

Program EVALPLOT  
(Version 2018-1)

by

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(Present Contact Information)

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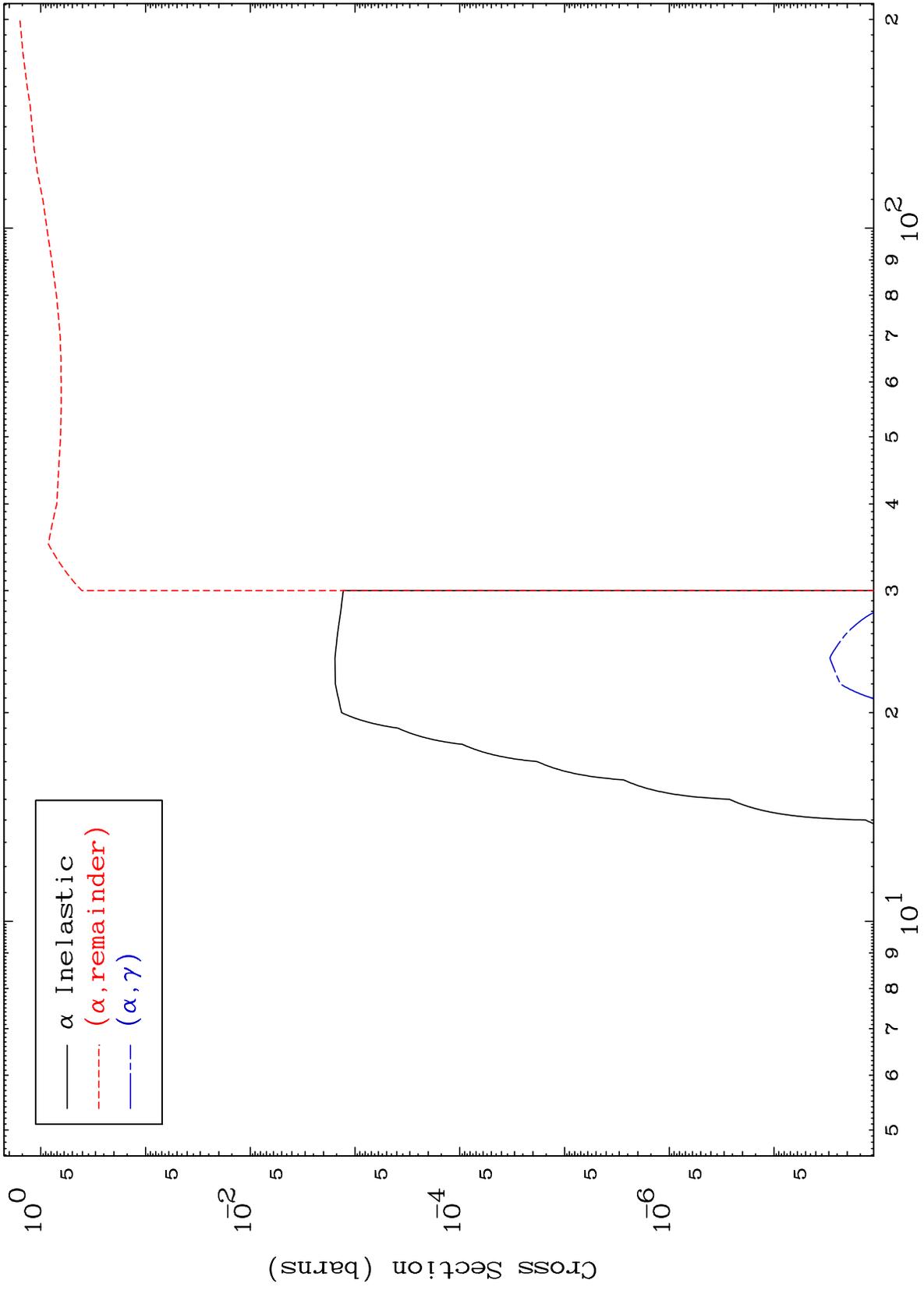
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

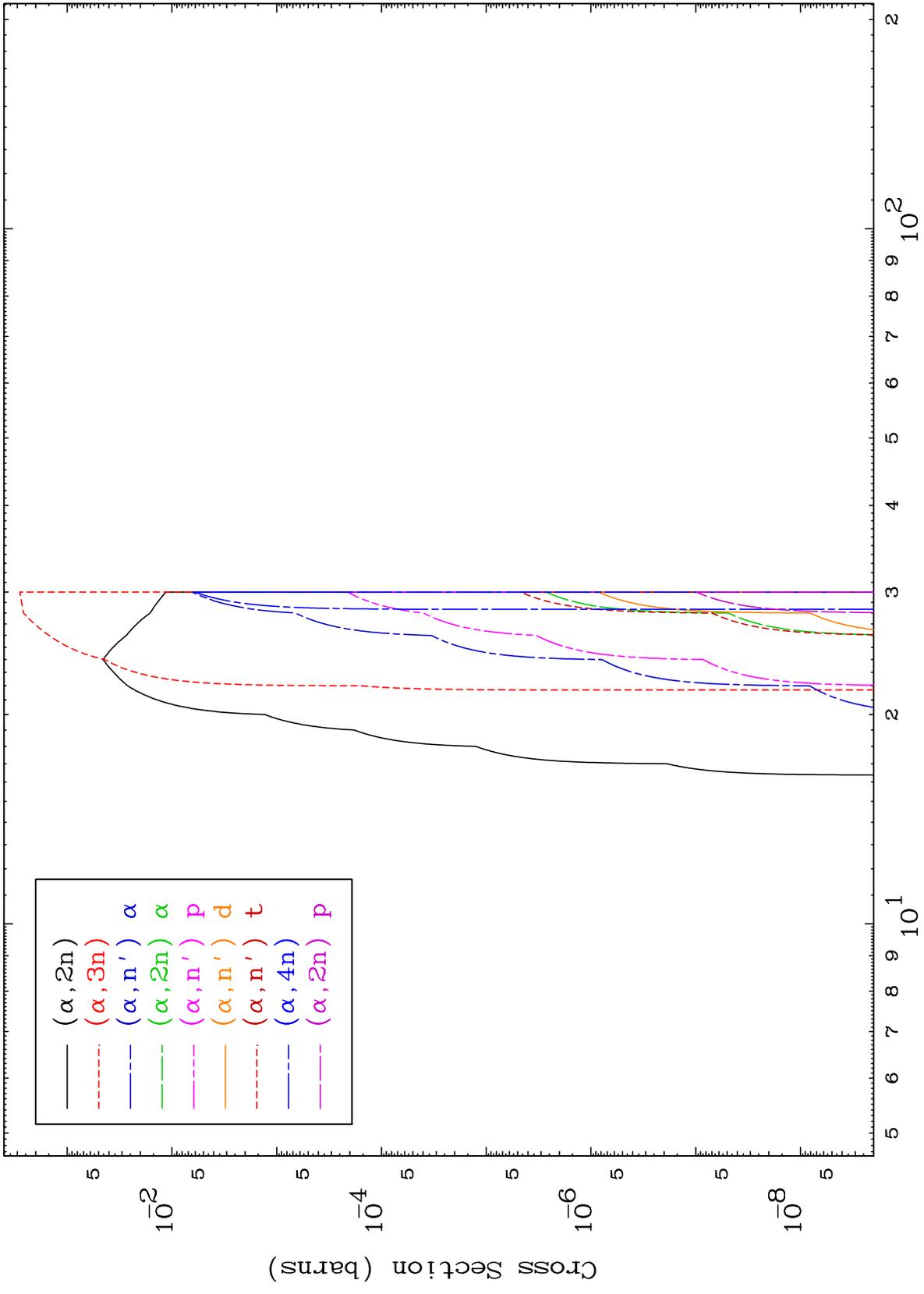
Press Mouse Button to Start

MAT 9240

$\alpha$  Major  
0 Kelvin Cross Sections

92-U -239

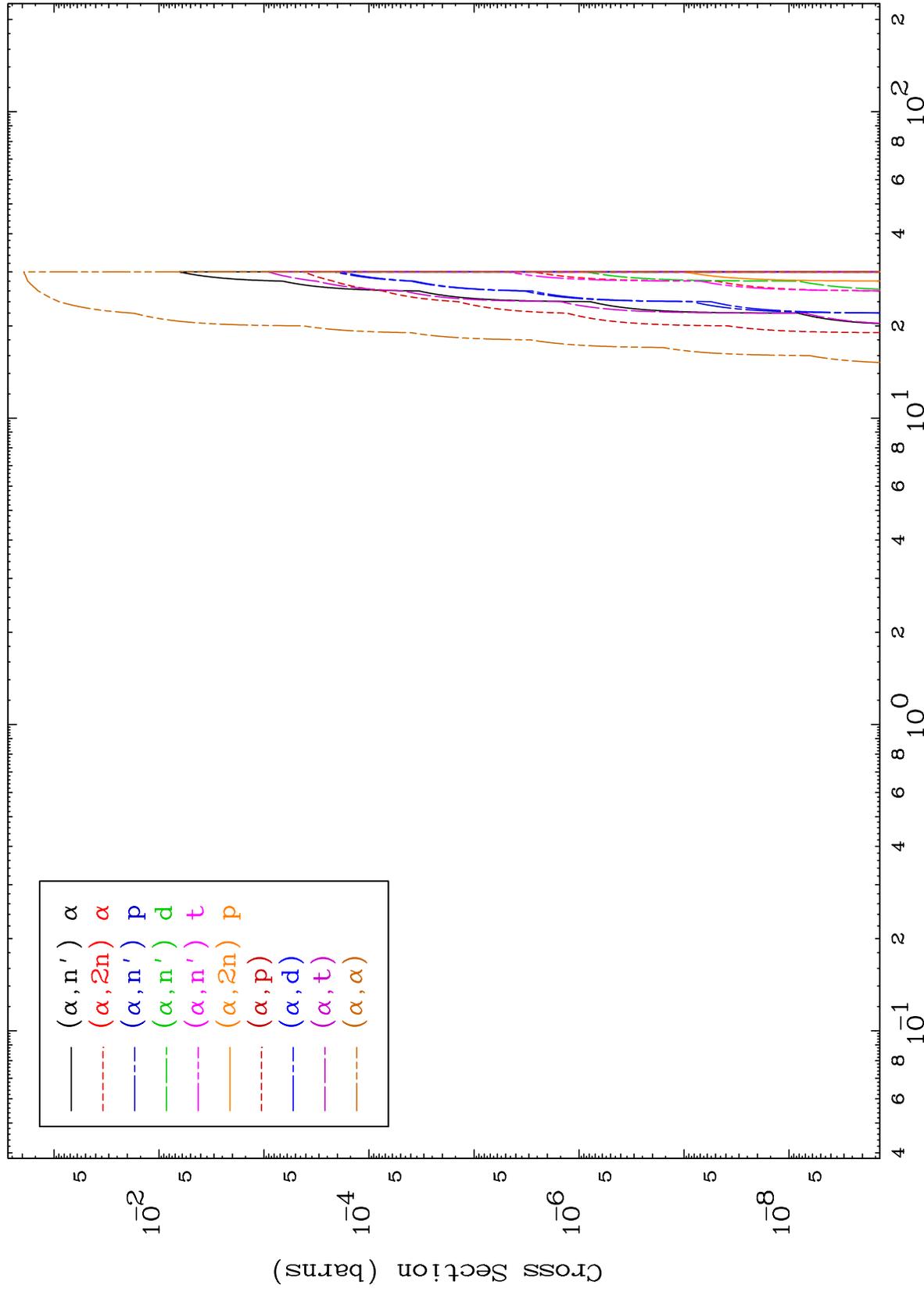




MAT 9240

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

92-U -239



3

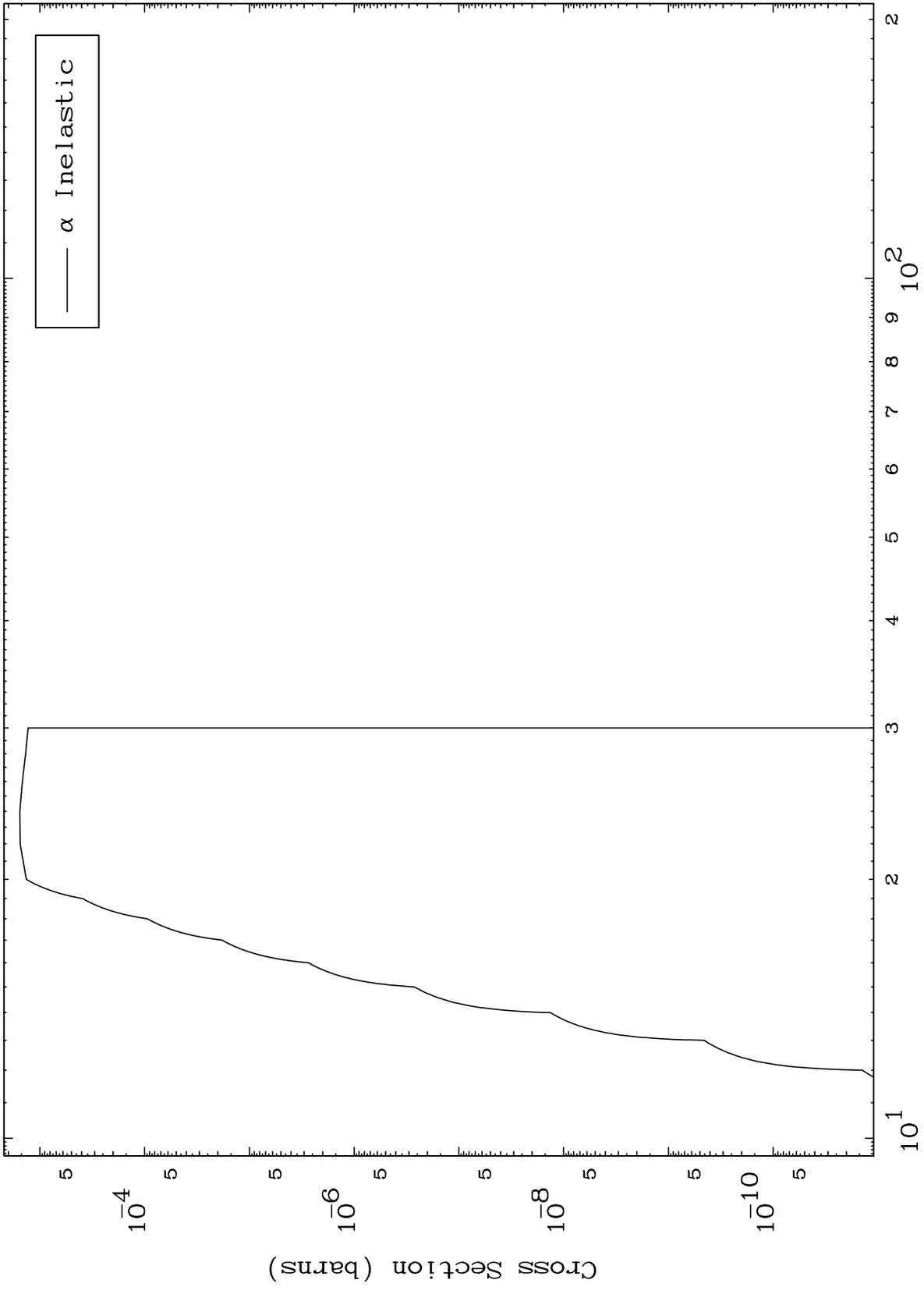
Incident Energy (MeV)

92-U -239

MAT 9240

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

92-U -239



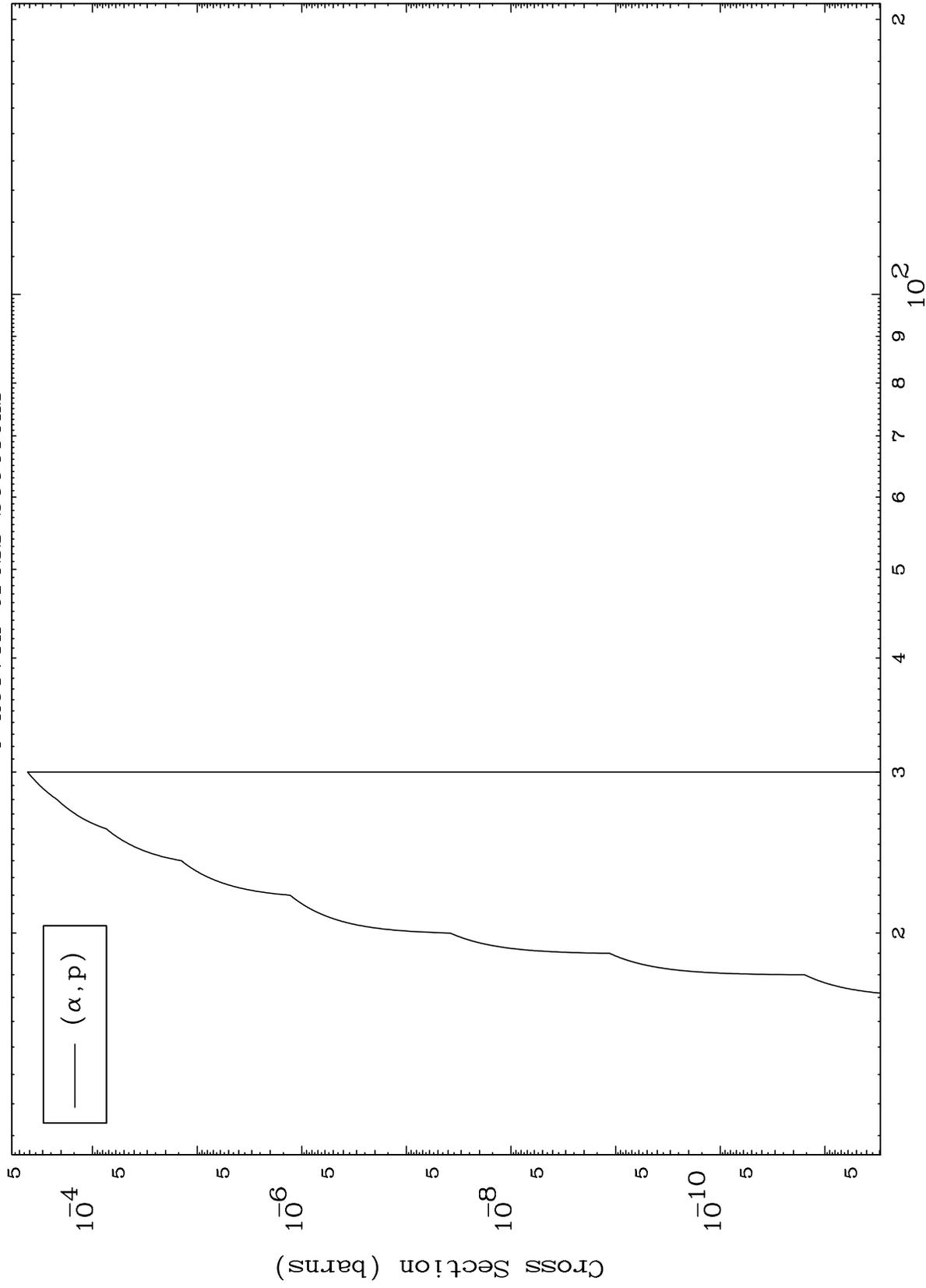
Incident Energy (MeV)

92-U -239

MAT 9240

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

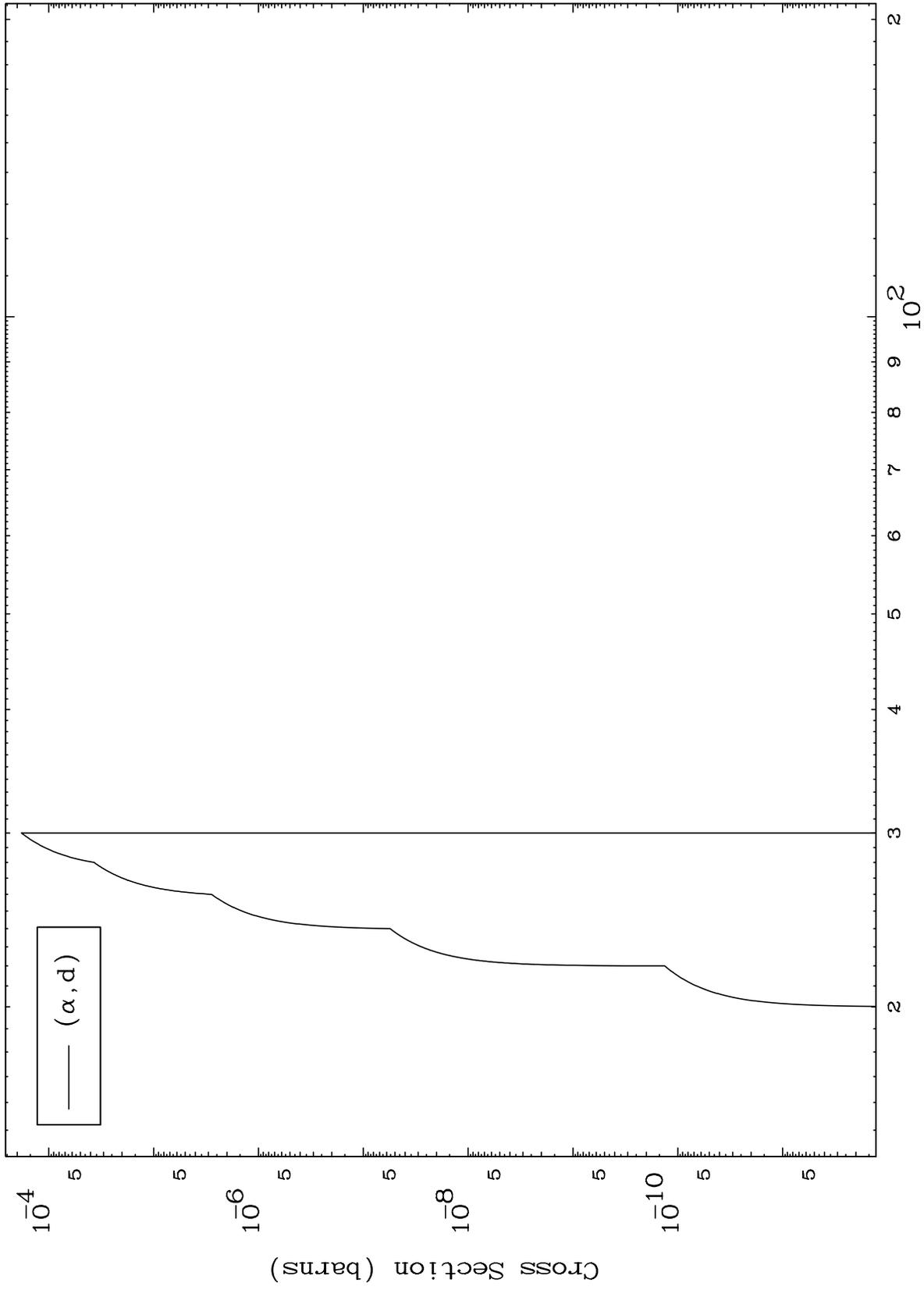
92-U -239



5

Incident Energy (MeV)

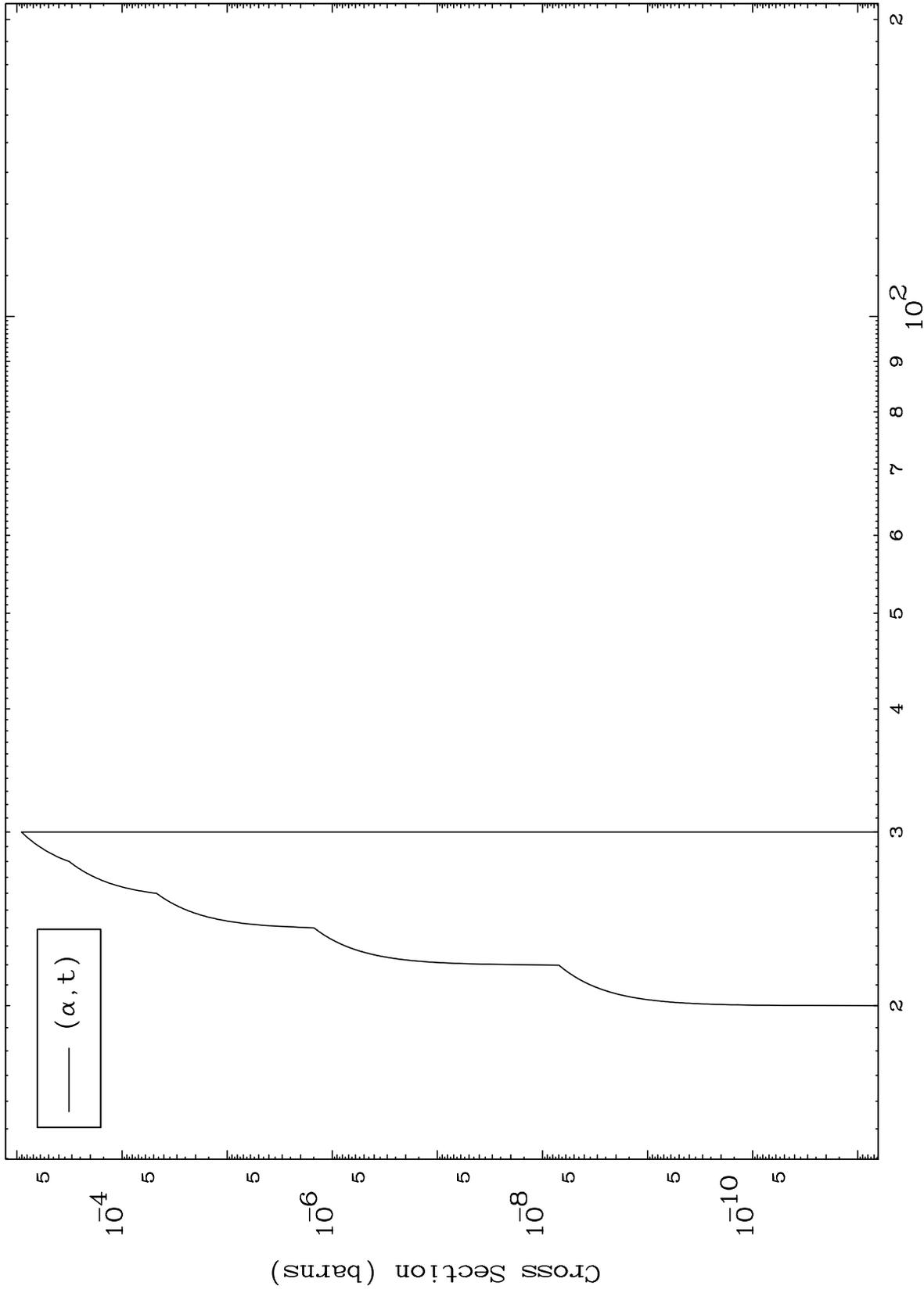
92-U -239



MAT 9240

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

92-U -239



7

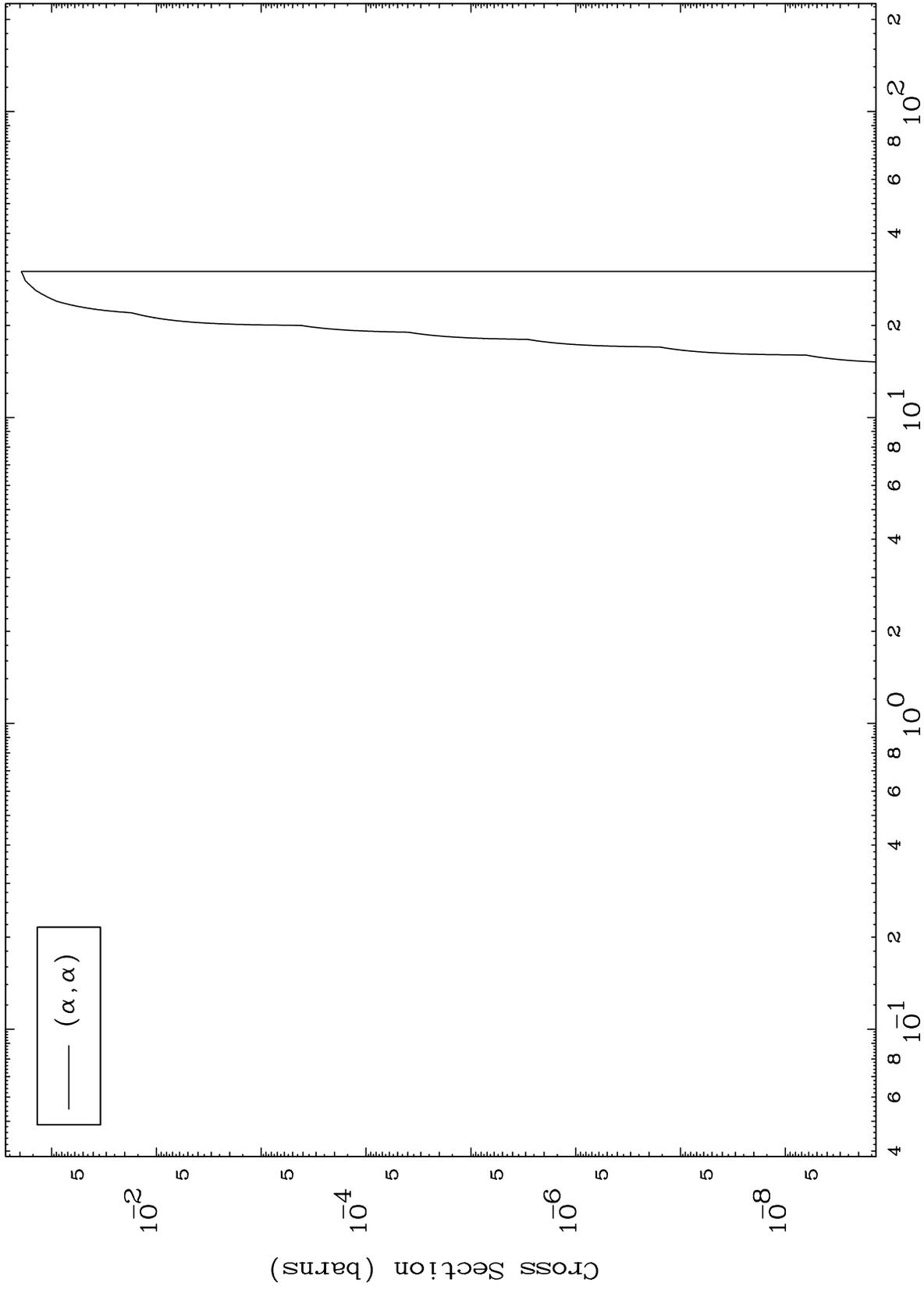
Incident Energy (MeV)

92-U -239

MAT 9240

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

92-U -239



8

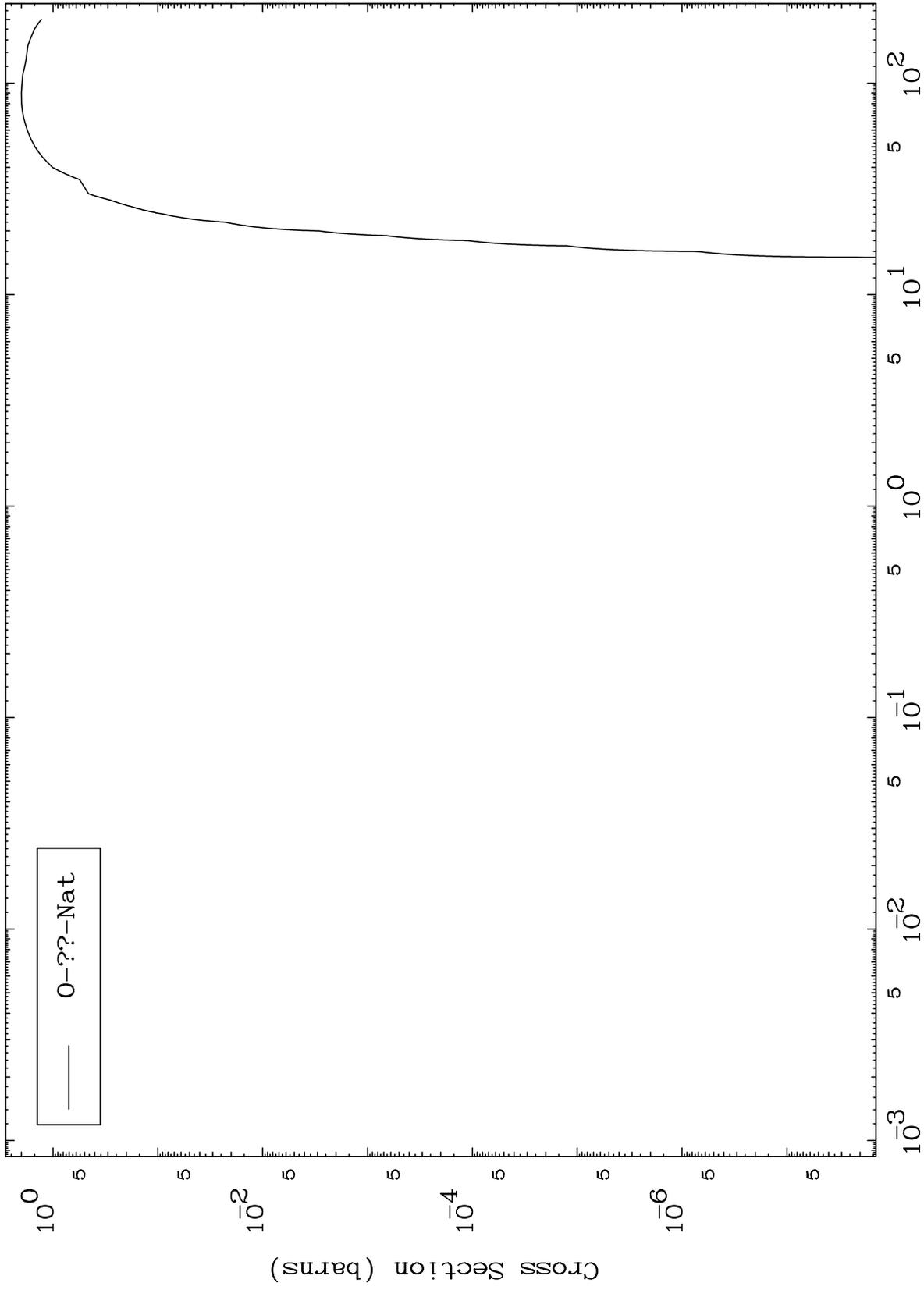
Incident Energy (MeV)

92-U -239

MAT 9240

92-U -239

# $\alpha$ Fission Radionuclide Production Cross Section



92-U -239

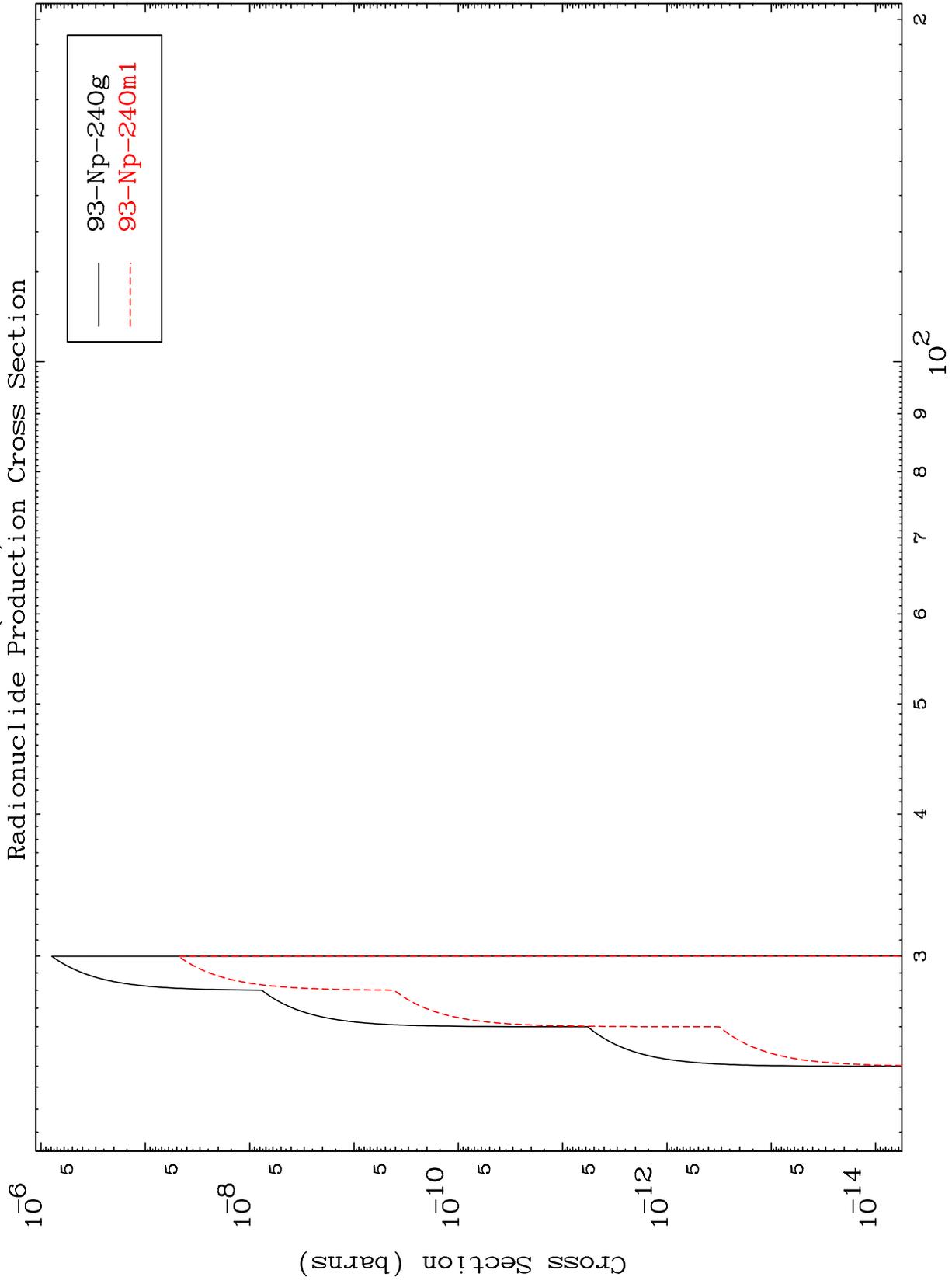
Incident Energy (MeV)

9

MAT 9240

( $\alpha, n'$ ) d

92-U -239



10

Incident Energy (MeV)

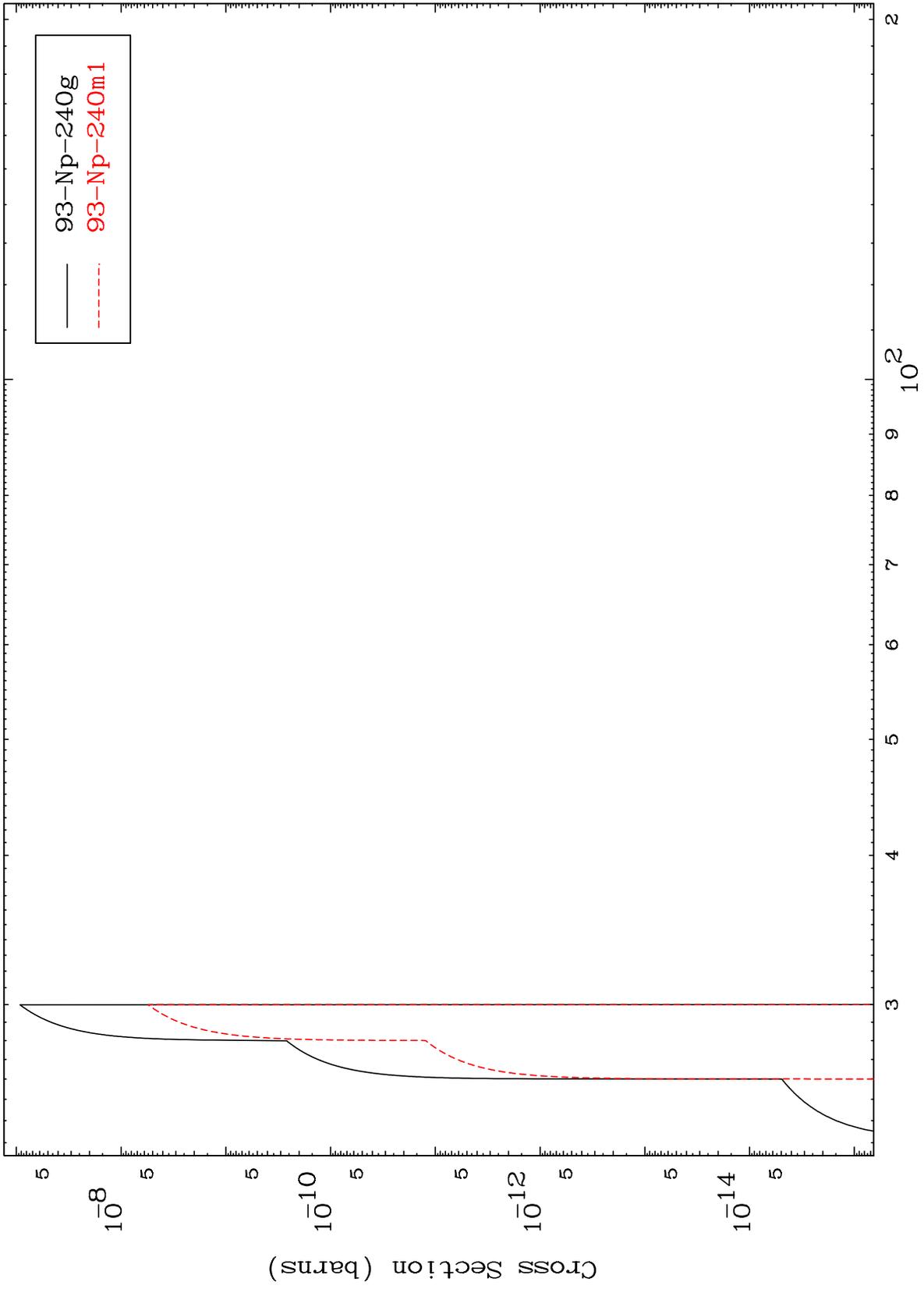
92-U -239

MAT 9240

( $\alpha, 2n$ ) p

92-U -239

Radionuclide Production Cross Section



11

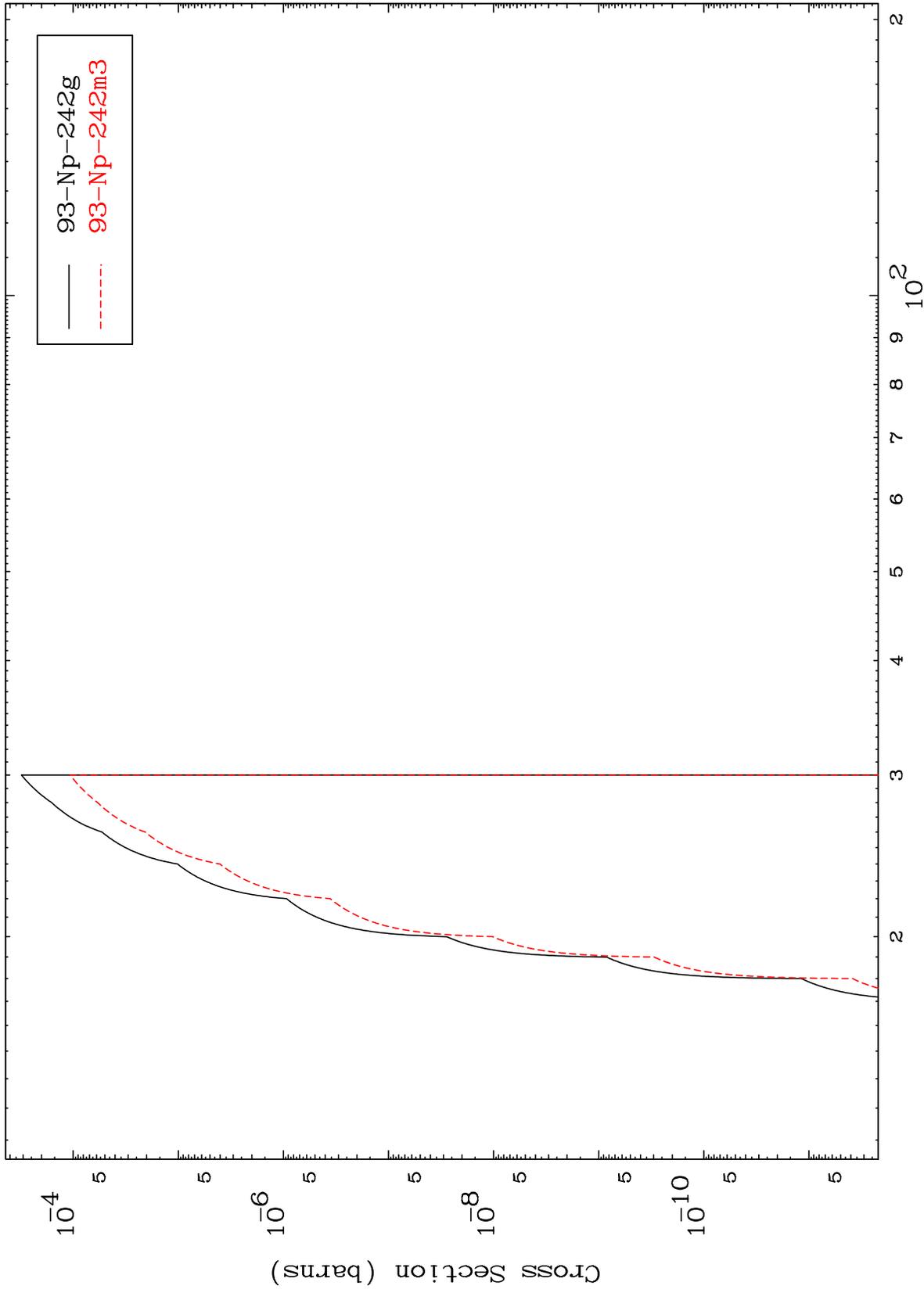
Incident Energy (MeV)

92-U -239

MAT 9240

92-U -239

( $\alpha, p$ )  
Radionuclide Production Cross Section



12

92-U -239

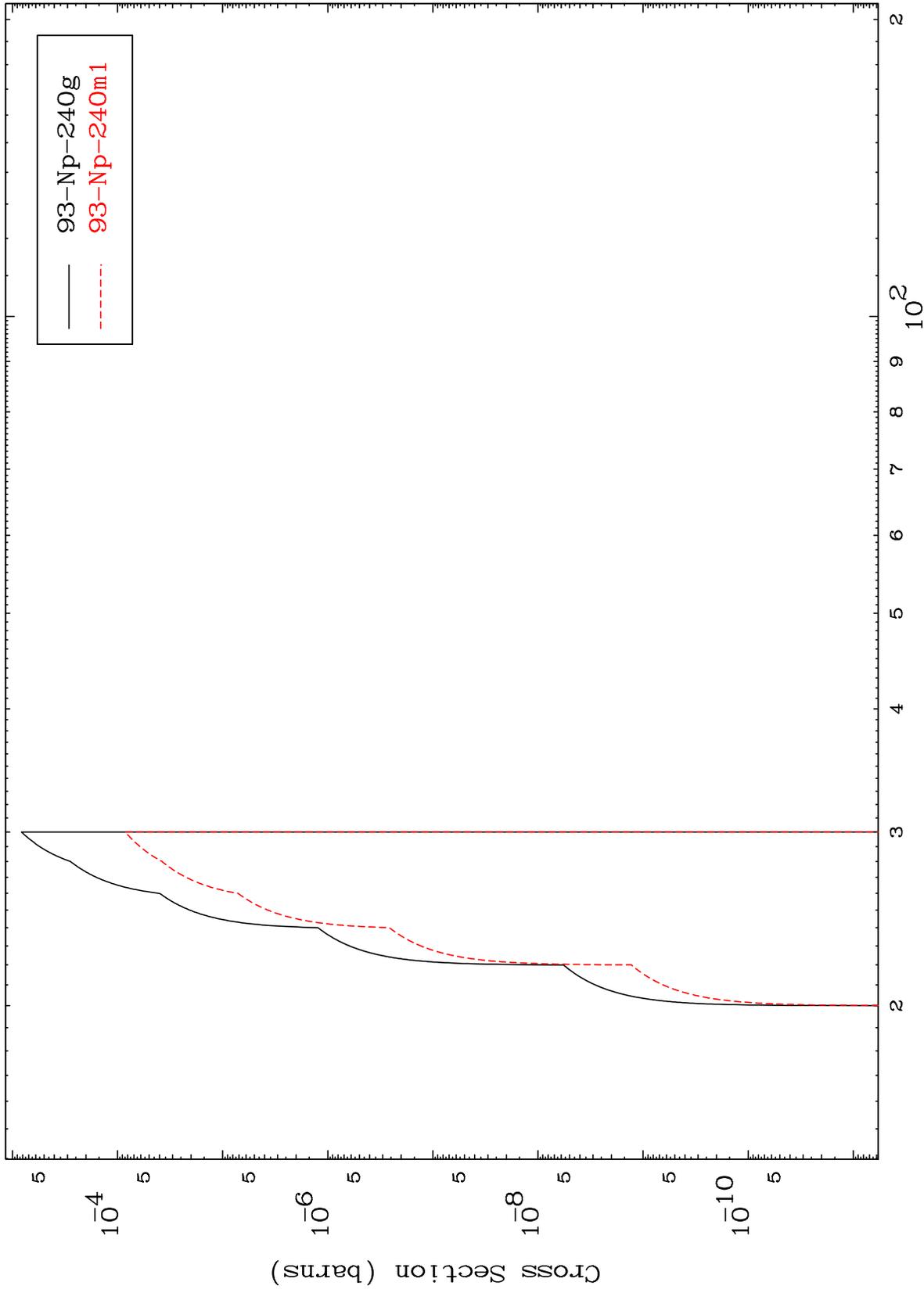
Incident Energy (MeV)

MAT 9240

( $\alpha, t$ )

92-U -239

Radionuclide Production Cross Section



13

Incident Energy (MeV)

92-U -239