

Program EVALPLOT
(Version 2018-1)

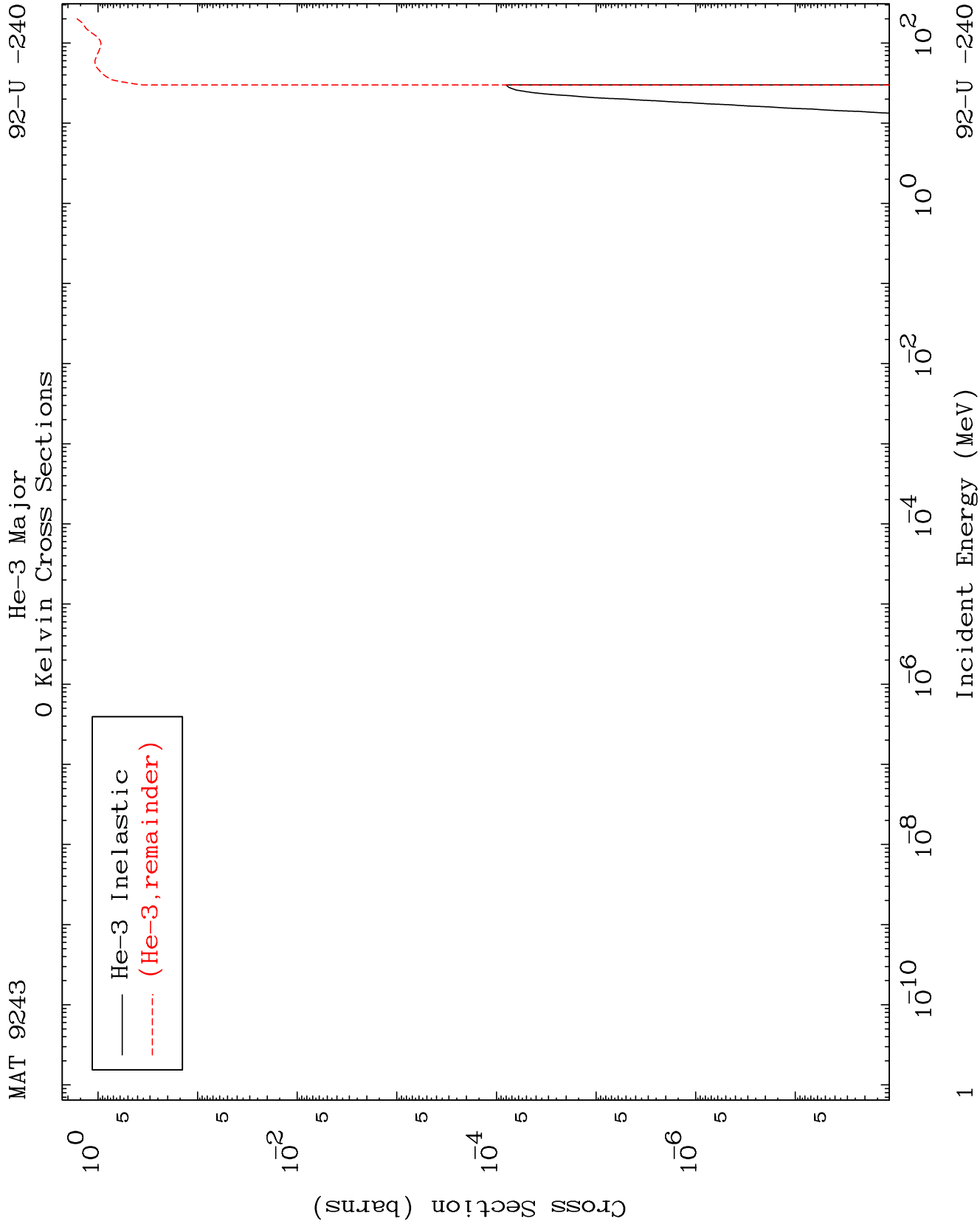
by

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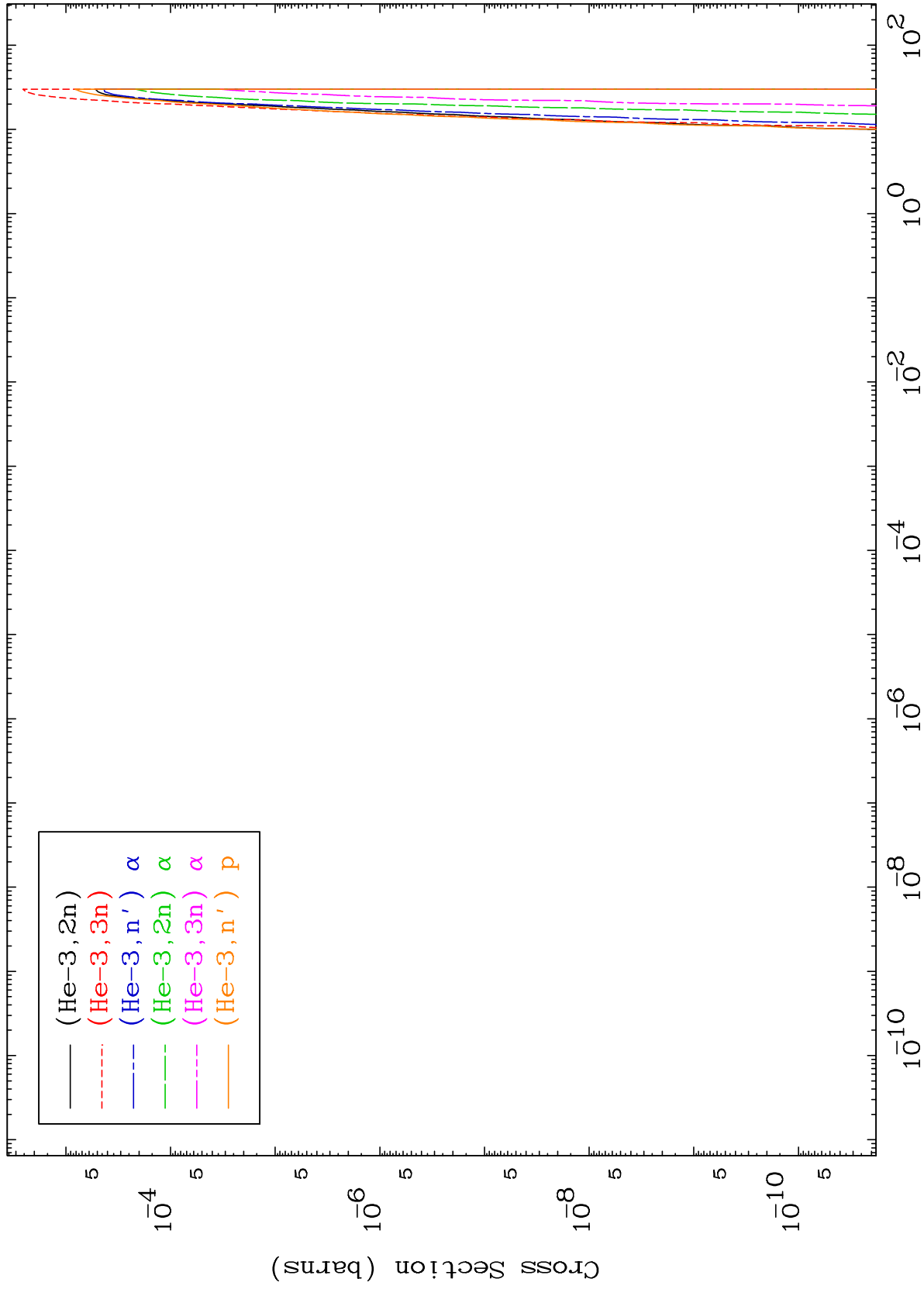
Press Mouse Button to Start



MAT 9243

He-3 Neutron Production
0 Kelvin Cross Sections

92-U -240



2

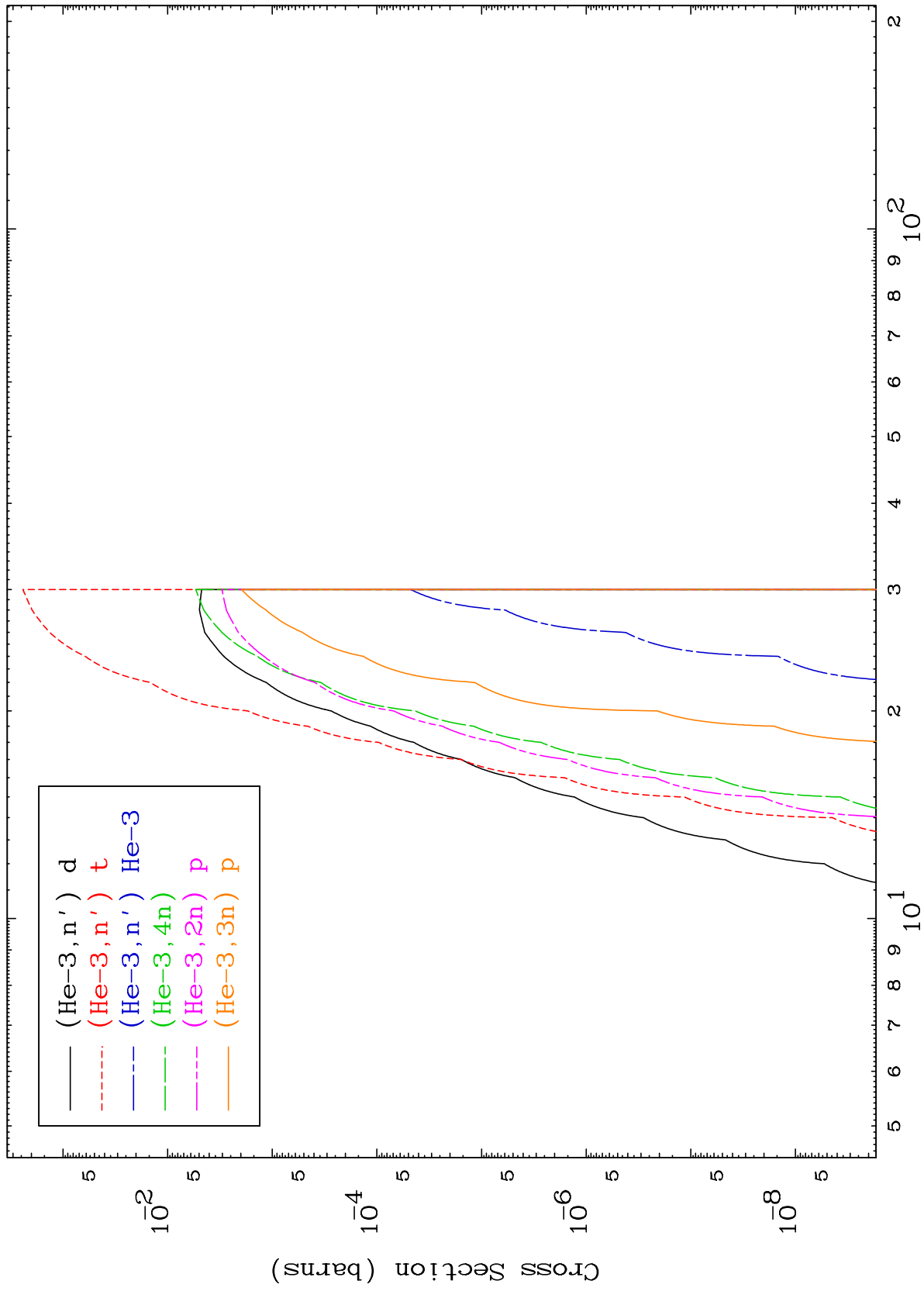
Incident Energy (MeV)

92-U -240

MAT 9243

He-3 Neutron Production
0 Kelvin Cross Sections

92-U -240



3

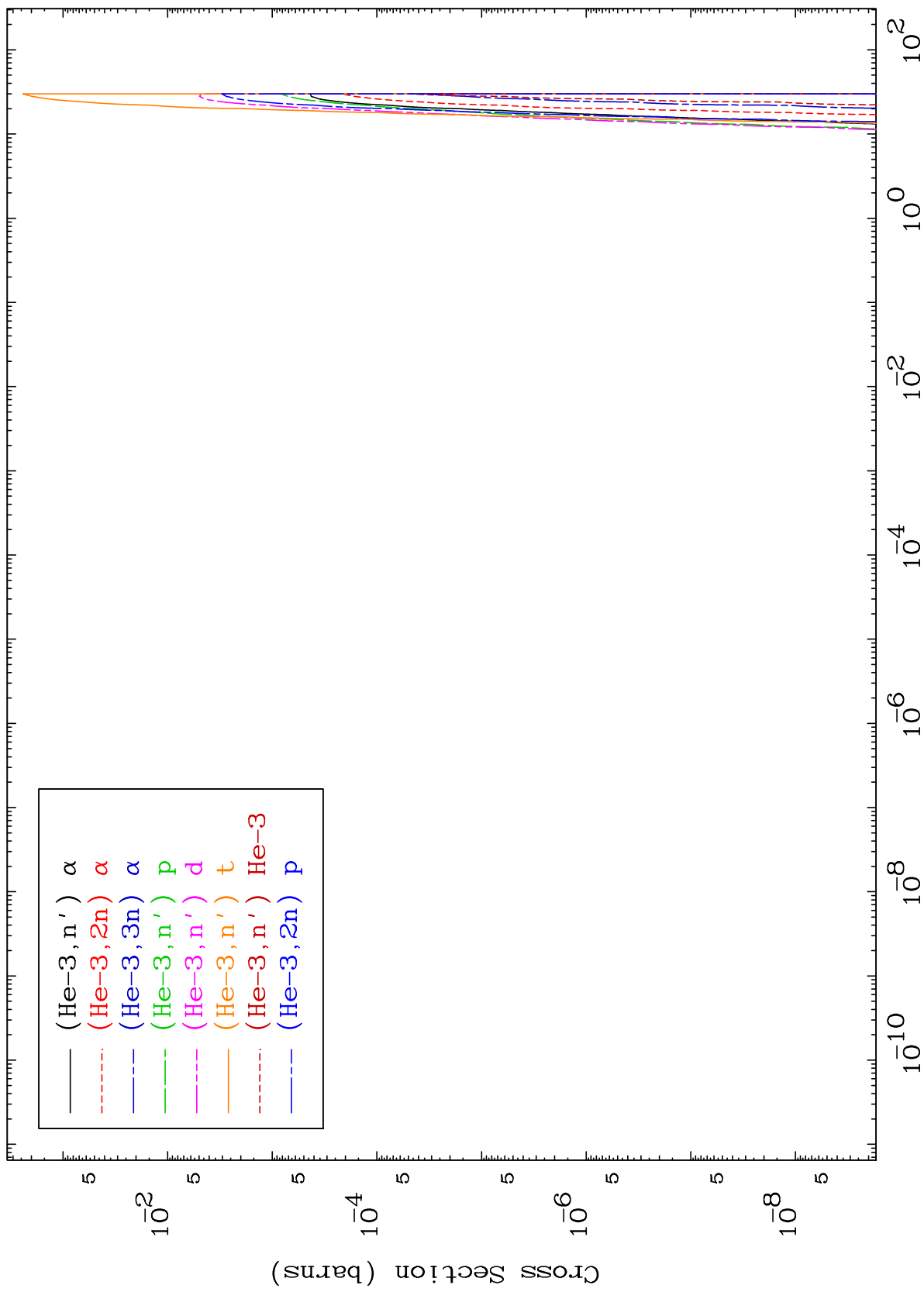
Incident Energy (MeV)

92-U -240

MAT 9243

He-3 Charged Particle
0 Kelvin Cross Sections

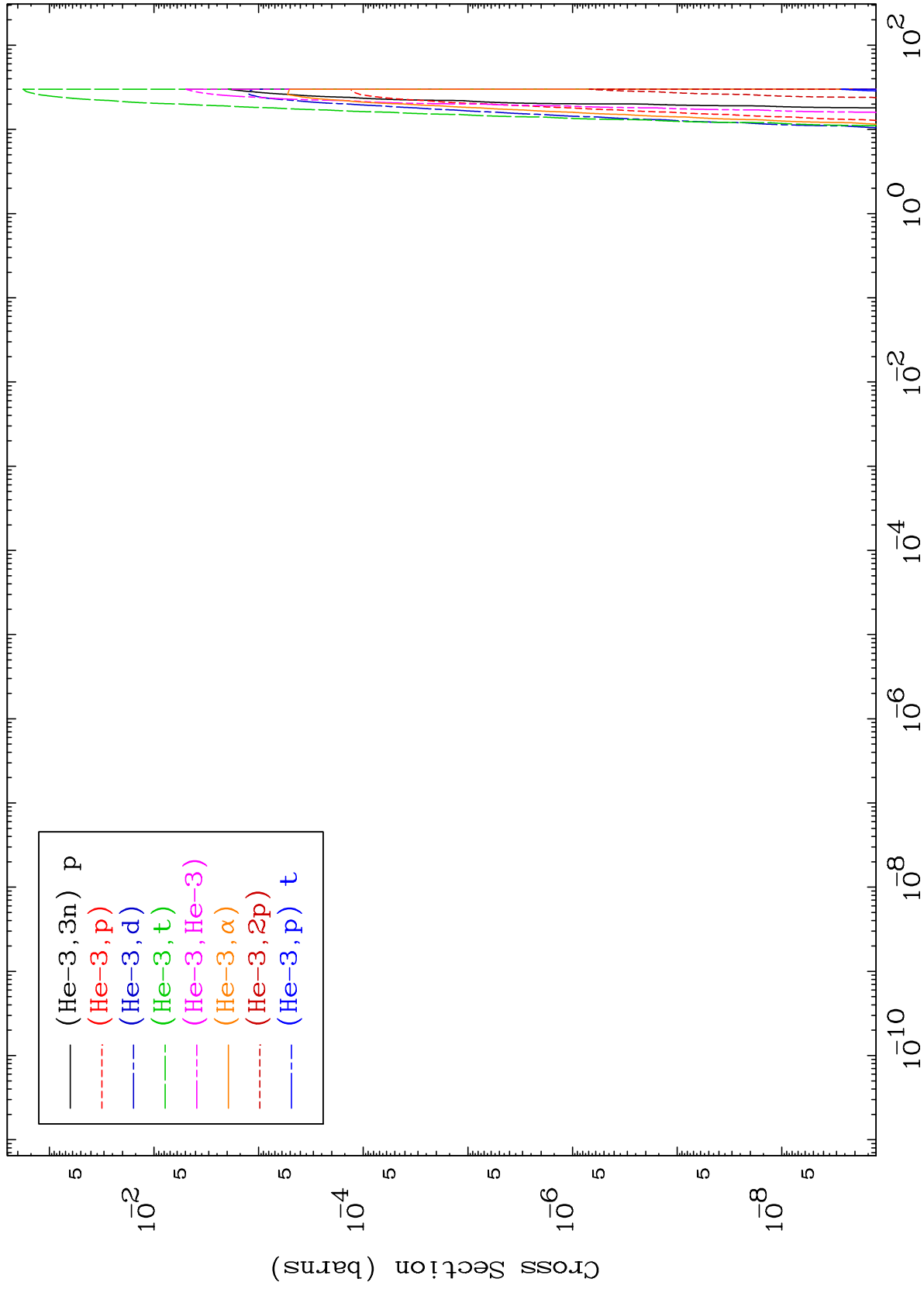
92-U -240



MAT 9243

He-3 Charged Particle
0 Kelvin Cross Sections

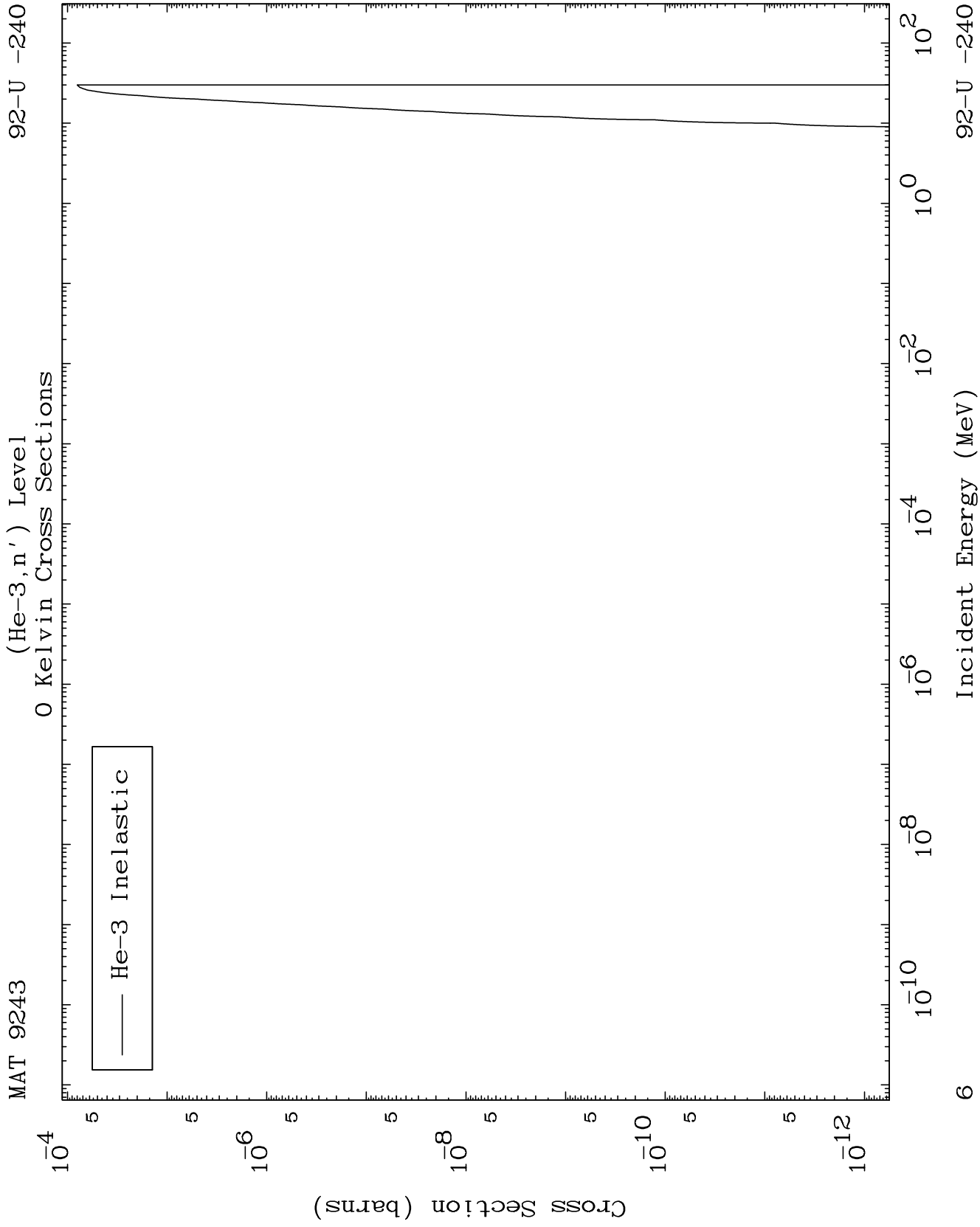
92-U -240



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Incident Energy (MeV)

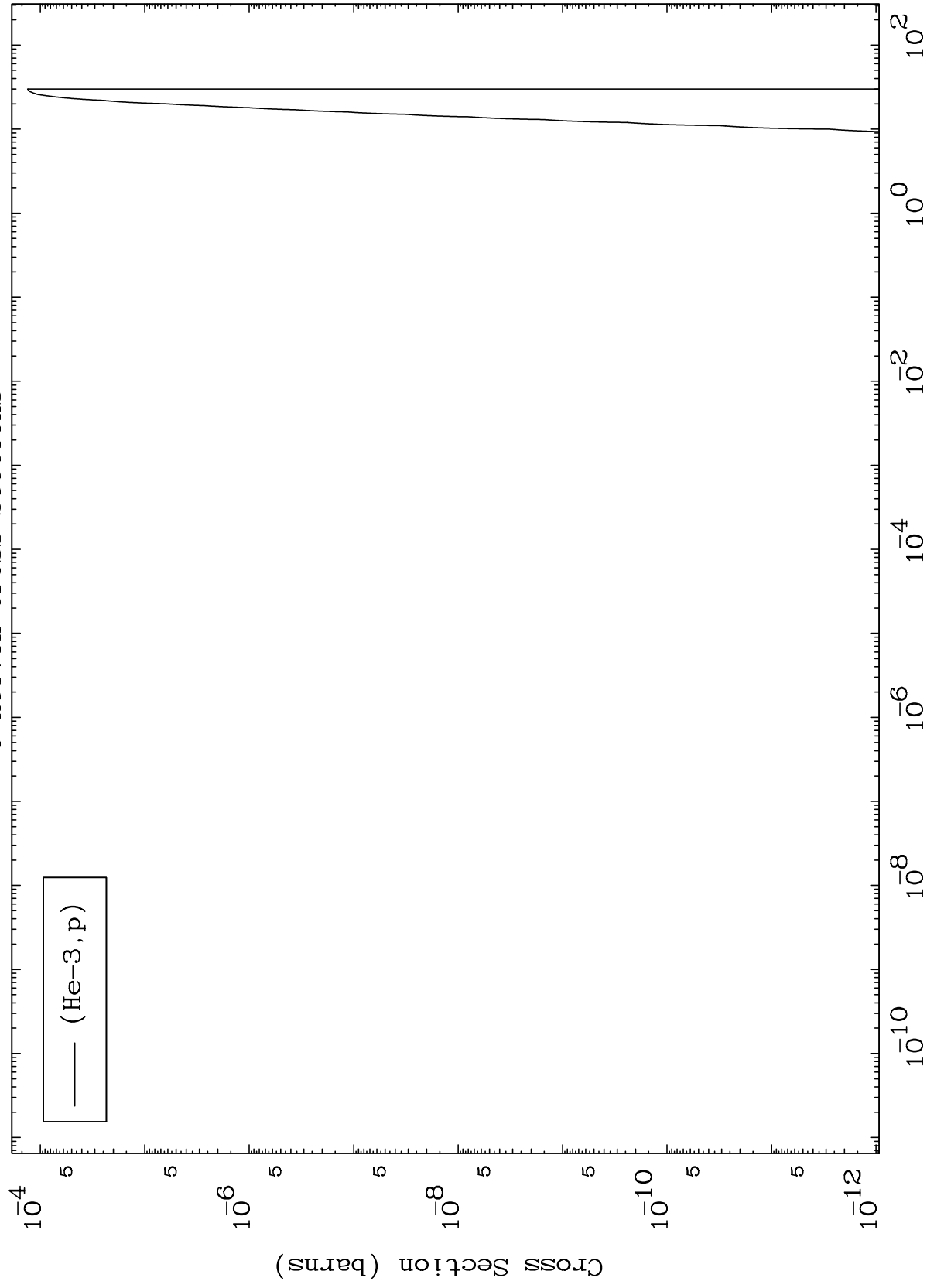
92-U -240



MAT 9243

(He-3,p) Levels
0 Kelvin Cross Sections

92-U -240



7

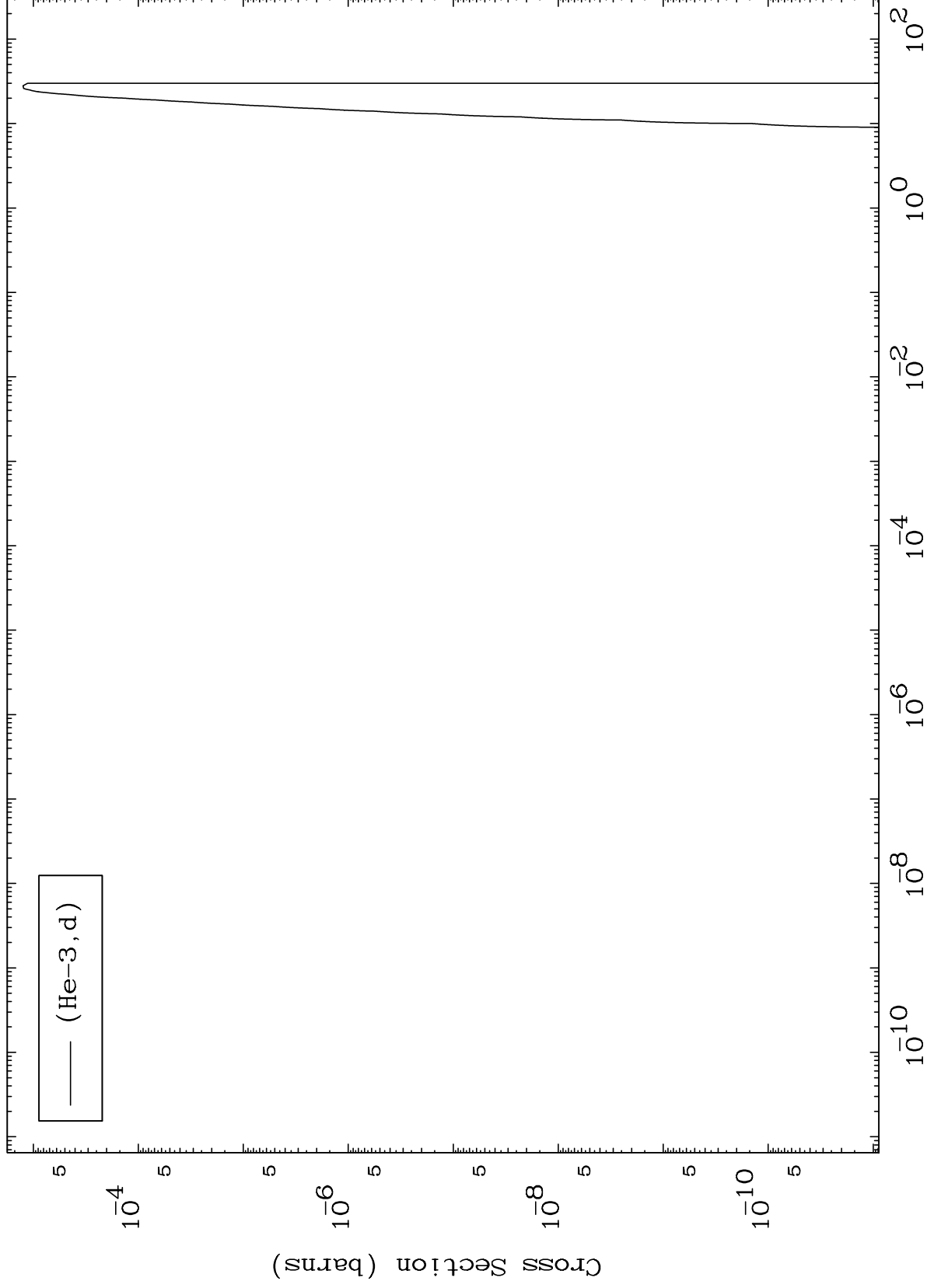
Incident Energy (MeV)

92-U -240

MAT 9243

(He-3,d) Levels
0 Kelvin Cross Sections

92-U -240



8

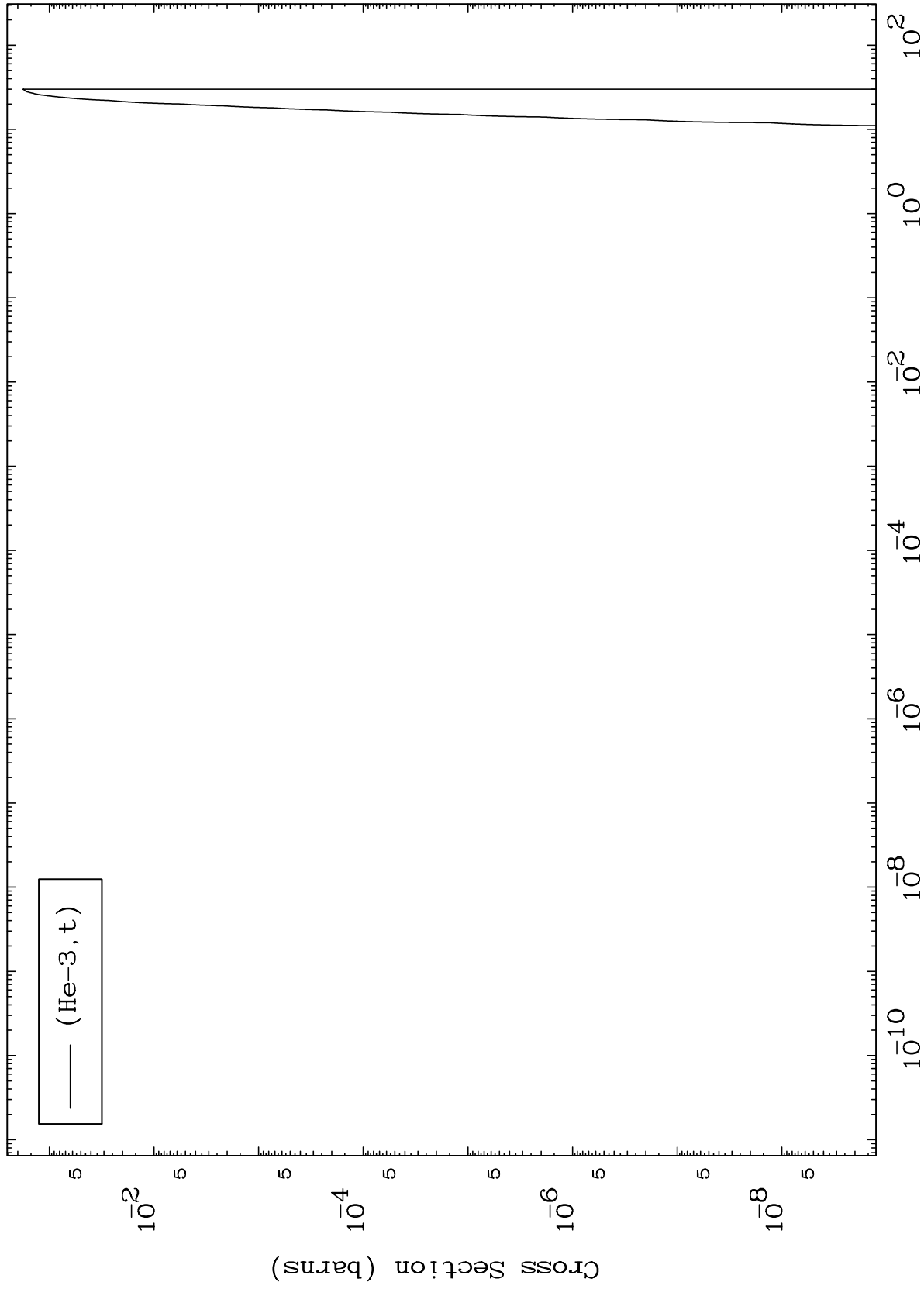
Incident Energy (MeV)

92-U -240

MAT 9243

(He-3,t) Levels
0 Kelvin Cross Sections

92-U -240



9

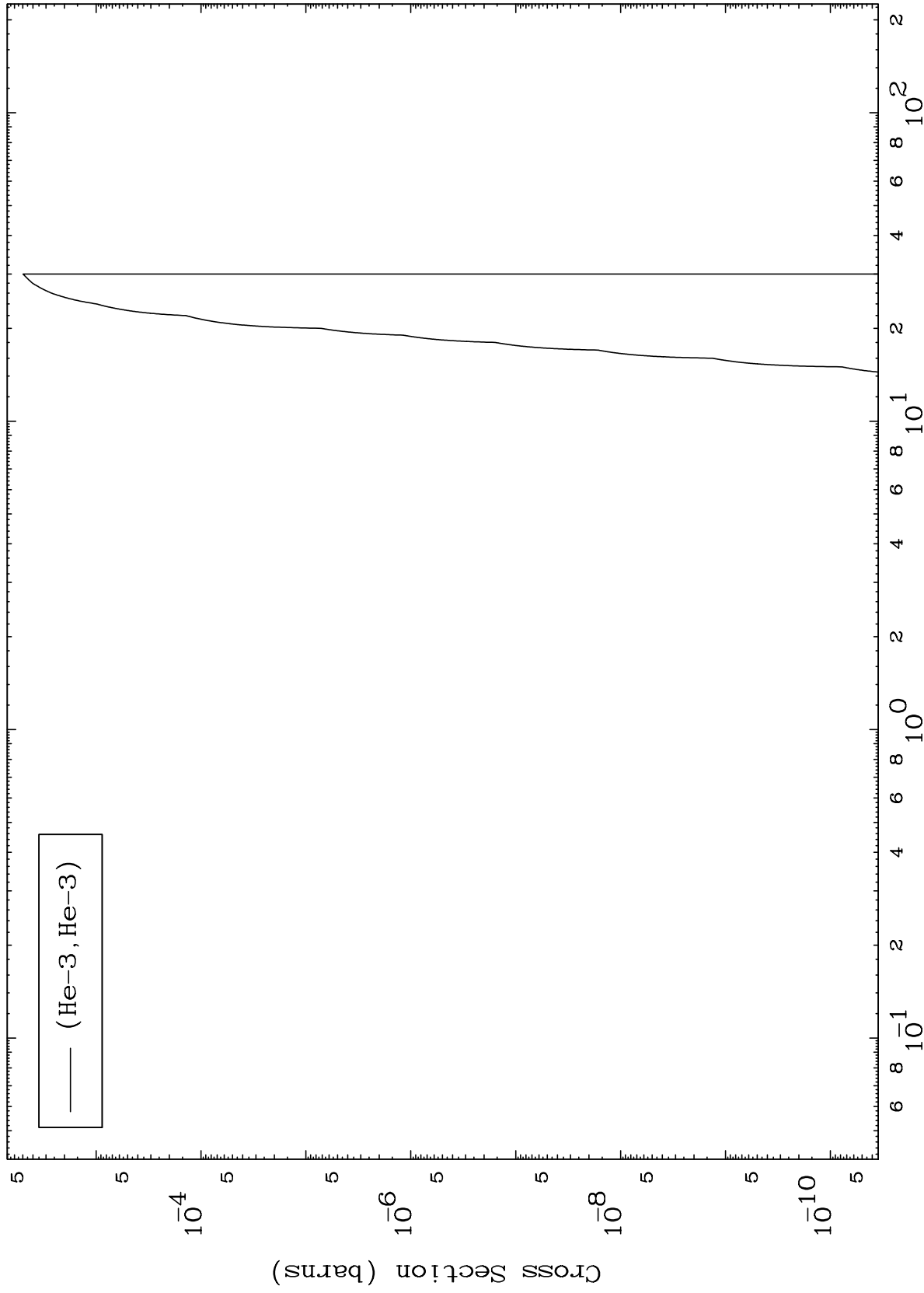
Incident Energy (MeV)

92-U -240

MAT 9243

(He-3, He3) Levels
0 Kelvin Cross Sections

92-U -240



10

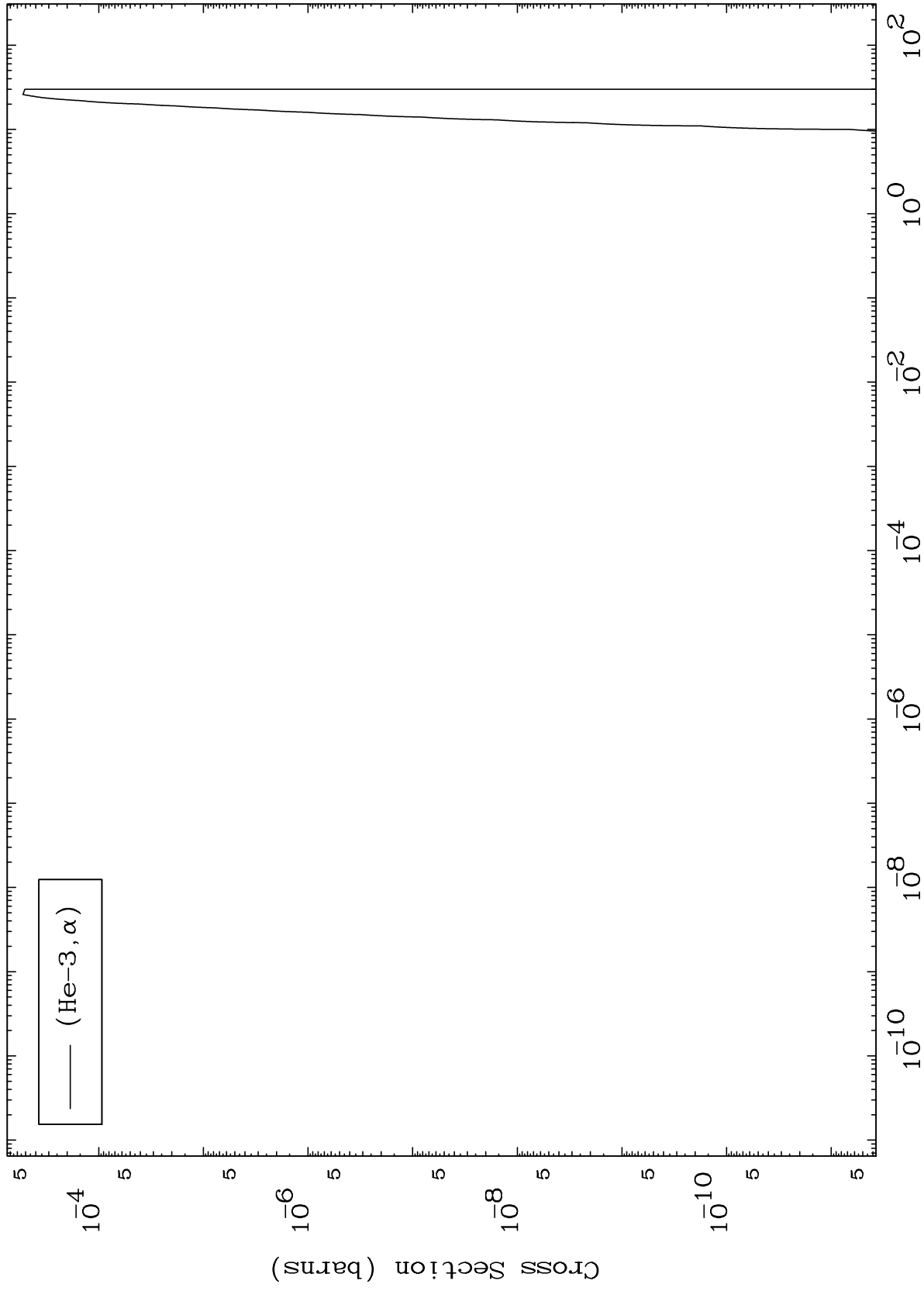
Incident Energy (MeV)

92-U -240

MAT 9243

(He-3, α) Levels
0 Kelvin Cross Sections

92-U -240



11

Incident Energy (MeV)

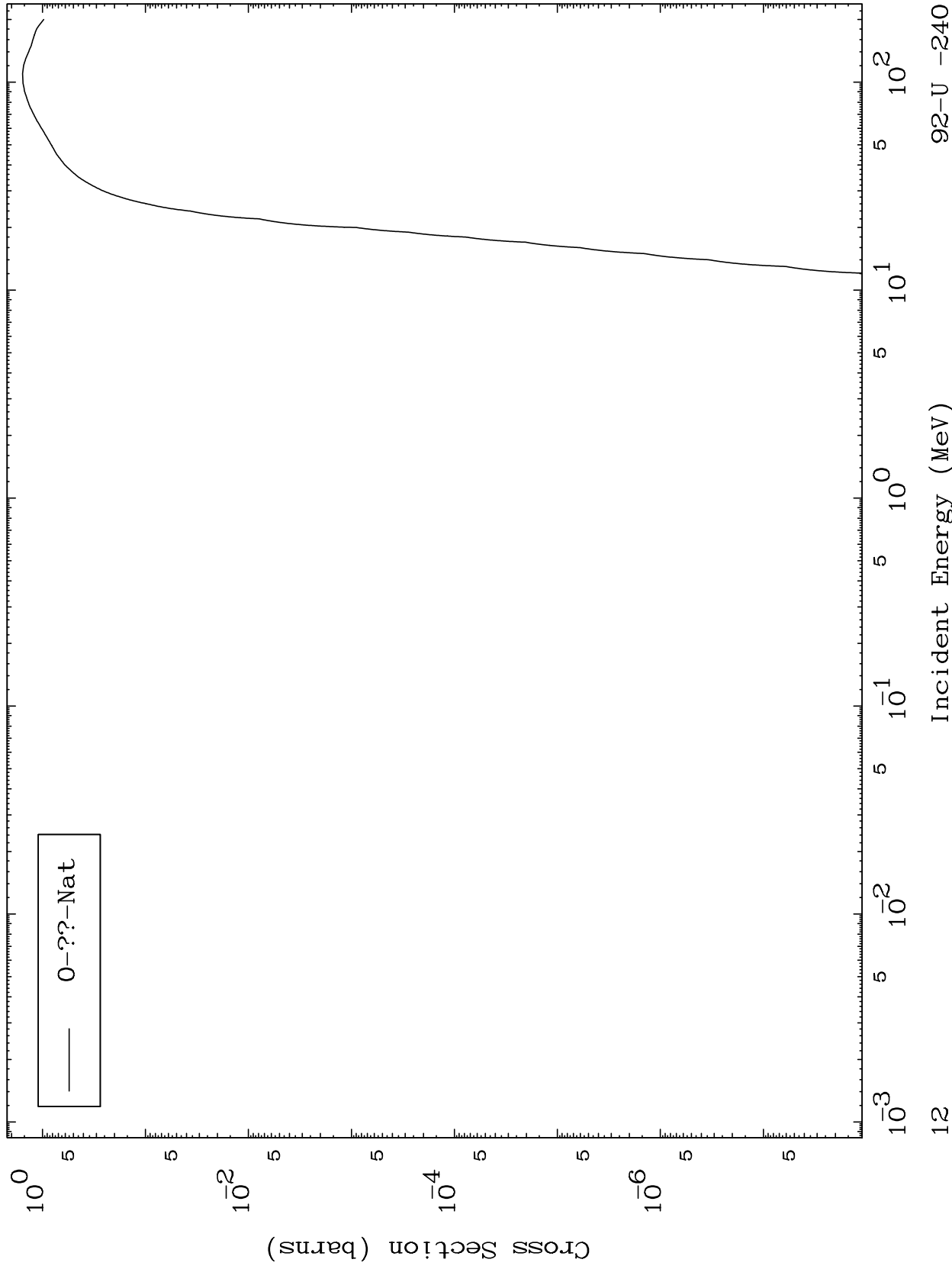
92-U -240

MAT 9243

He-3 Fission

92-U -240

Radionuclide Production Cross Section

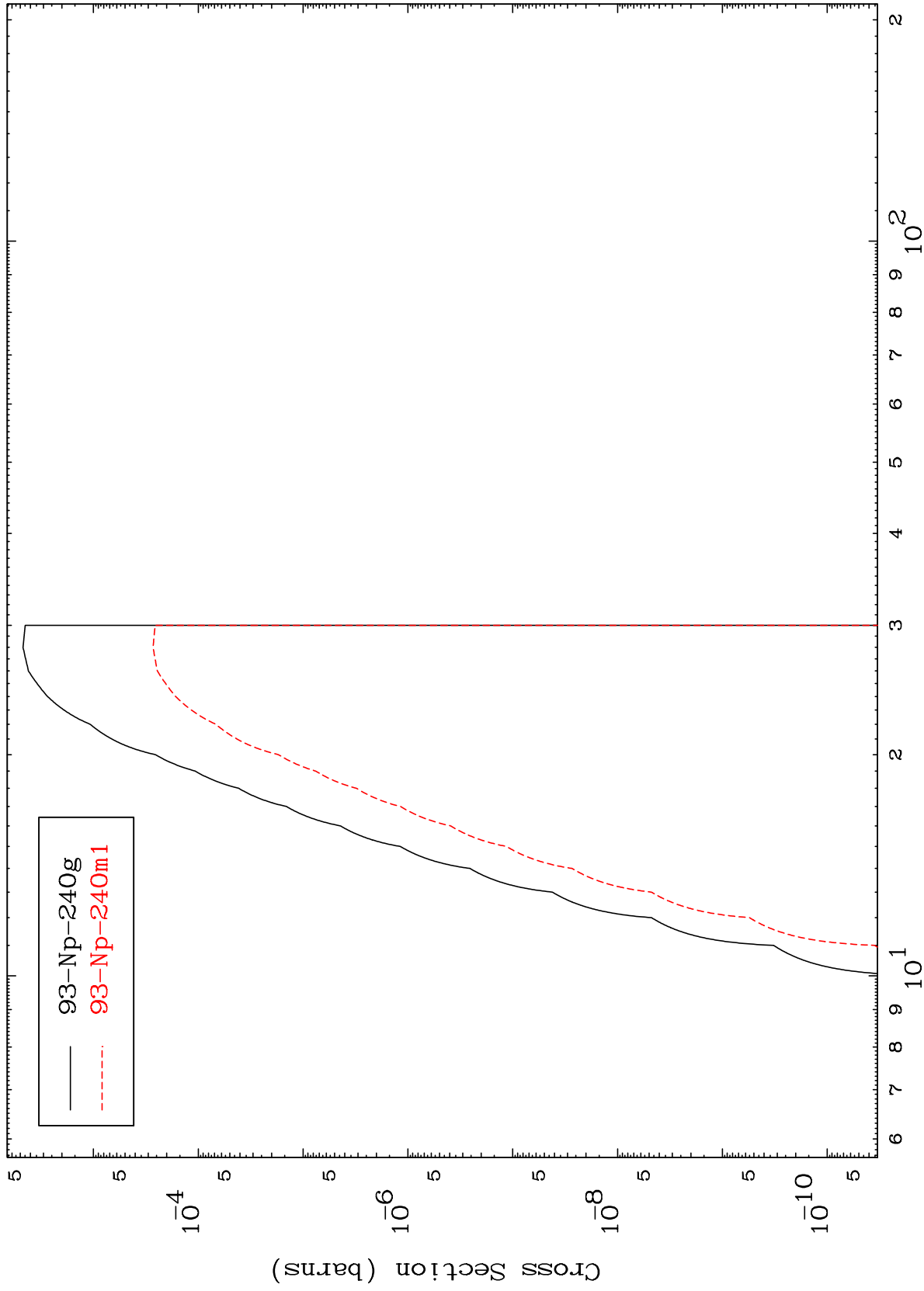


MAT 9243

(He-3,n') d

92-U -240

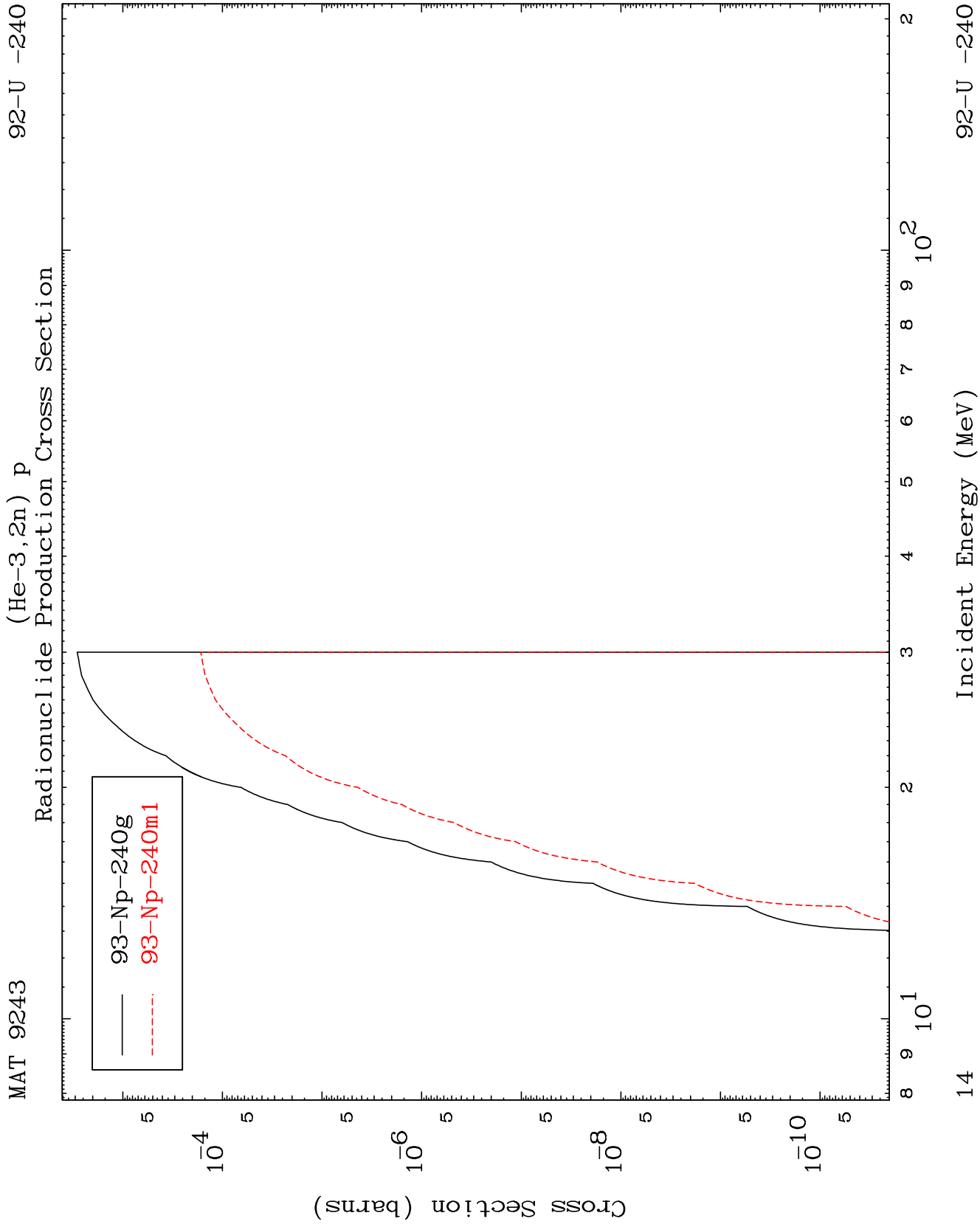
Radionuclide Production Cross Section

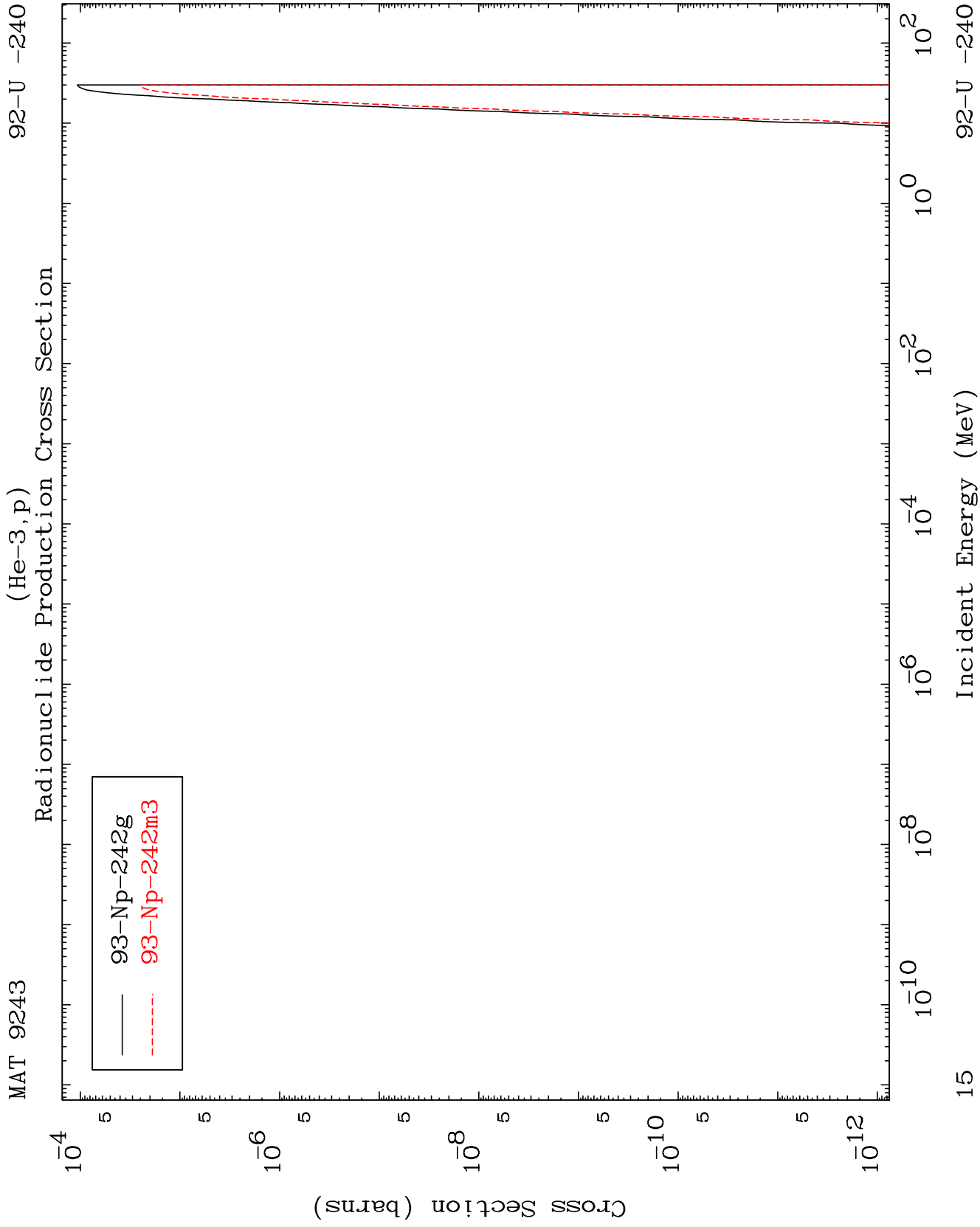


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Incident Energy (MeV)

92-U -240



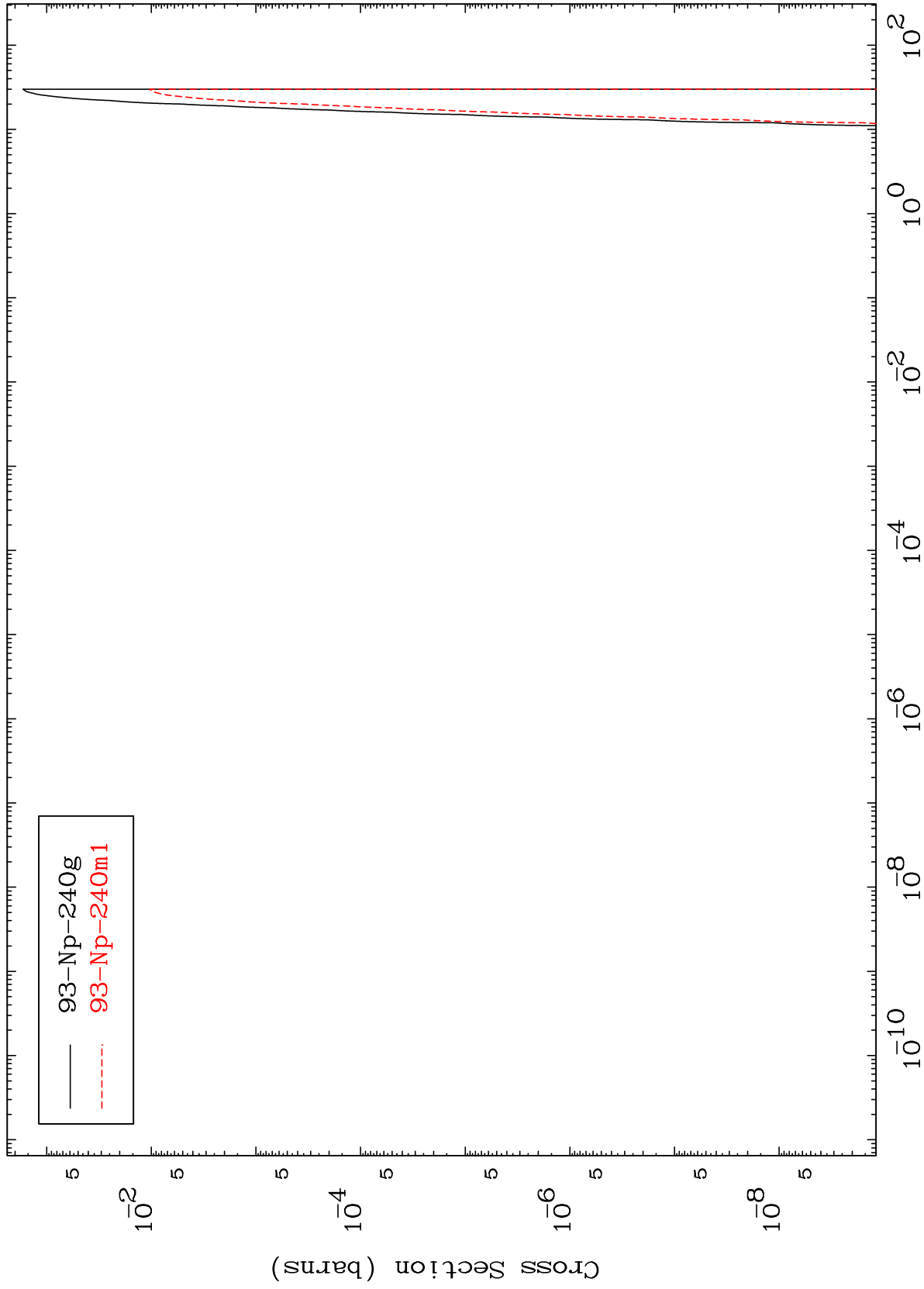


MAT 9243

(He-3, t)

92-U -240

Radionuclide Production Cross Section



16

Incident Energy (MeV)

92-U -240