

Program EVALPLOT  
(Version 2018-1)

by

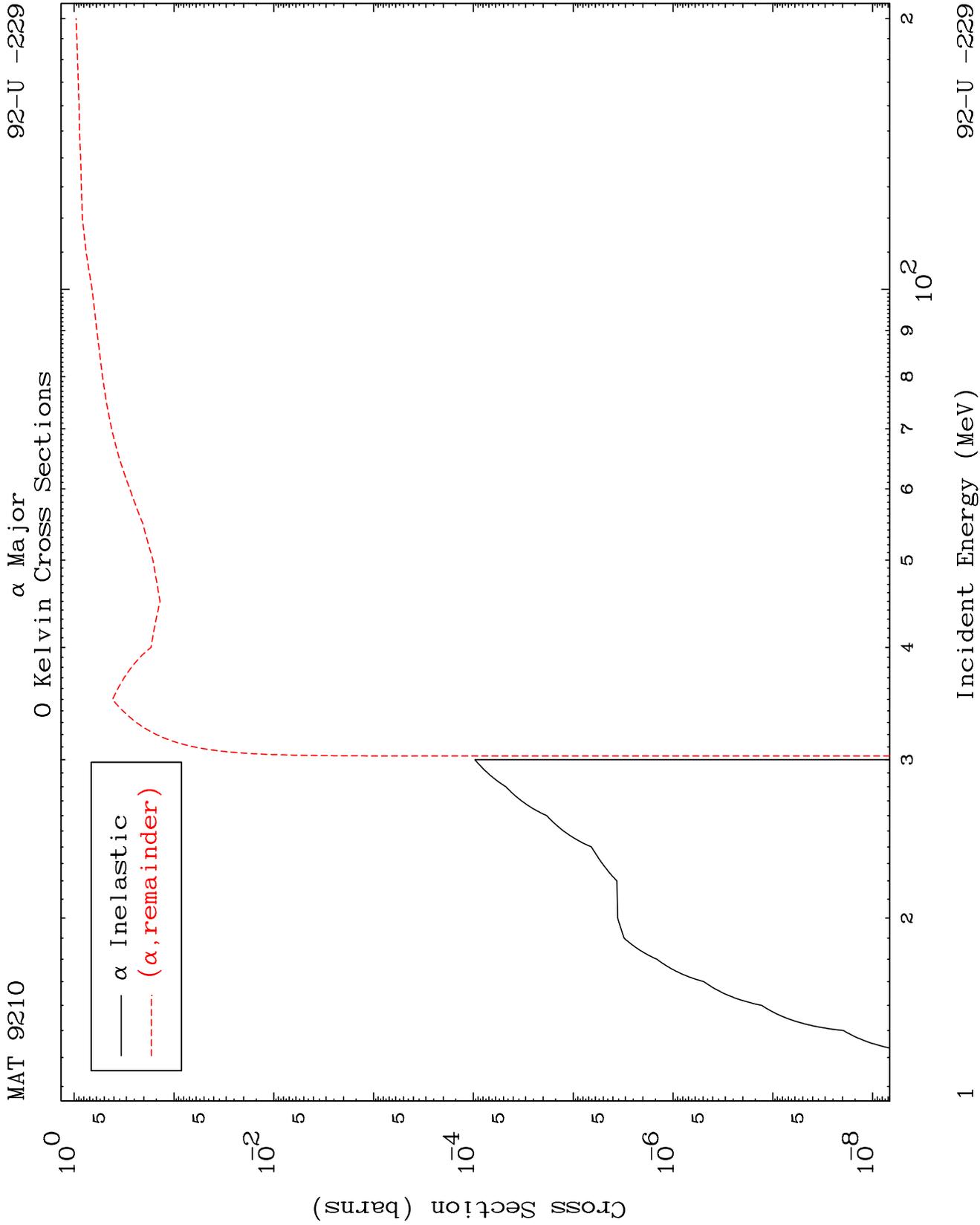
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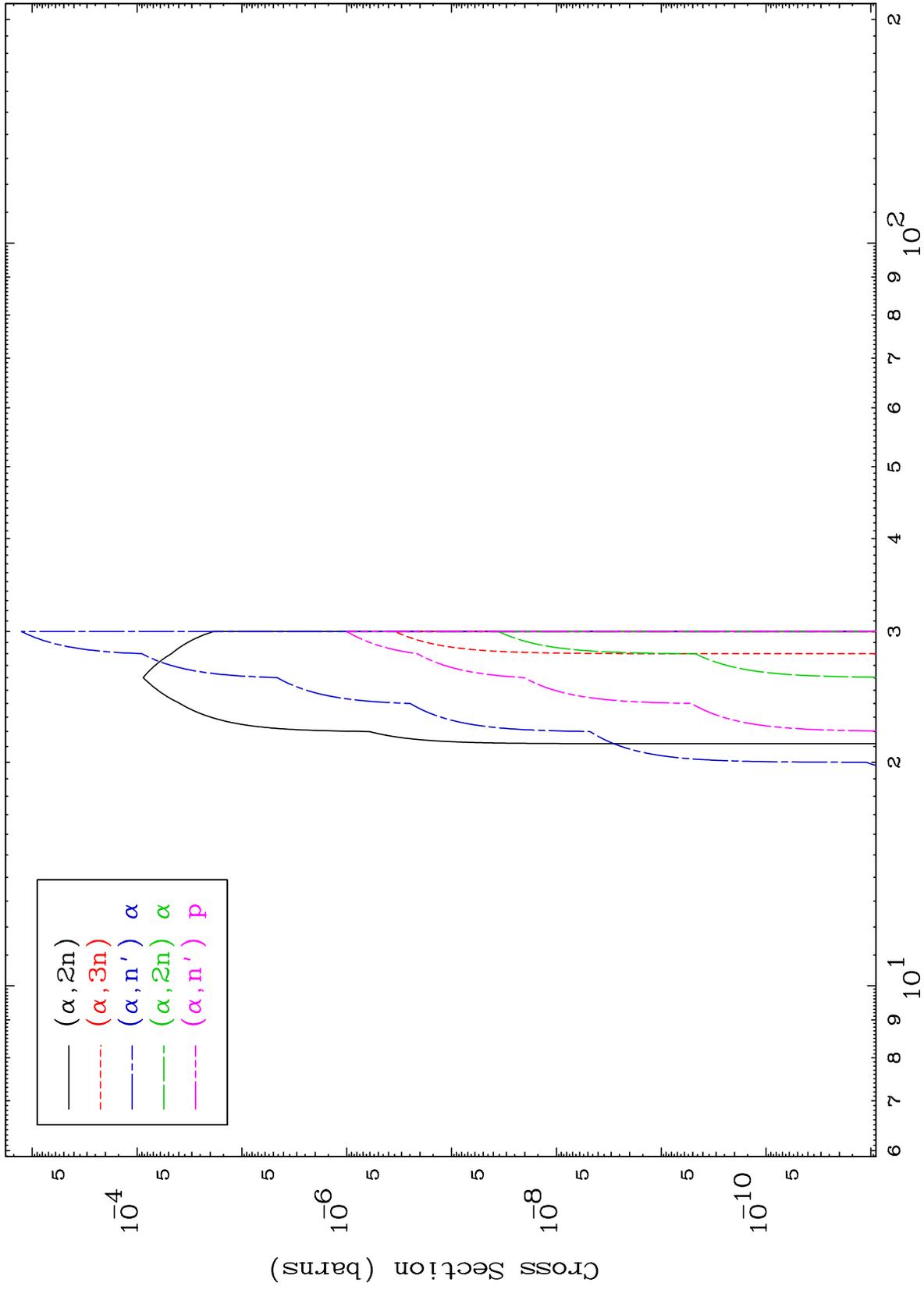
Press Mouse Button to Start



MAT 9210

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

92-U -229



2

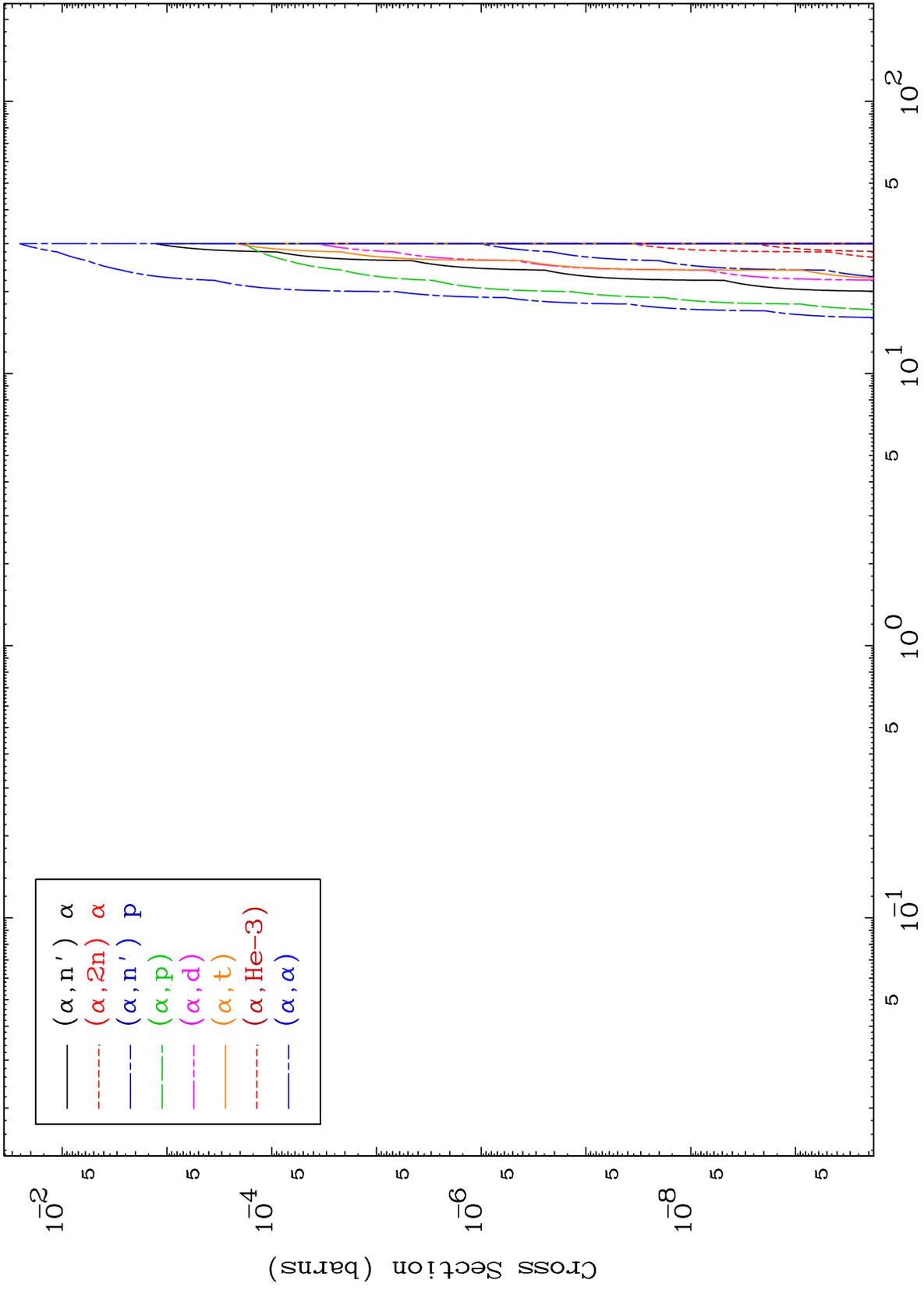
Incident Energy (MeV)

92-U -229

MAT 9210

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

92-U -229



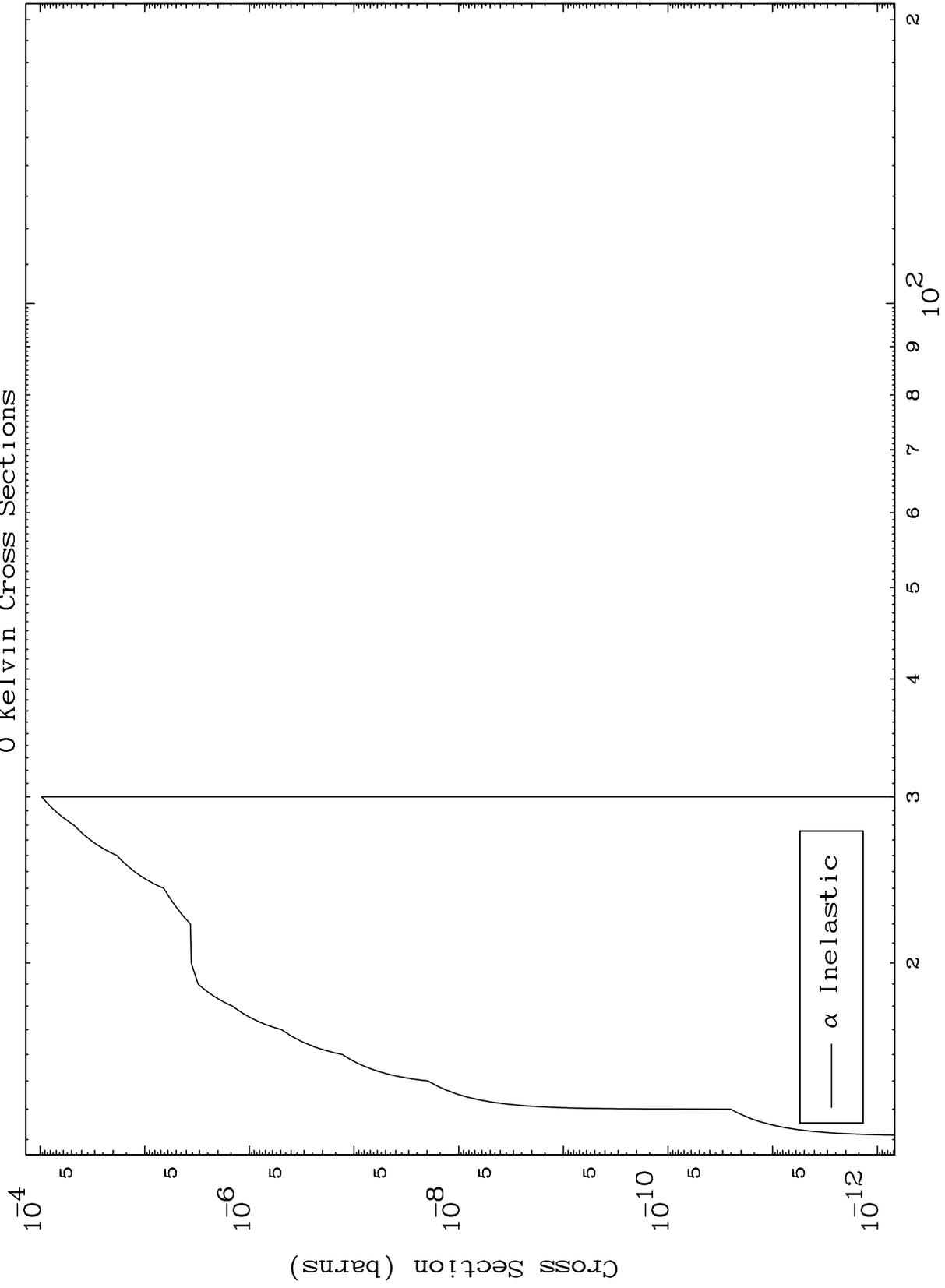
3

92-U -229

MAT 9210

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

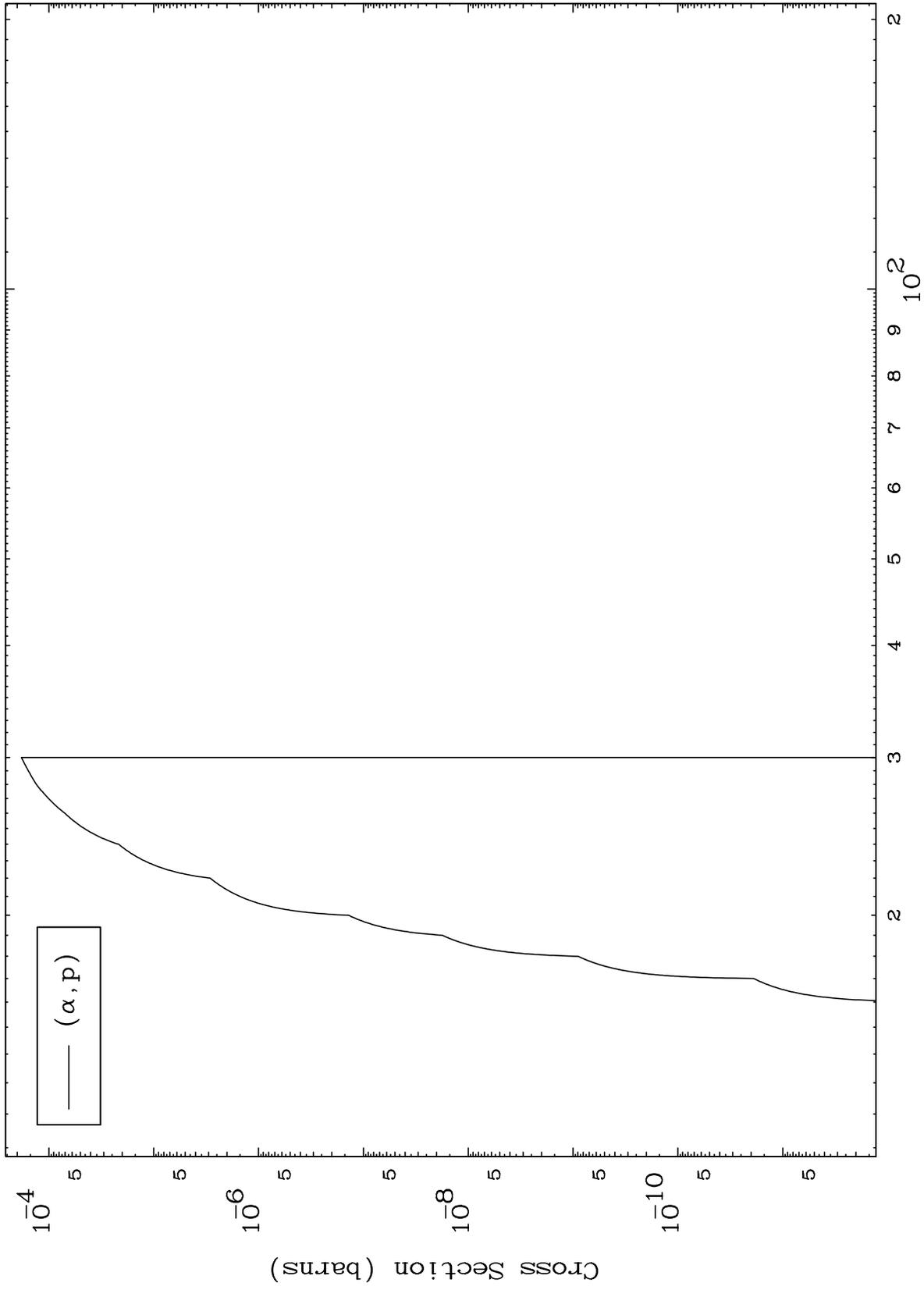
92-U -229



4

Incident Energy (MeV)

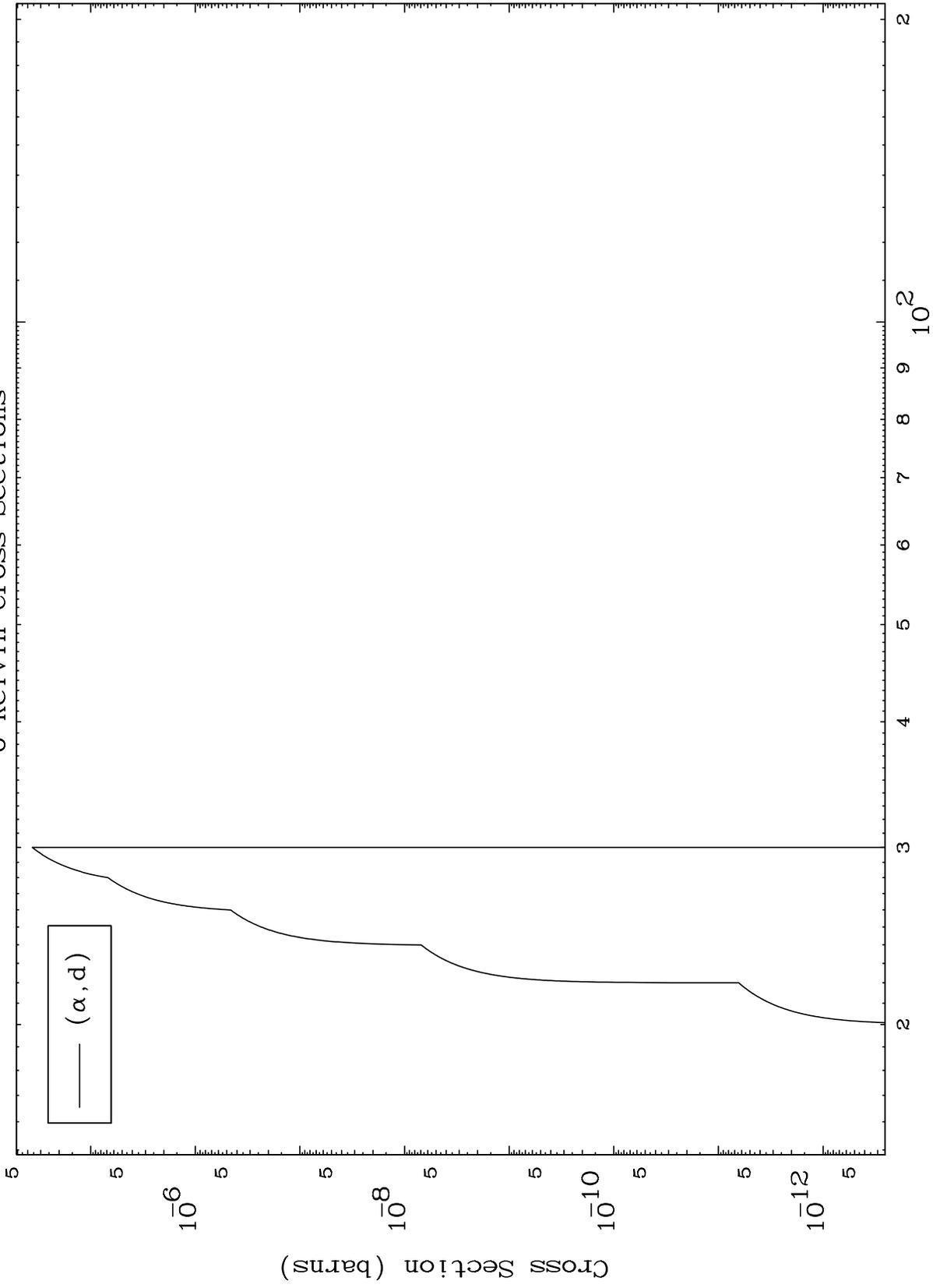
92-U -229



MAT 9210

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

92-U -229



6

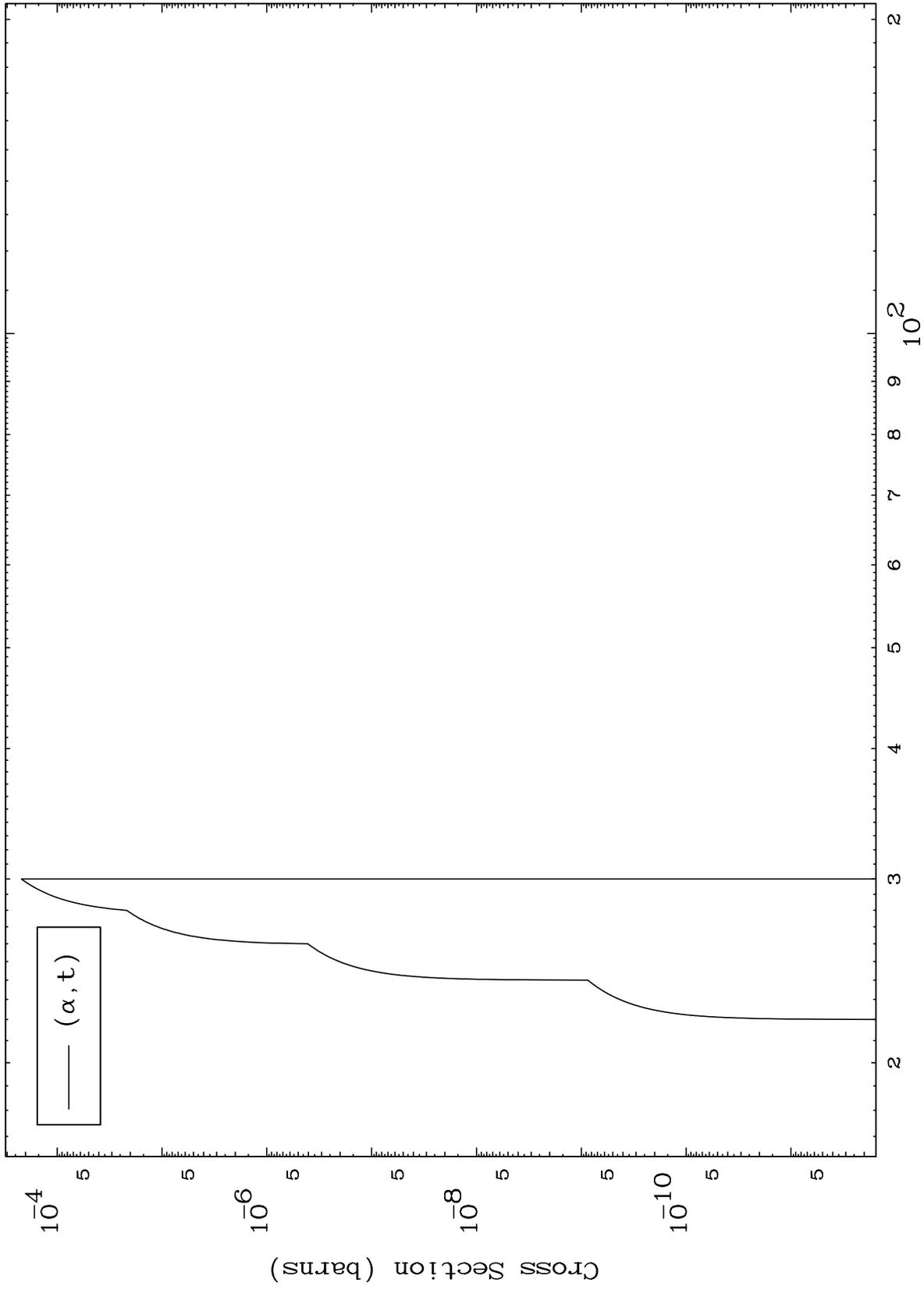
Incident Energy (MeV)

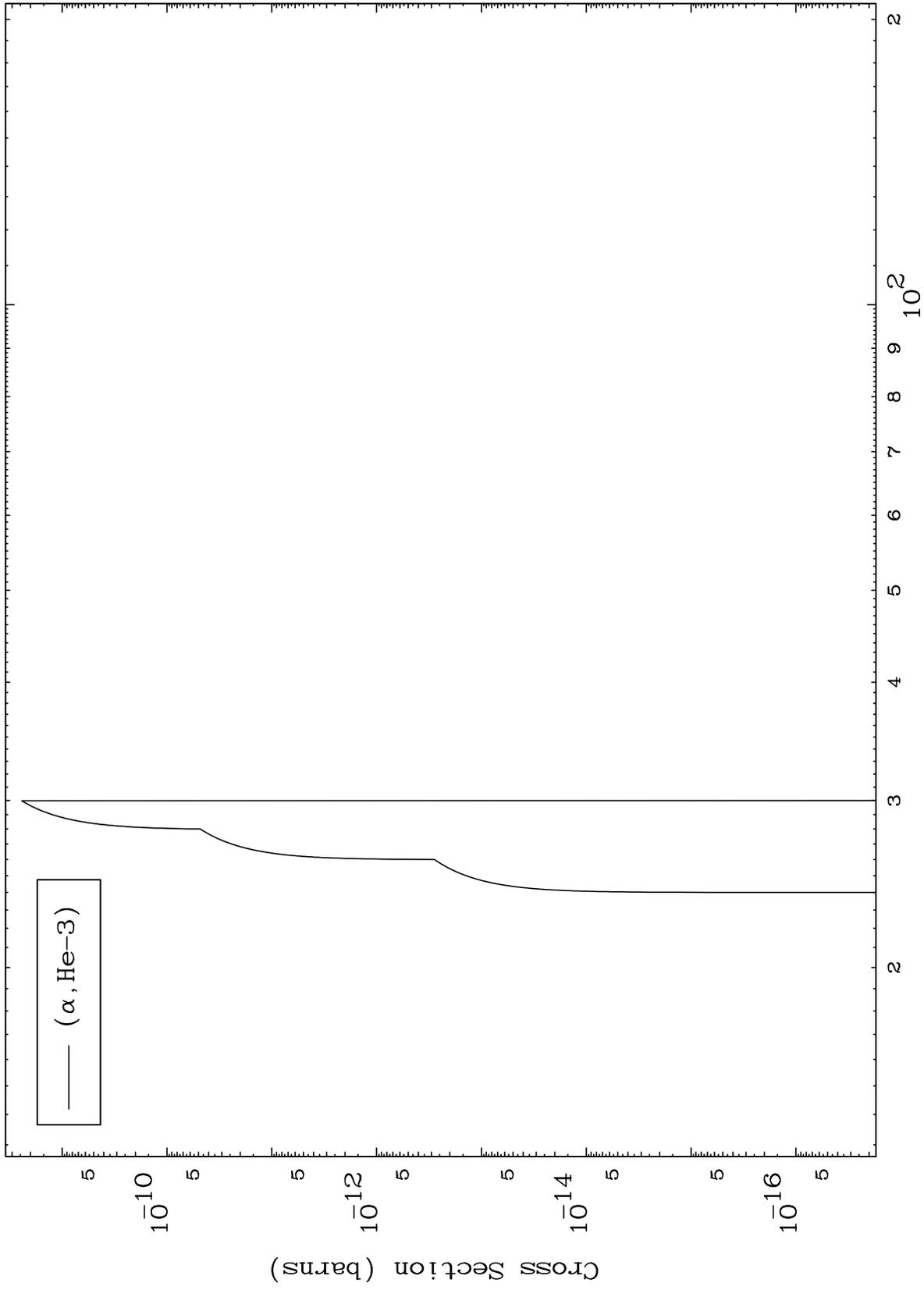
92-U -229

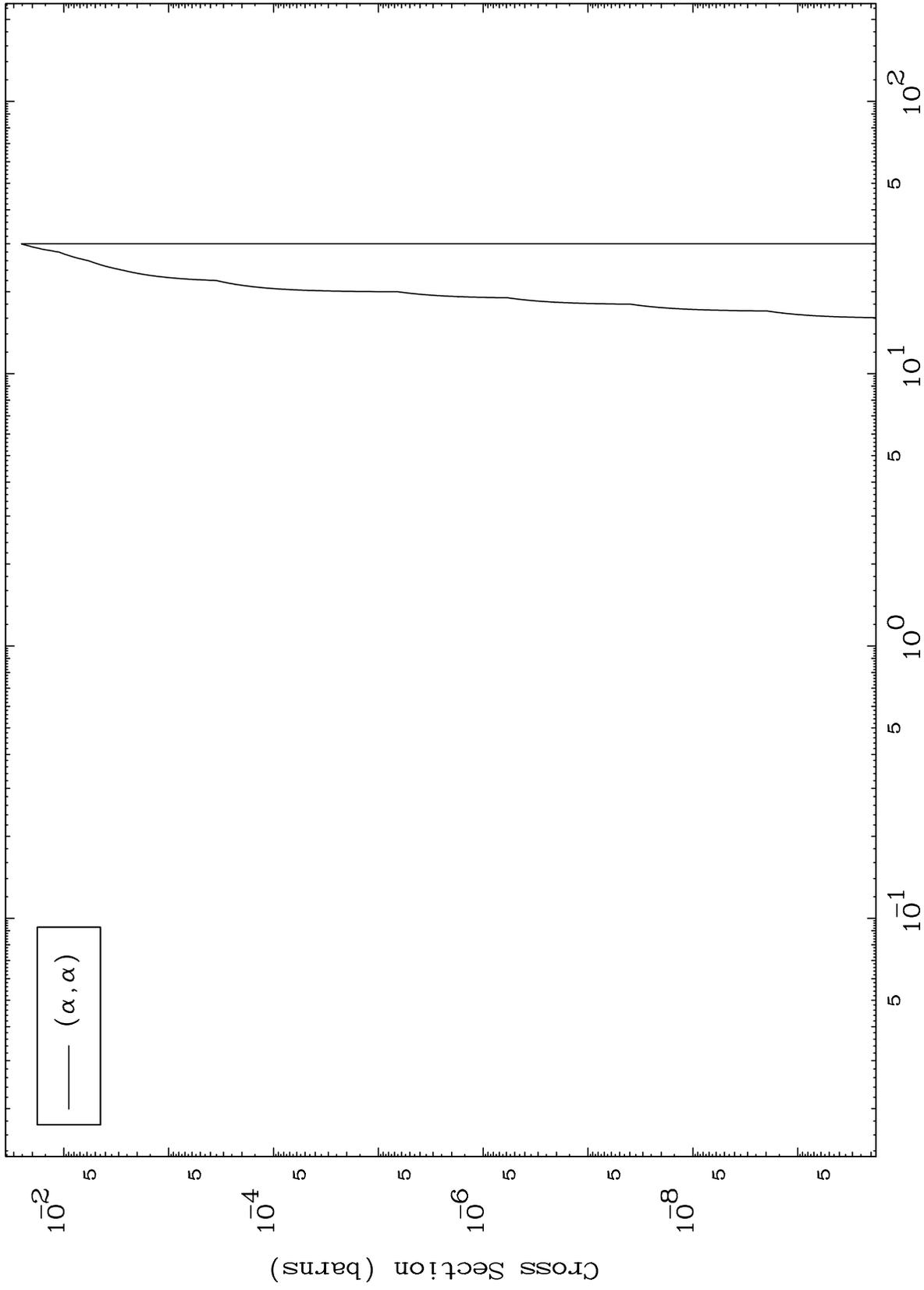
MAT 9210

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

92-U -229



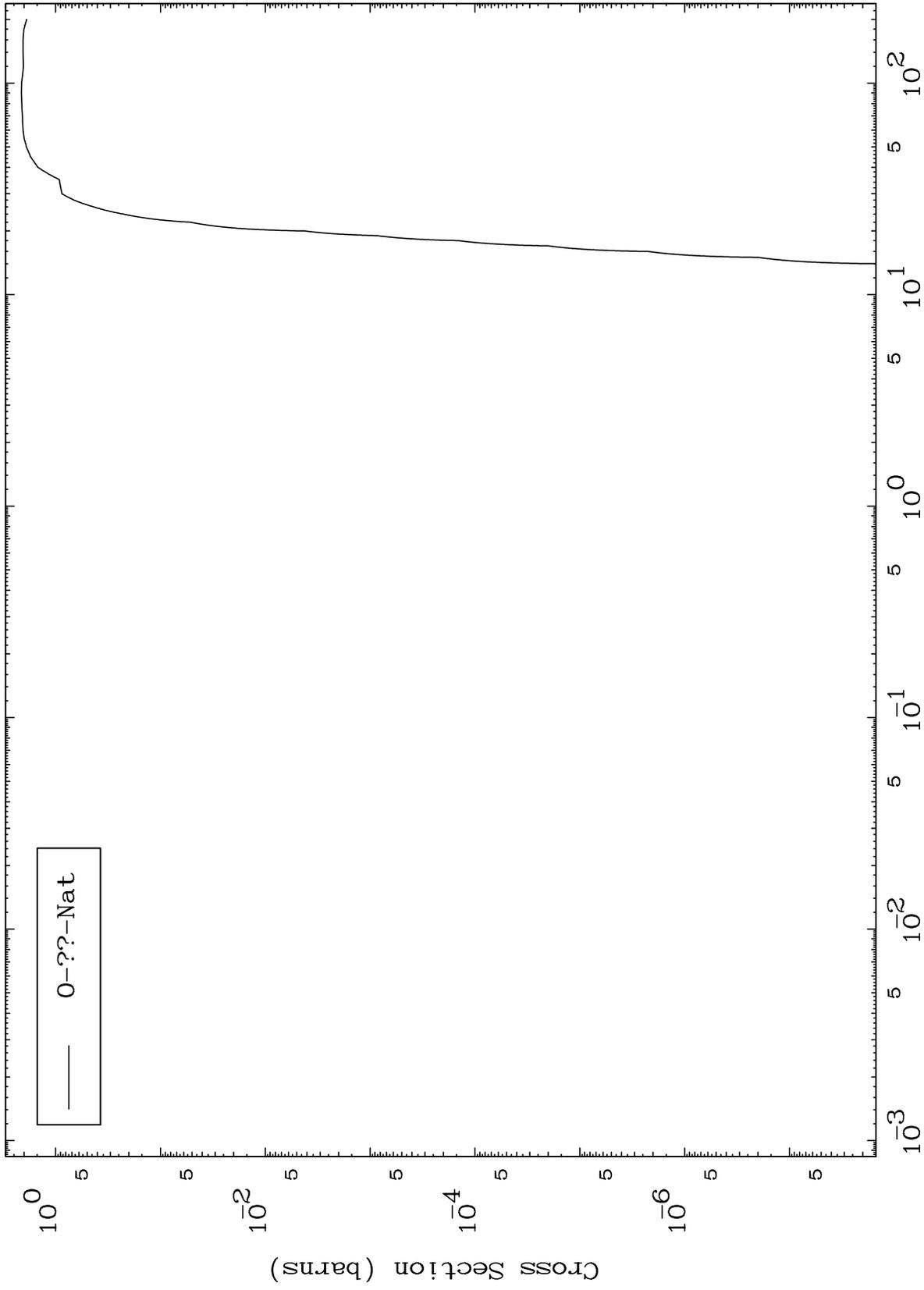




MAT 9210

92-U -229

$\alpha$  Fission  
Radionuclide Production Cross Section



Incident Energy (MeV)

92-U -229