

Program EVALPLOT  
(Version 2018-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

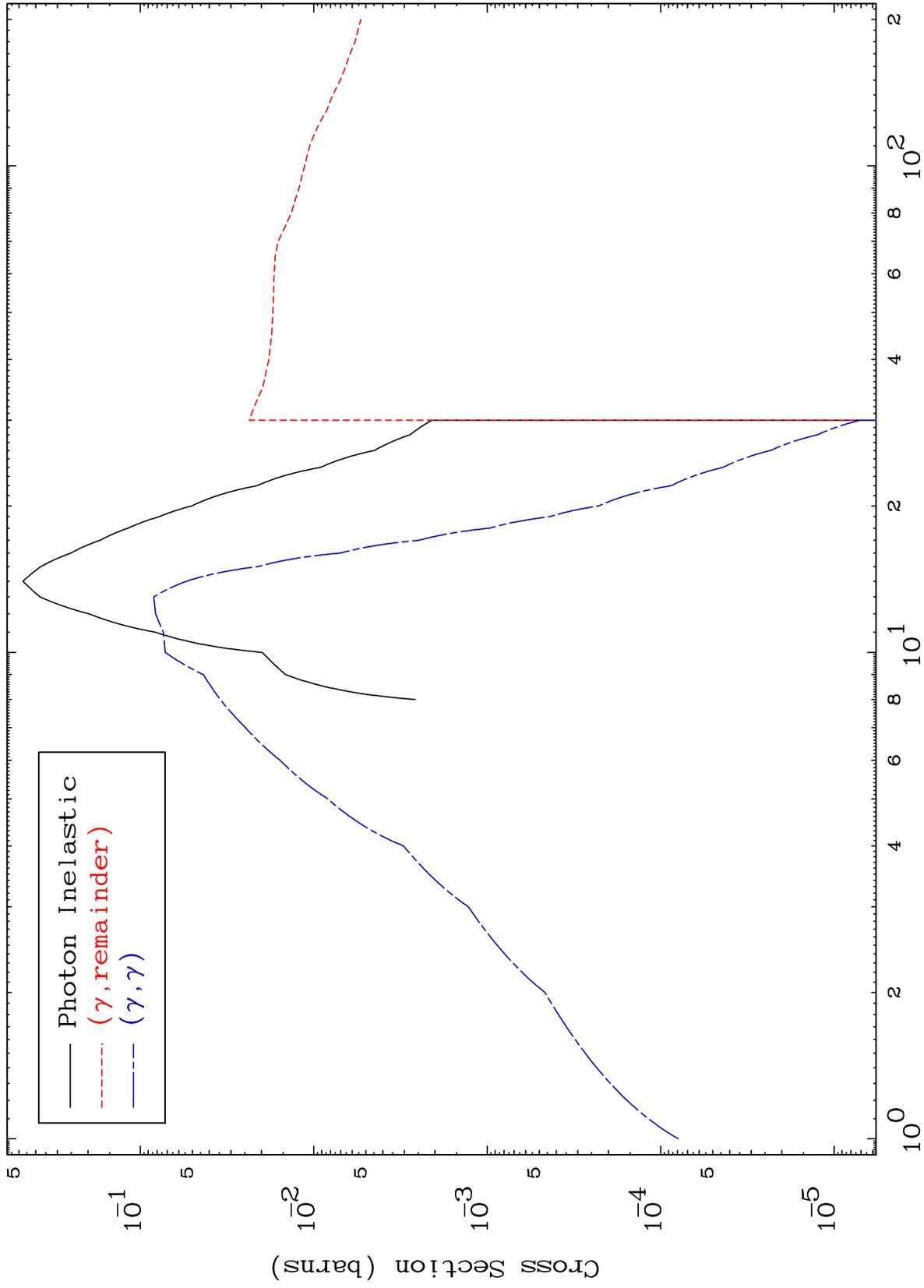
E.Mail: [redcullen1@comcast.net](mailto:redcullen1@comcast.net)  
Web: [redcullen1.net/HOMEPAGE.NEW](http://redcullen1.net/HOMEPAGE.NEW)

Press Mouse Button to Start

MAT 8613

Photon Major  
0 Kelvin Cross Sections

86-Rn-207



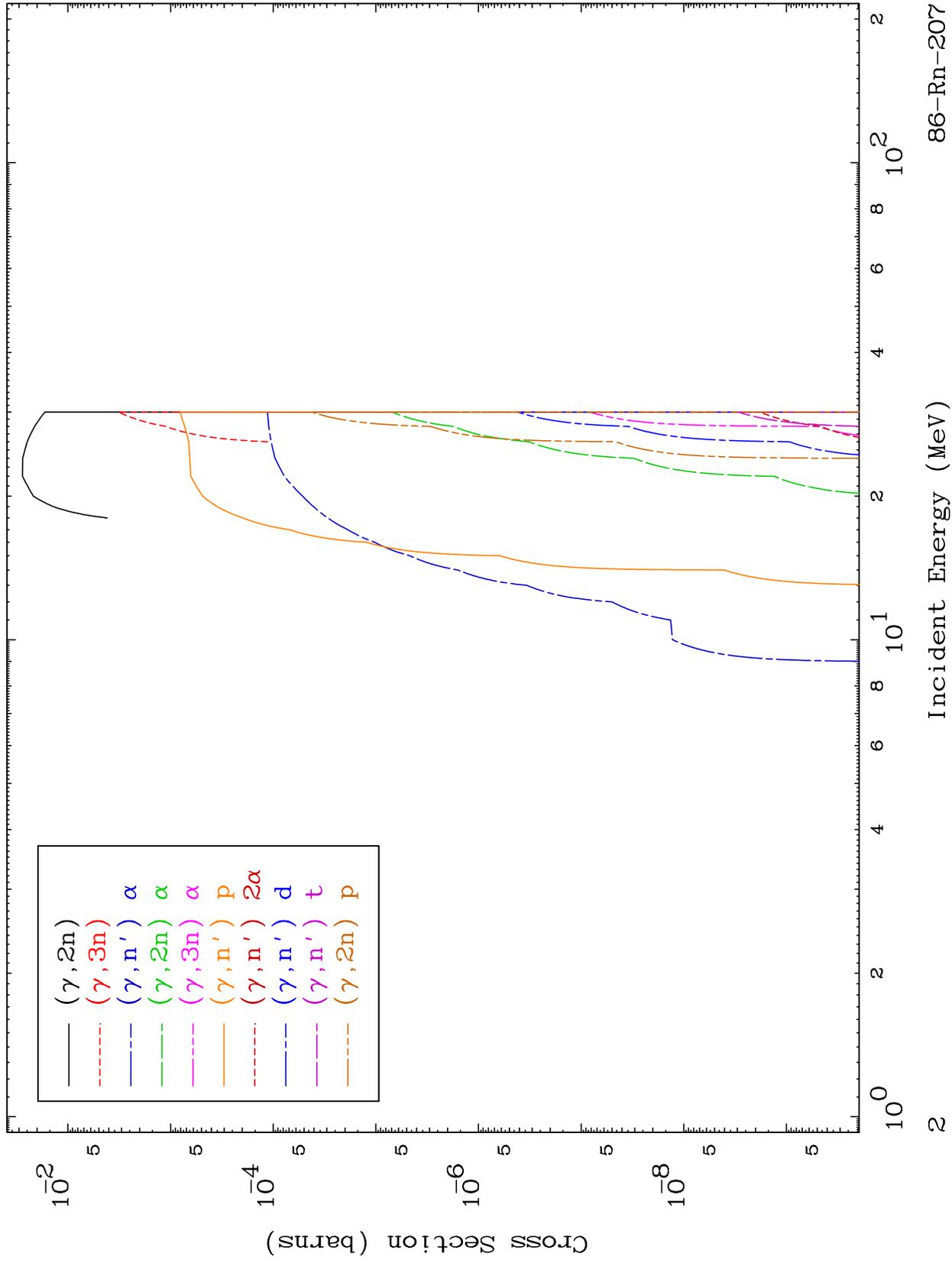
Incident Energy (MeV)

86-Rn-207

MAT 8613

Photon Neutron Production  
0 Kelvin Cross Sections

86-Rn-207

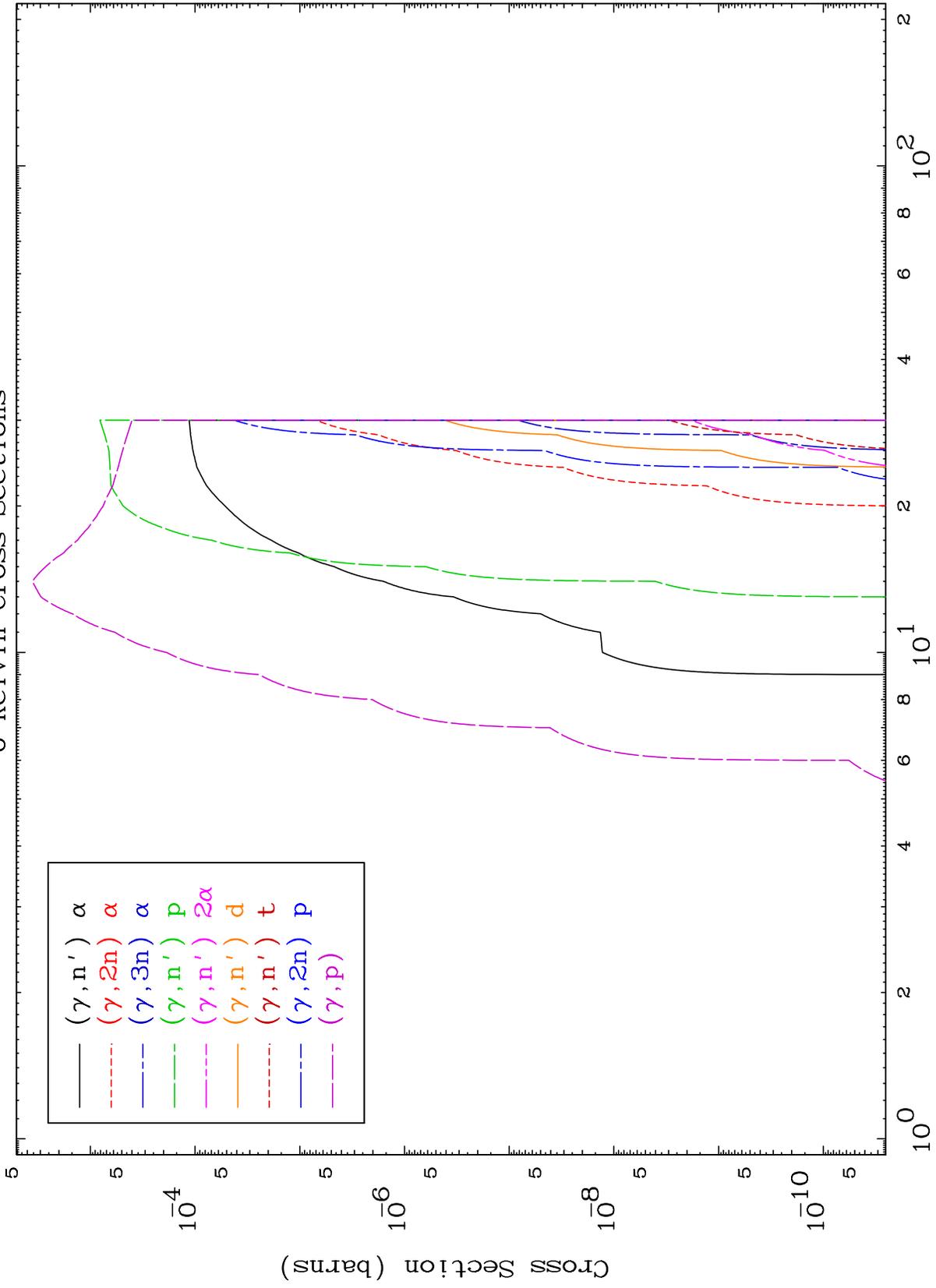


86-Rn-207

MAT 8613

Photon Charged Particle  
0 Kelvin Cross Sections

86-Rn-207



3

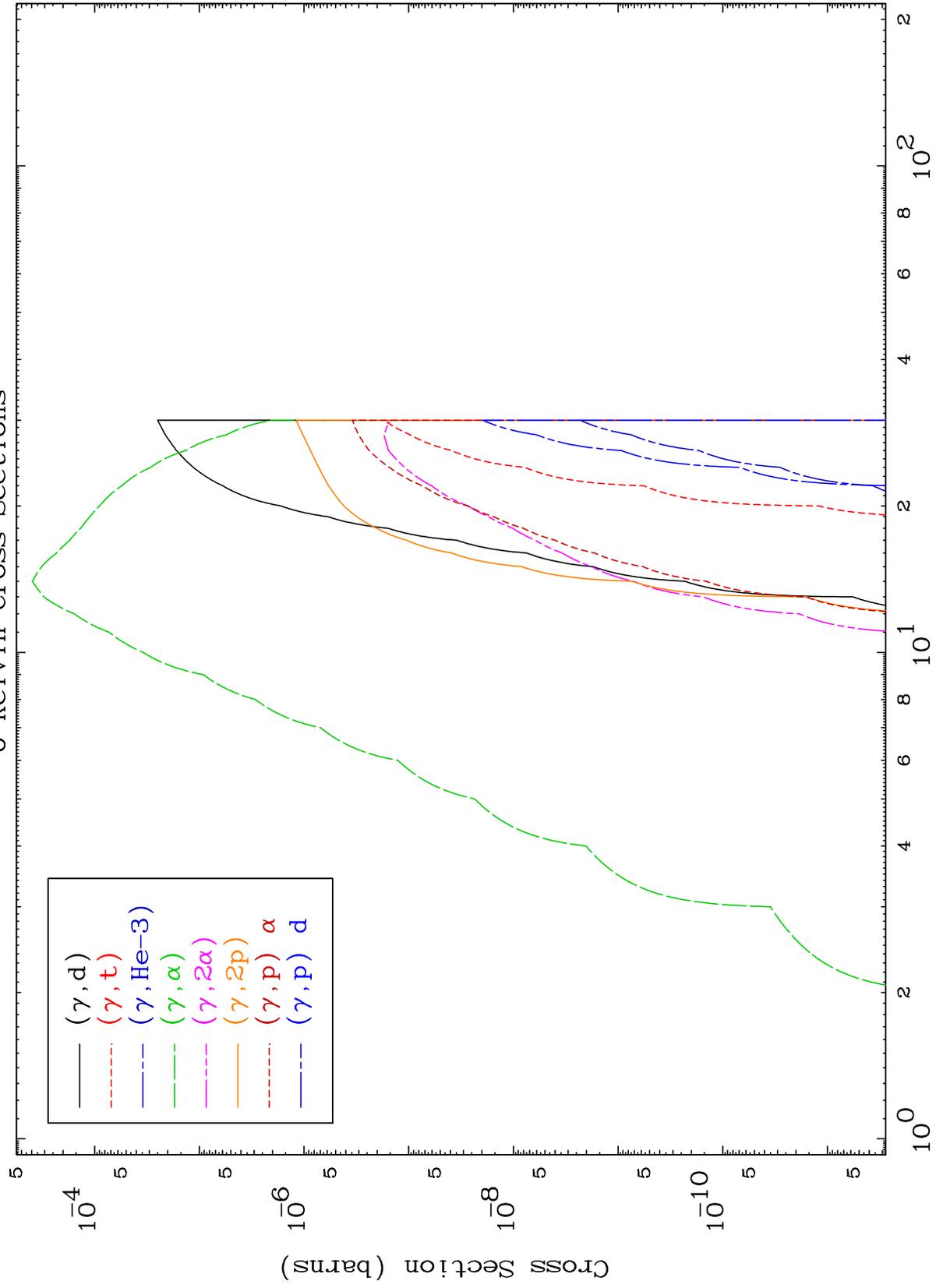
Incident Energy (MeV)

86-Rn-207

MAT 8613

Photon Charged Particle  
0 Kelvin Cross Sections

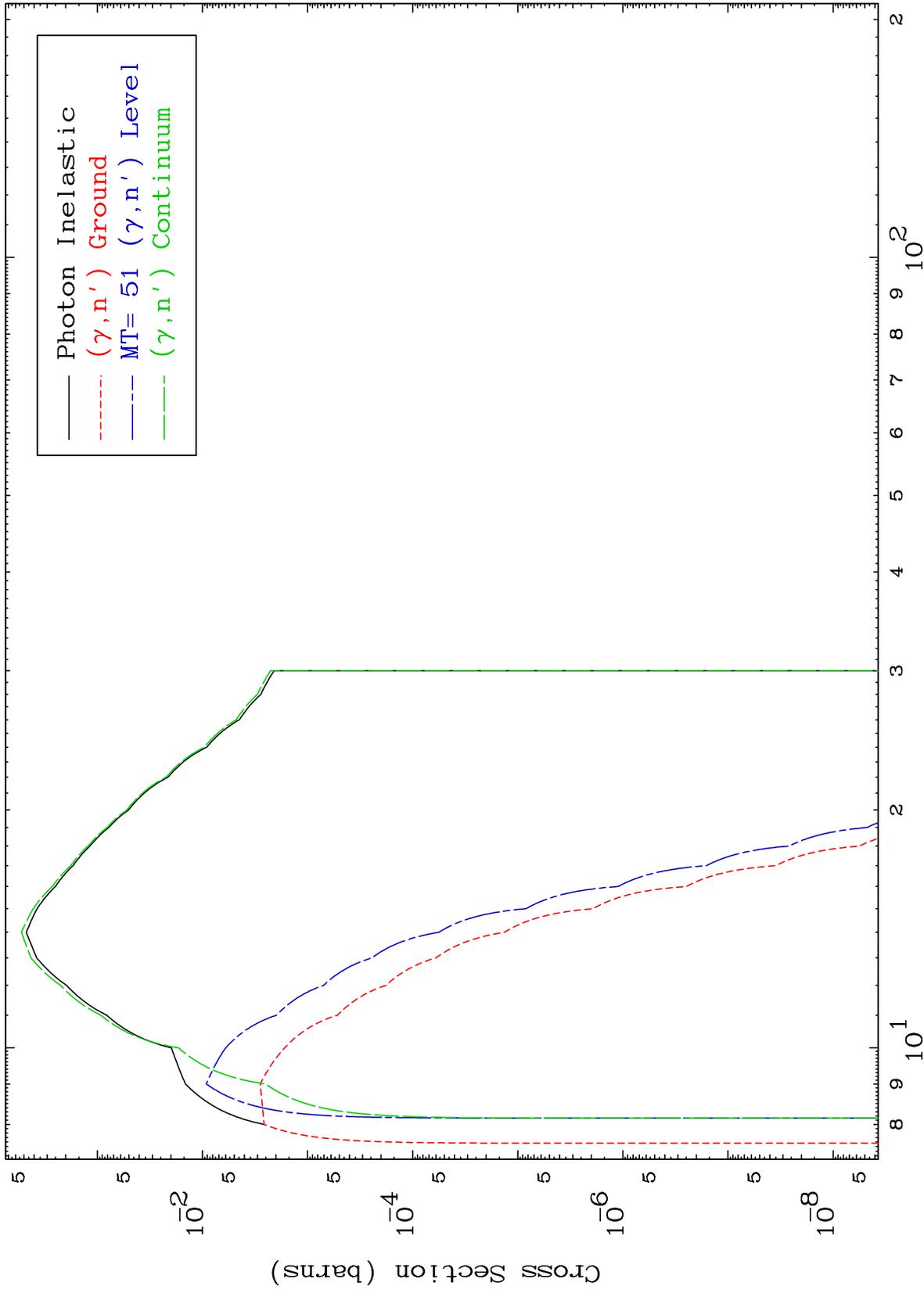
86-Rn-207



86-Rn-207

Incident Energy (MeV)

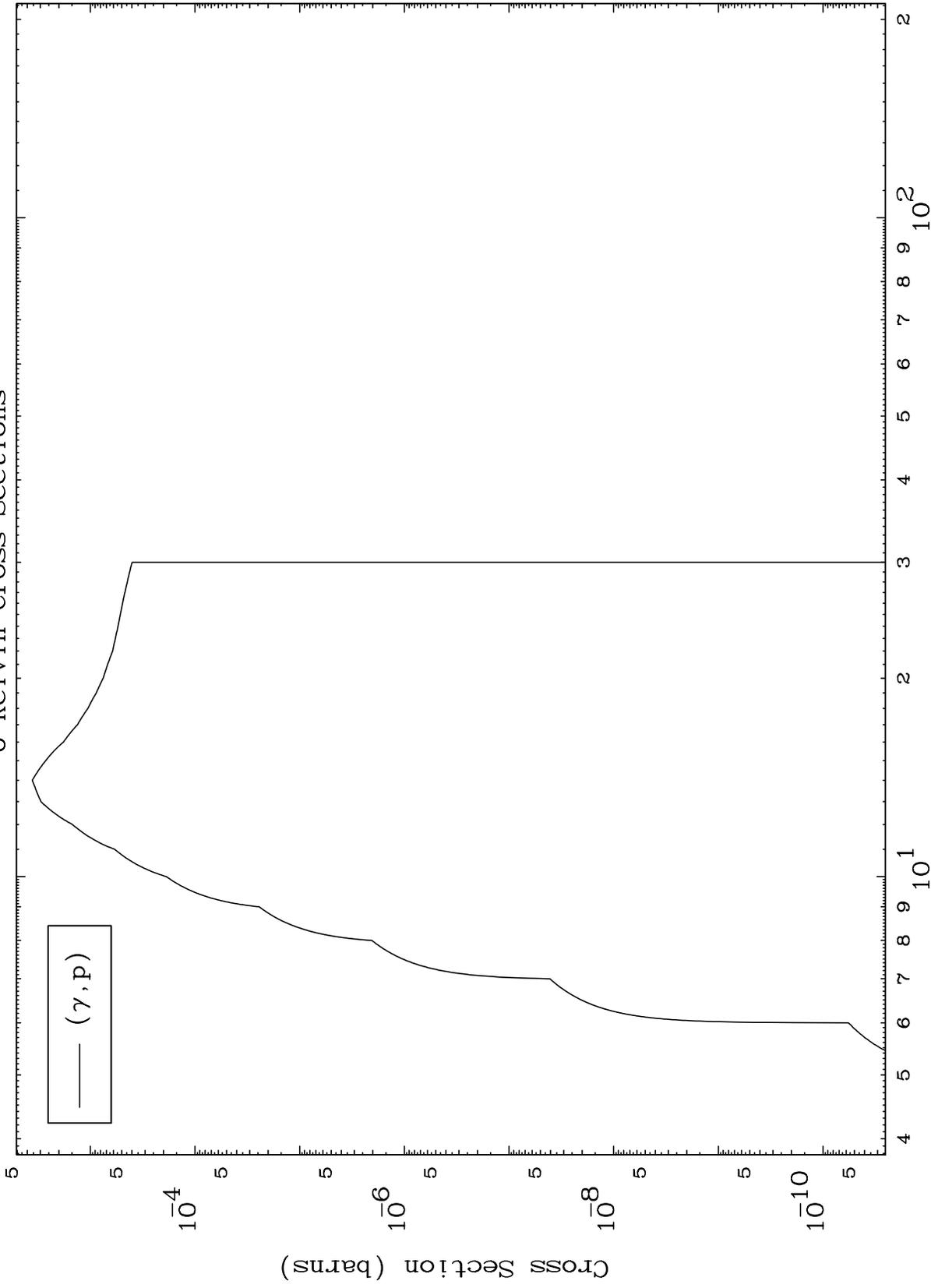
4



MAT 8613

( $\gamma, p$ ) Levels  
0 Kelvin Cross Sections

86-Rn-207



6

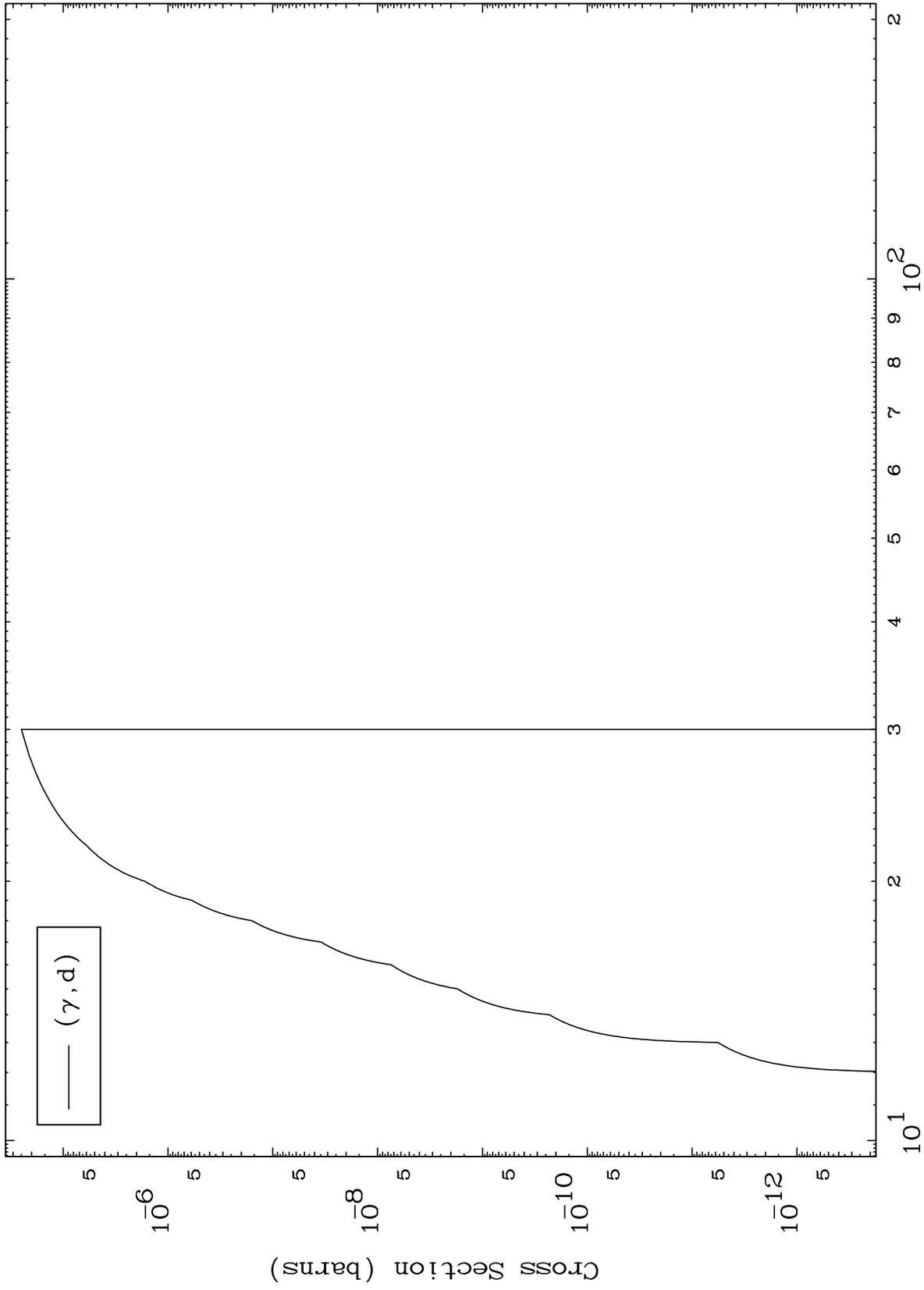
Incident Energy (MeV)

86-Rn-207

MAT 8613

( $\gamma, d$ ) Levels  
0 Kelvin Cross Sections

86-Rn-207



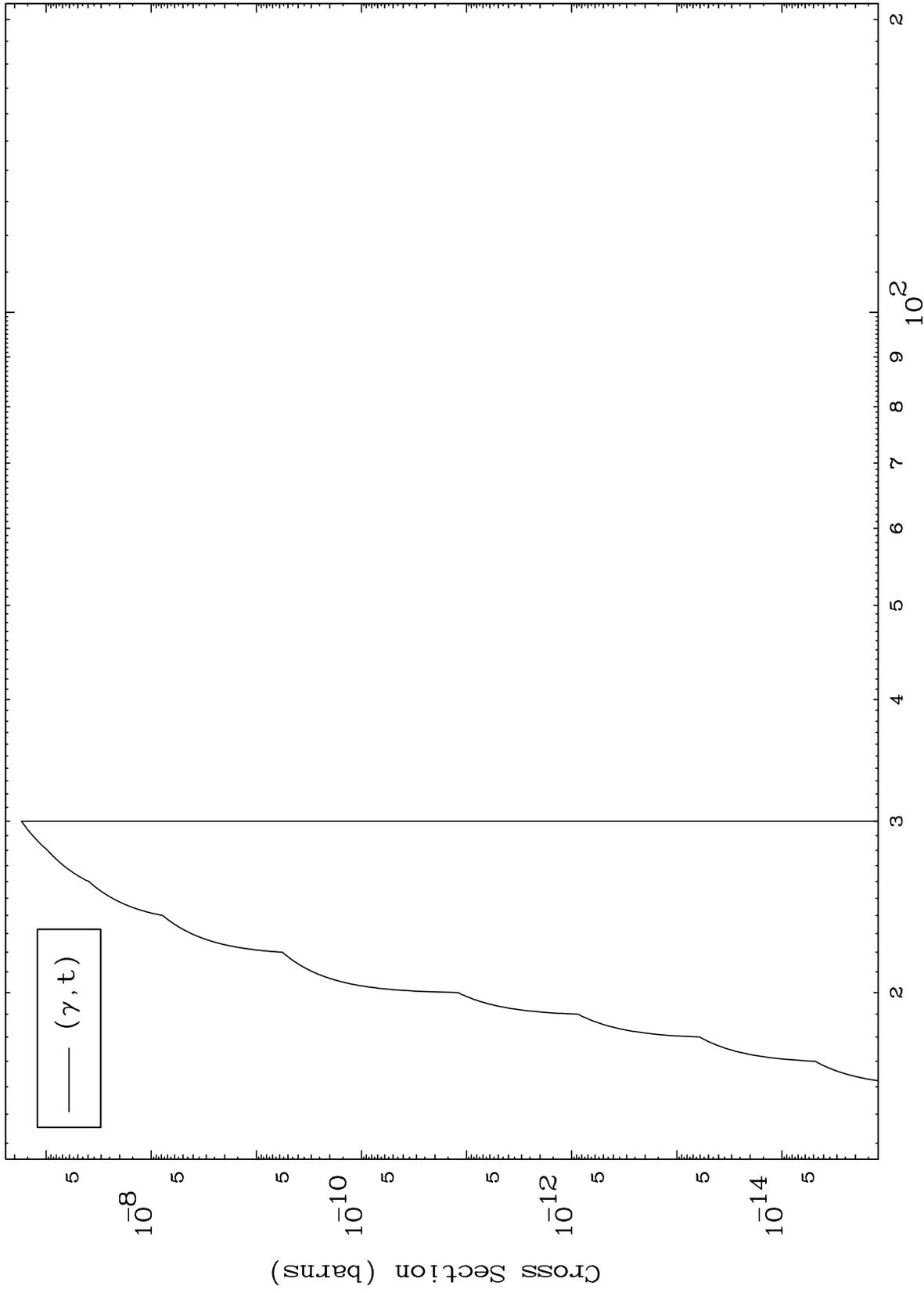
Incident Energy (MeV)

86-Rn-207

MAT 8613

( $\gamma, t$ ) Levels  
0 Kelvin Cross Sections

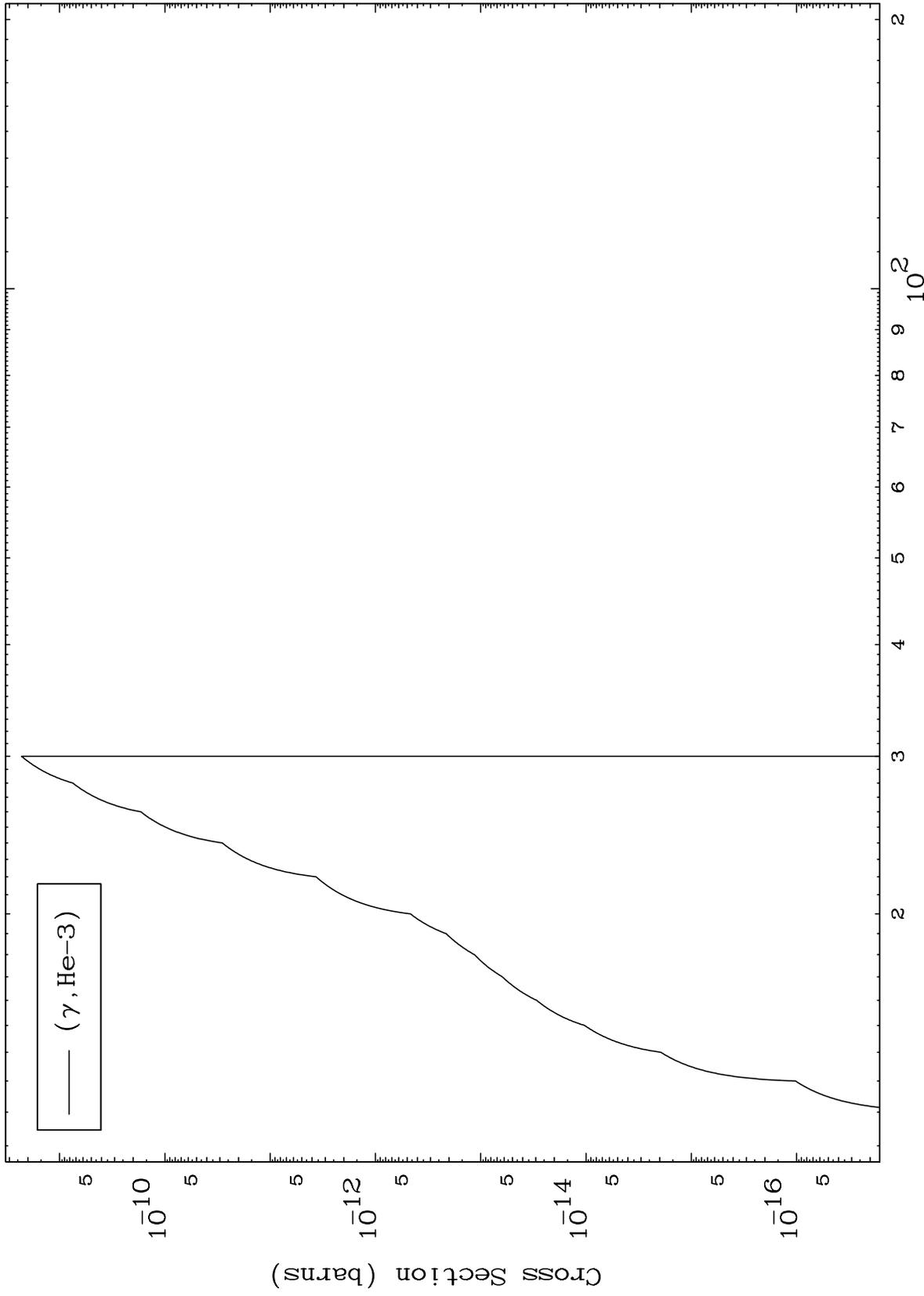
86-Rn-207



8

Incident Energy (MeV)

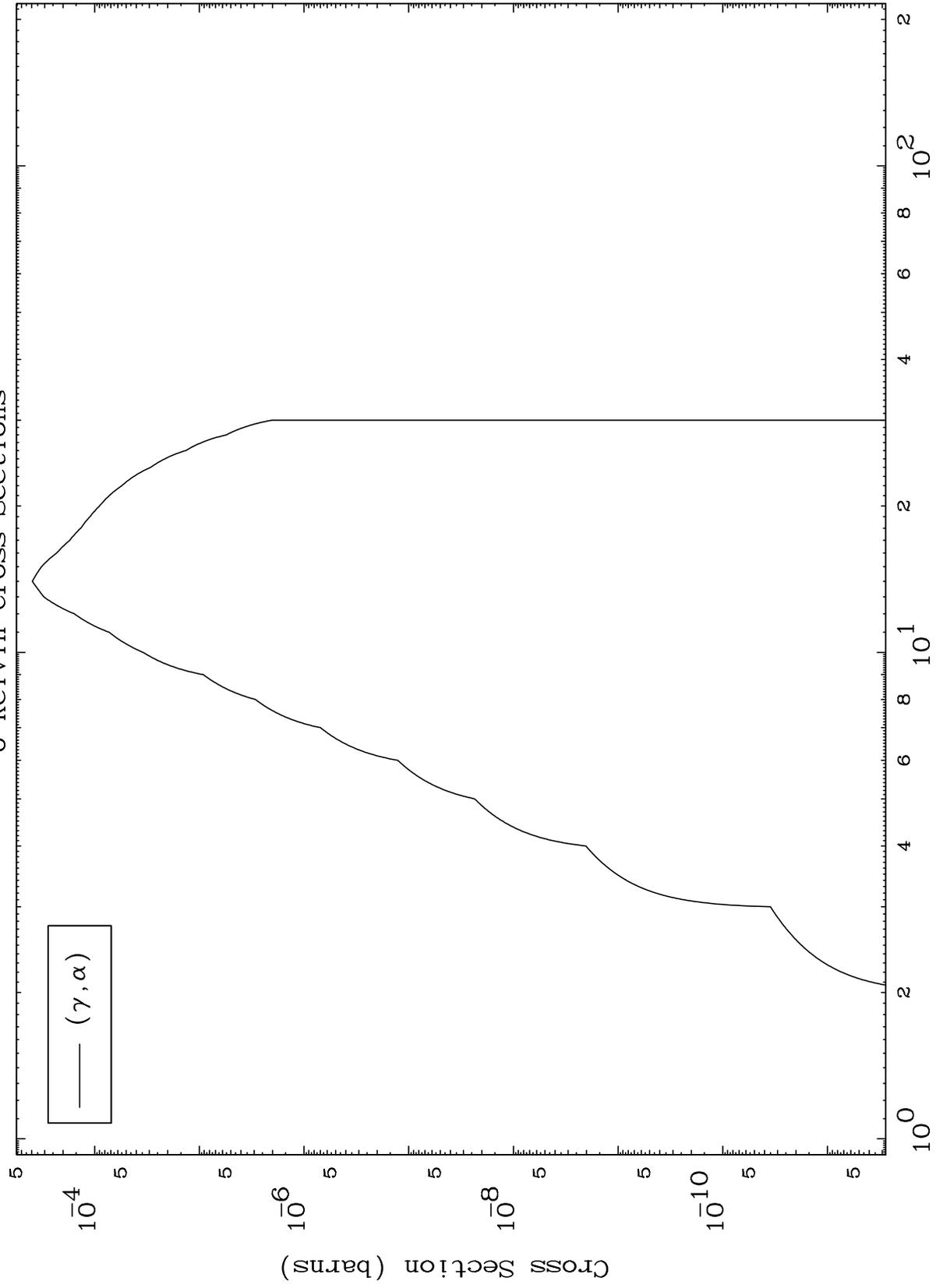
86-Rn-207



MAT 8613

( $\gamma, \alpha$ ) Levels  
0 Kelvin Cross Sections

86-Rn-207



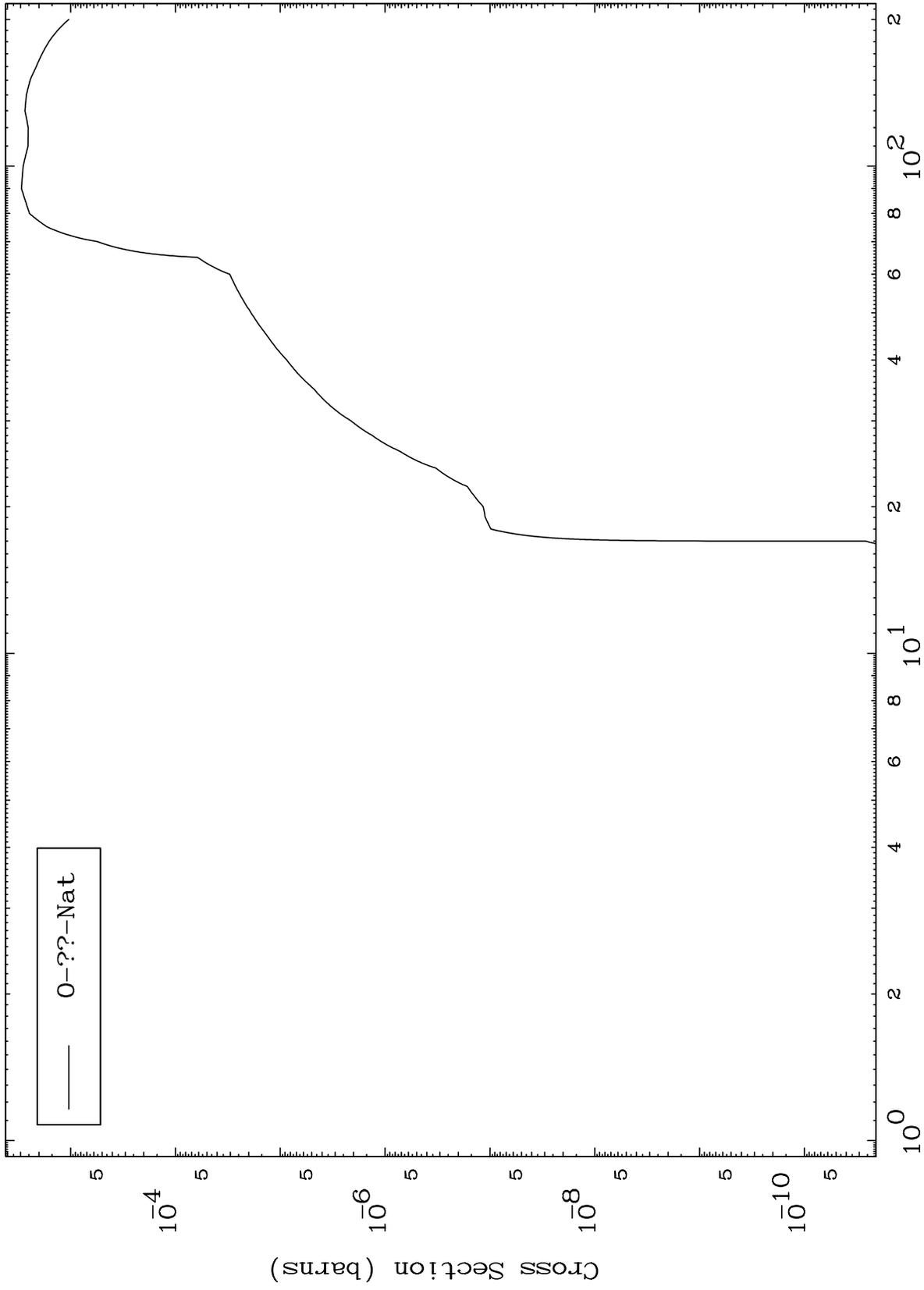
Incident Energy (MeV)

86-Rn-207

MAT 8613

Photon Fission  
Radionuclide Production Cross Section

86-Rn-207



11

Incident Energy (MeV)

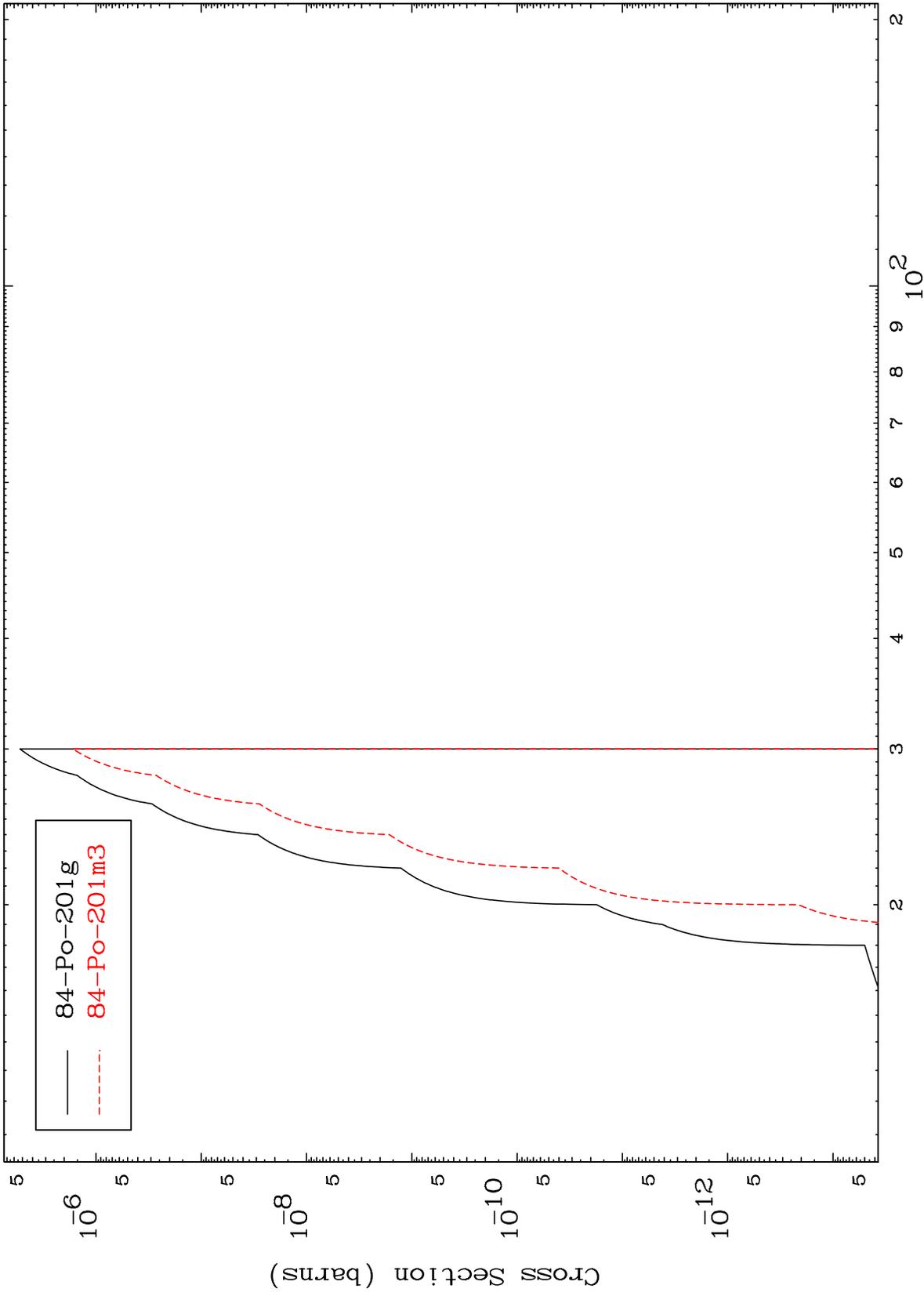
86-Rn-207

MAT 8613

( $\gamma, 2n$ )  $\alpha$

86-Rn-207

Radionuclide Production Cross Section



12

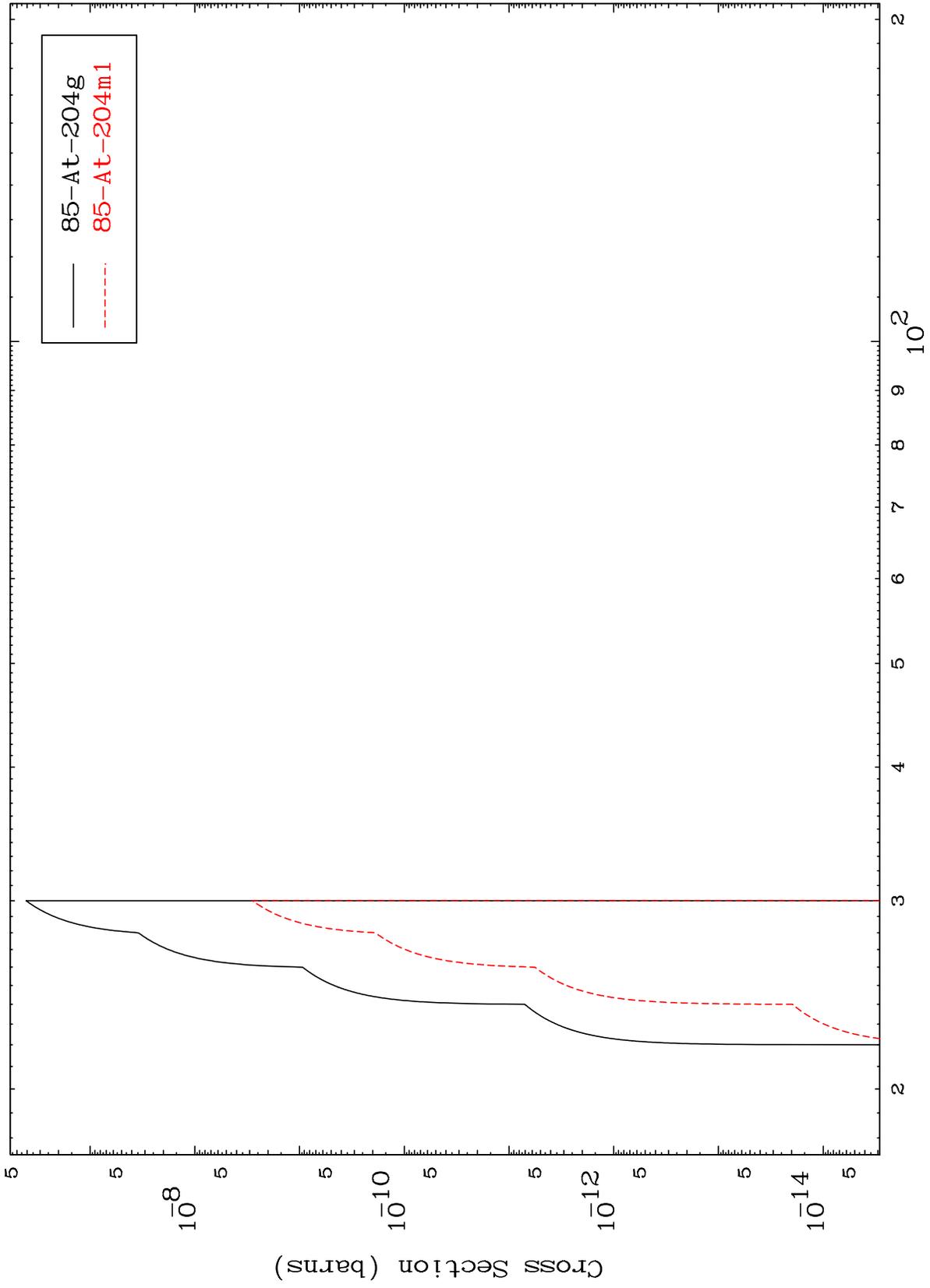
86-Rn-207

MAT 8613

( $\gamma, n'$ ) d

86-Rn-207

Radionuclide Production Cross Section



13

Incident Energy (MeV)

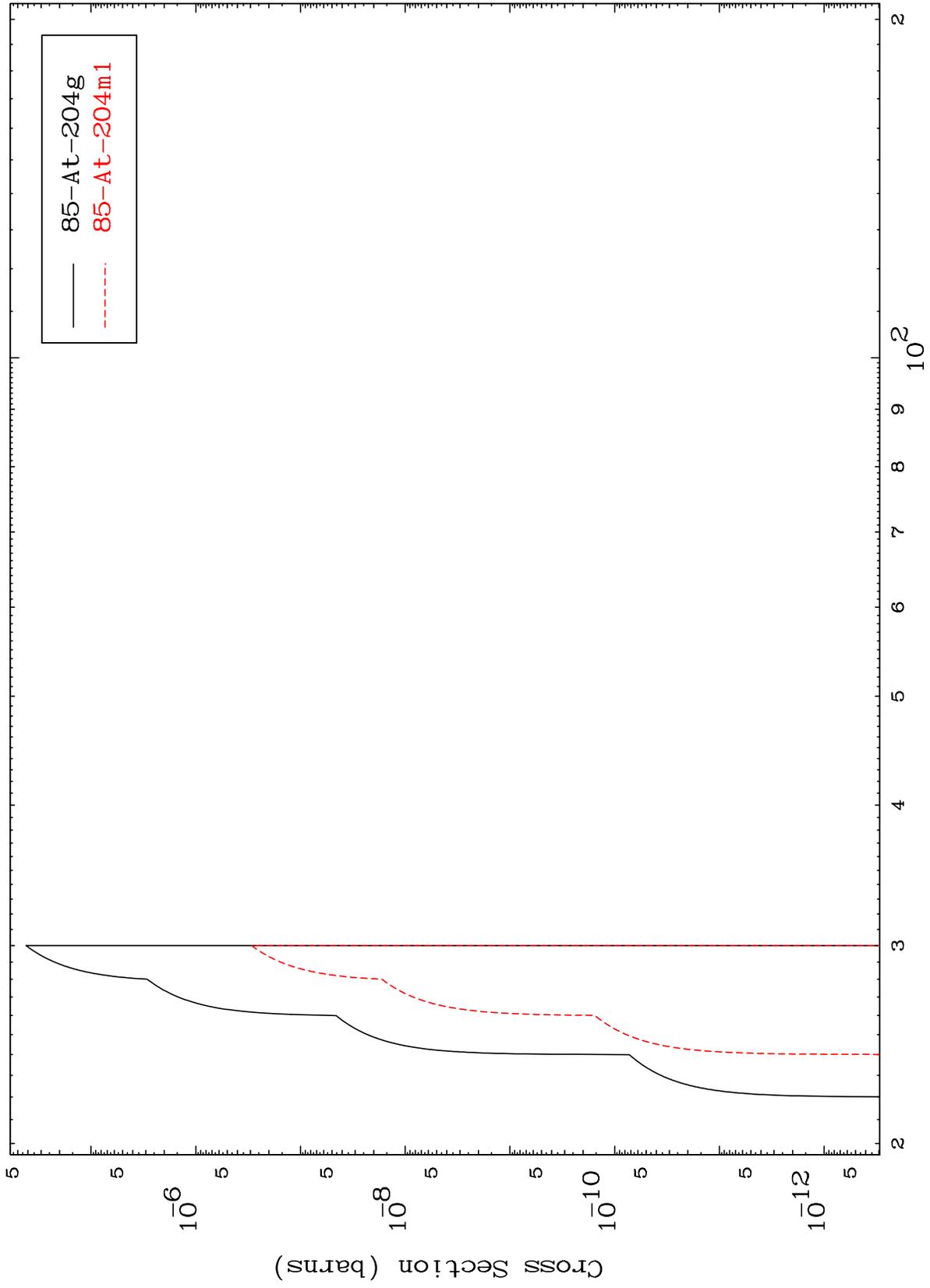
86-Rn-207

MAT 8613

( $\gamma, 2n$ ) p

86-Rn-207

Radionuclide Production Cross Section



14

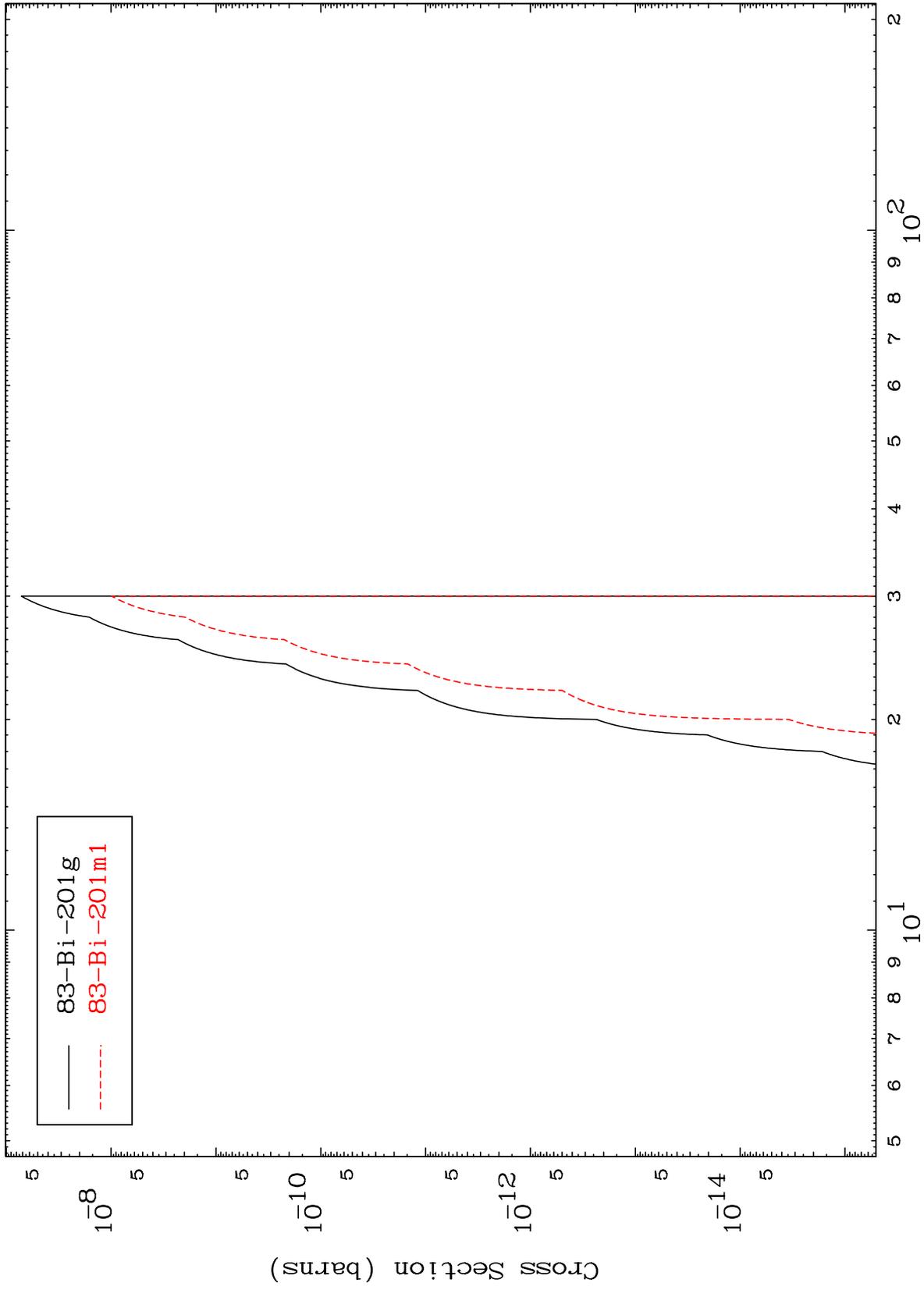
Incident Energy (MeV)

86-Rn-207

MAT 8613

86-Rn-207

$(\gamma, n')$  p  $\alpha$   
Radionuclide Production Cross Section



15

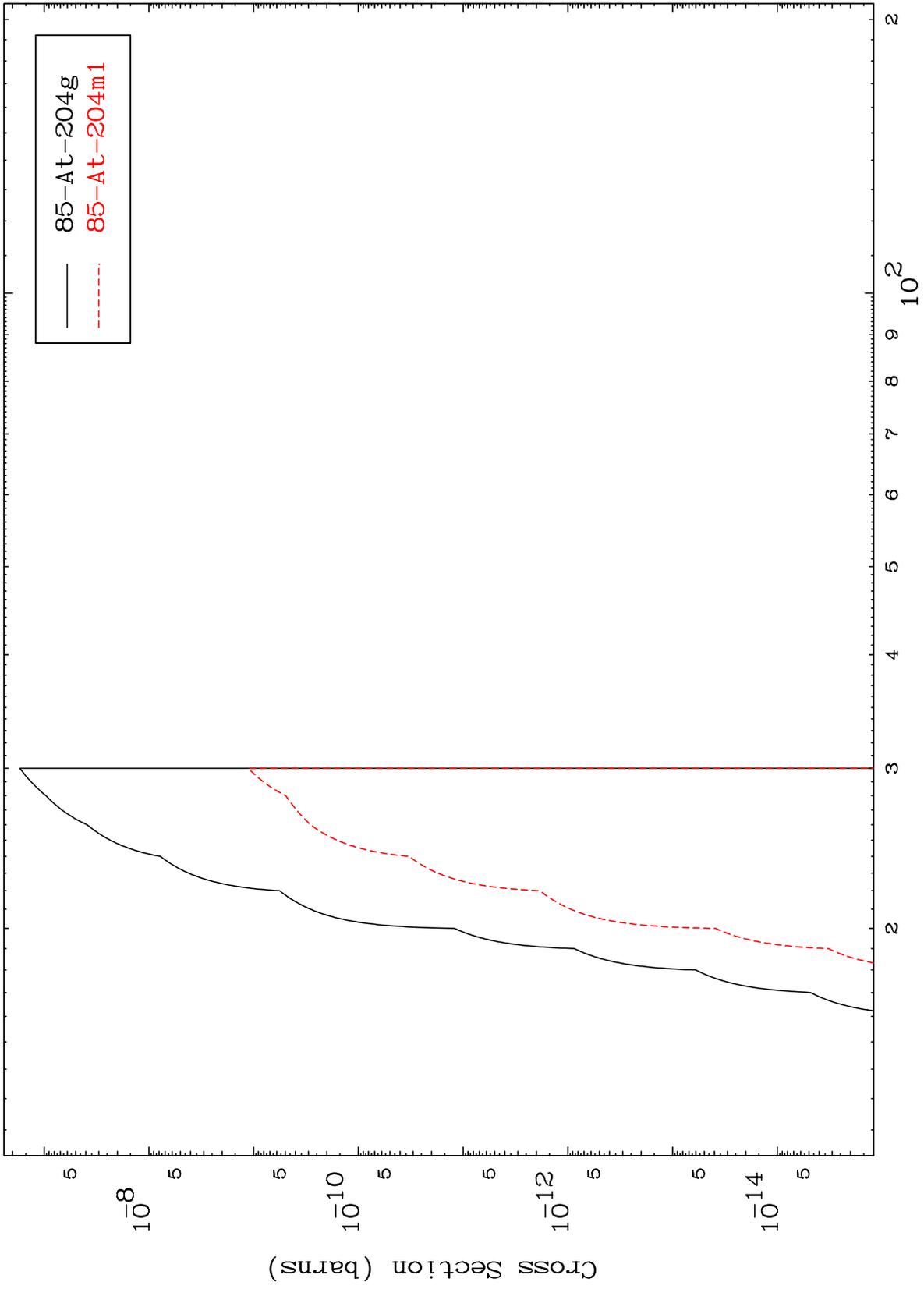
Incident Energy (MeV)

86-Rn-207

MAT 8613

86-Rn-207

( $\gamma, t$ )  
Radionuclide Production Cross Section



16

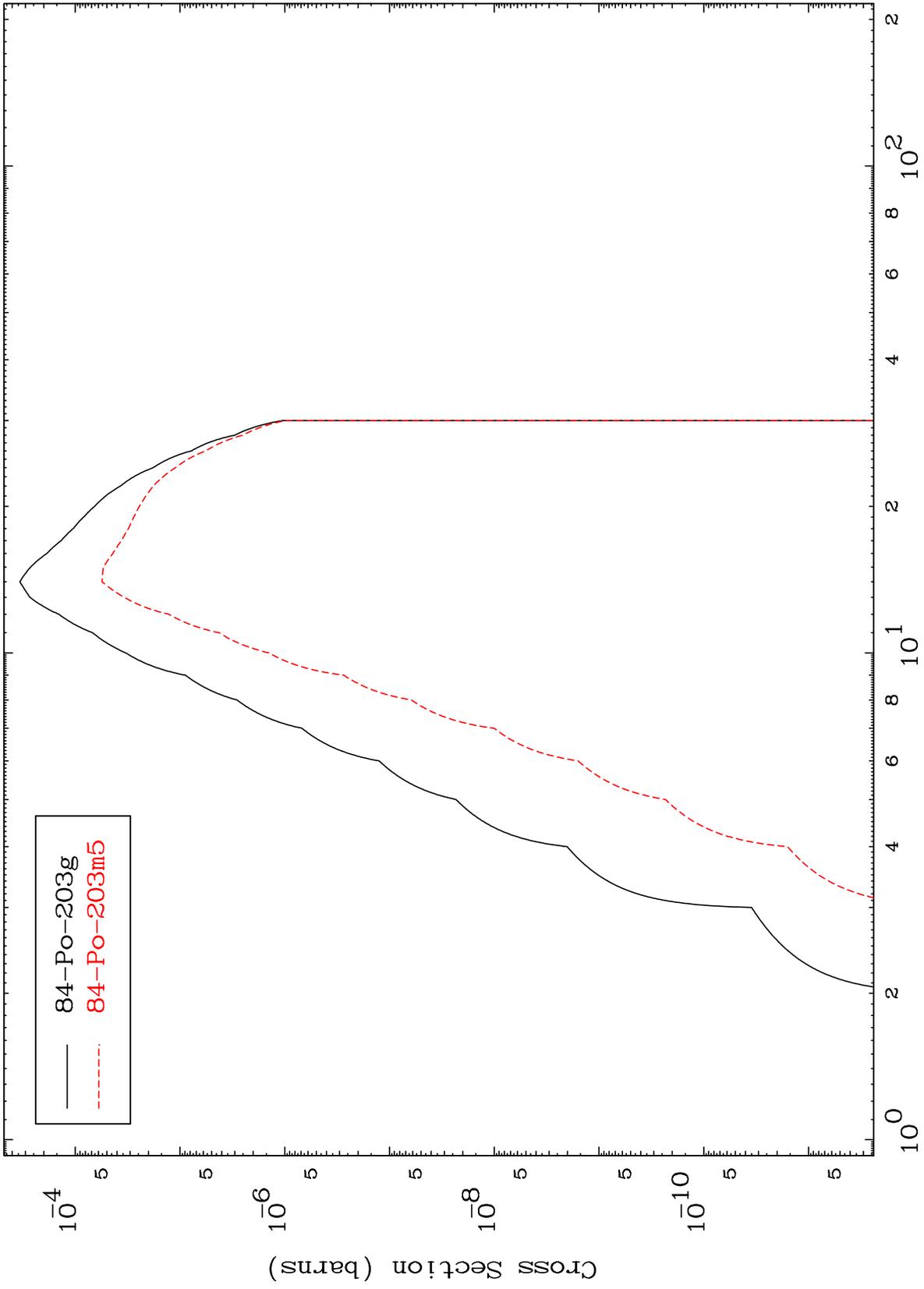
Incident Energy (MeV)

86-Rn-207

MAT 8613

Radionuclide Production Cross Section  
( $\gamma, \alpha$ )

86-Rn-207



17

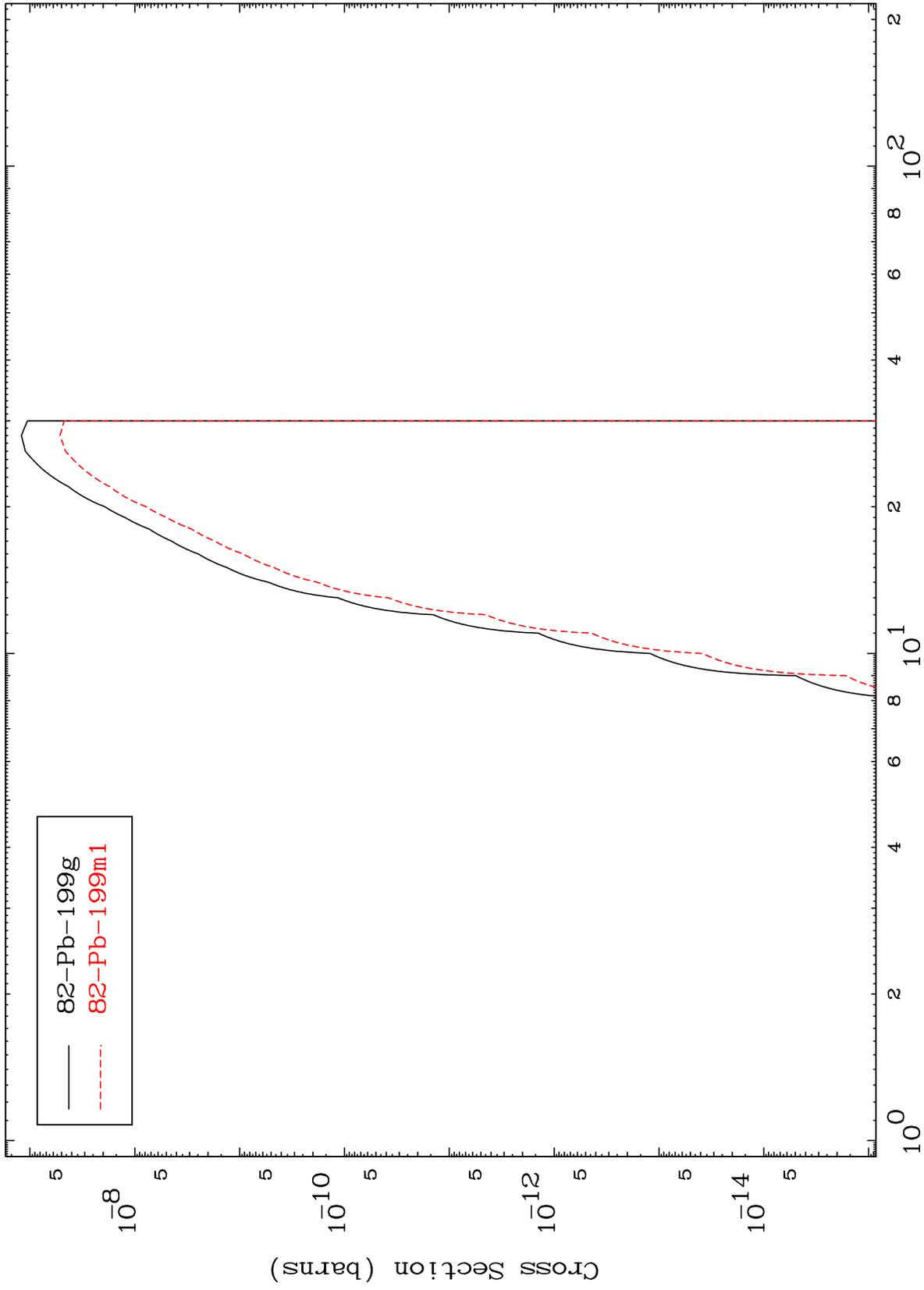
Incident Energy (MeV)

86-Rn-207

MAT 8613

86-Rn-207

Radionuclide Production Cross Section  
( $\gamma, 2\alpha$ )



18

Incident Energy (MeV)

86-Rn-207