

Program EVALPLOT
(Version 2018-1)

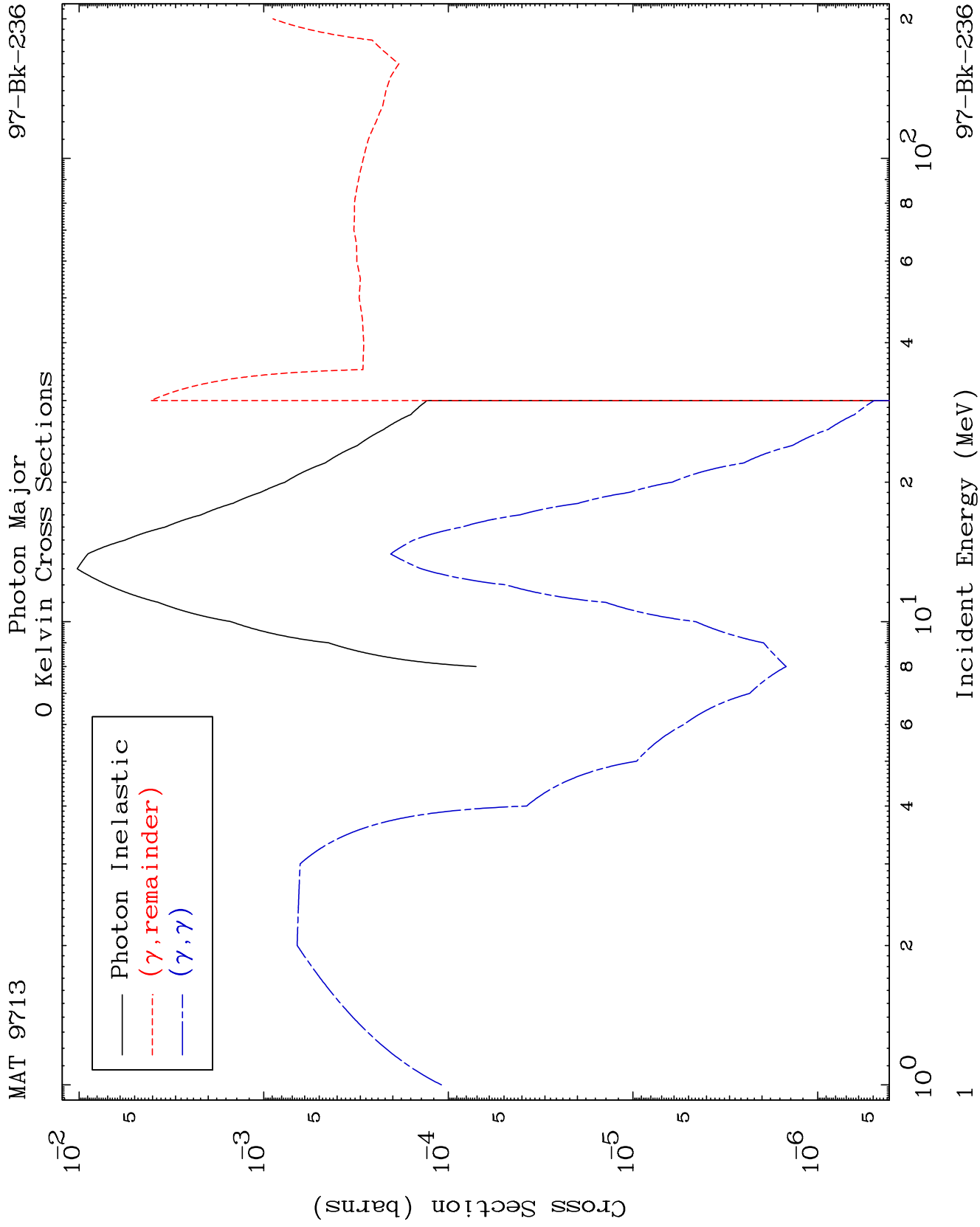
by

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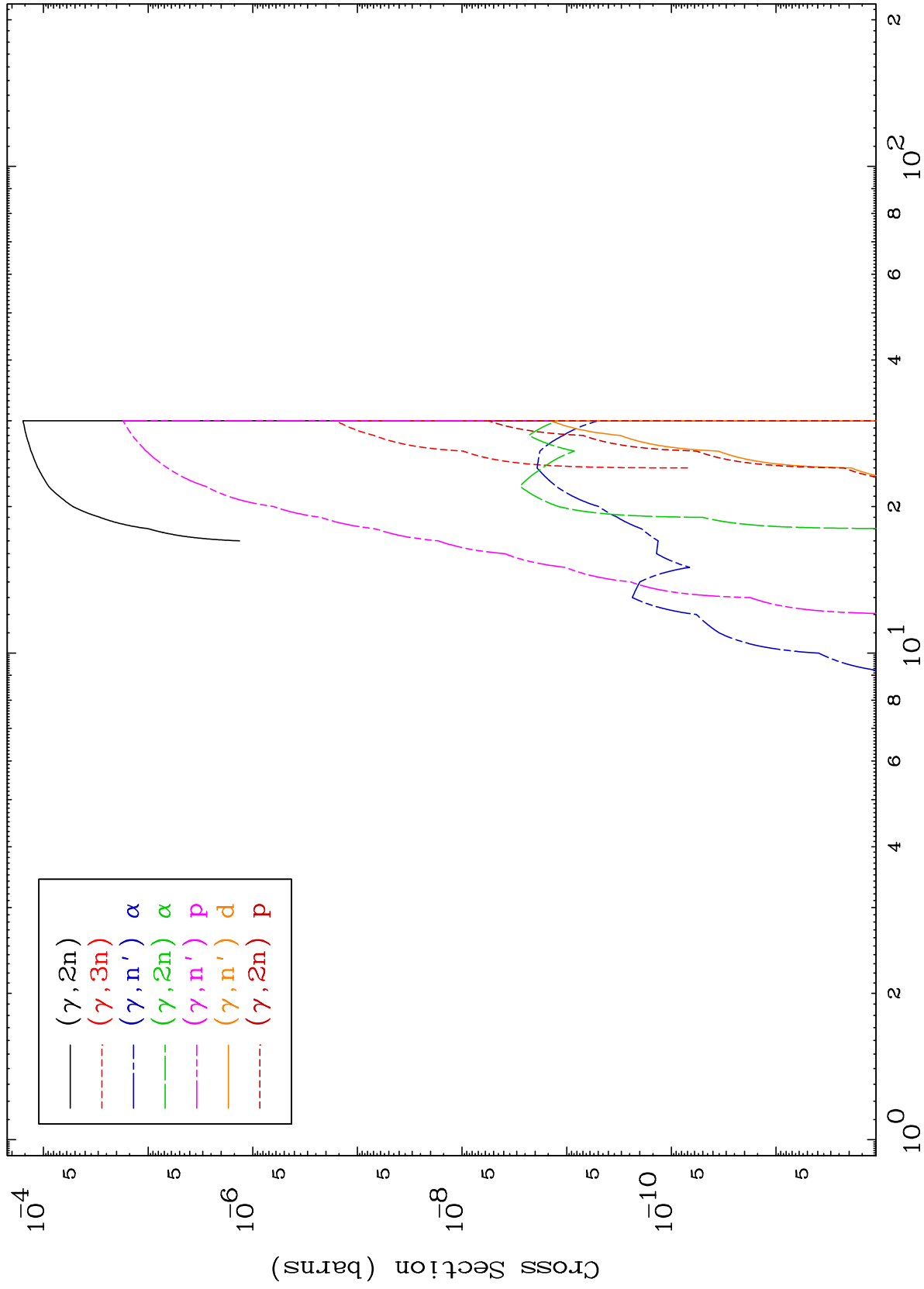
Press Mouse Button to Start



MAT 9713

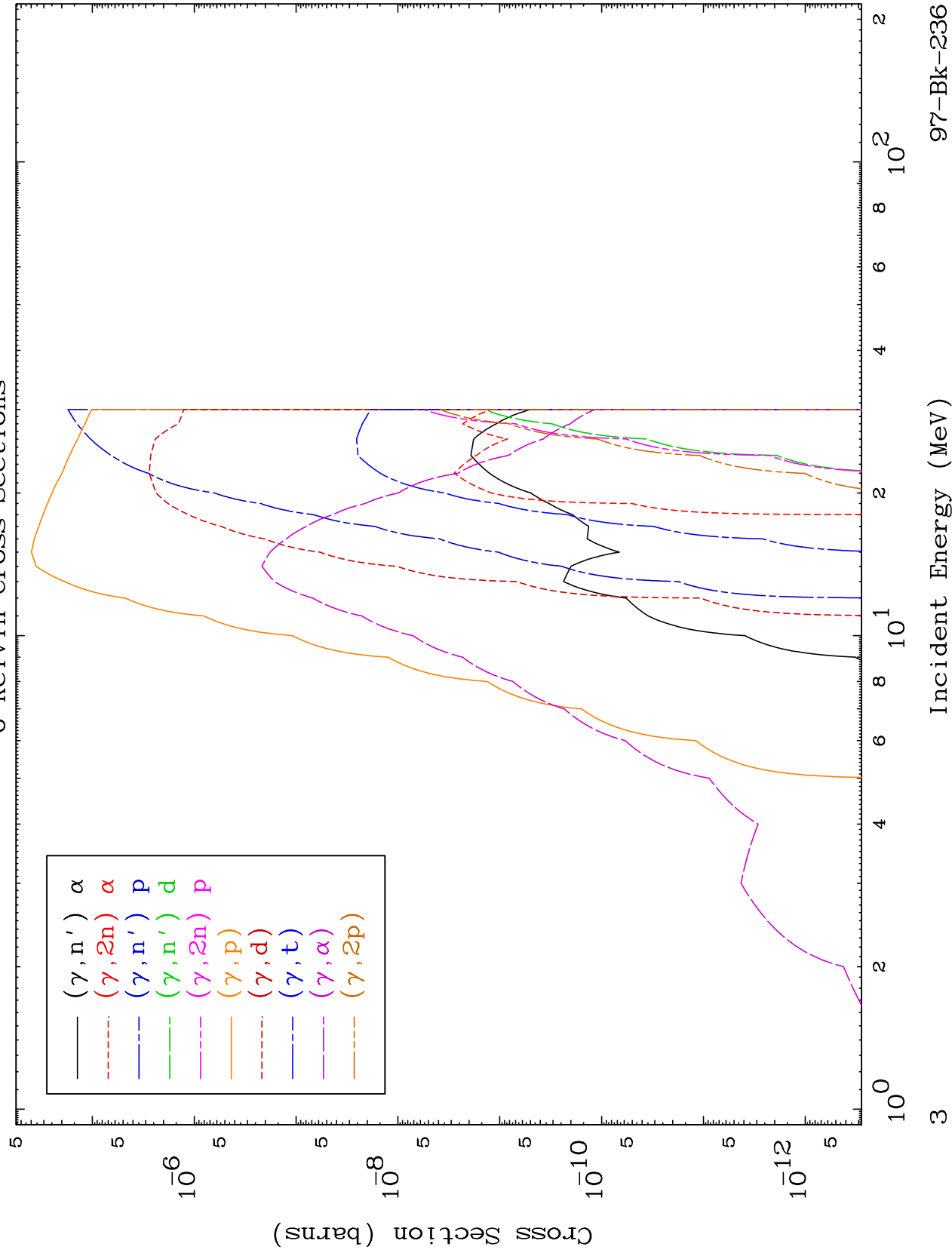
Photon Neutron Production
0 Kelvin Cross Sections

97-Bk-236



Incident Energy (MeV)

97-Bk-236

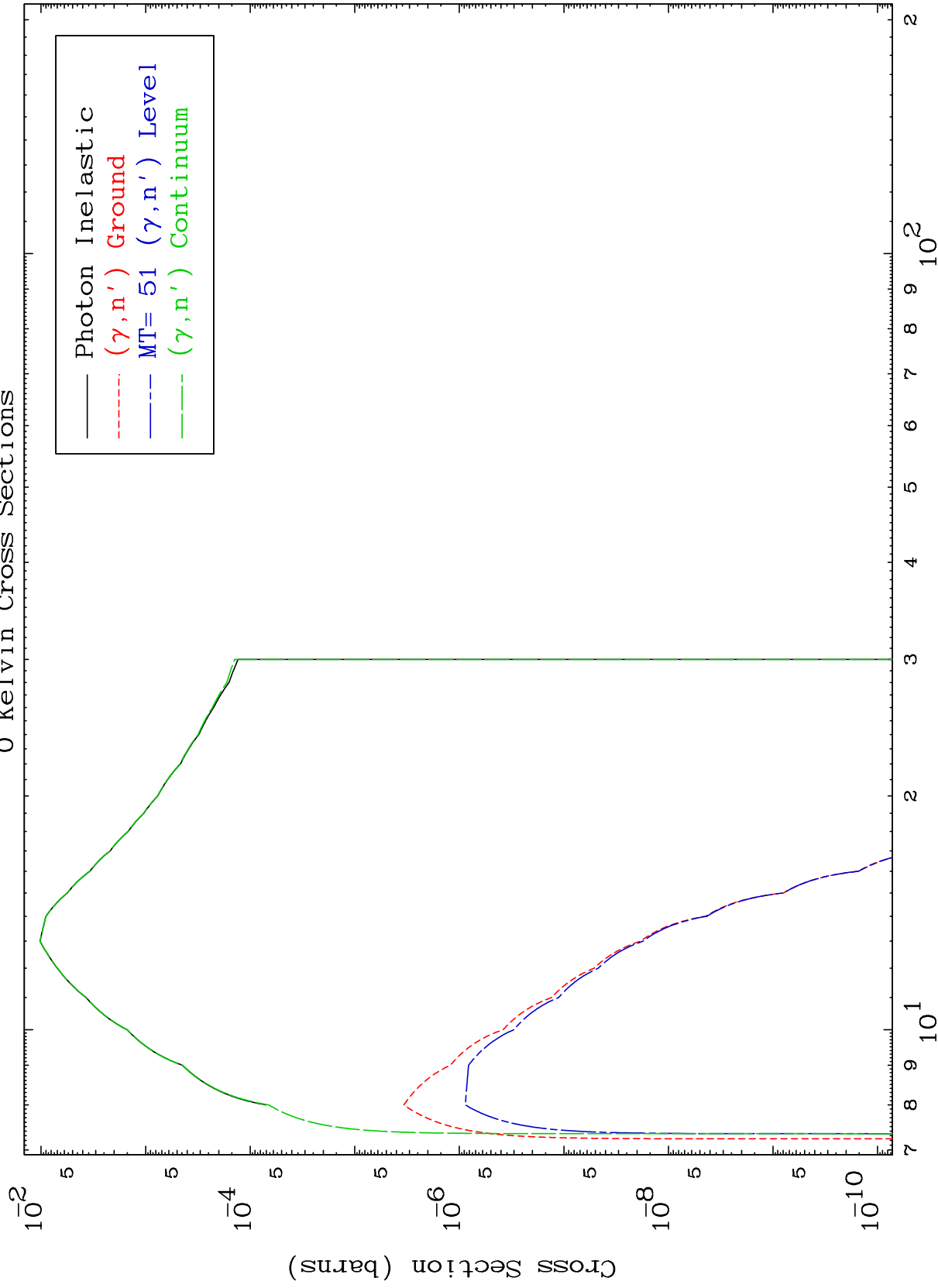


MAT 9713

(γ, n') Level

97-Bk-236

0 Kelvin Cross Sections



4

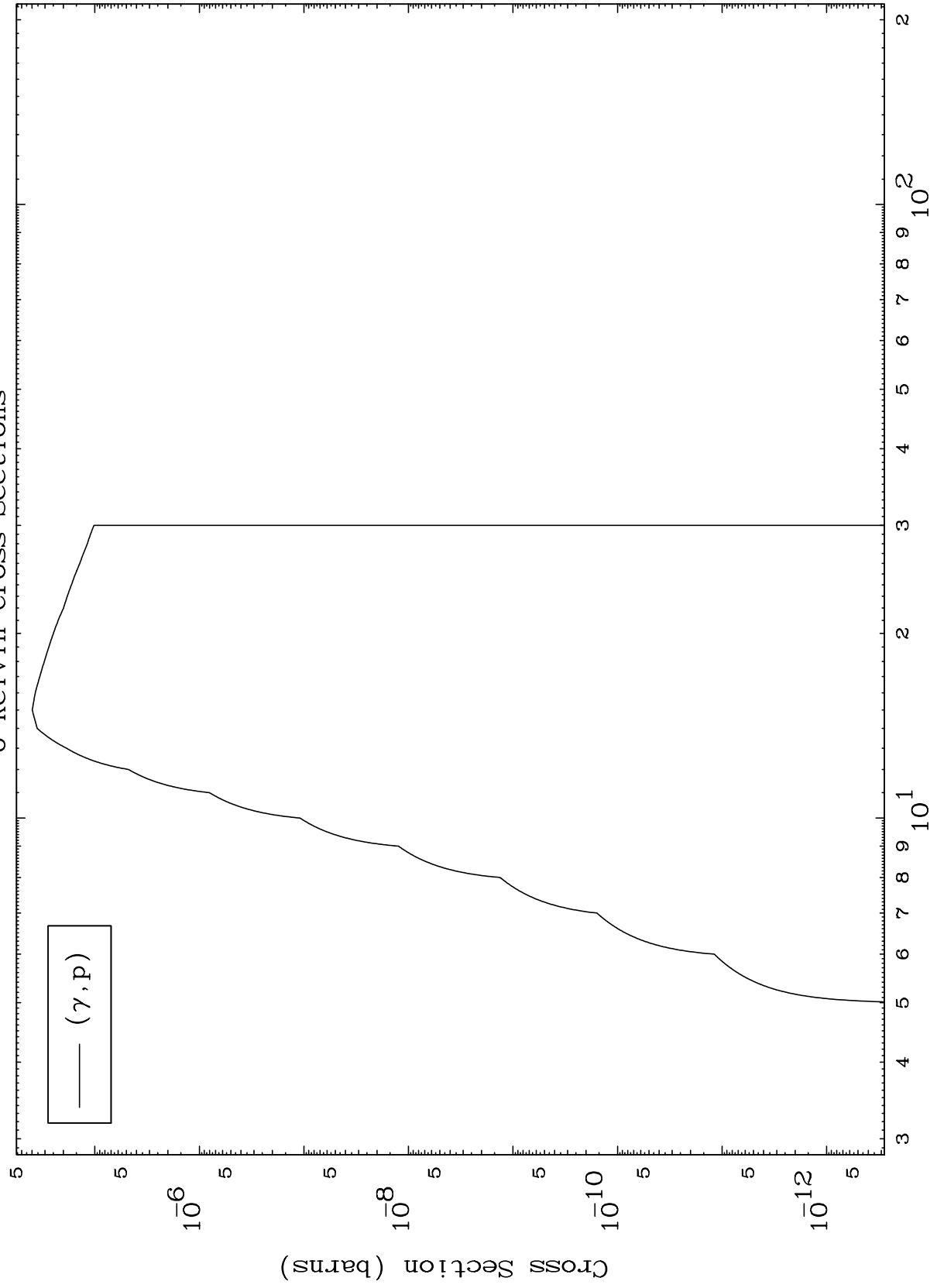
Incident Energy (MeV)

97-Bk-236

MAT 9713

97-Bk-236

(γ ,p) Levels
0 Kelvin Cross Sections



5

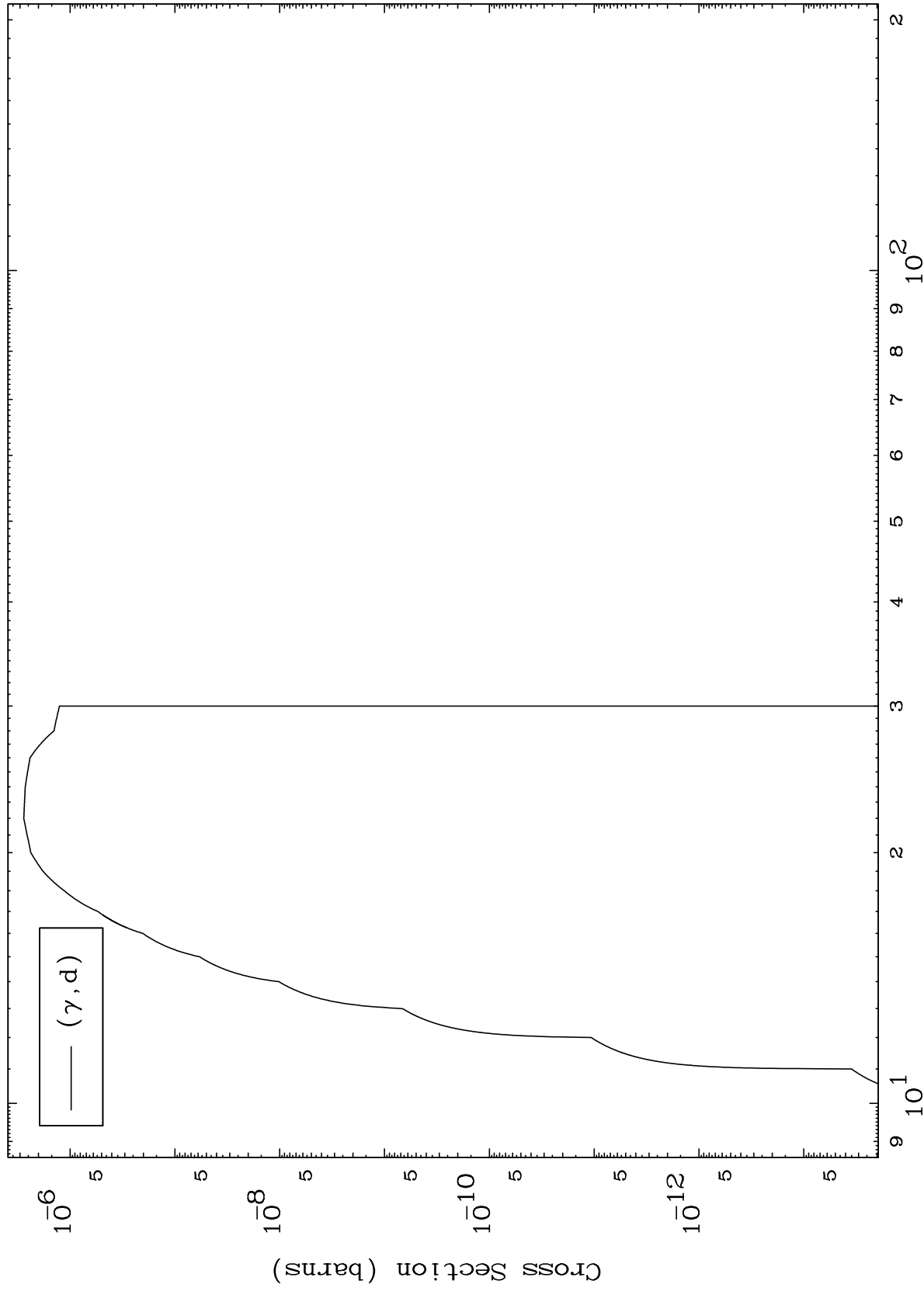
Incident Energy (MeV)

97-Bk-236

MAT 9713

97-Bk-236

(γ, d) Levels
0 Kelvin Cross Sections



6

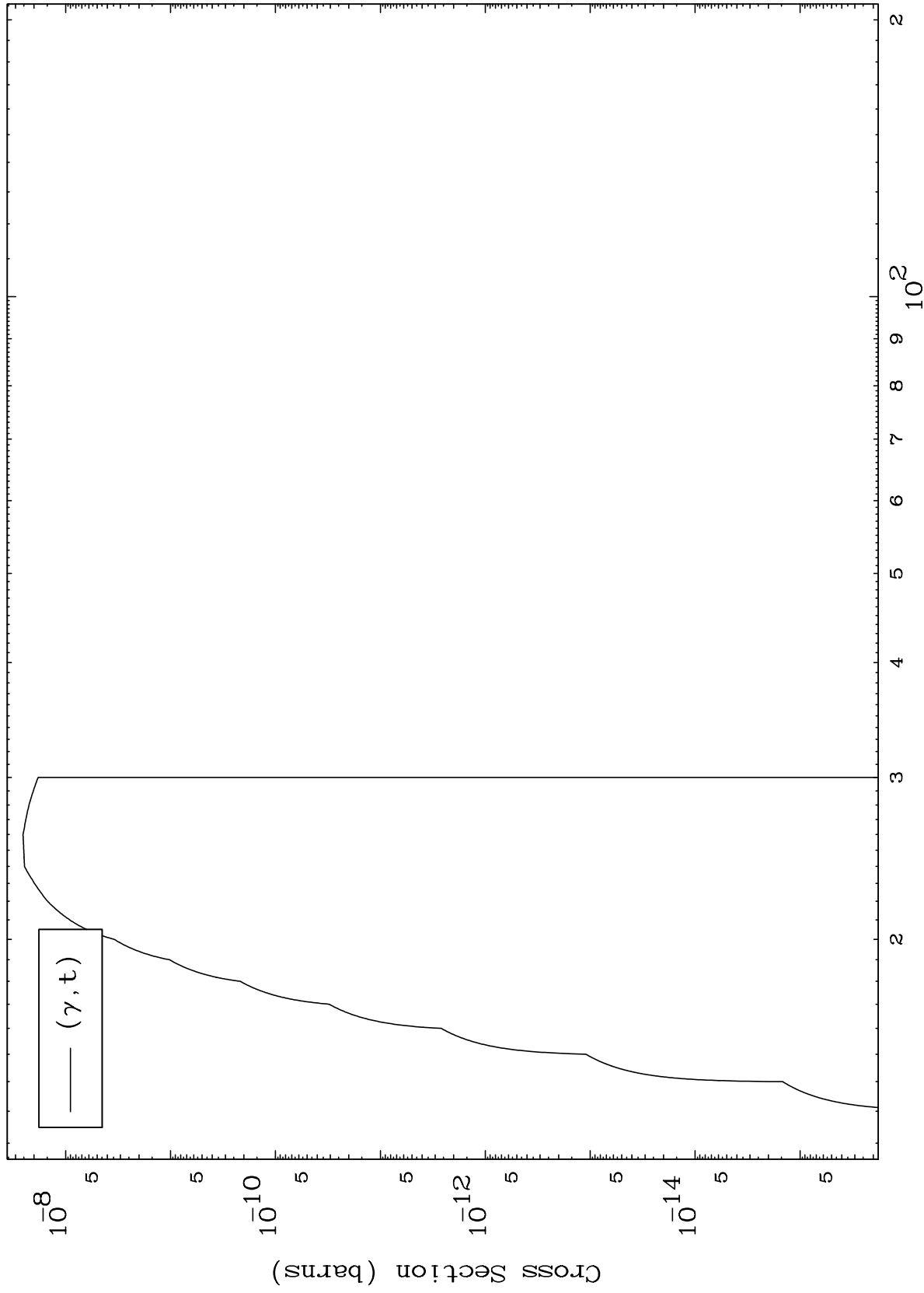
Incident Energy (MeV)

97-Bk-236

MAT 9713

97-Bk-236

(γ, t) Levels
0 Kelvin Cross Sections



7

Incident Energy (MeV)

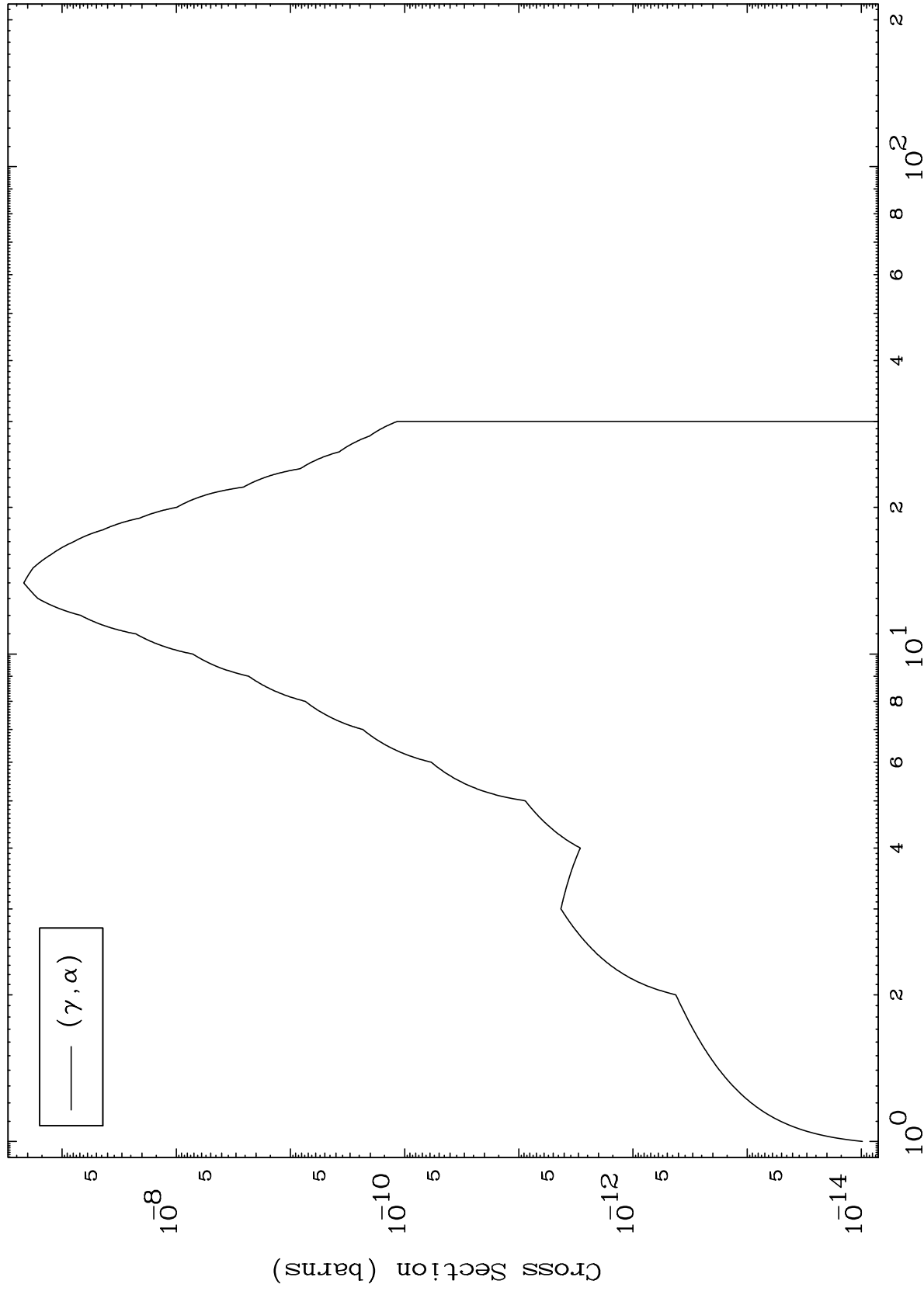
97-Bk-236

MAT 9713

(γ, α) Levels

97-Bk-236

0 Kelvin Cross Sections



8

Incident Energy (MeV)

97-Bk-236

MAT 9713

97-Bk-236

Photon Fission
Radionuclide Production Cross Section

