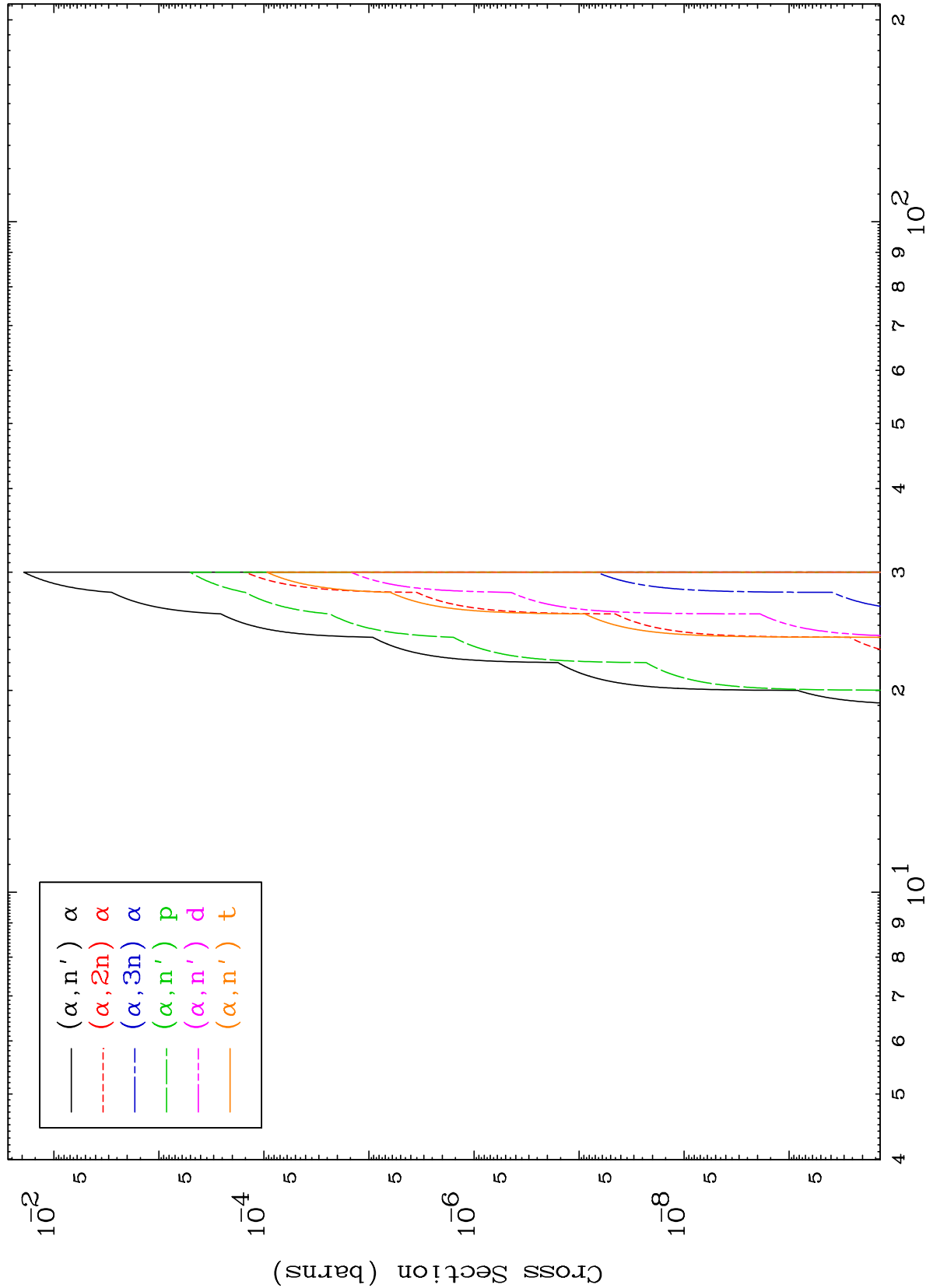
90-Th-239



2

Incident Energy (MeV)

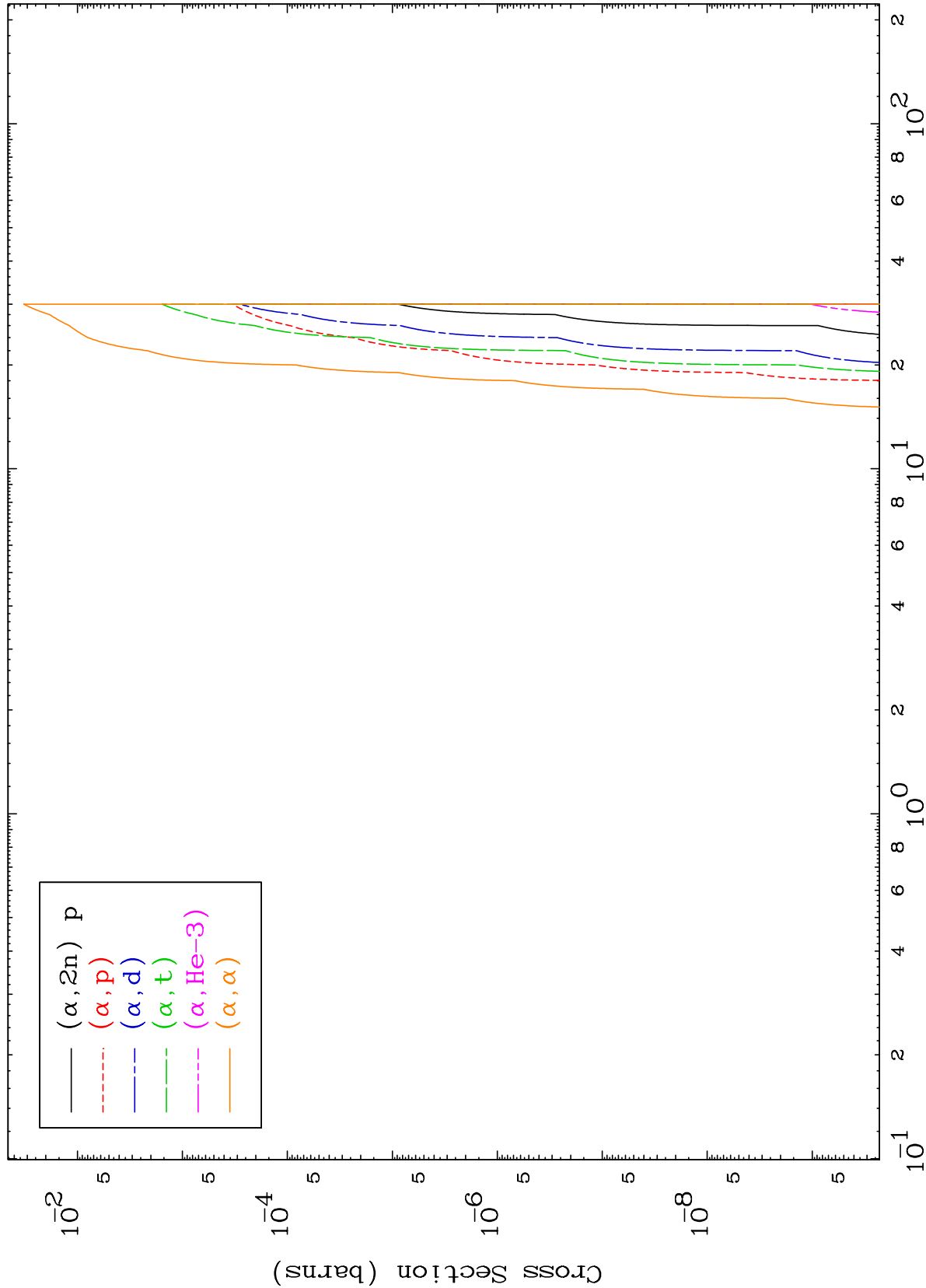
90-Th-239



MAT 9061

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

90-Th-239



4

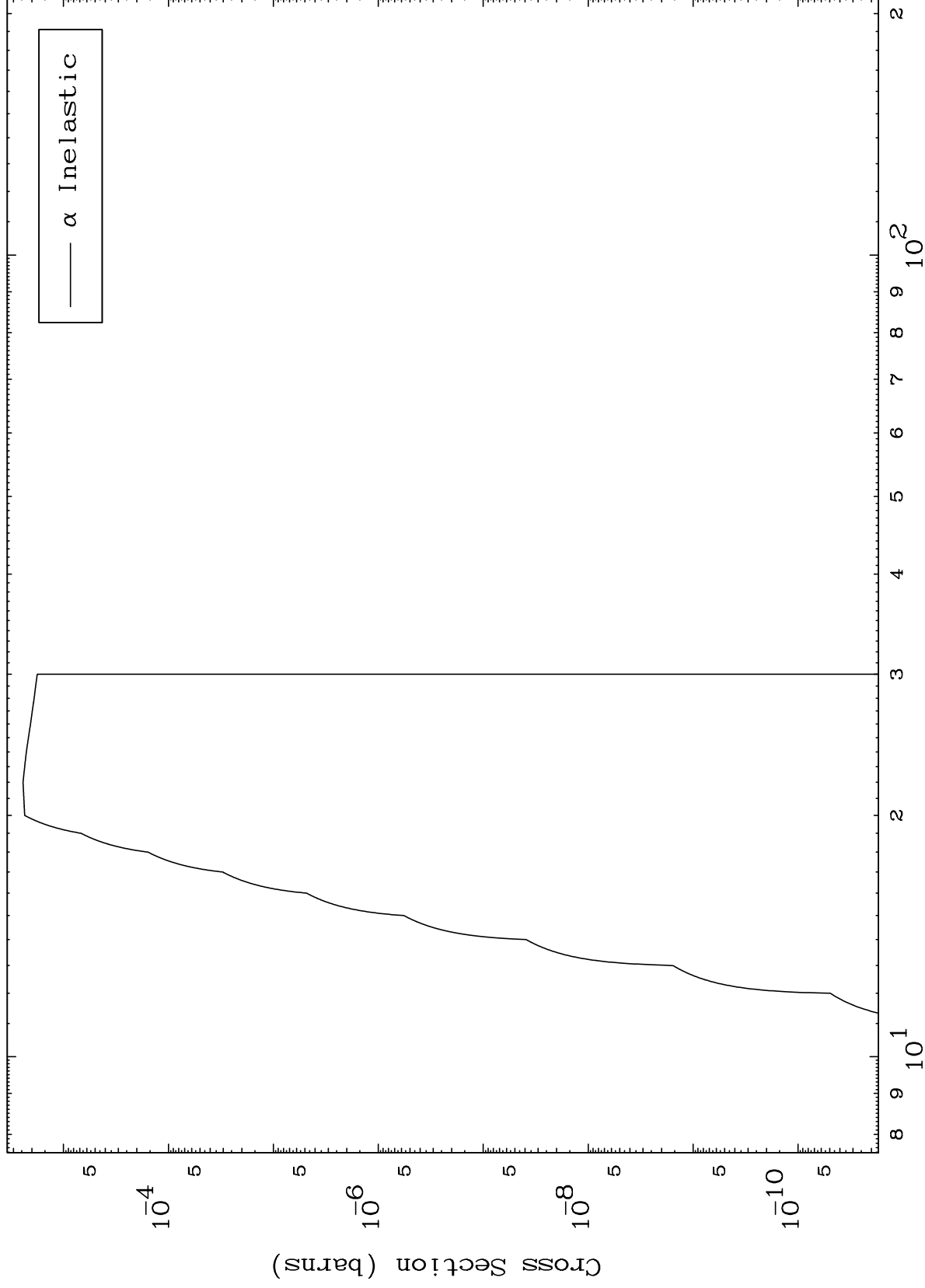
Incident Energy (MeV)

90-Th-239

MAT 9061

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

90-Th-239



5

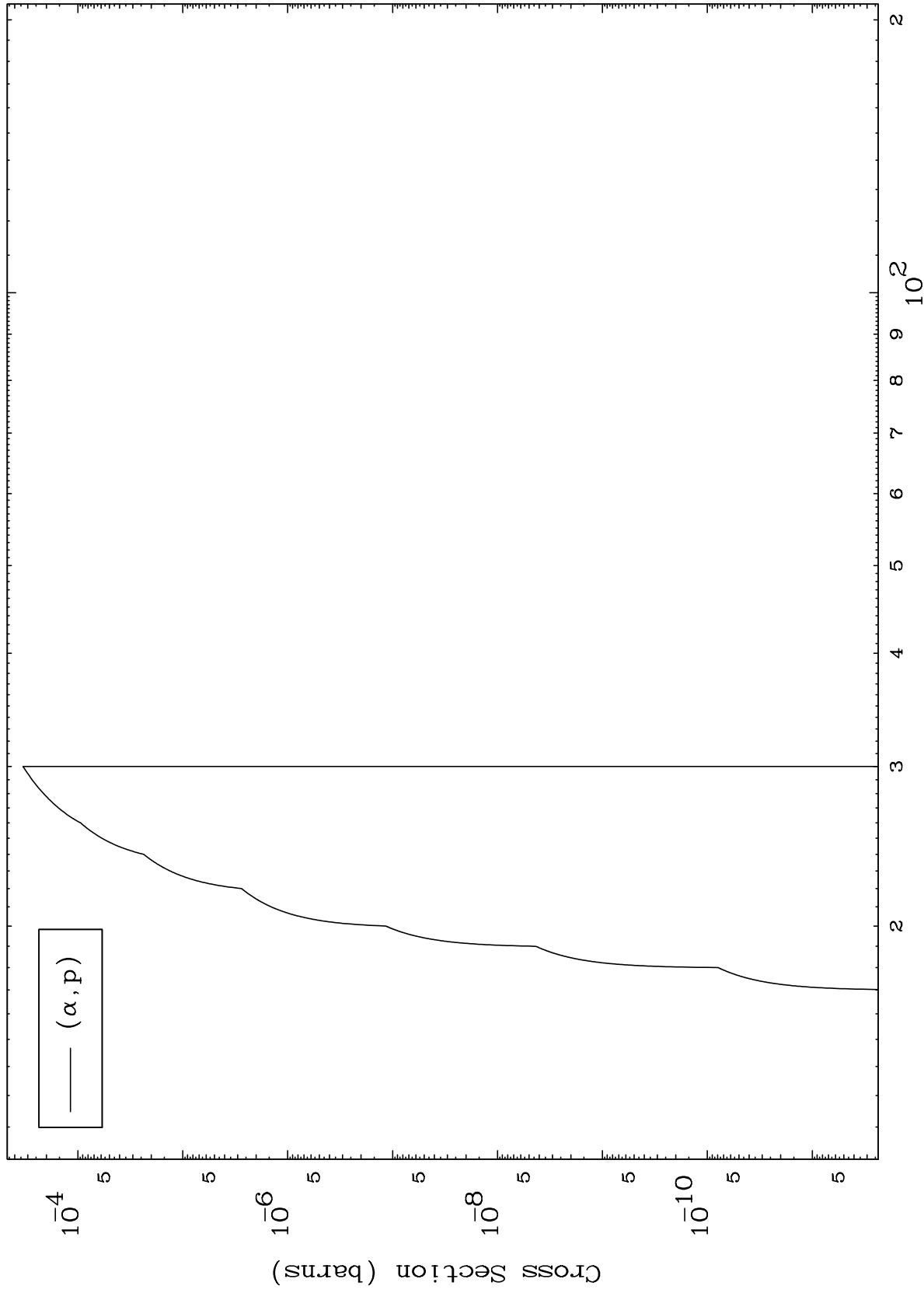
Incident Energy (MeV)

90-Th-239

MAT 9061

( $\alpha$ ,p) Levels  
0 Kelvin Cross Sections

90-Th-239

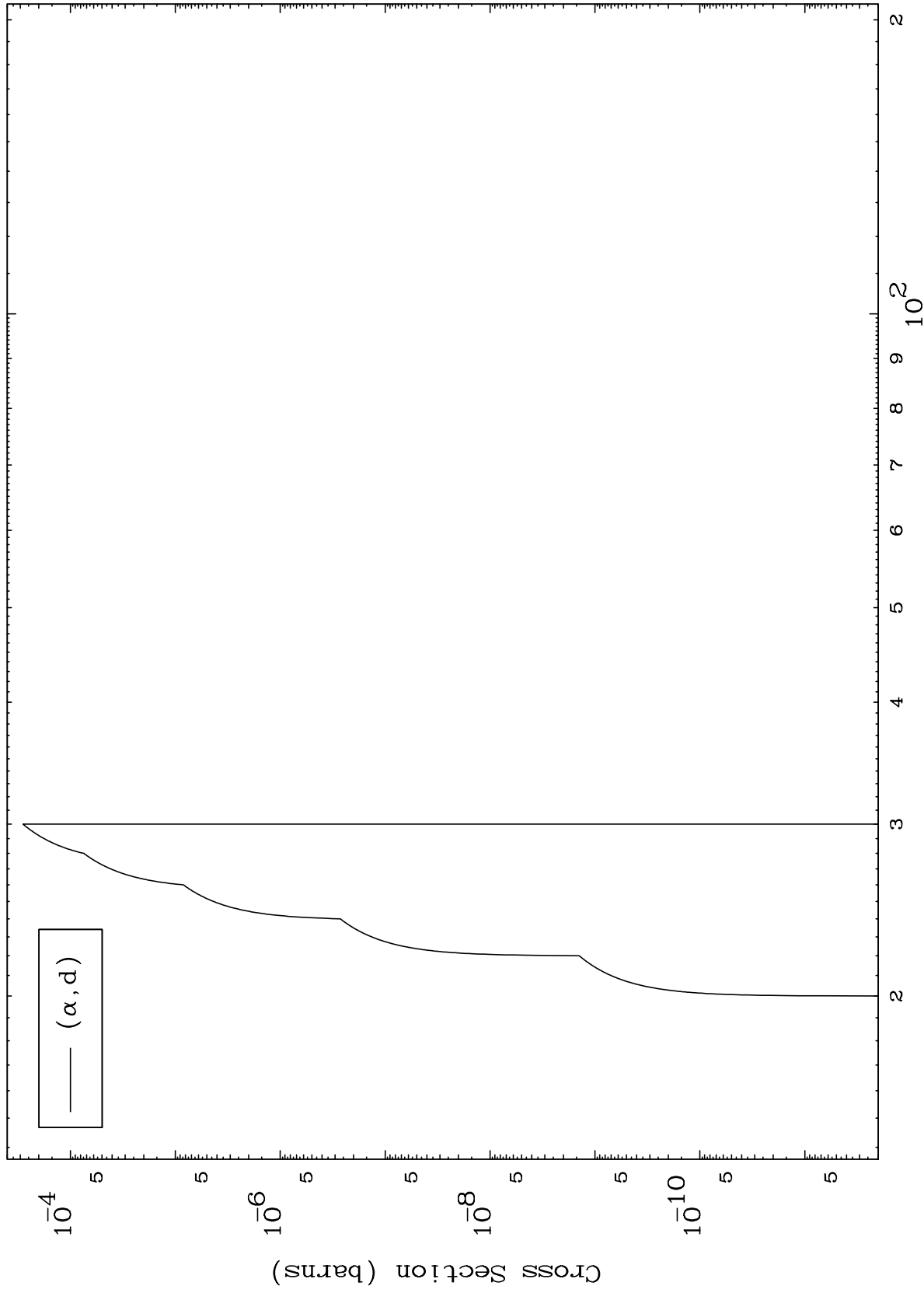


6

MAT 9061

( $\alpha$ ,d) Levels  
0 Kelvin Cross Sections

90-Th-239



7

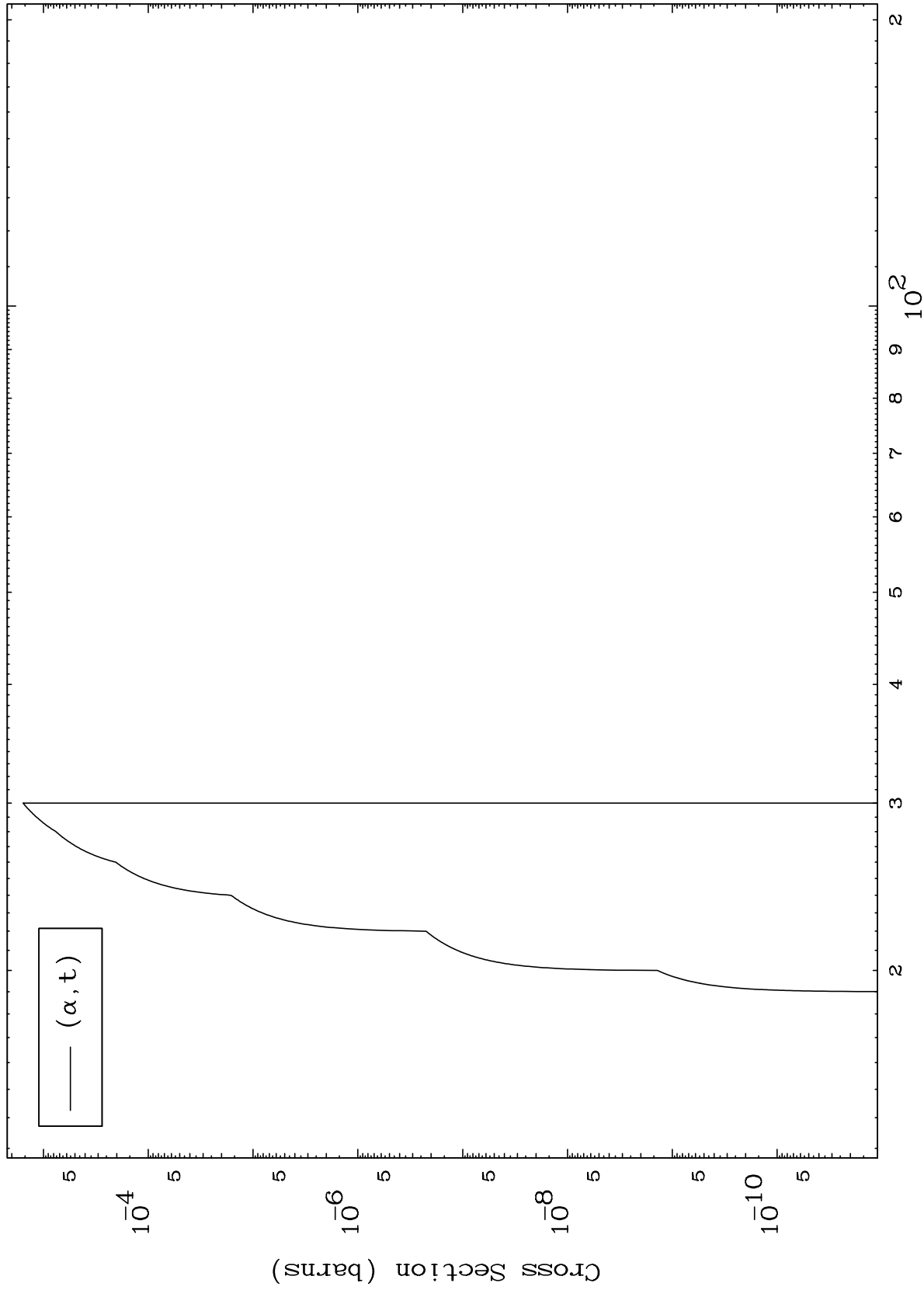
Incident Energy (MeV)

90-Th-239

MAT 9061

90-Th-239

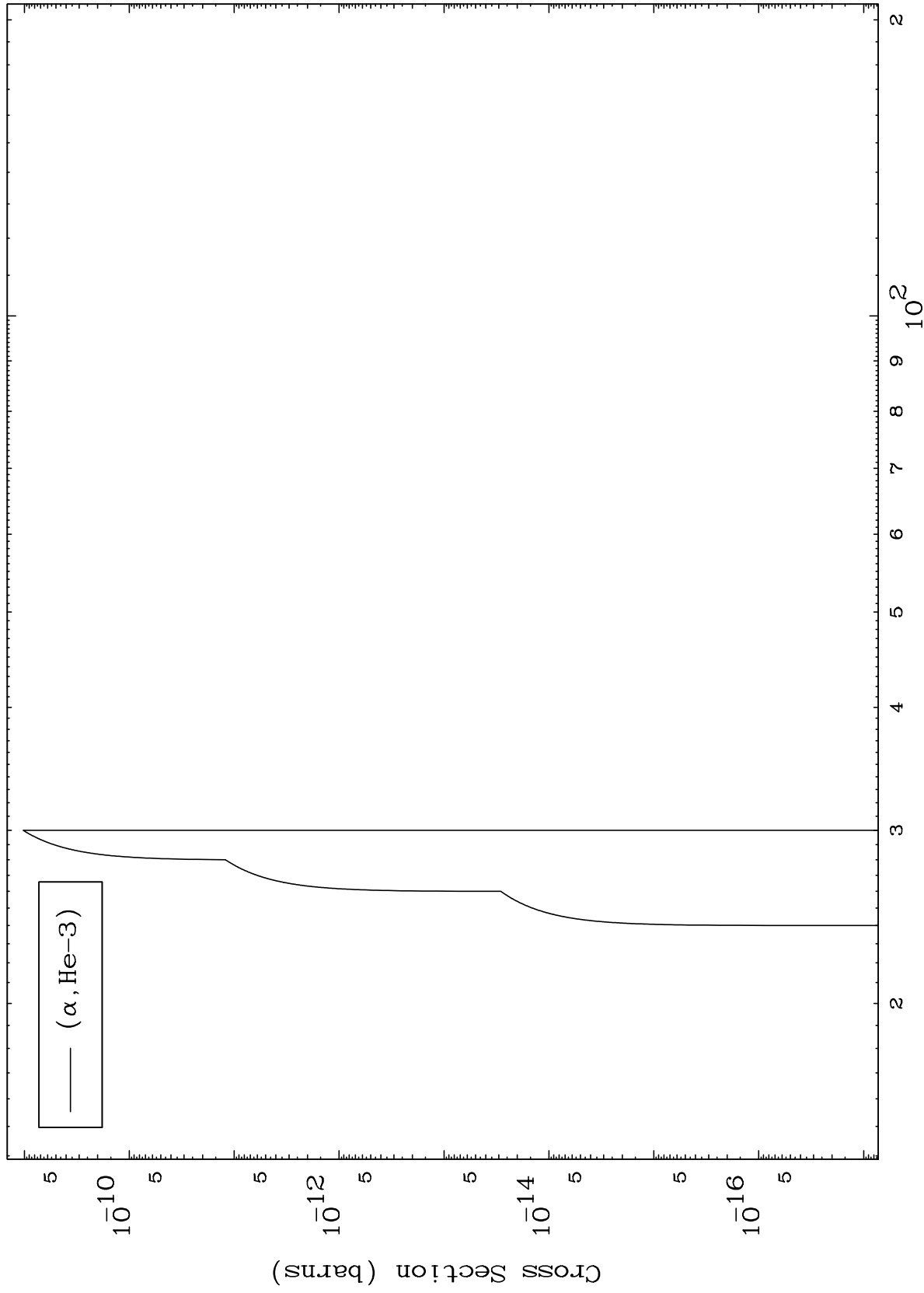
( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections



8

90-Th-239

( $\alpha$ ,He3) Levels  
0 Kelvin Cross Sections

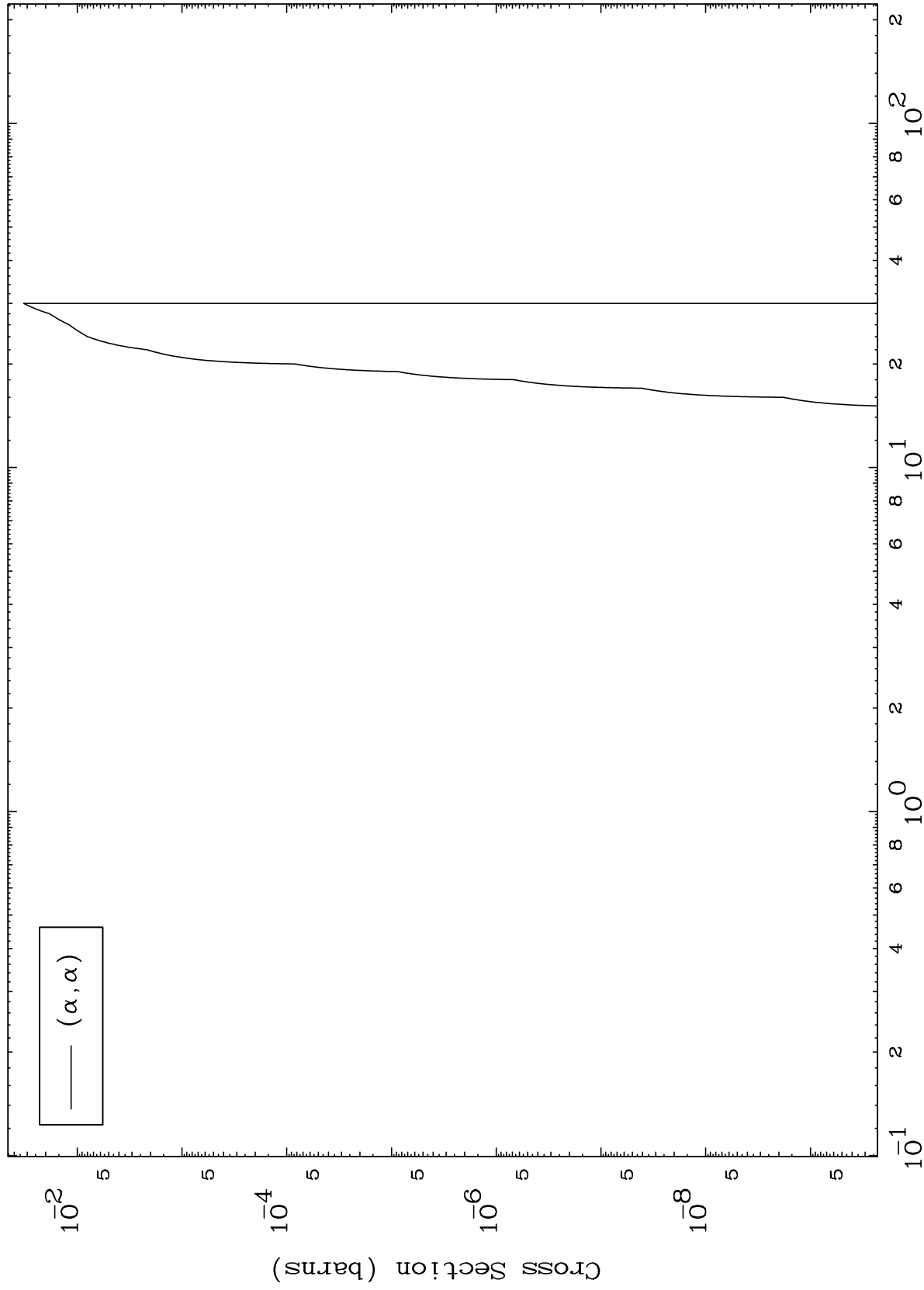


MAT 9061

( $\alpha, \alpha$ ) Levels

90-Th-239

0 Kelvin Cross Sections



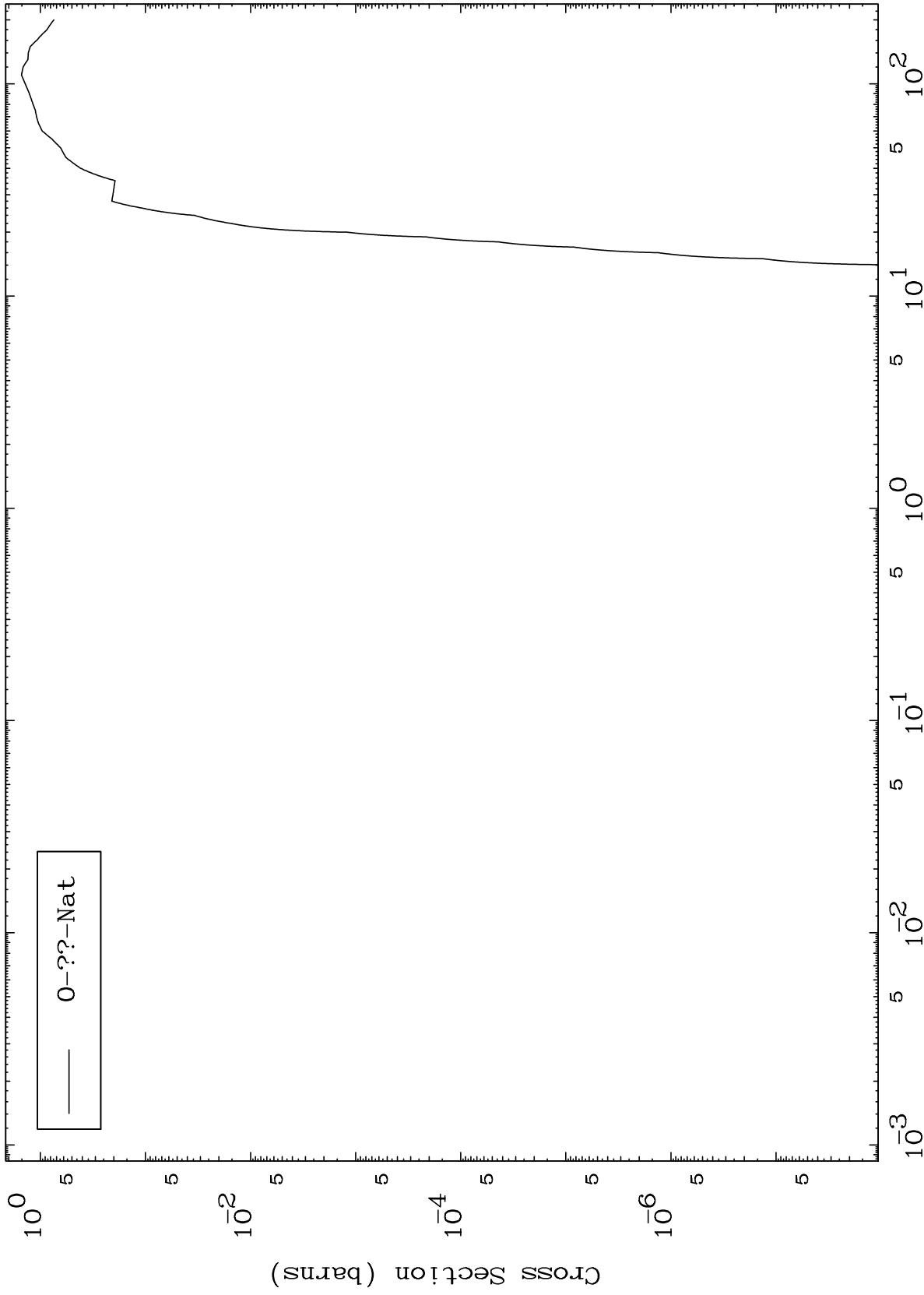
Incident Energy (MeV)

90-Th-239

MAT 9061

$\alpha$  Fission  
Radionuclide Production Cross Section

90-Th-239



Incident Energy (MeV)

90-Th-239