

Program EVALPLOT  
(Version 2018-1)

by

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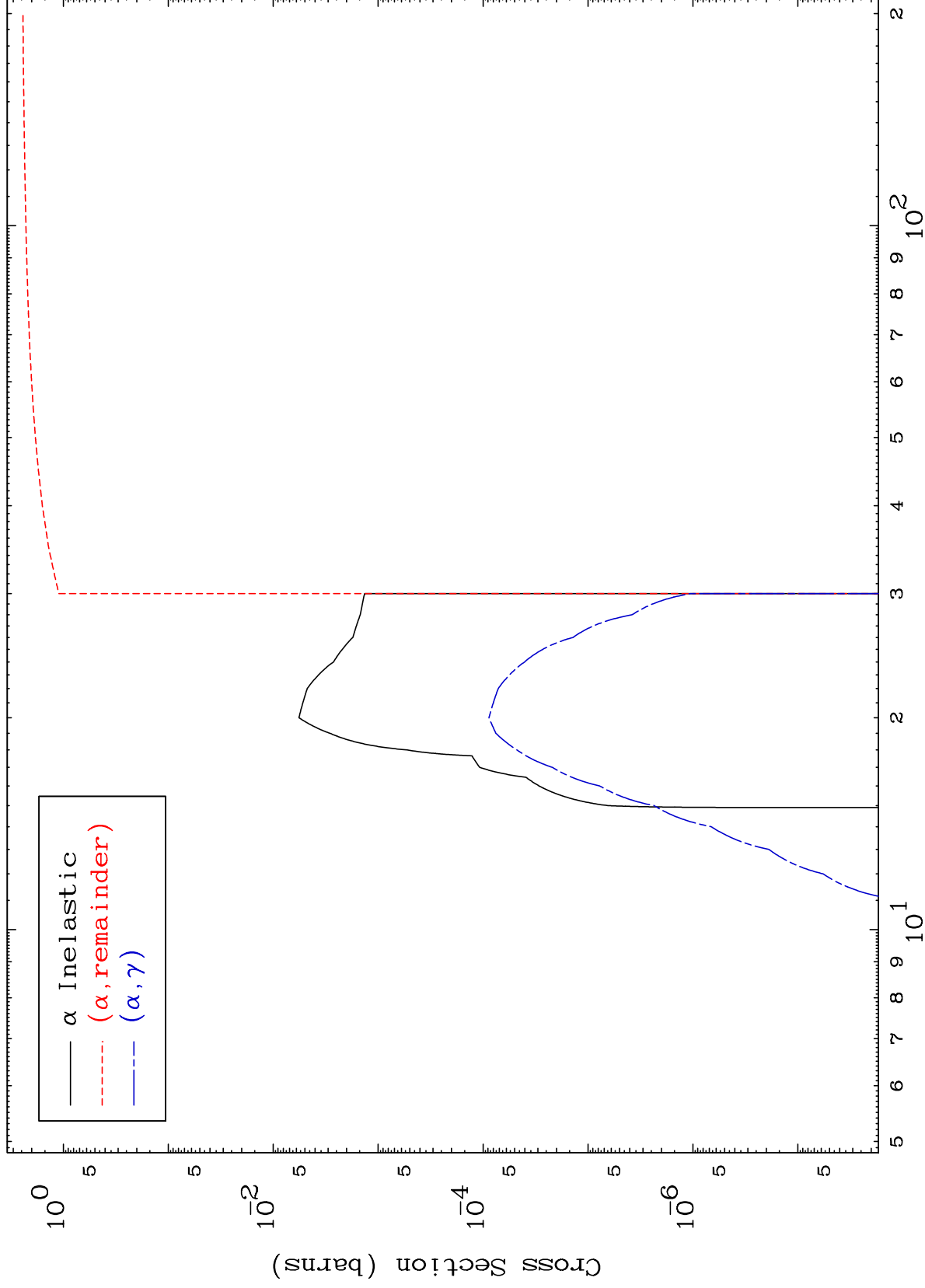
Press Mouse Button to Start

MAT 6974

$\alpha$  Major

0 Kelvin Cross Sections

70-Yb-151



1

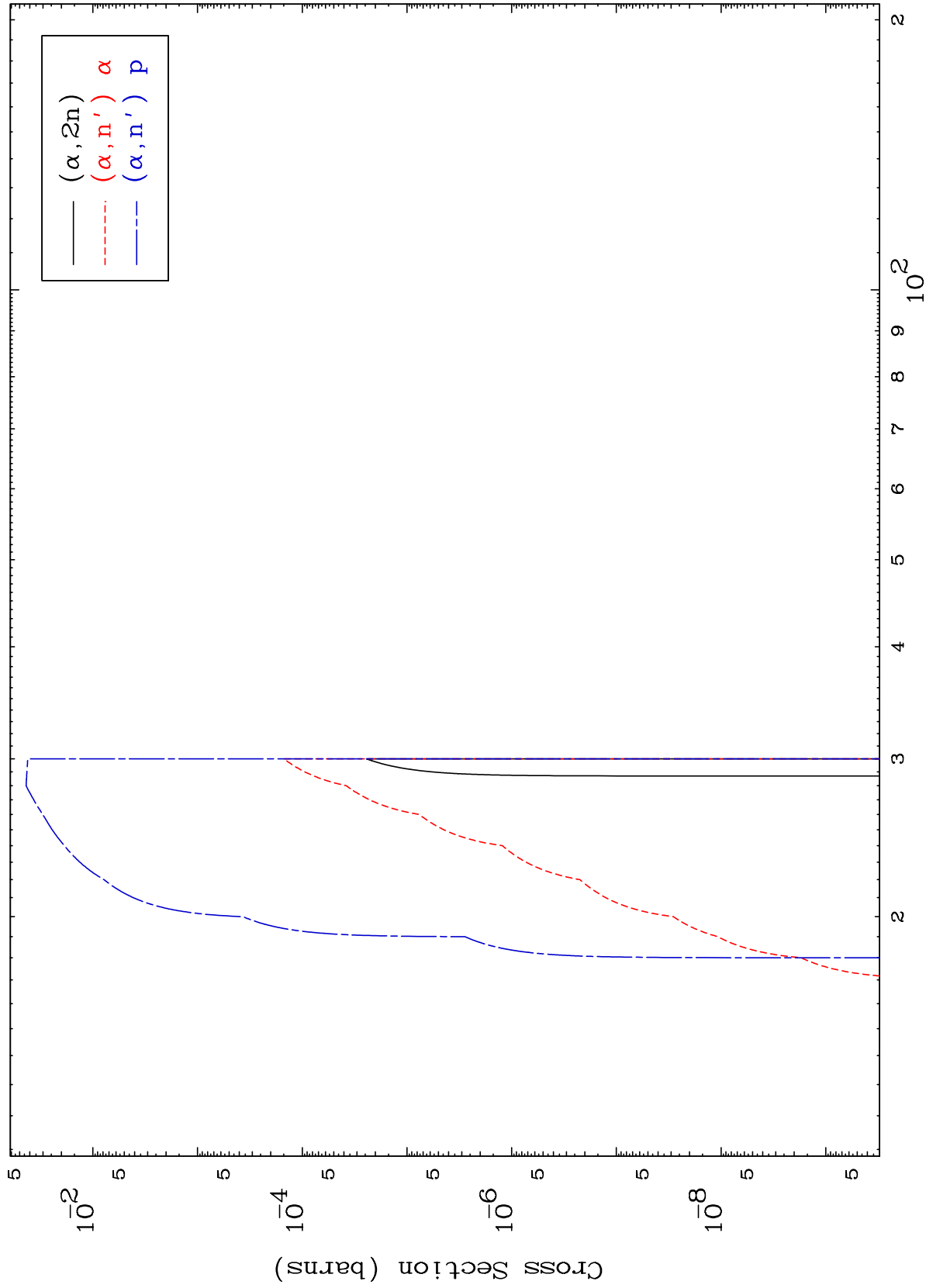
Incident Energy (MeV)

70-Yb-151

MAT 6974

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

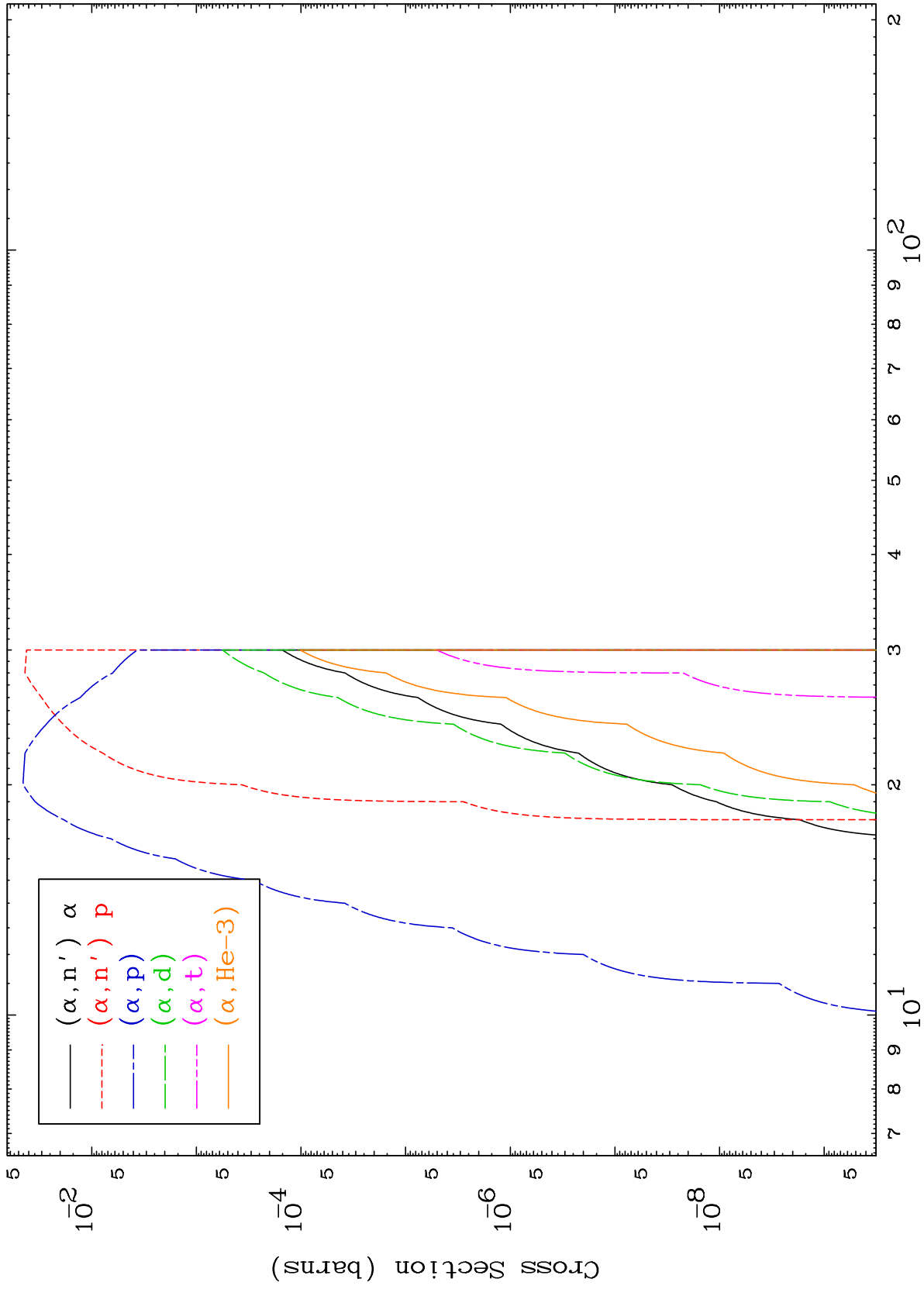
70-Yb-151

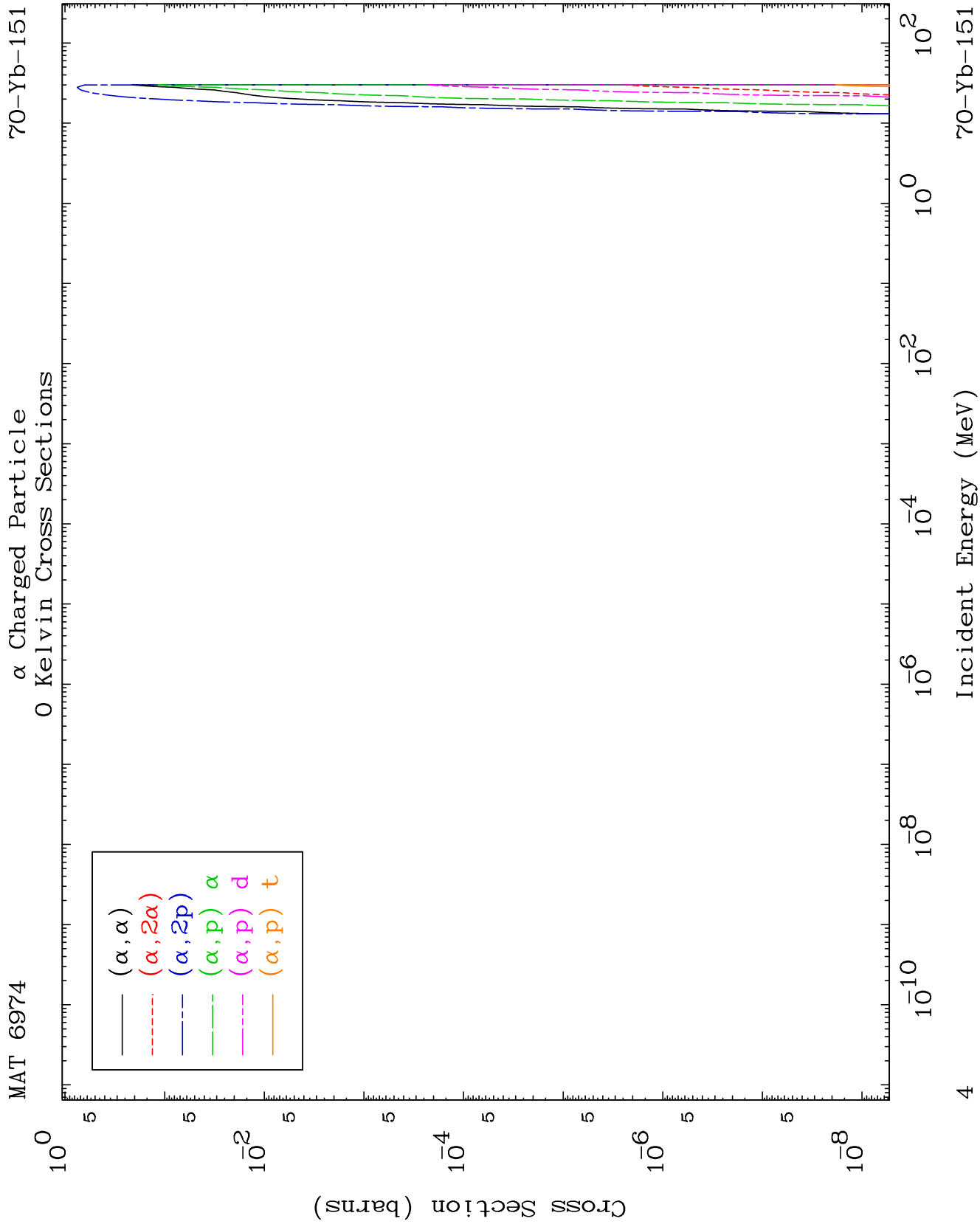


2

Incident Energy (MeV)

70-Yb-151

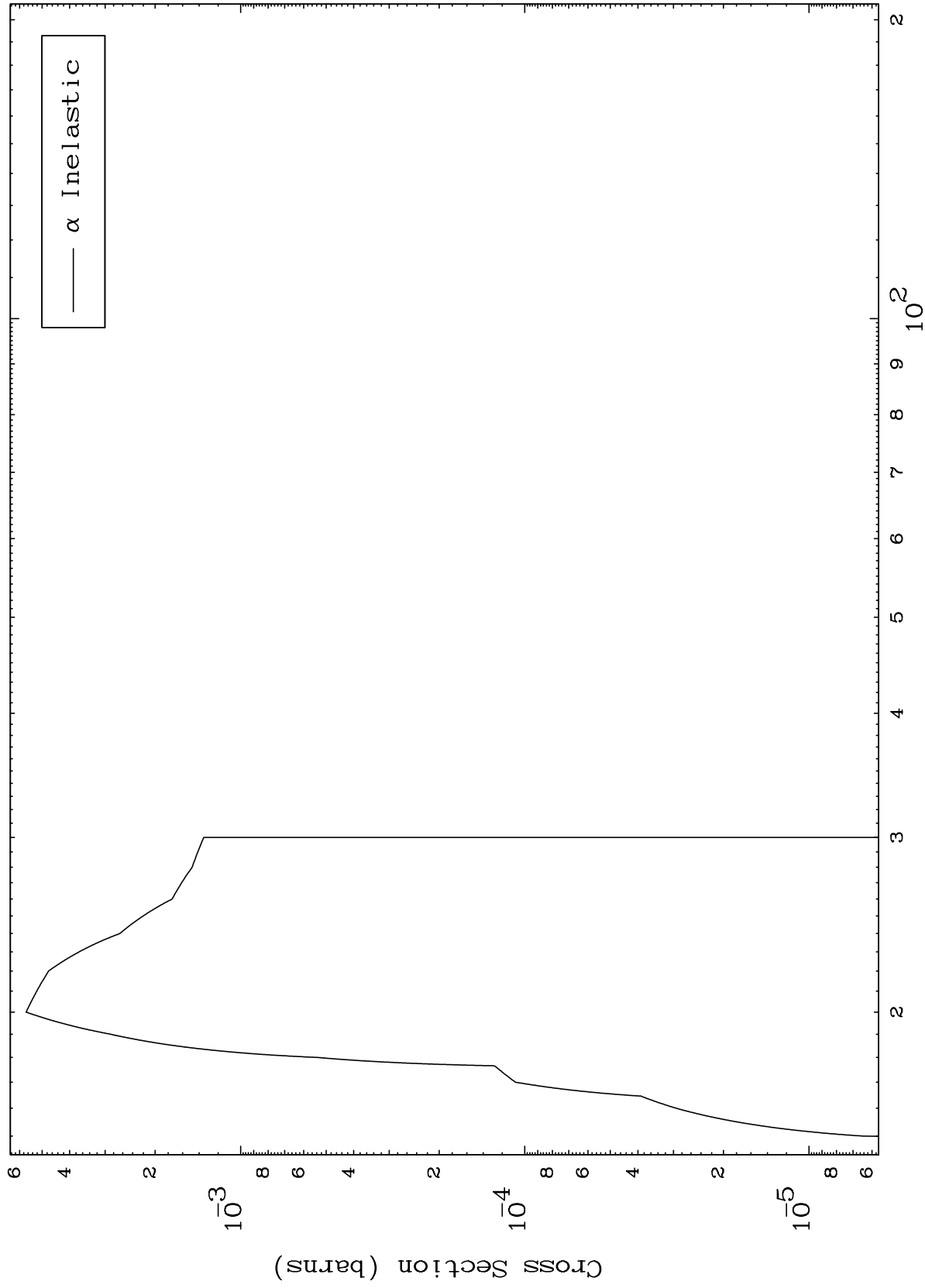




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( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

70-Yb-151



5

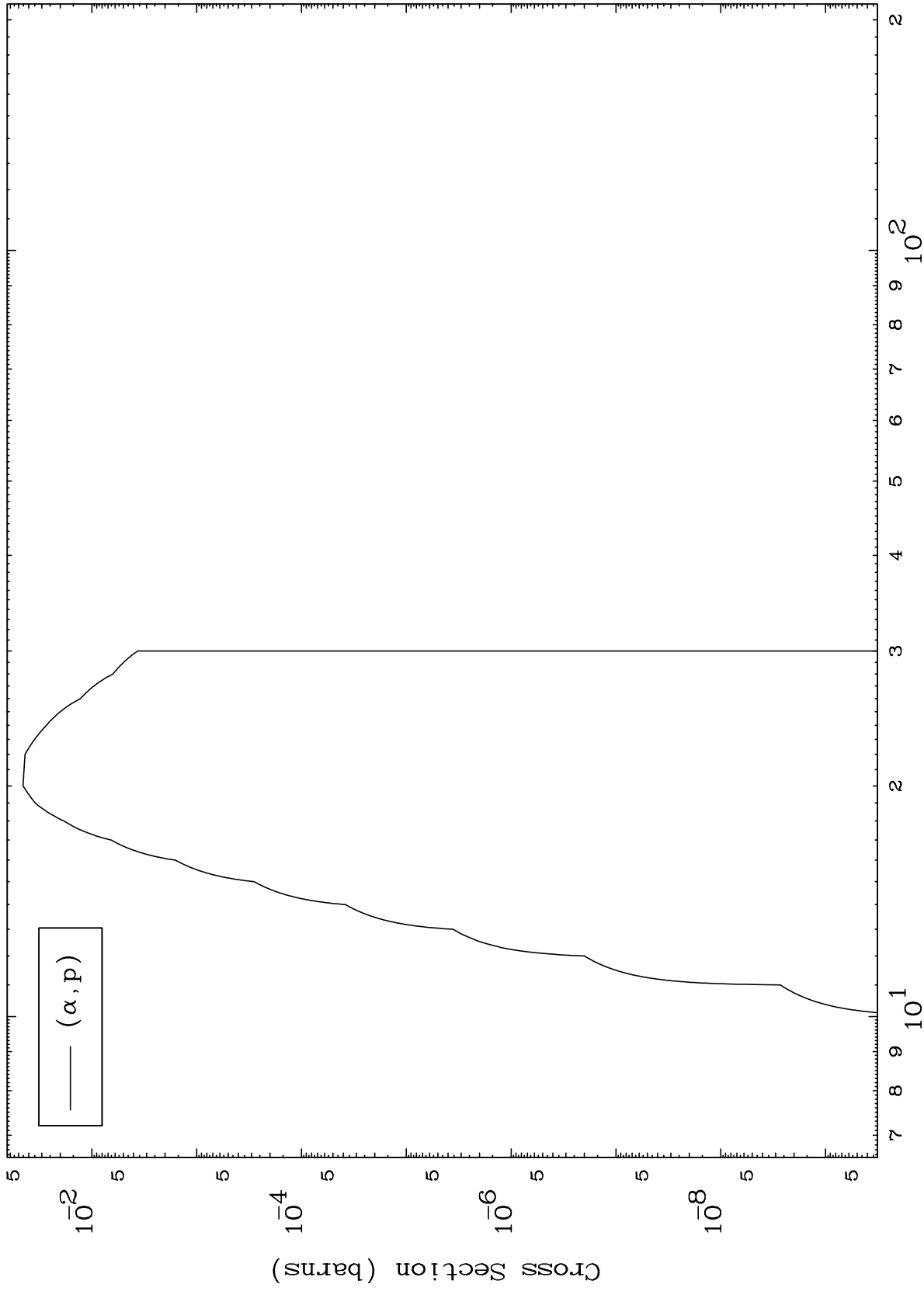
Incident Energy (MeV)

70-Yb-151

MAT 6974

70-Yb-151

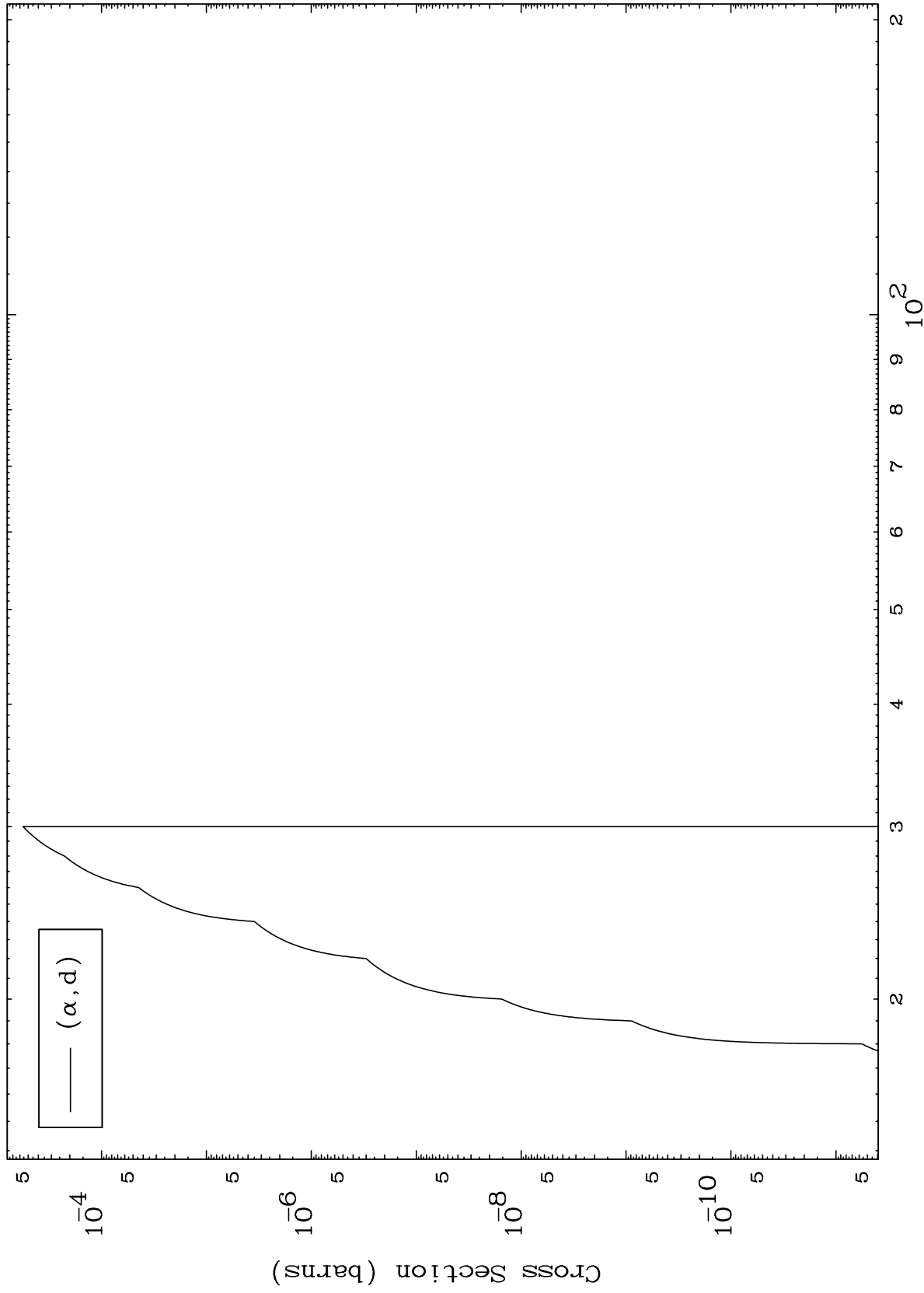
( $\alpha$ ,p) Levels  
0 Kelvin Cross Sections



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( $\alpha$ ,d) Levels  
0 Kelvin Cross Sections

70-Yb-151



7

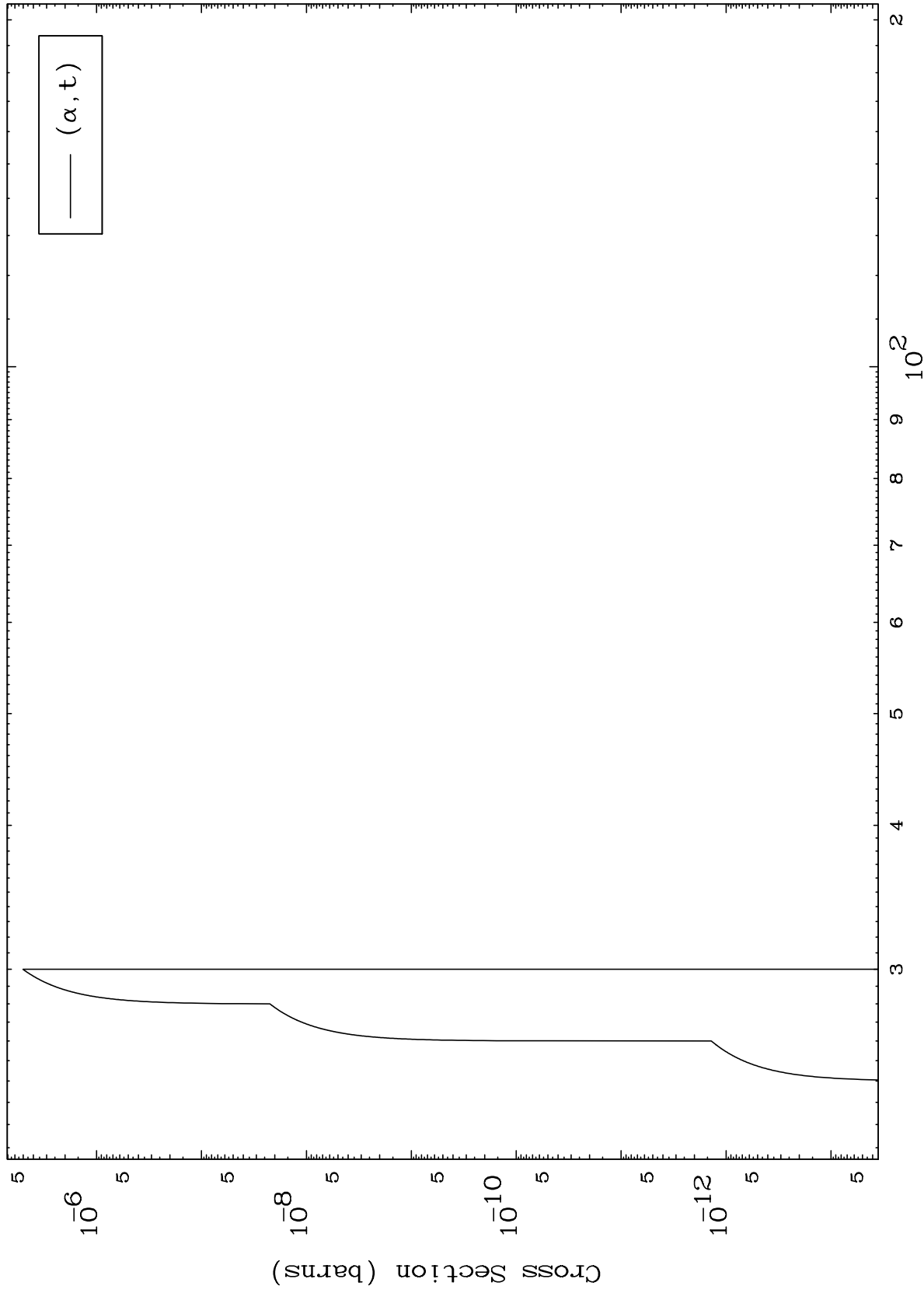
Incident Energy (MeV)

70-Yb-151

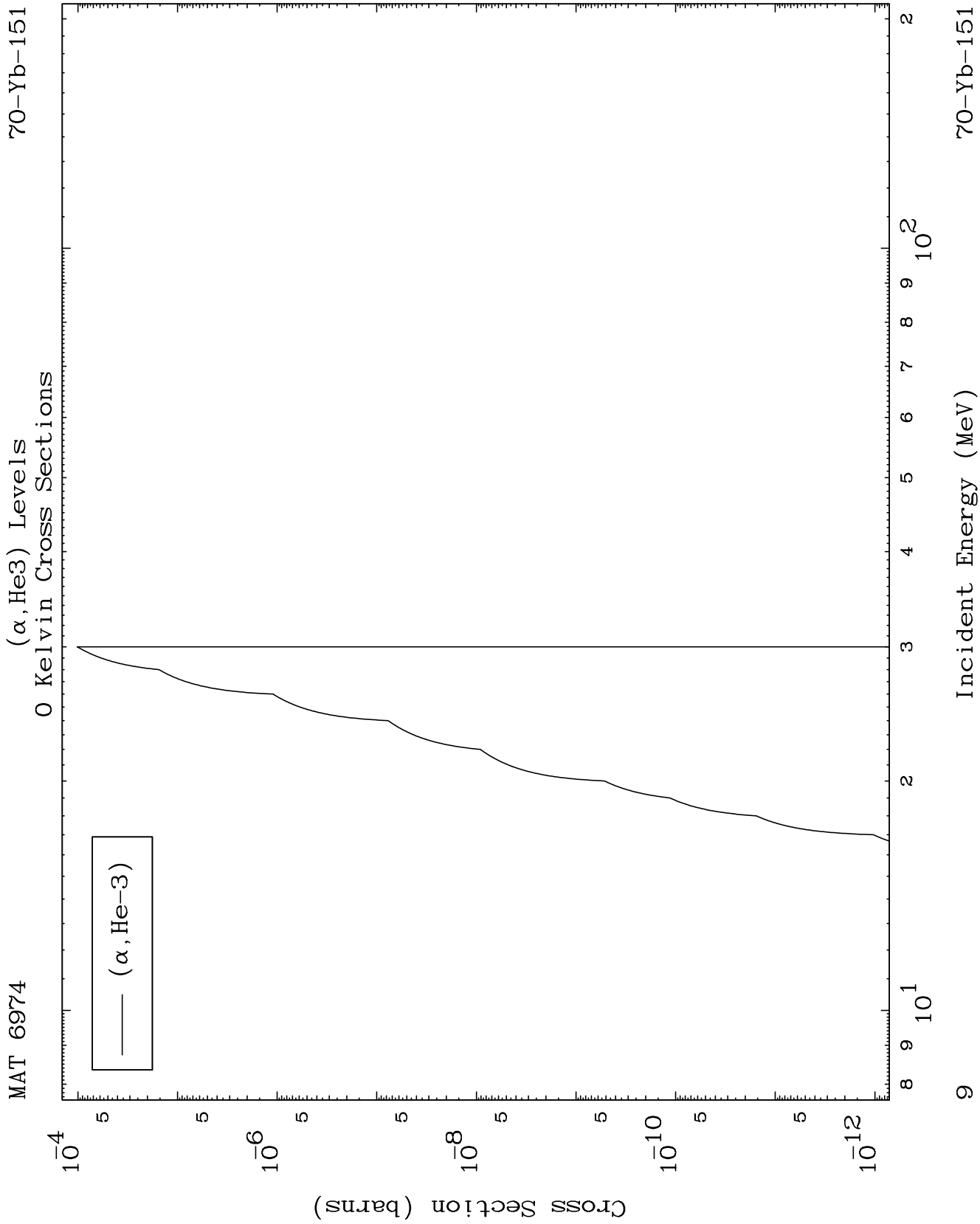
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( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

70-Yb-151



8

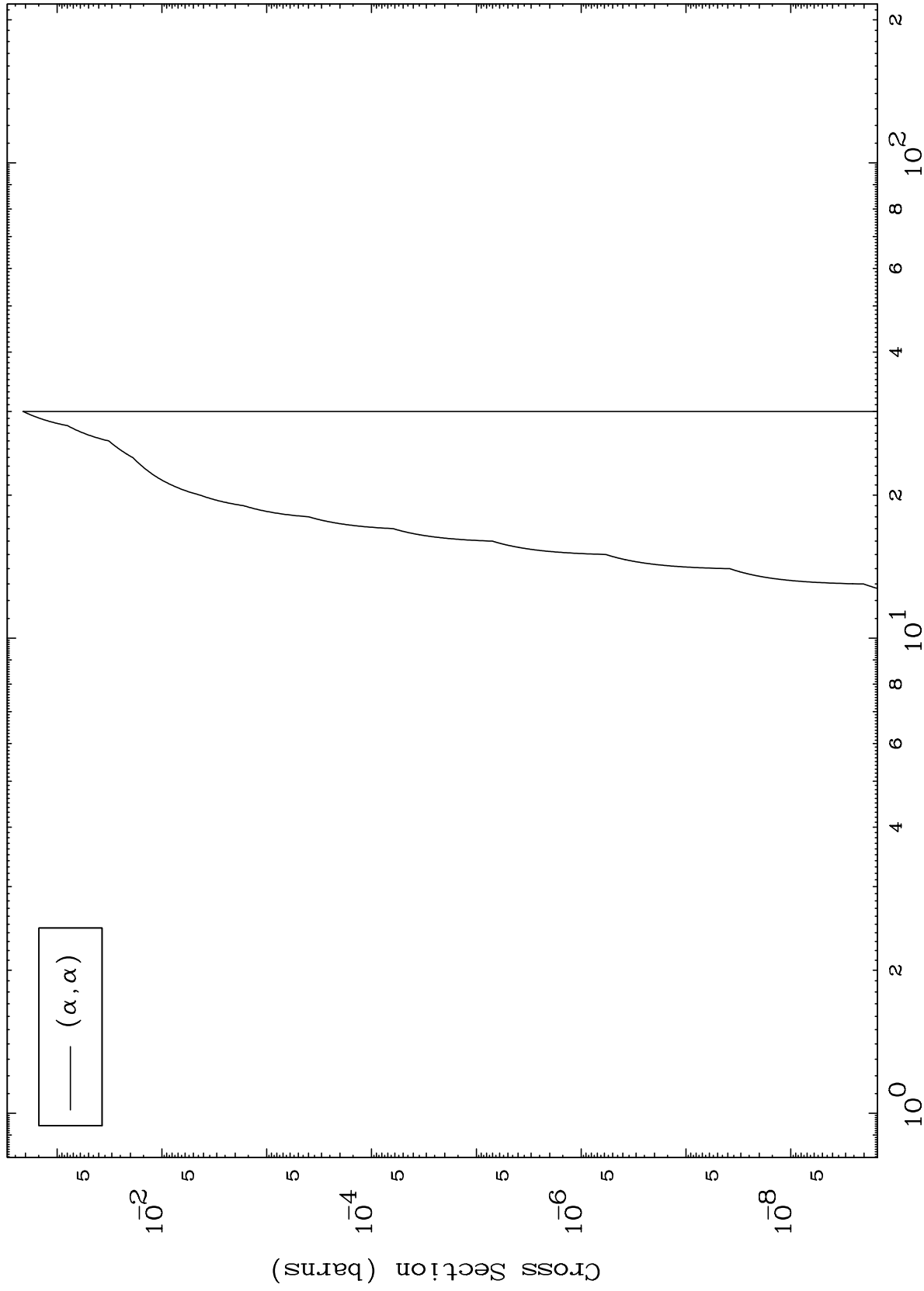


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$(\alpha, \alpha)$  Levels

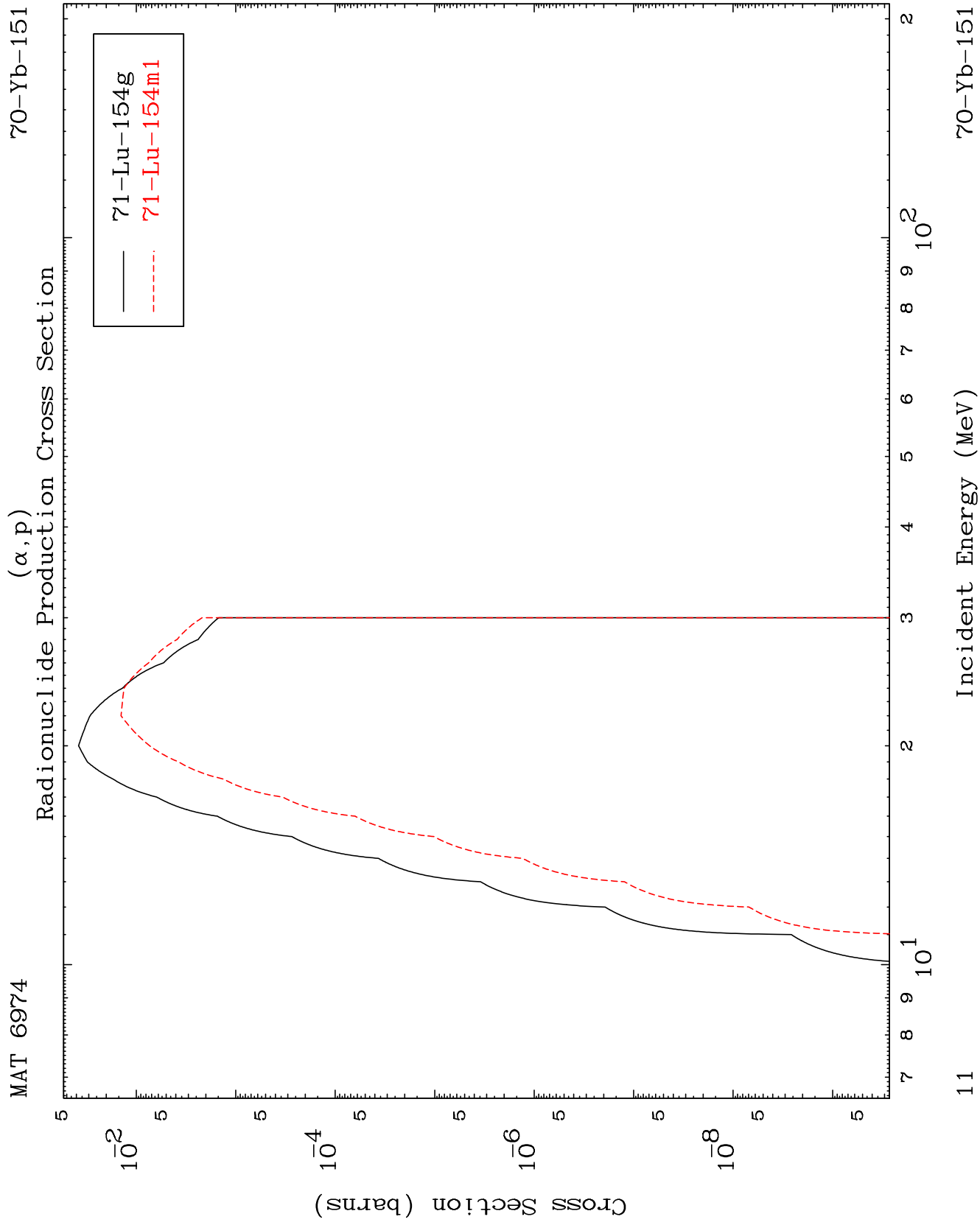
70-Yb-151

0 Kelvin Cross Sections



Incident Energy (MeV)

70-Yb-151

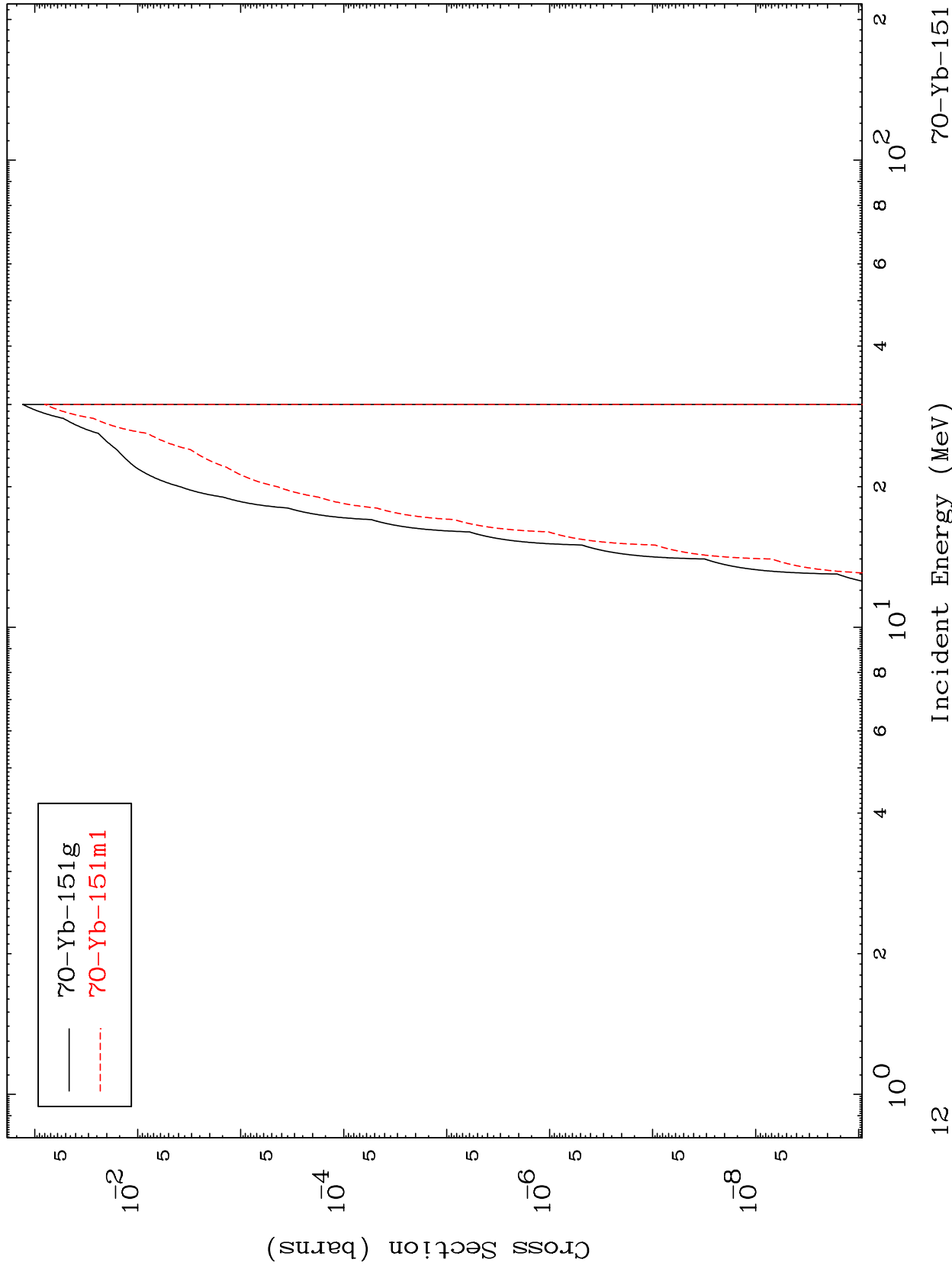


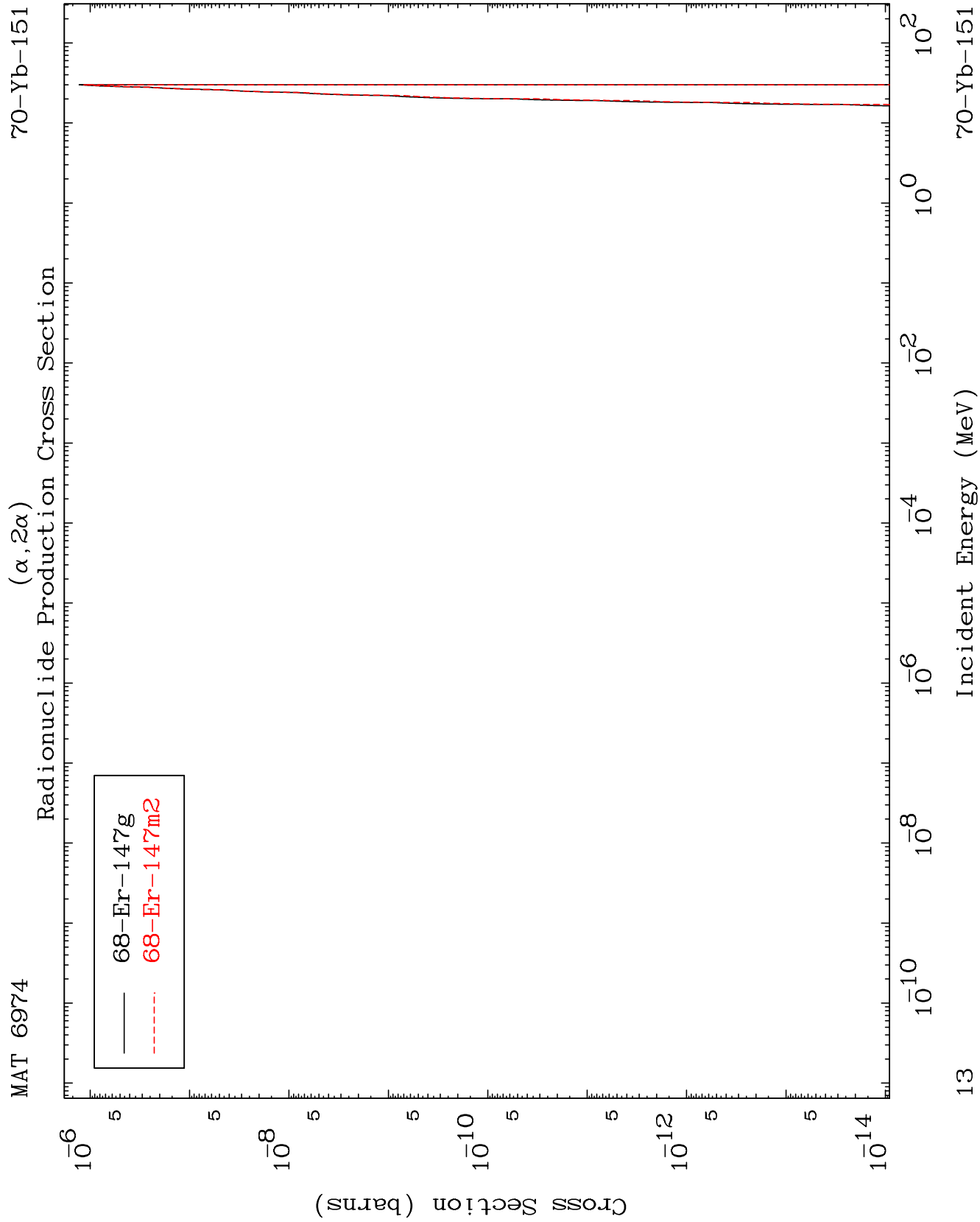
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$(\alpha, \alpha)$

$^{70}\text{Yb}-^{151}$

Radionuclide Production Cross Section



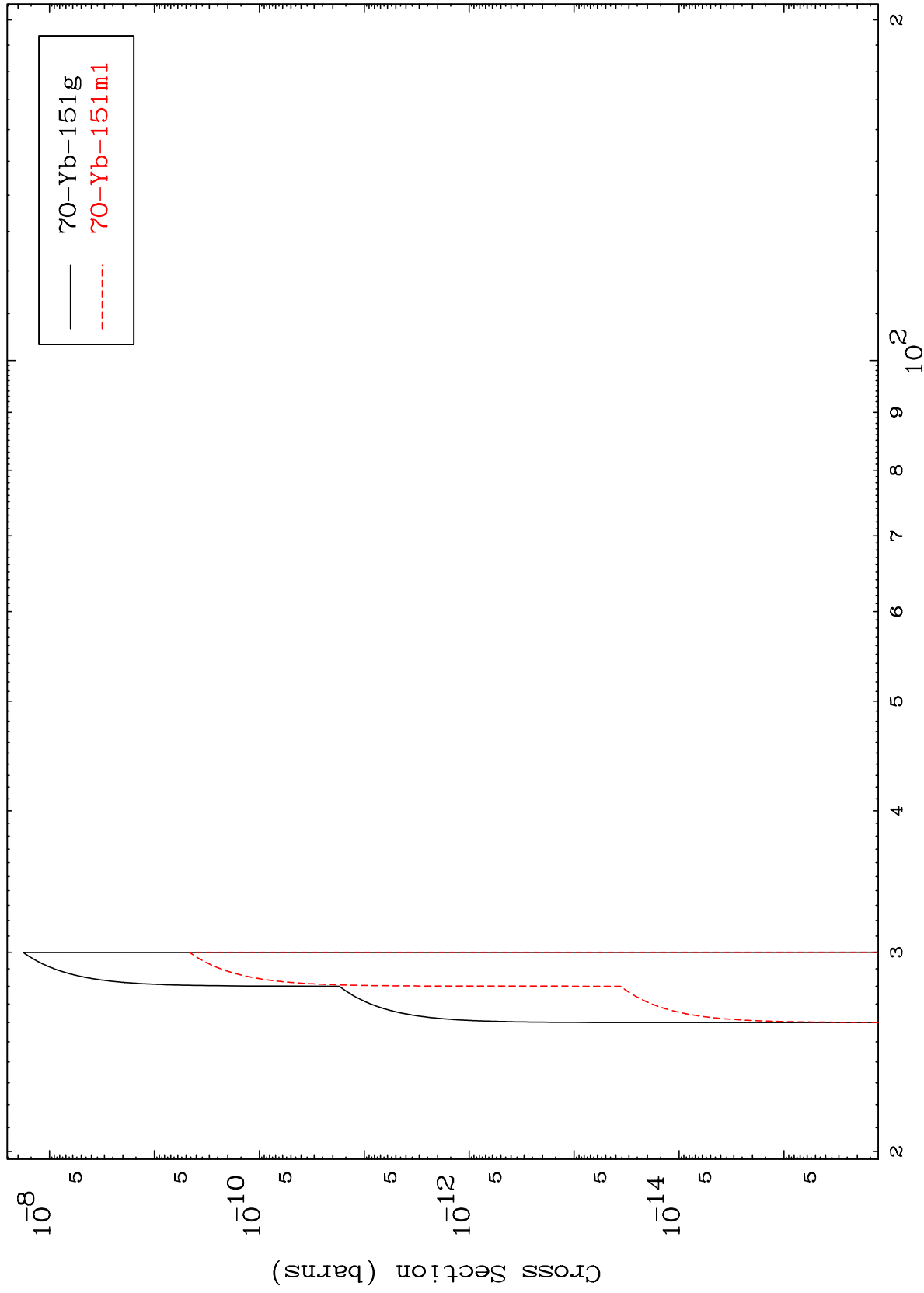


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( $\alpha$ ,p) t

$^{70}\text{Yb}-^{151}$

Radionuclide Production Cross Section



14

Incident Energy (MeV)

$^{70}\text{Yb}-^{151}$