

Program EVALPLOT  
(Version 2018-1)

by

Dermott E. Cullen  
(Present Contact Information)

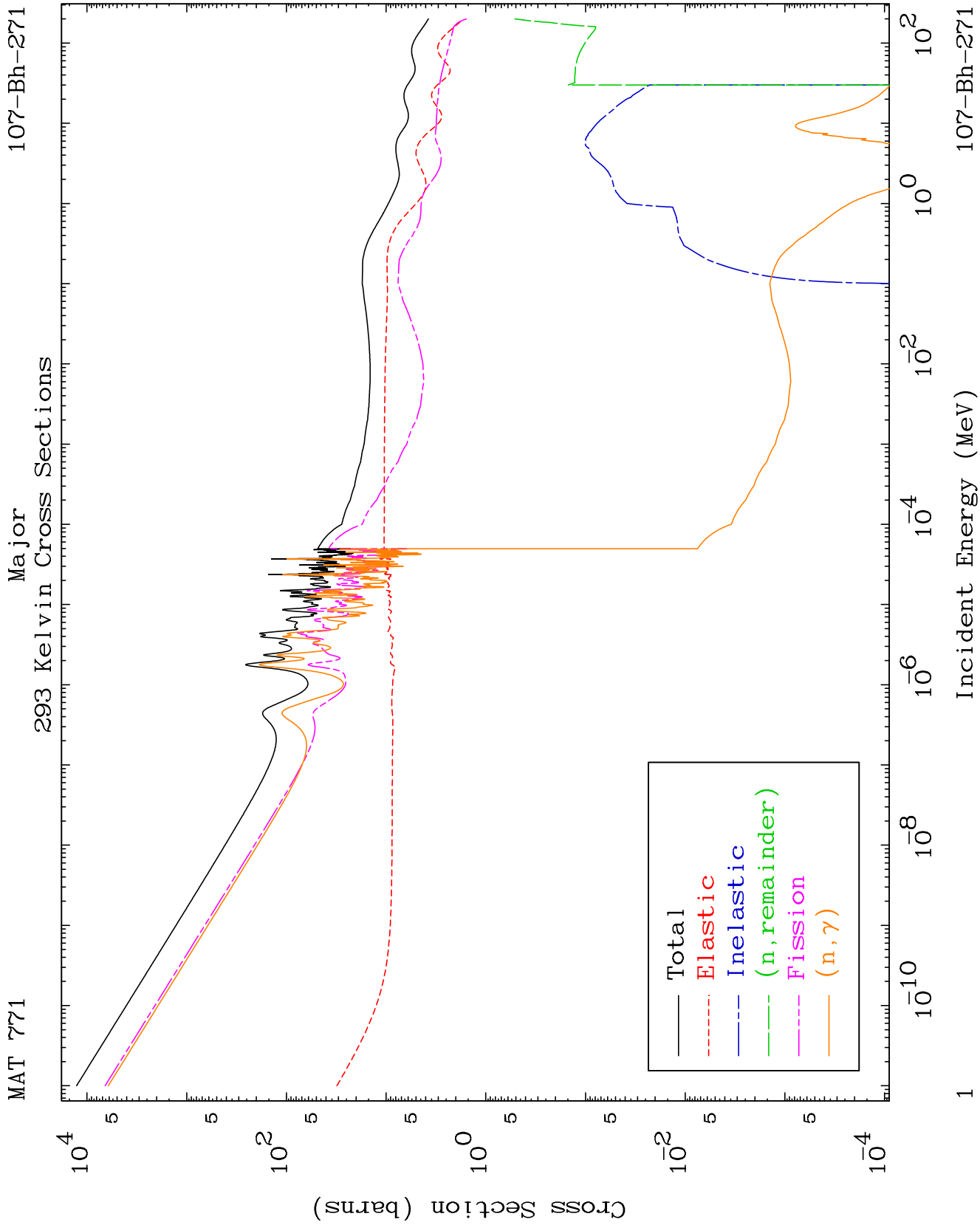
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E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

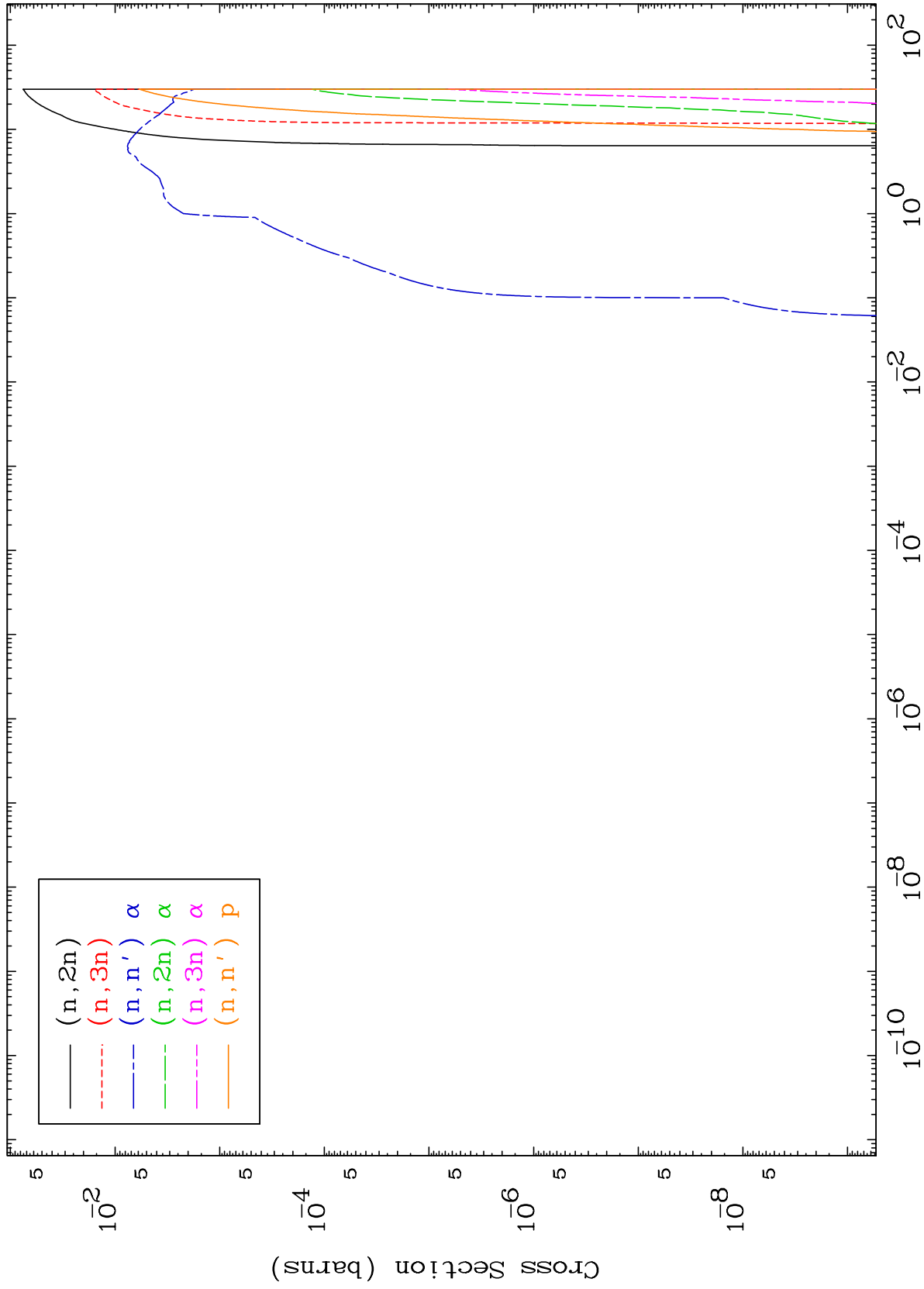
Press Mouse Button to Start



MAT 771

Neutron Production  
293 Kelvin Cross Sections

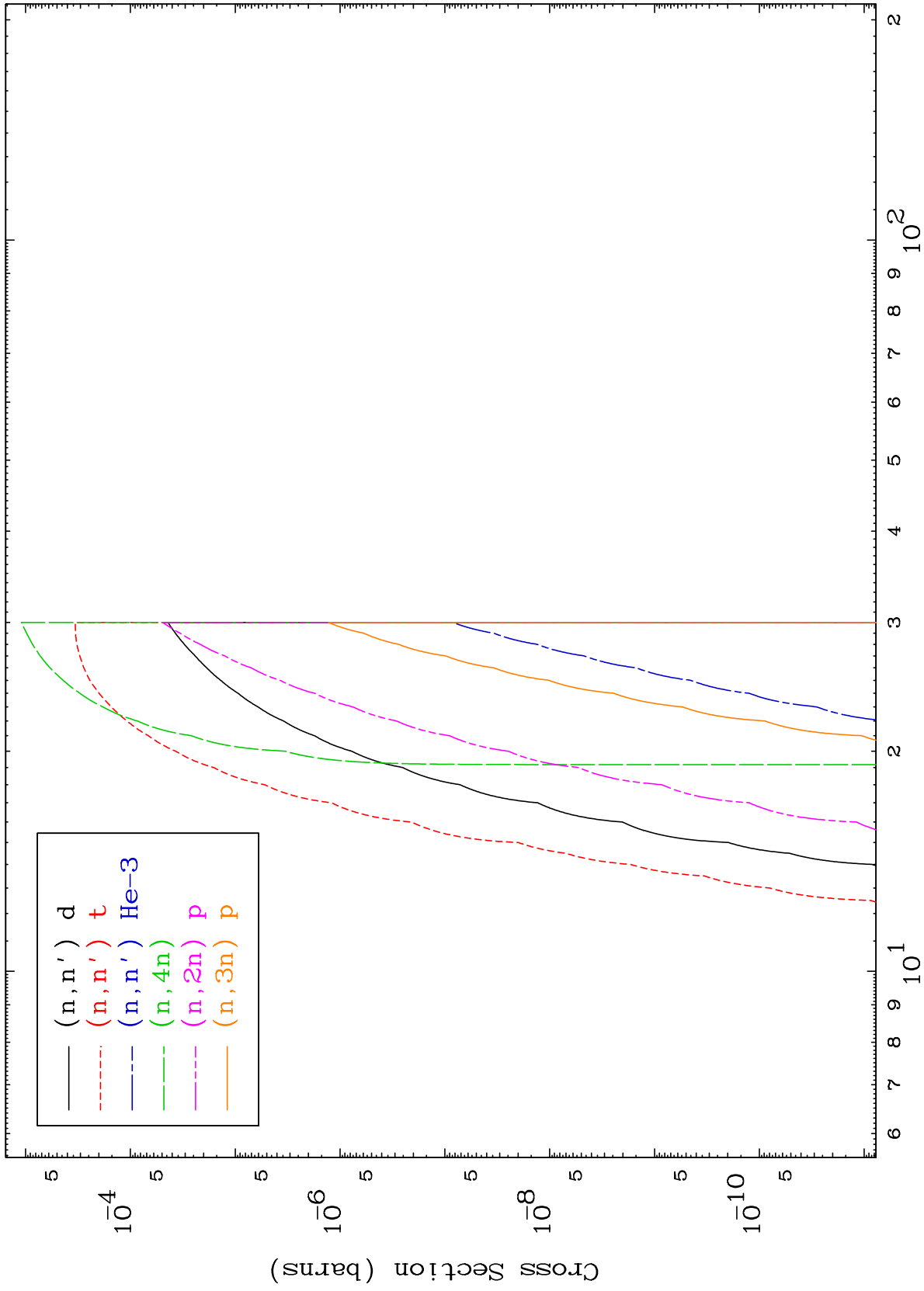
107-Bh-271



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Neutron Production  
293 Kelvin Cross Sections

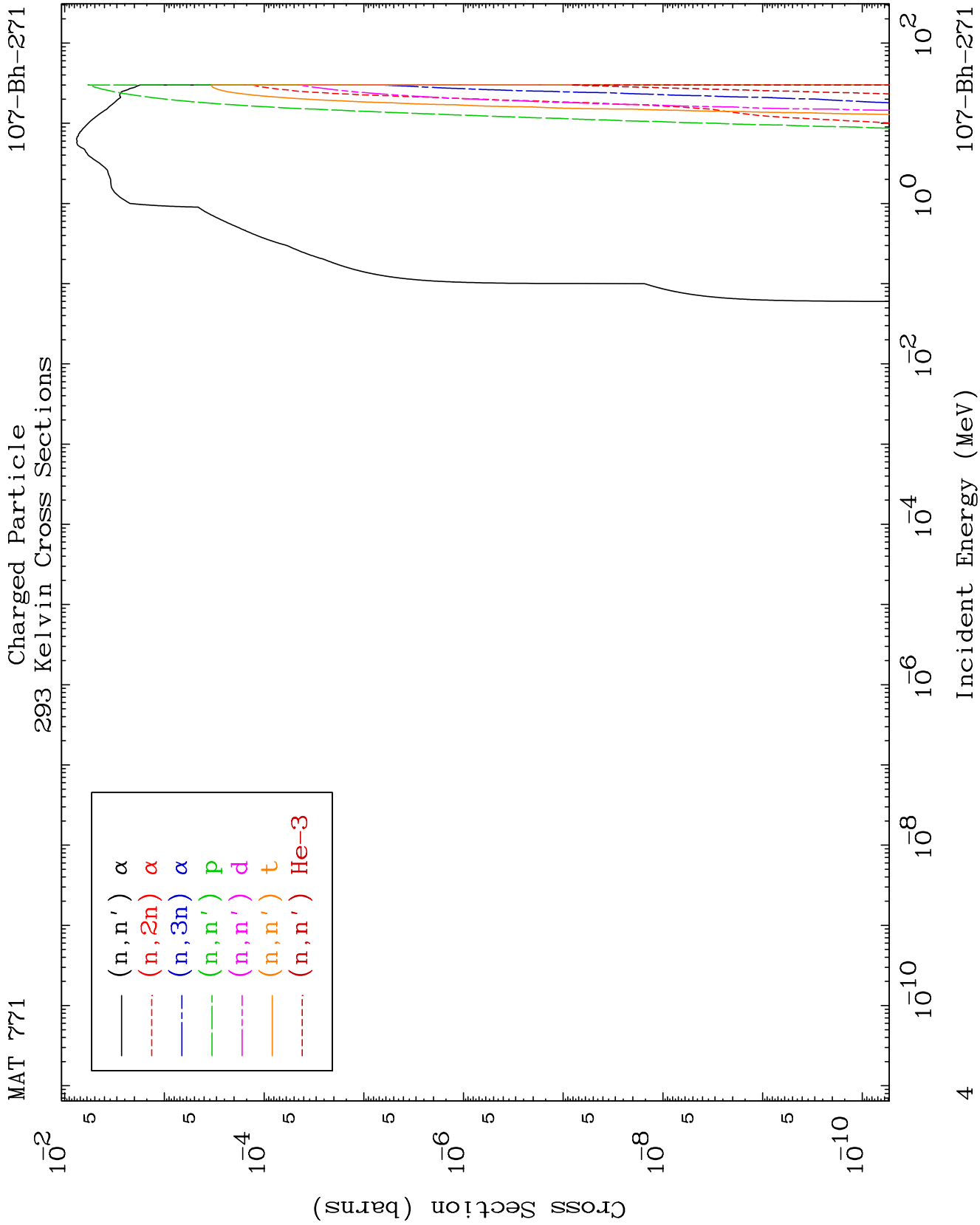
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3

Incident Energy (MeV)

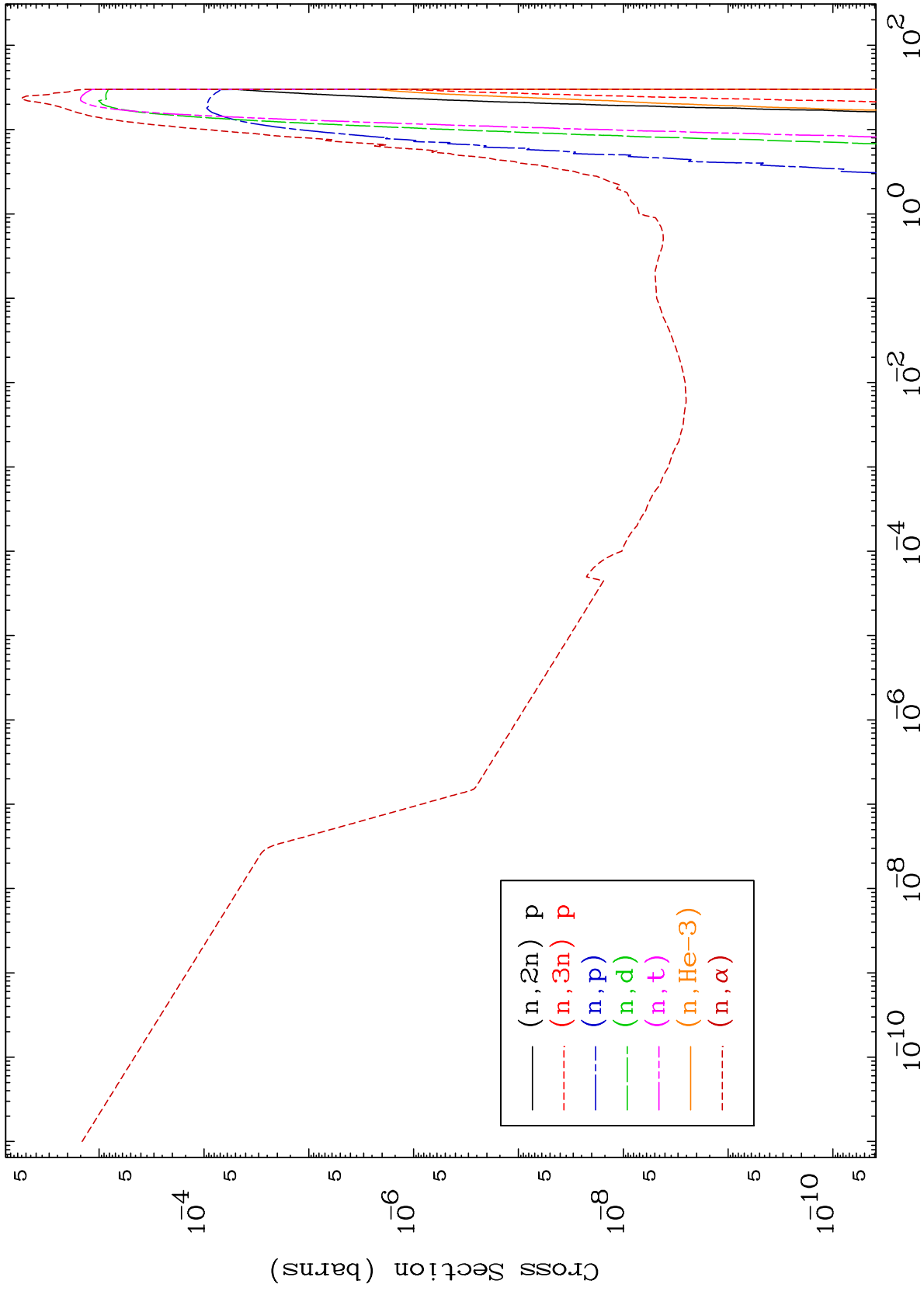
107-Bh-271



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293 Kelvin Cross Sections

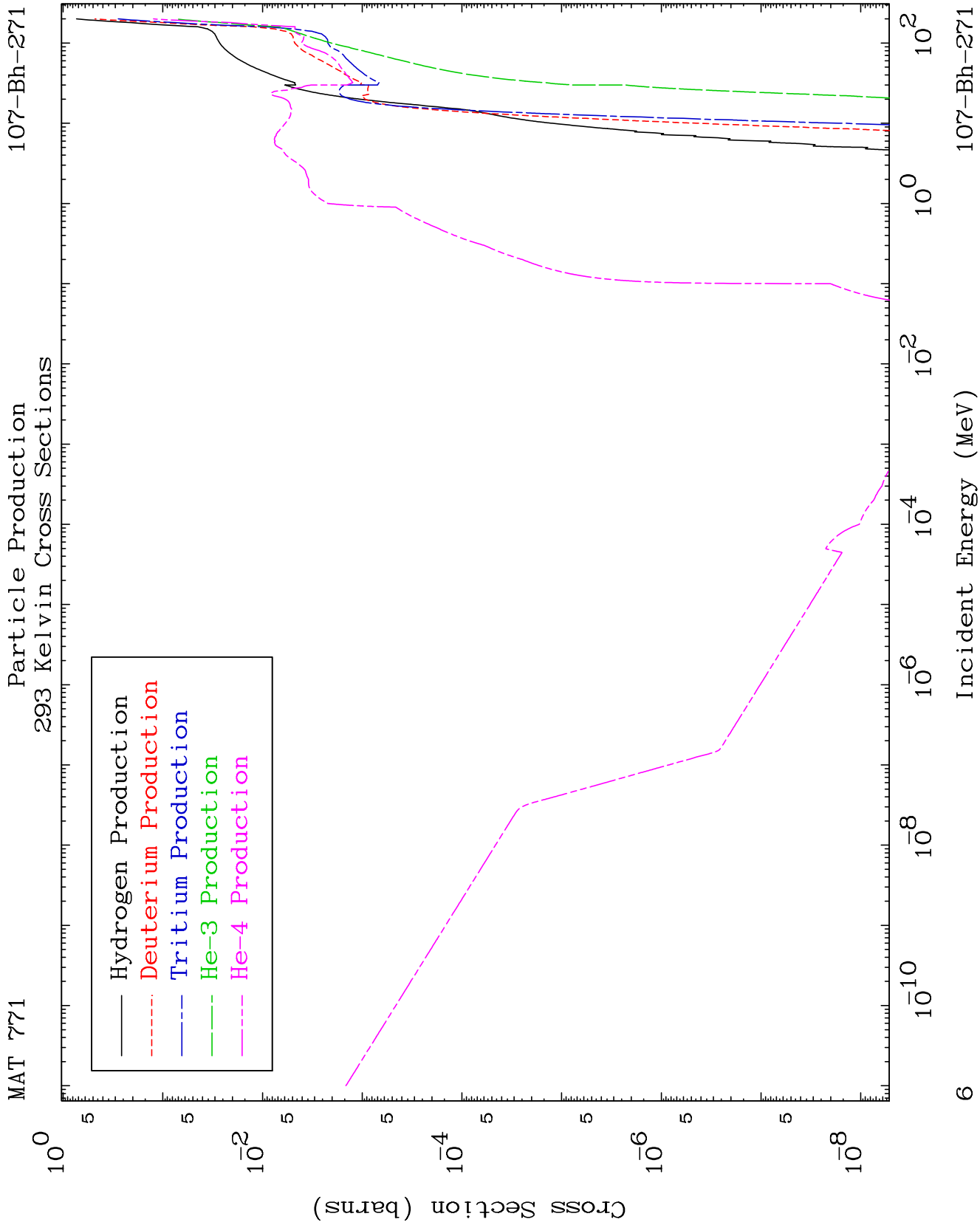
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5

Incident Energy (MeV)

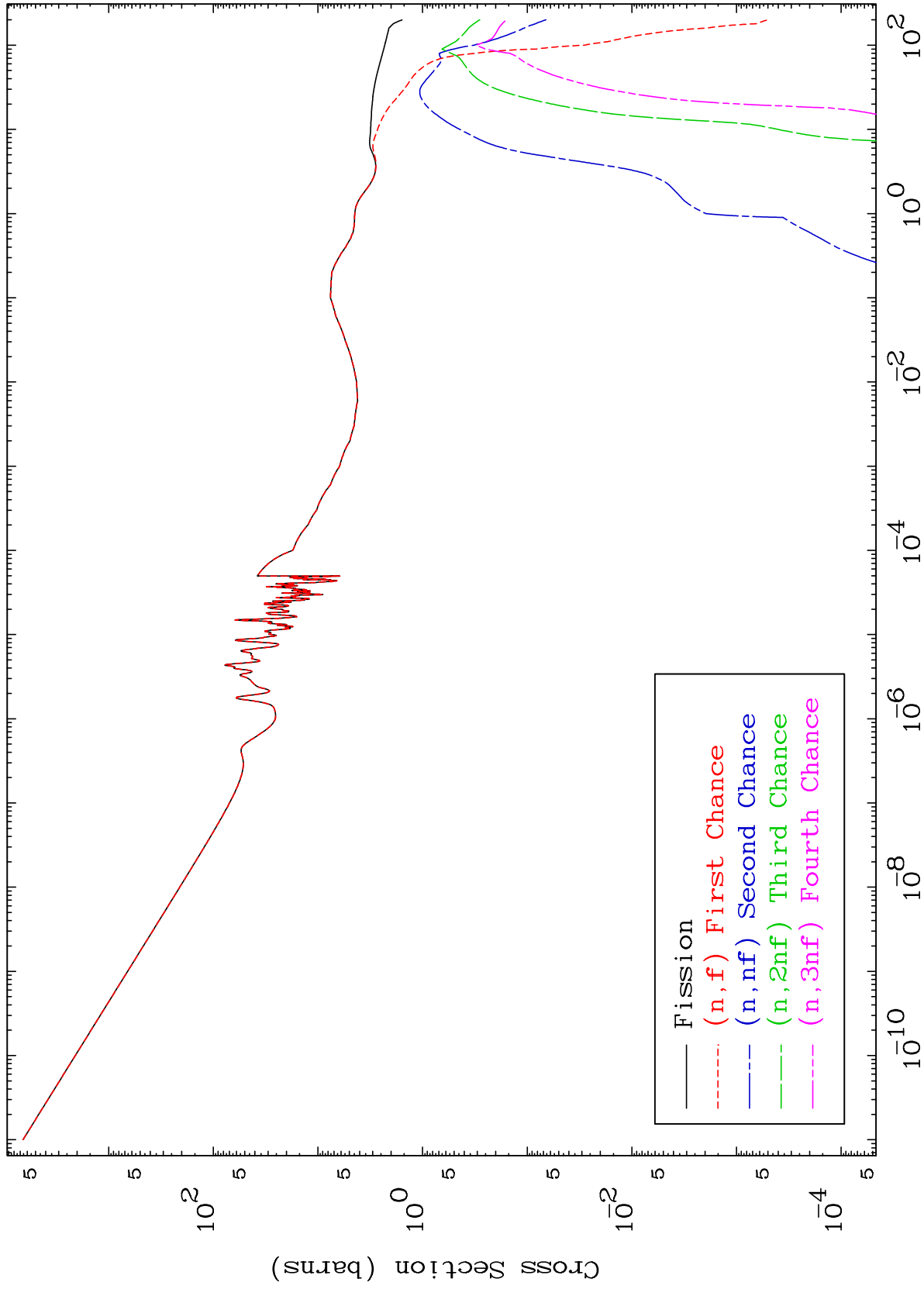
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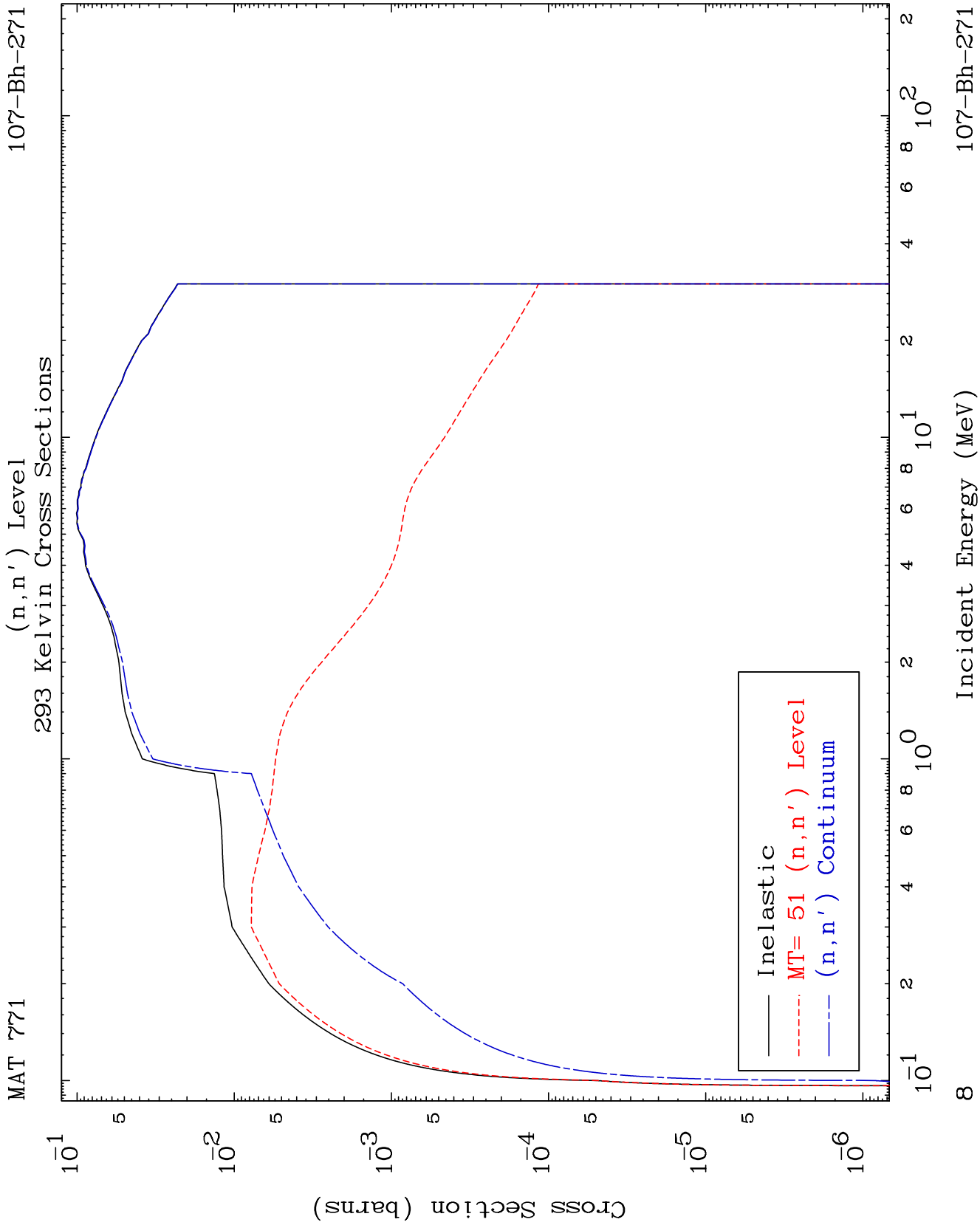


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Fission  
293 Kelvin Cross Sections

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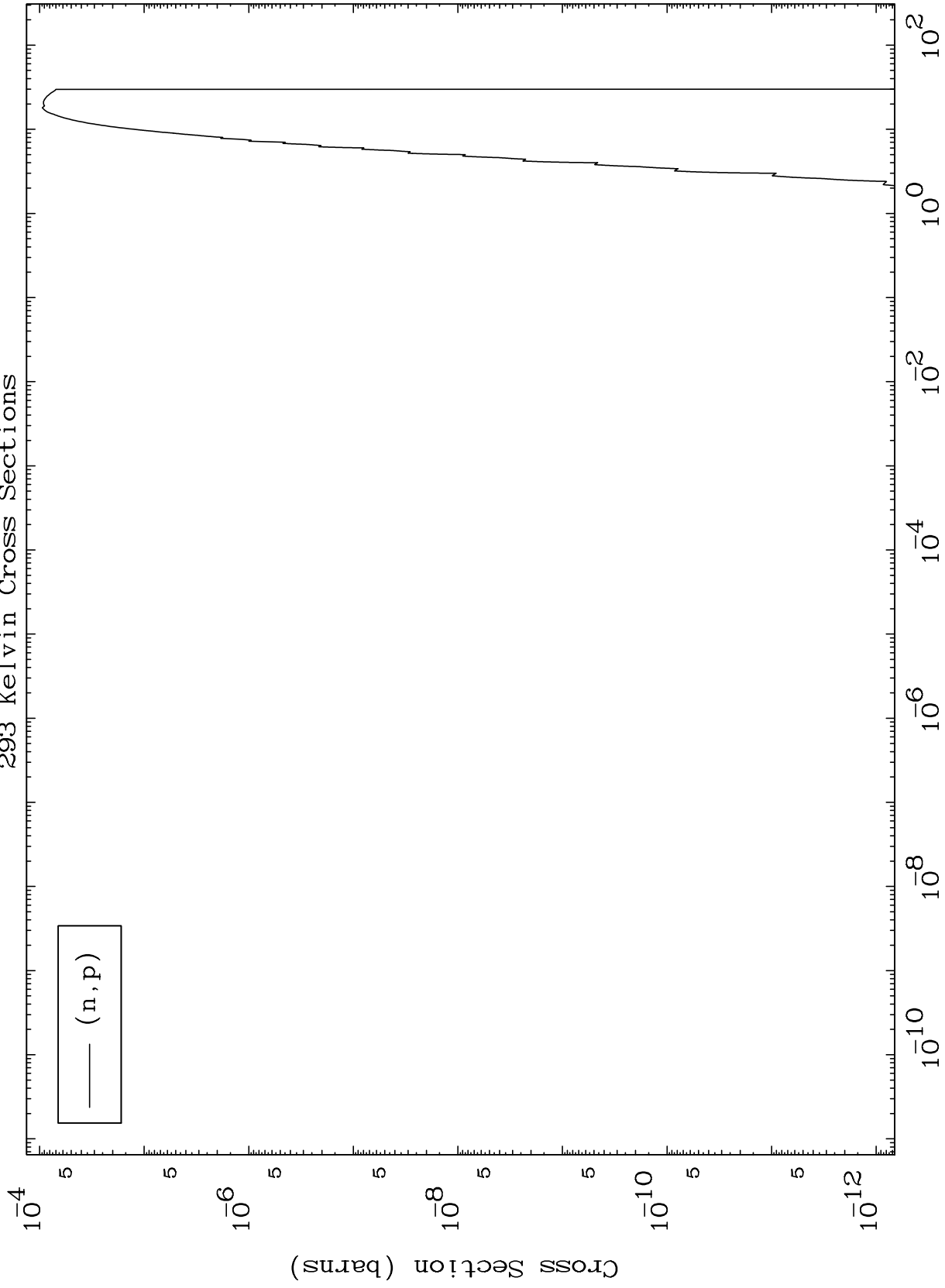




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(n,p) Levels  
293 Kelvin Cross Sections

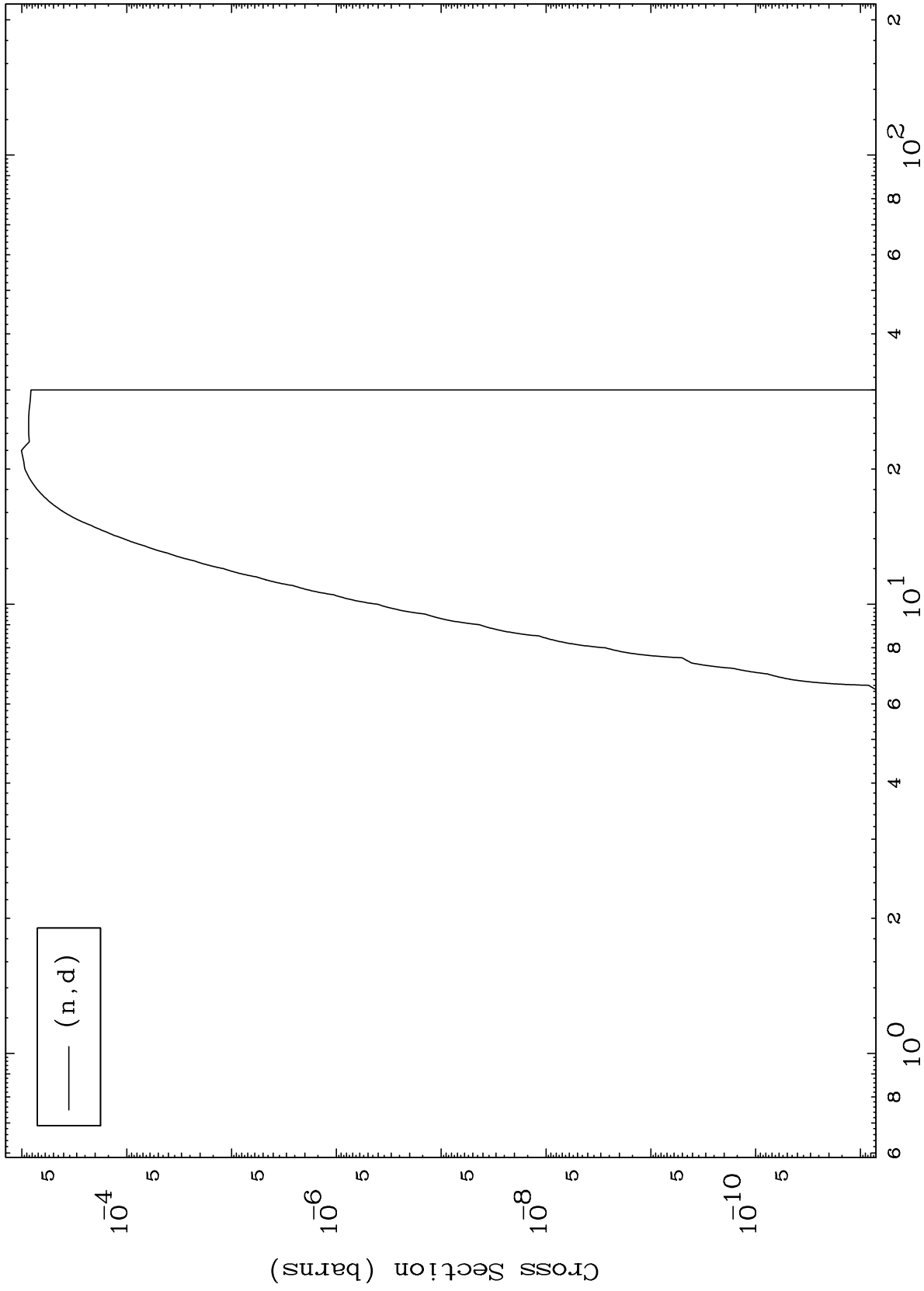
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(n,d) Levels  
293 Kelvin Cross Sections

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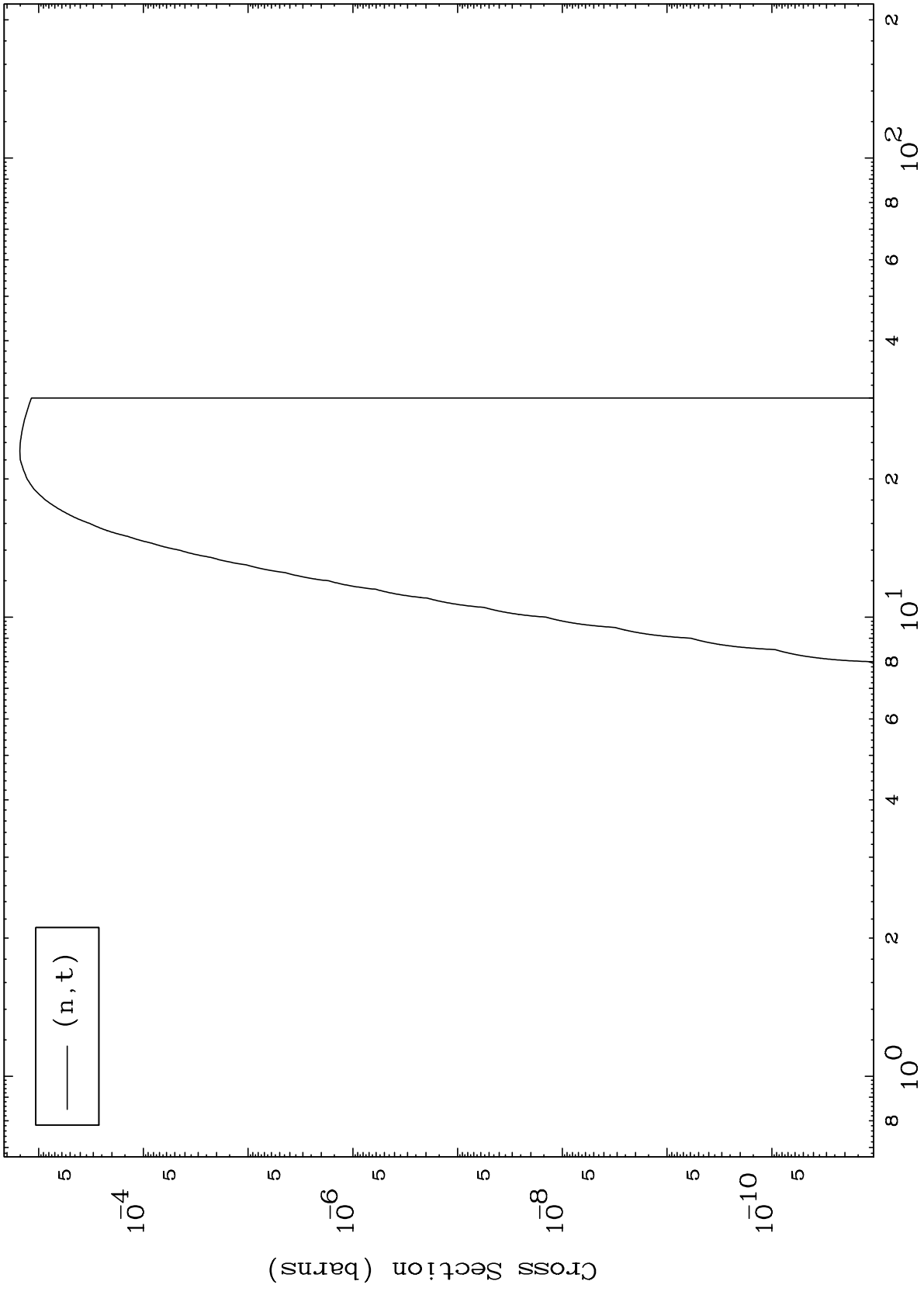
Incident Energy (MeV)

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(n,t) Levels  
293 Kelvin Cross Sections

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11

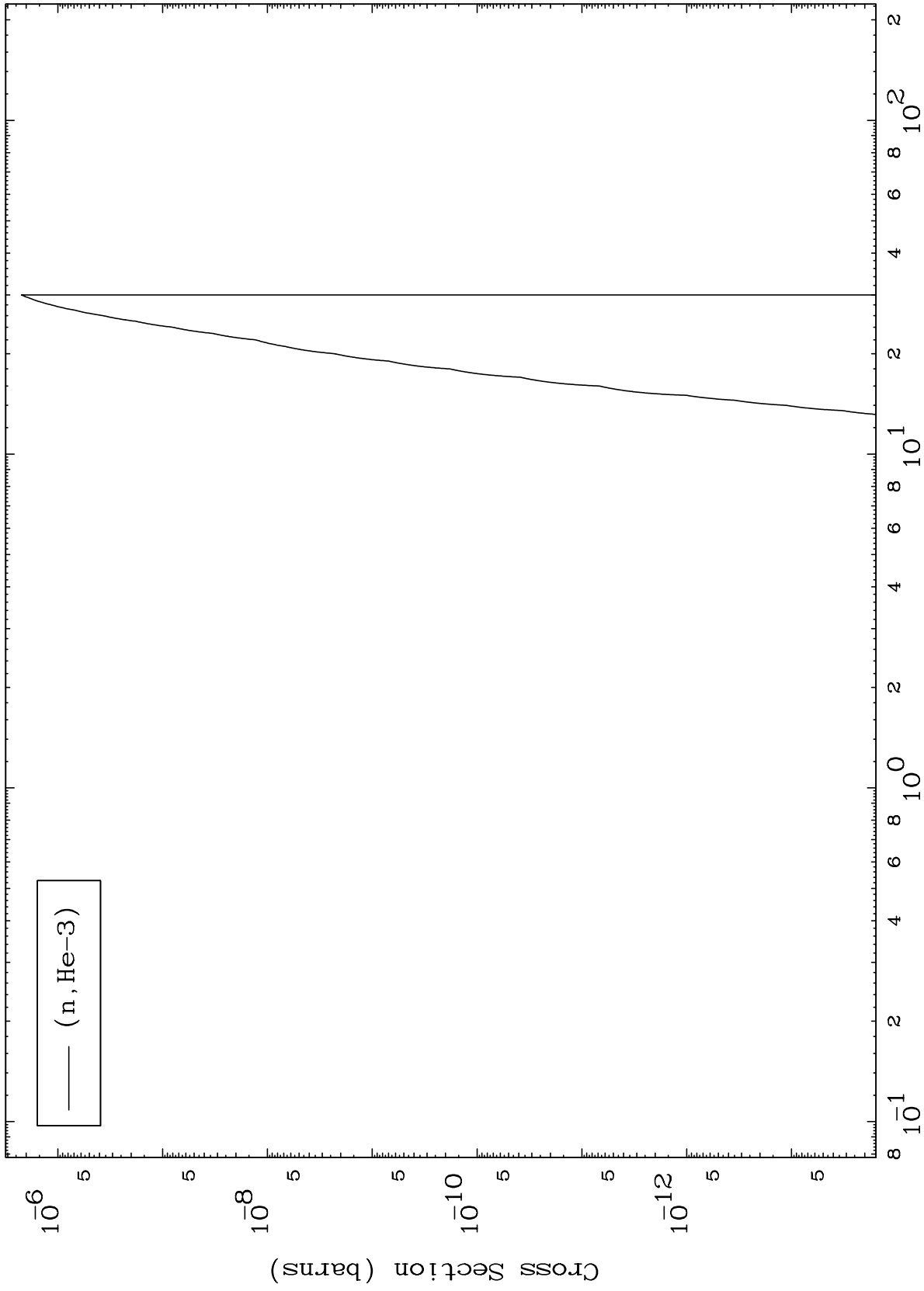
Incident Energy (MeV)

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(n,He3) Levels  
293 Kelvin Cross Sections

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(n, He-3)

12

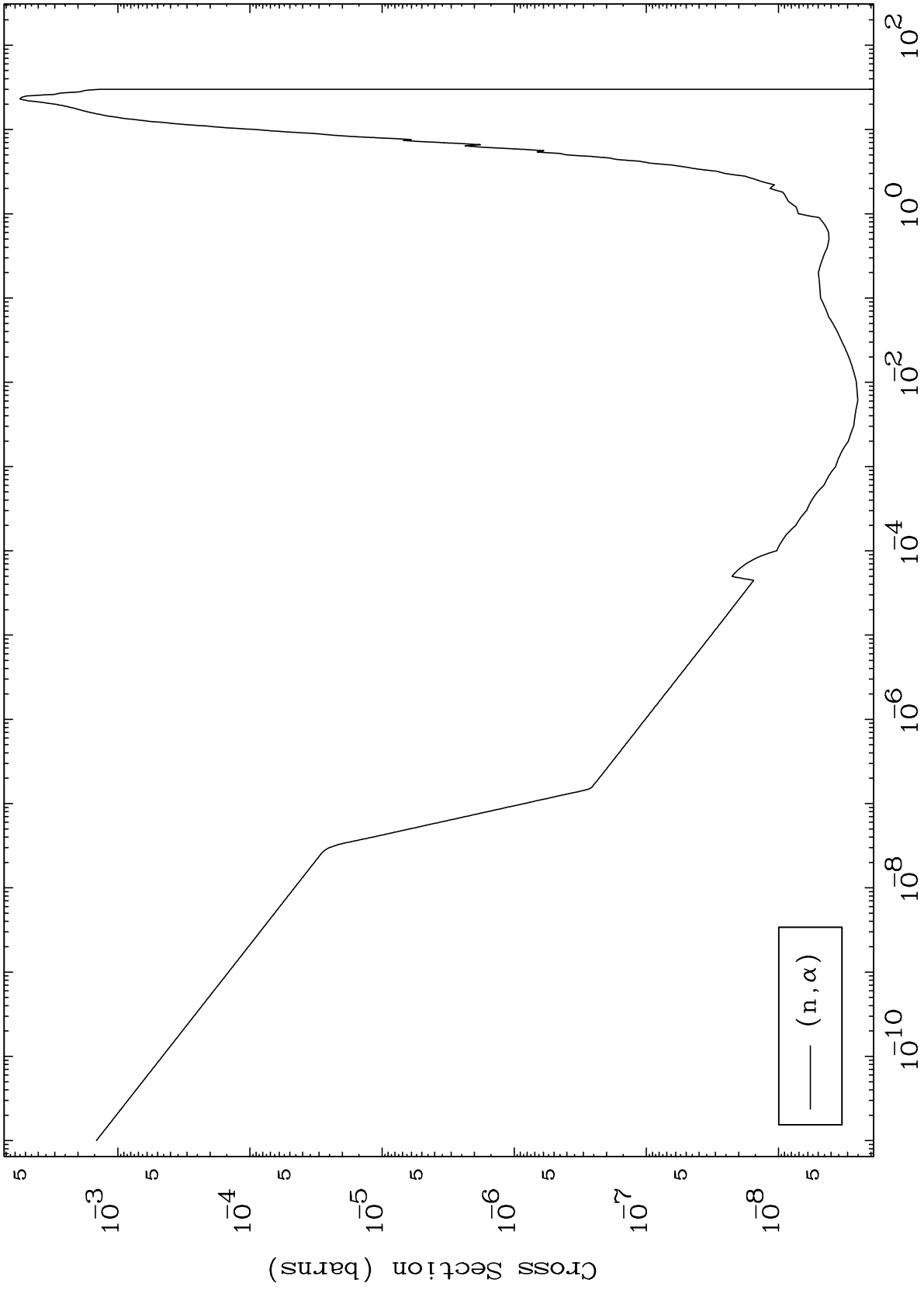
Incident Energy (MeV)

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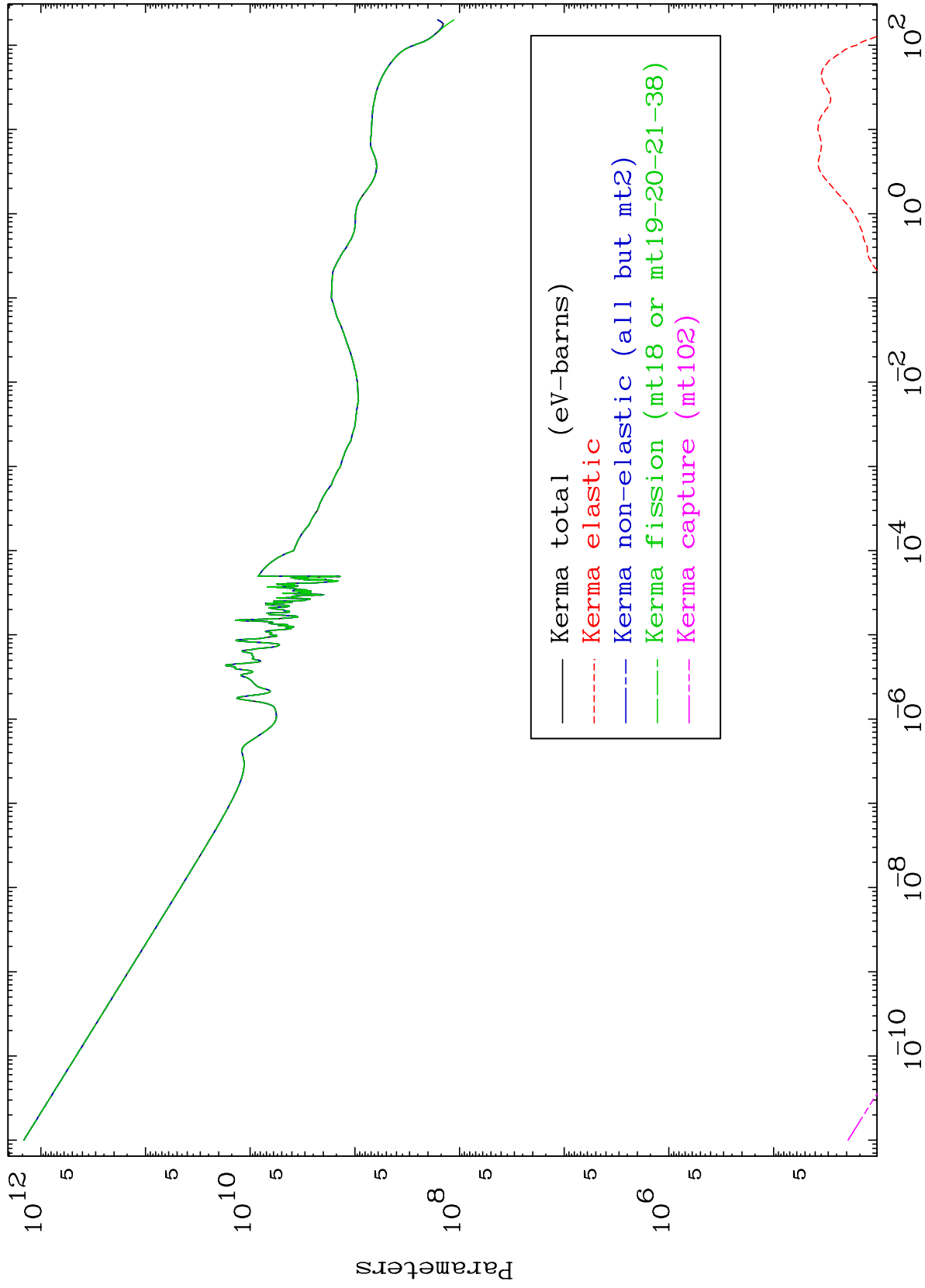
(n,α) Levels  
293 Kelvin Cross Sections

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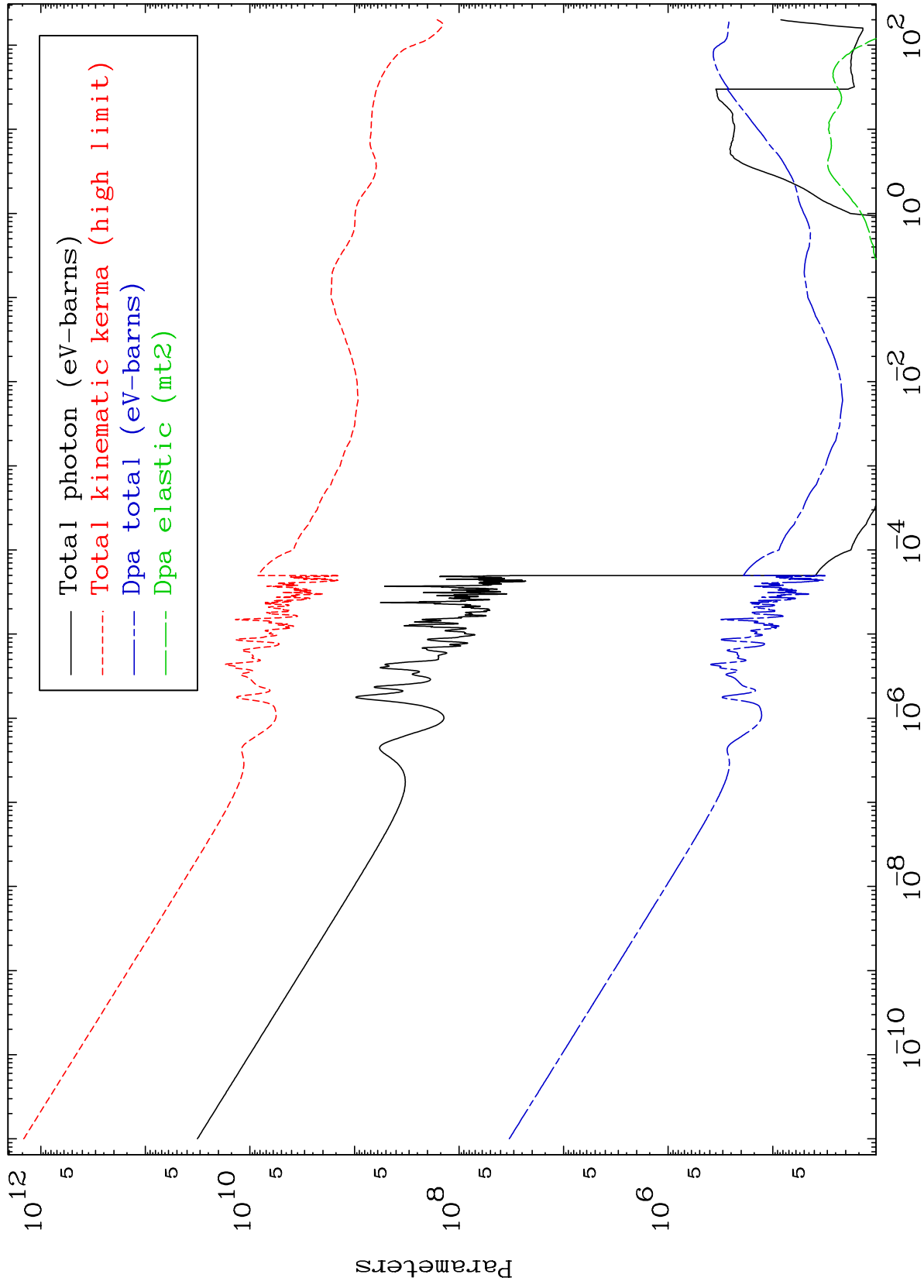
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Energy Release  
Parameters

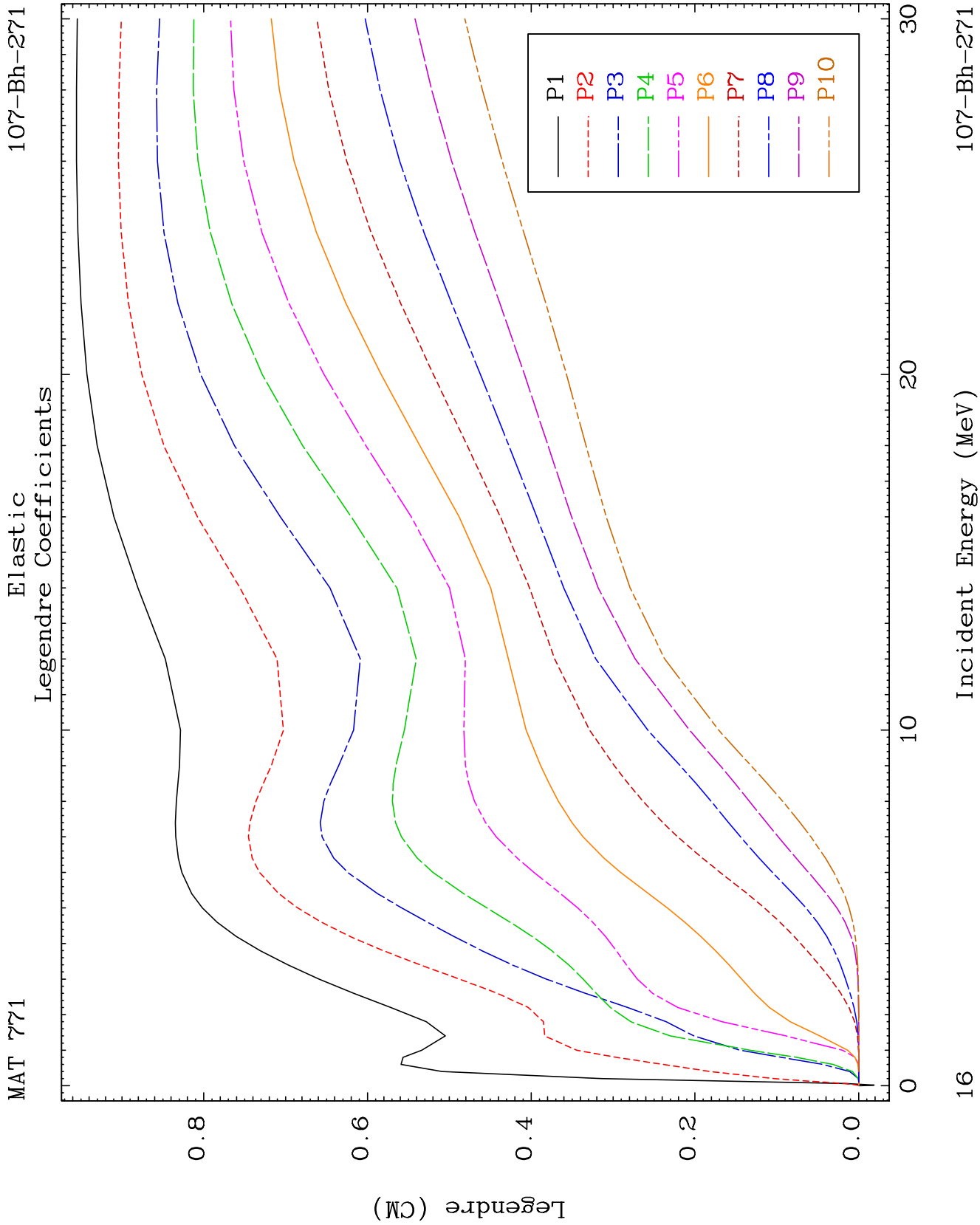
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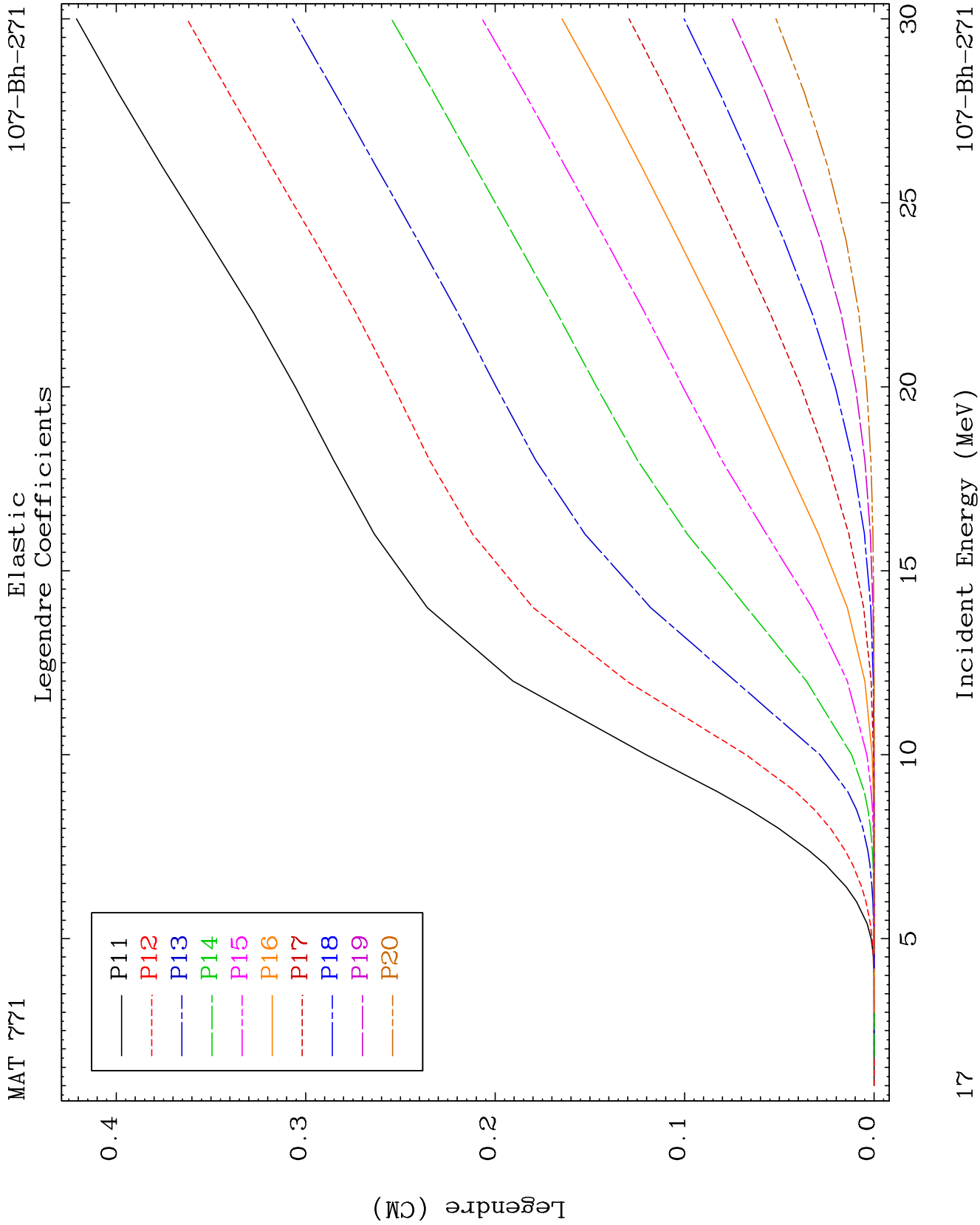


15

Incident Energy (MeV)

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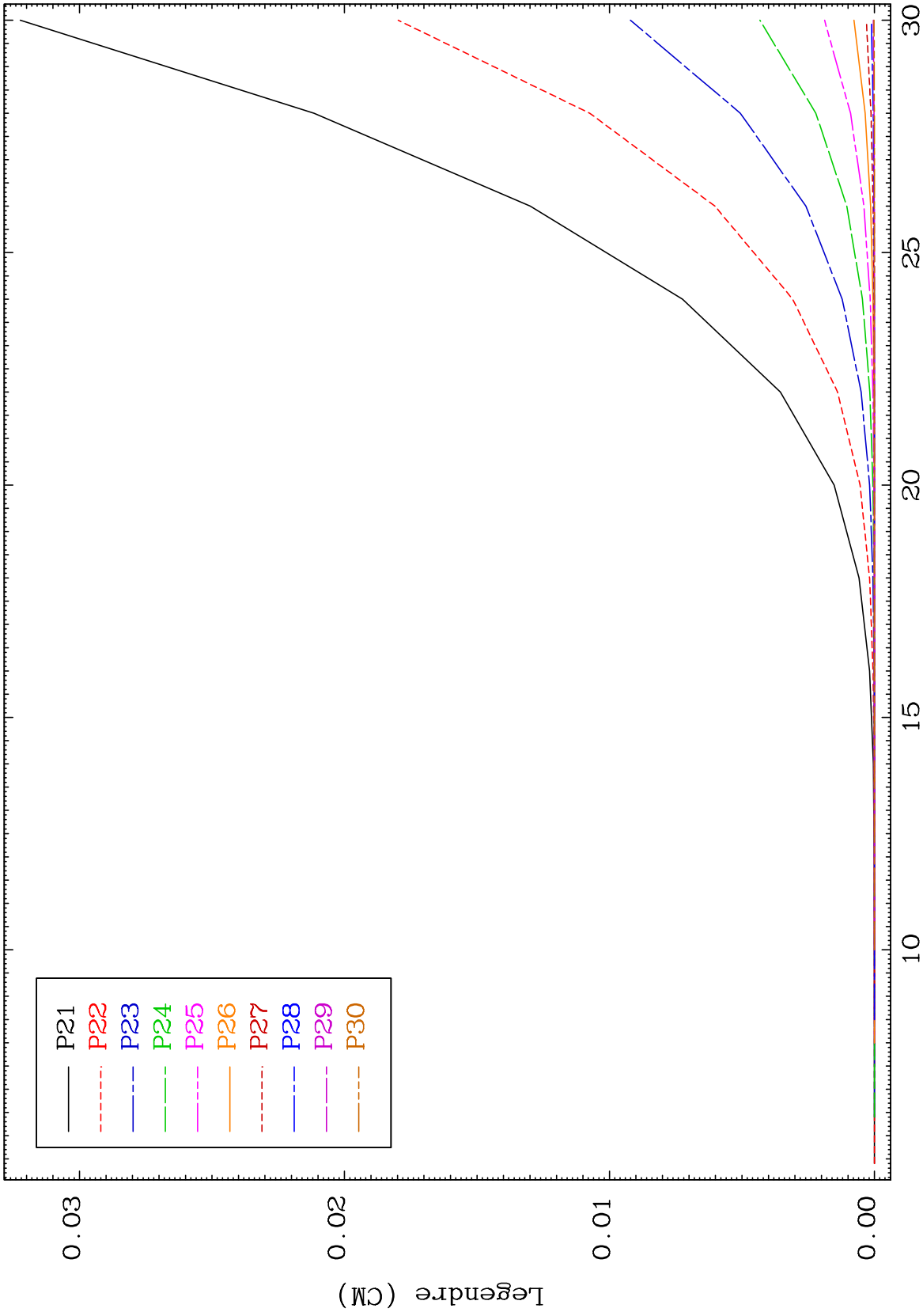




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### Elastic Legendre Coefficients

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18

Incident Energy (MeV)

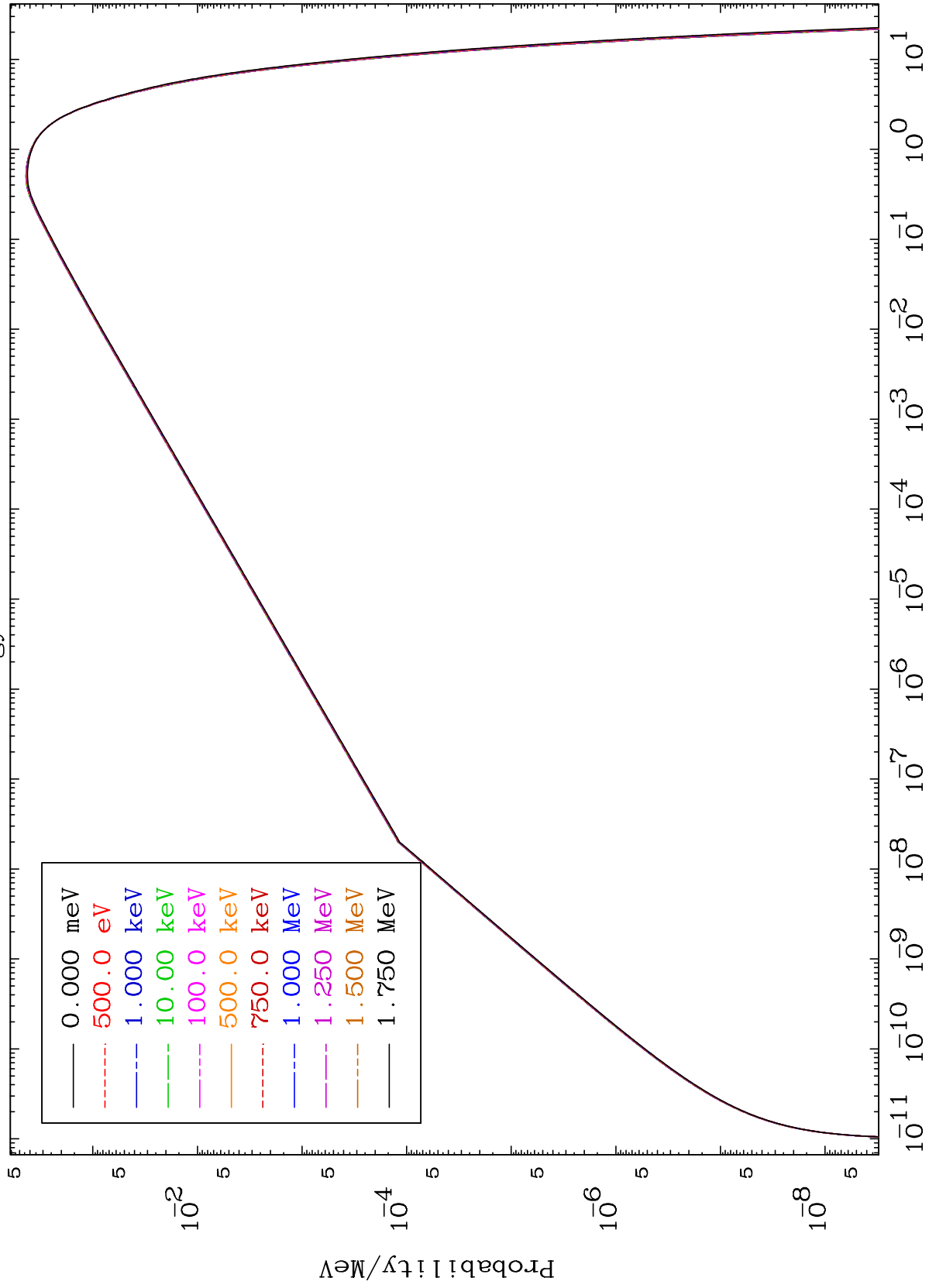
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### Fission Energy Distribution

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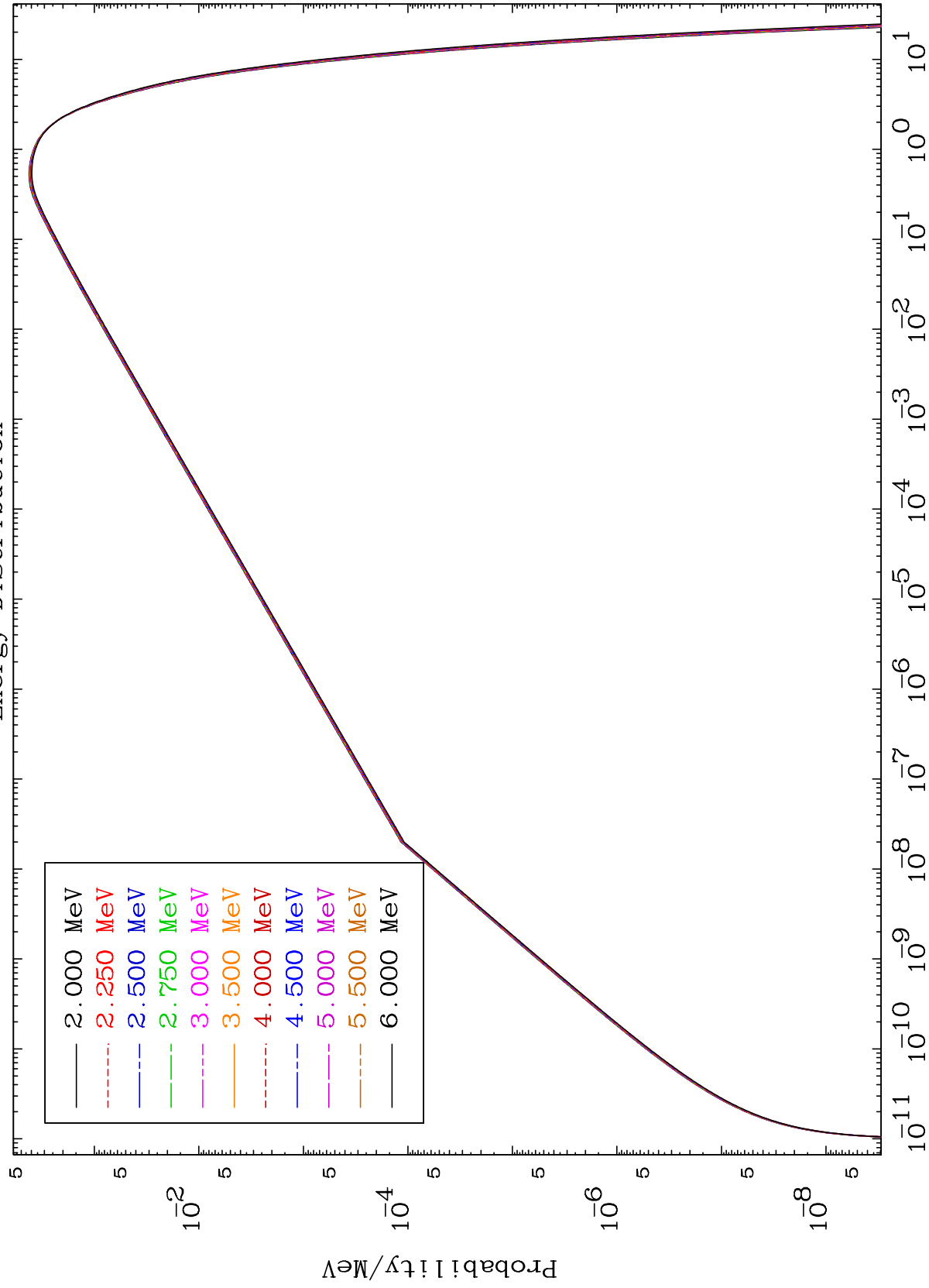


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# Fission Energy Distribution

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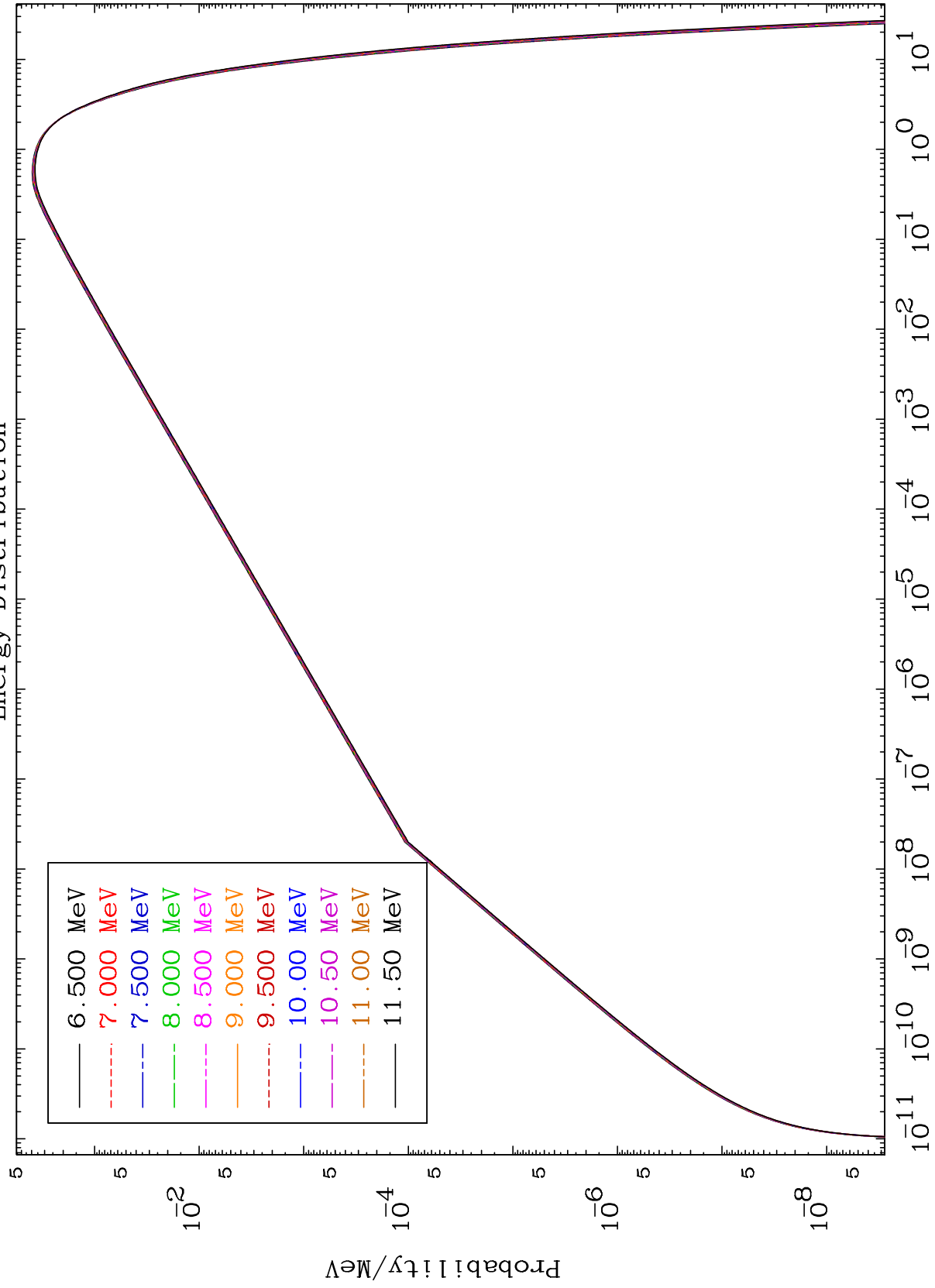


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# Fission Energy Distribution

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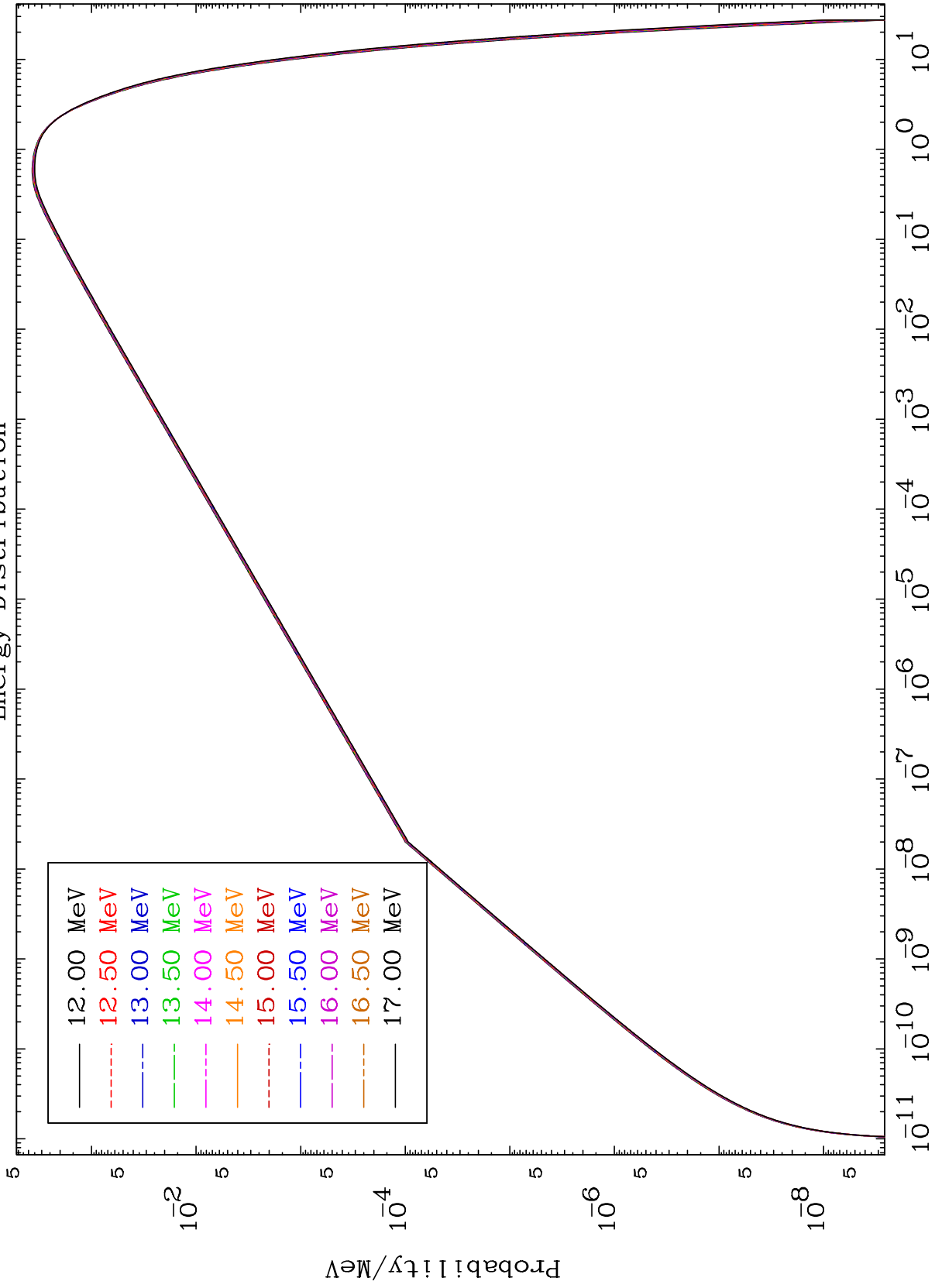


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# Fission Energy Distribution

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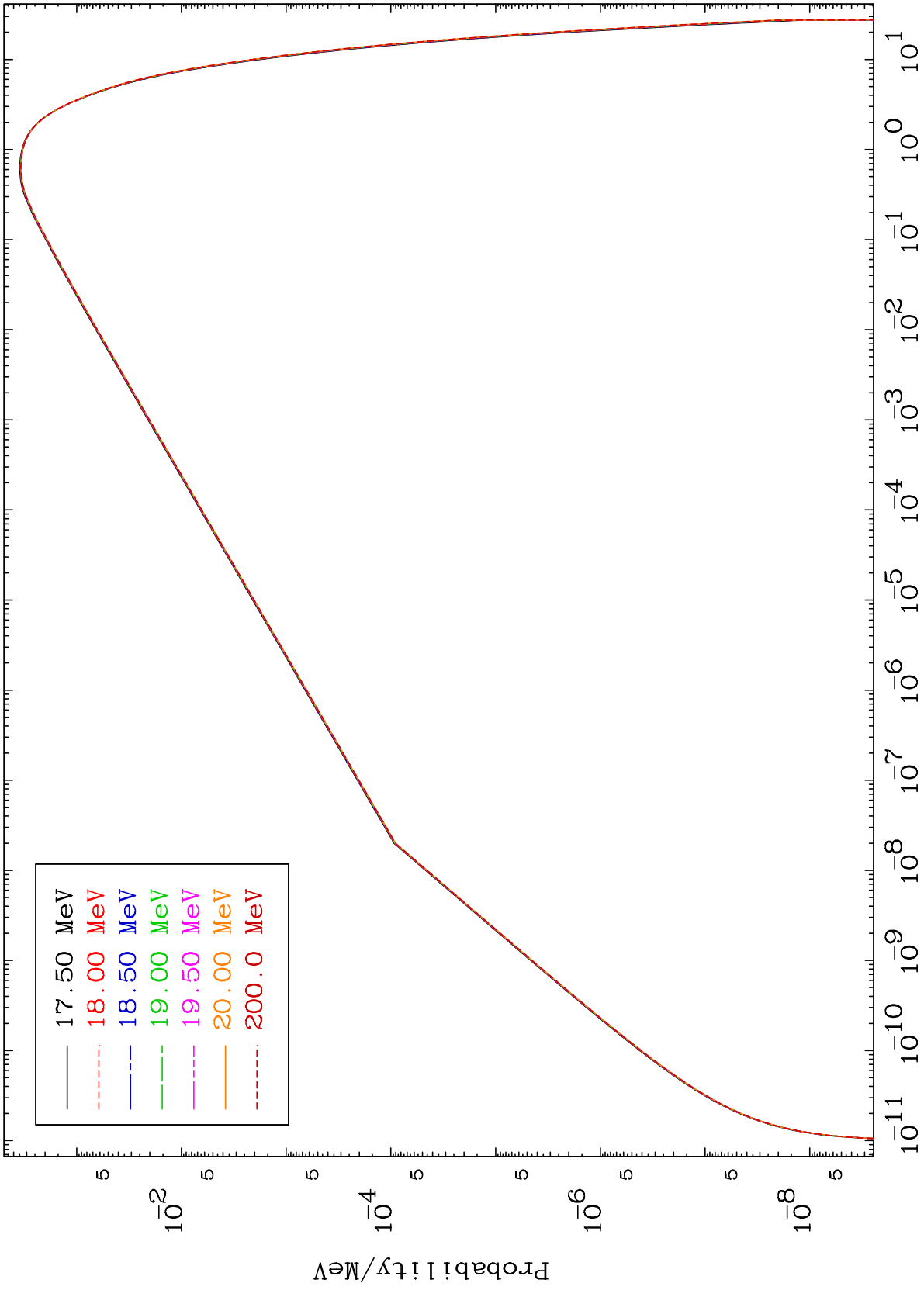


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### Fission Energy Distribution

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