

Program EVALPLOT
(Version 2018-1)

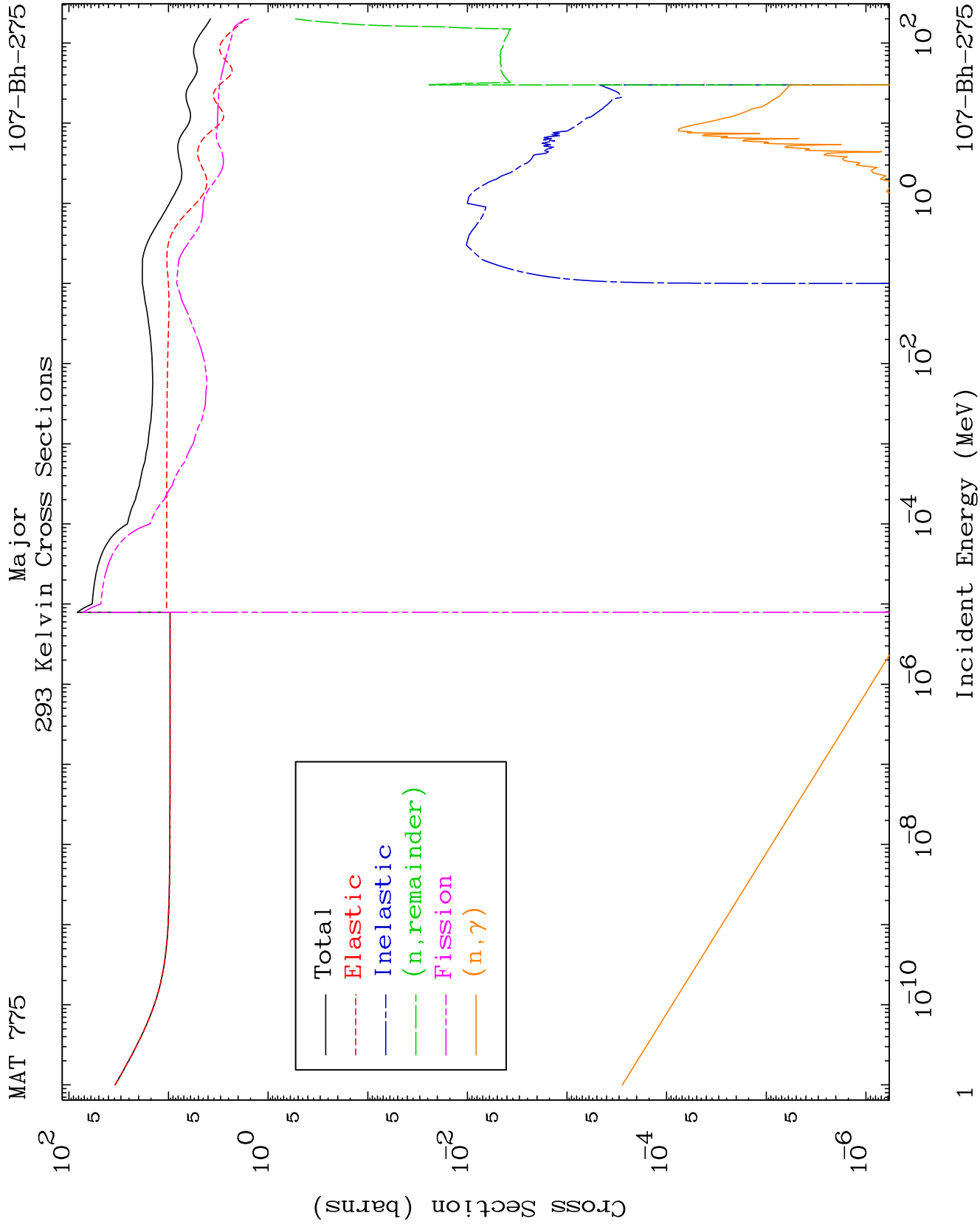
by

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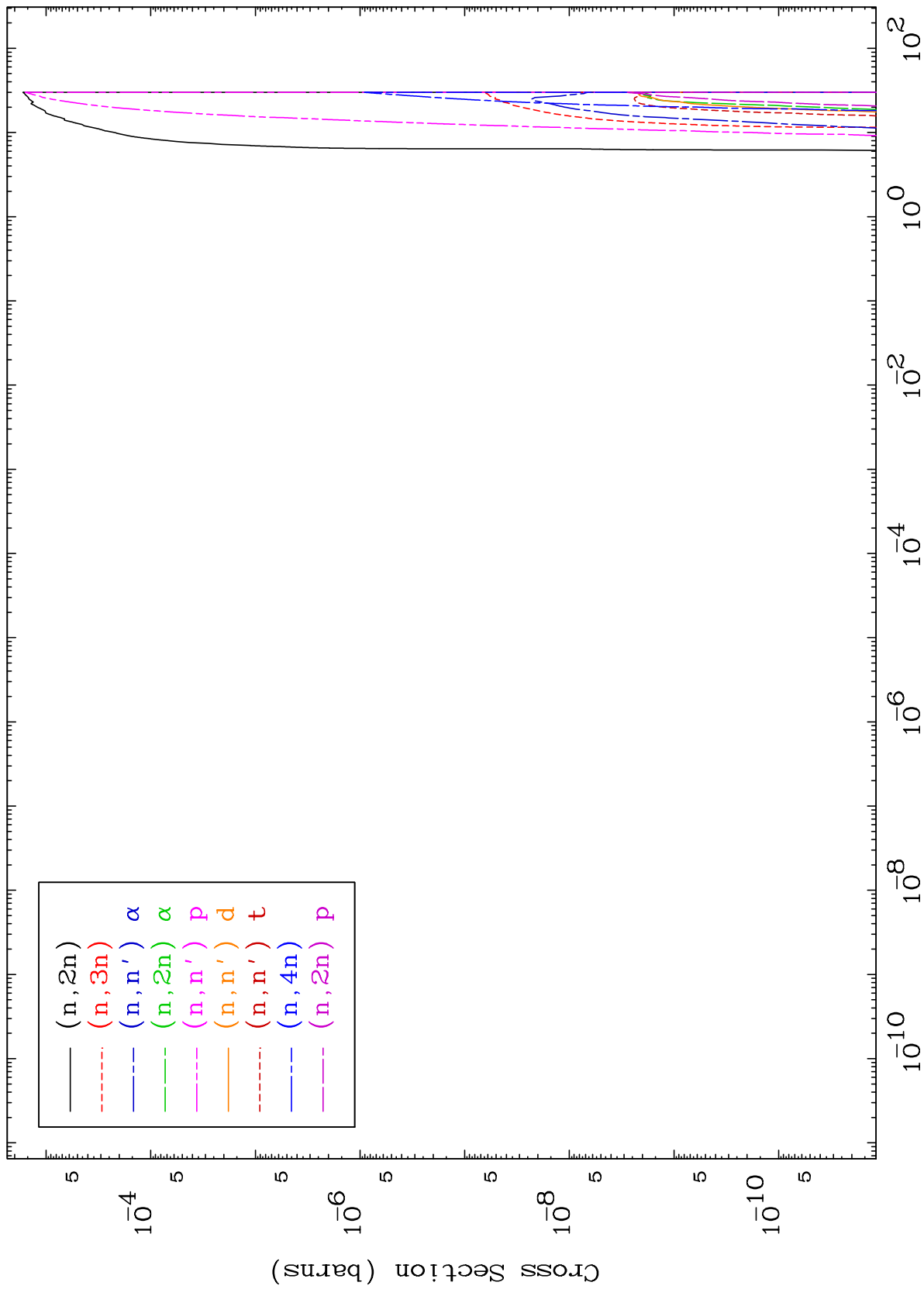
Press Mouse Button to Start



MAT 775

Neutron Production
293 Kelvin Cross Sections

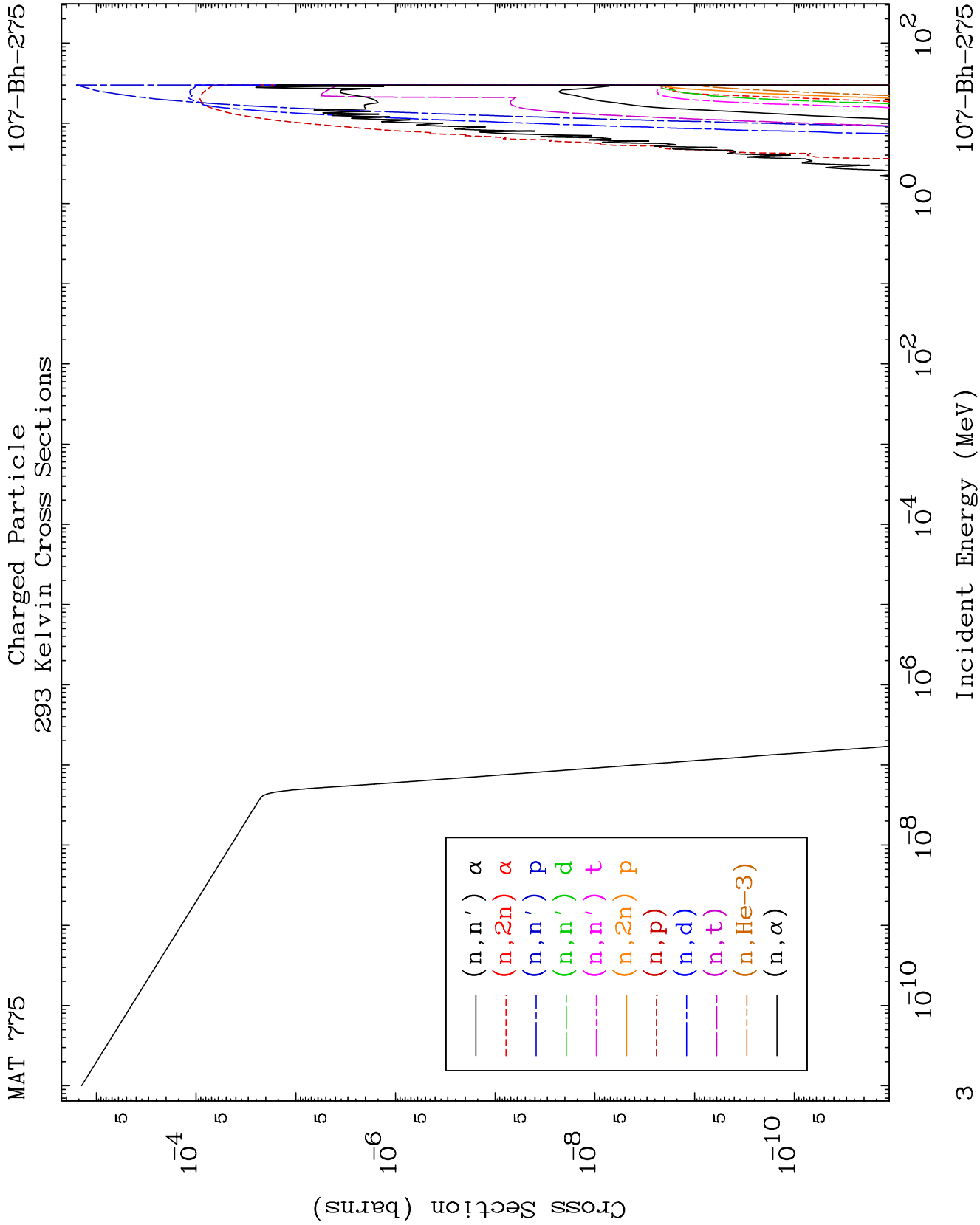
107-Bh-275

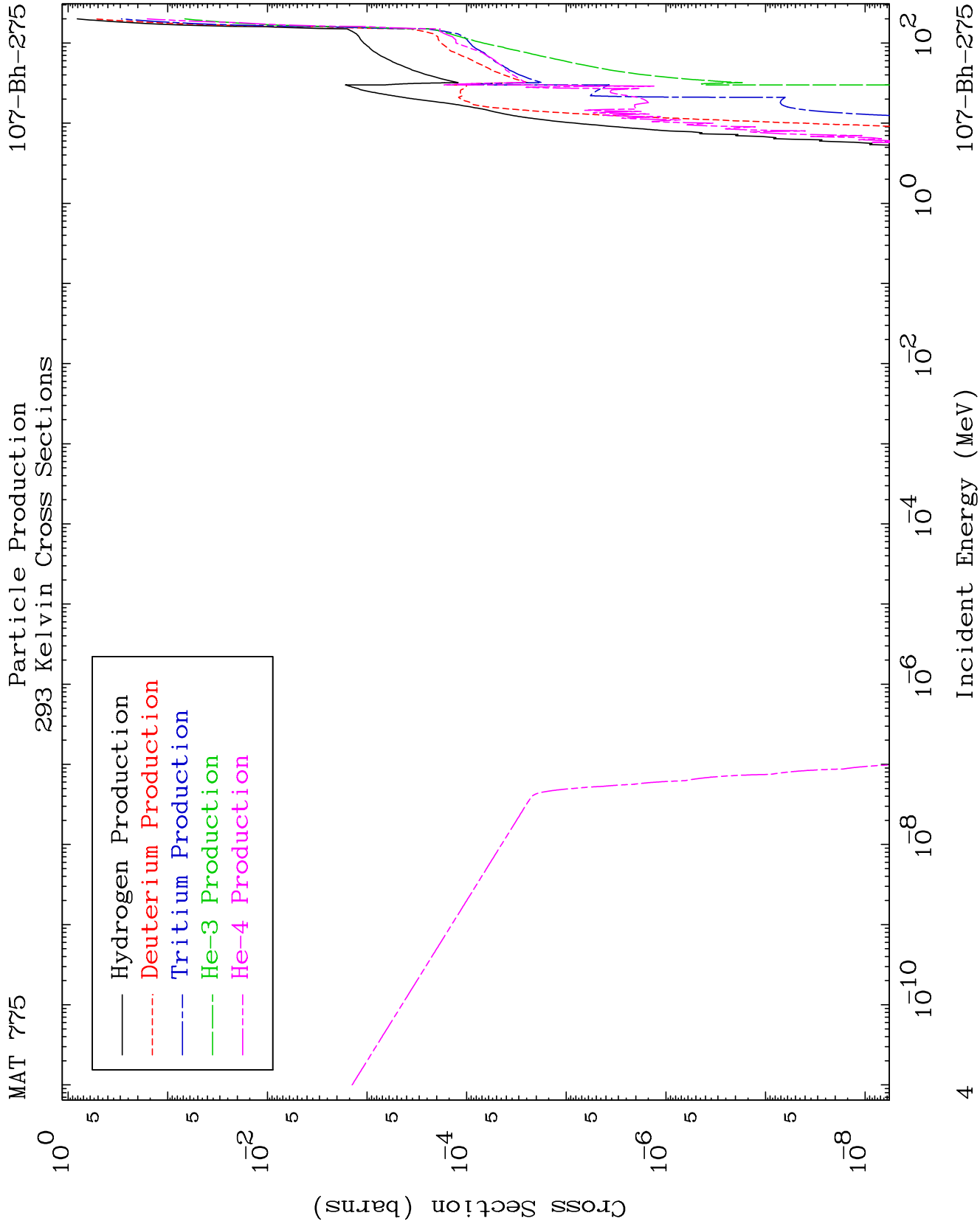


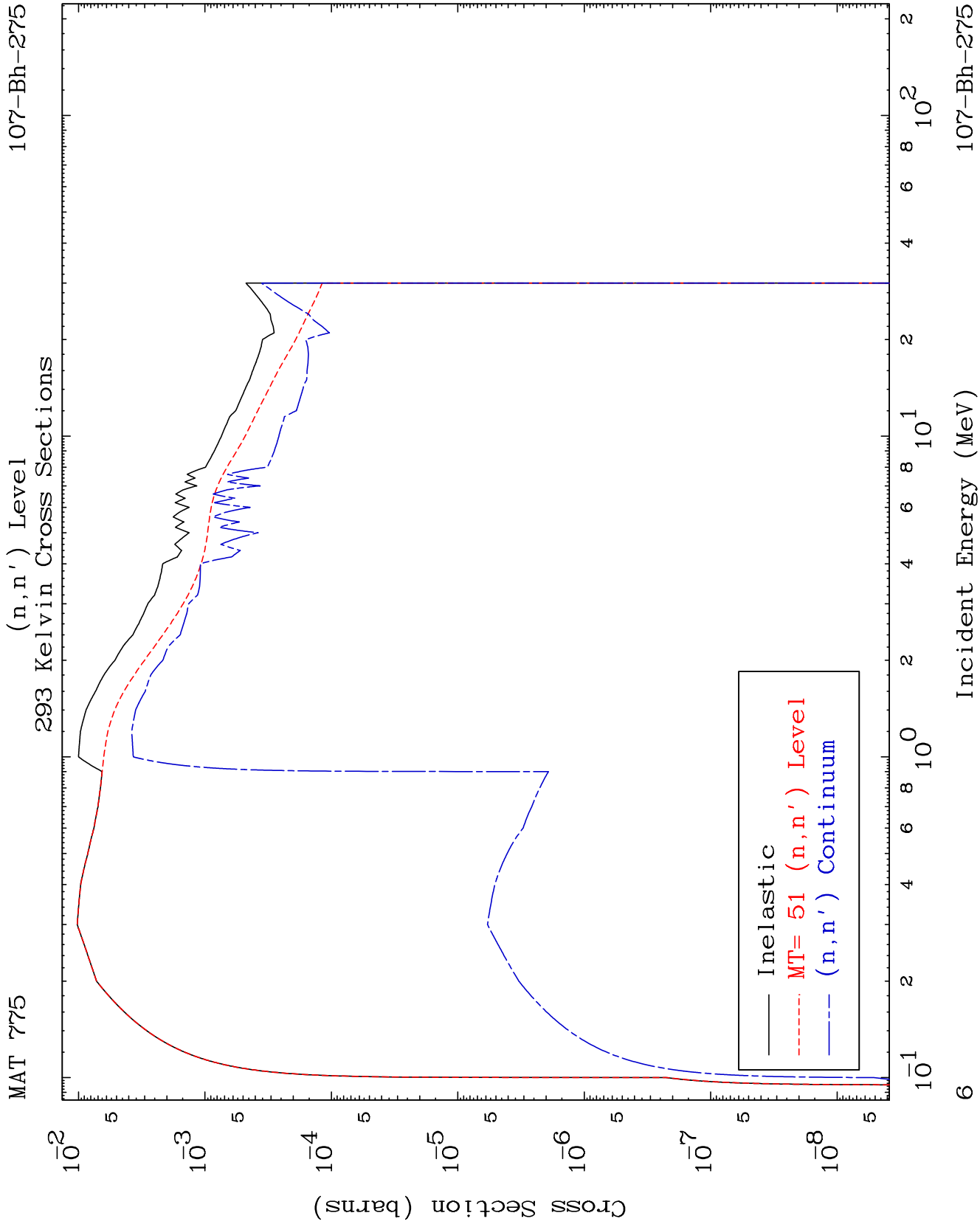
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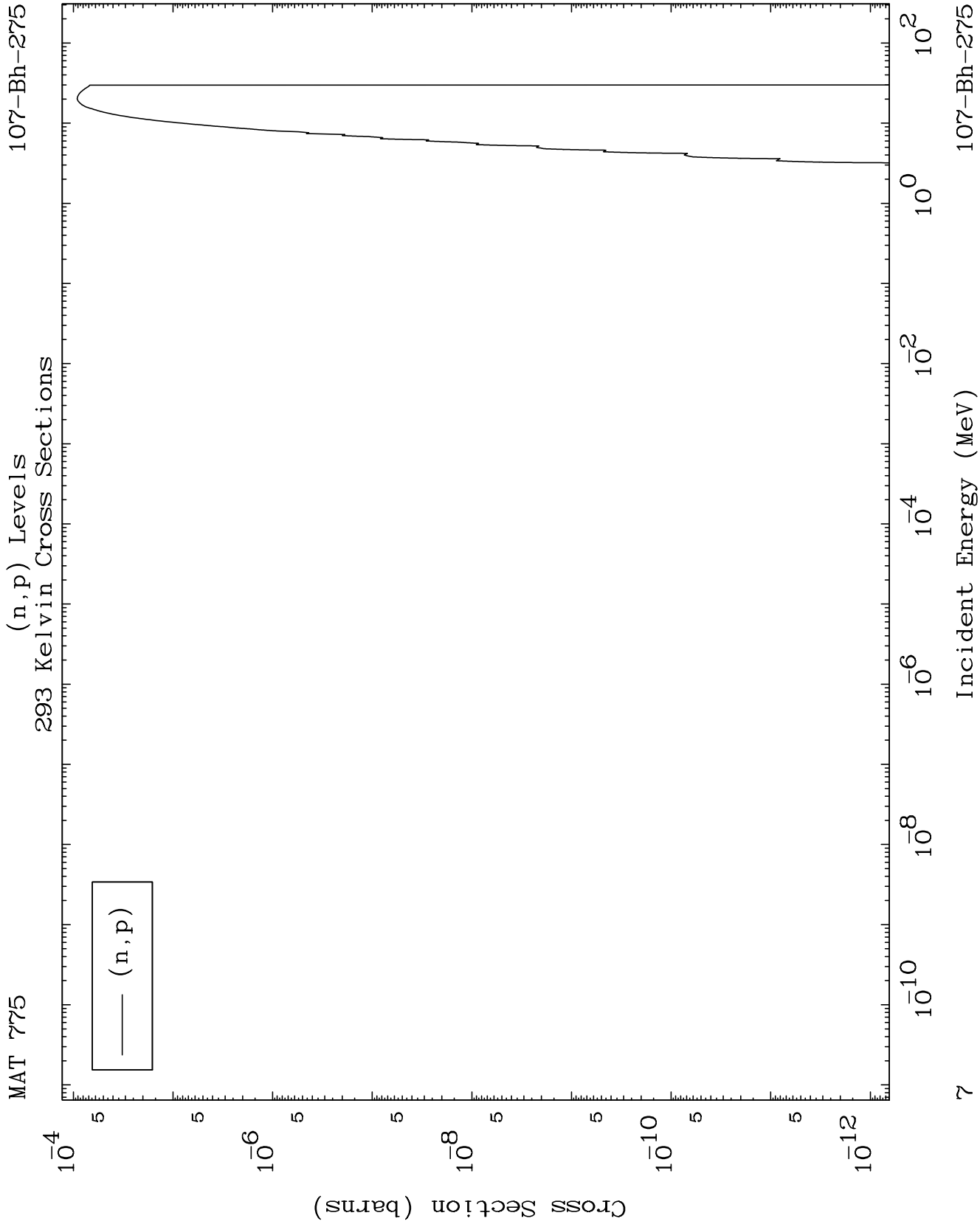
Incident Energy (MeV)

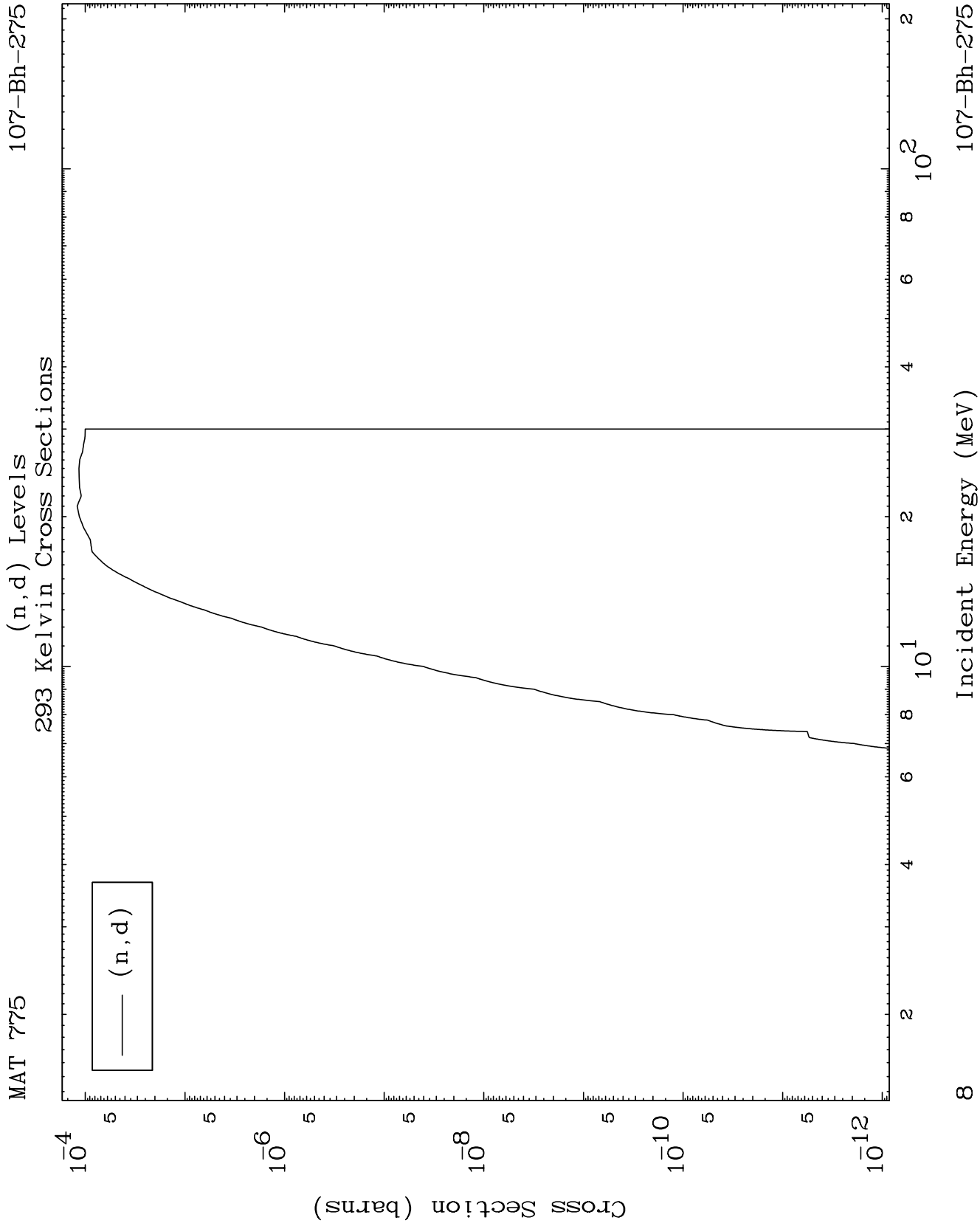
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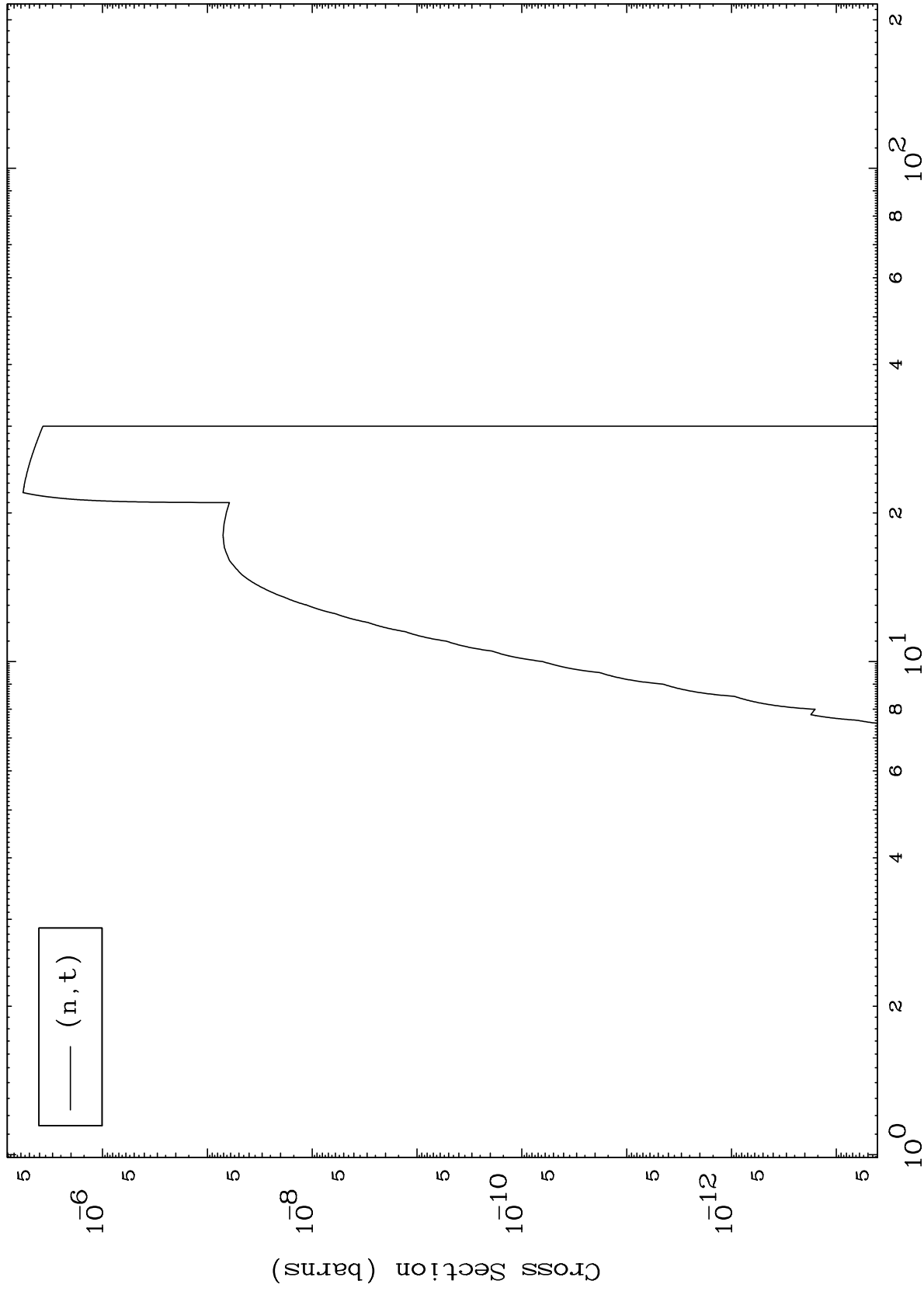




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(n,t) Levels
293 Kelvin Cross Sections

107-Bh-275



Incident Energy (MeV)

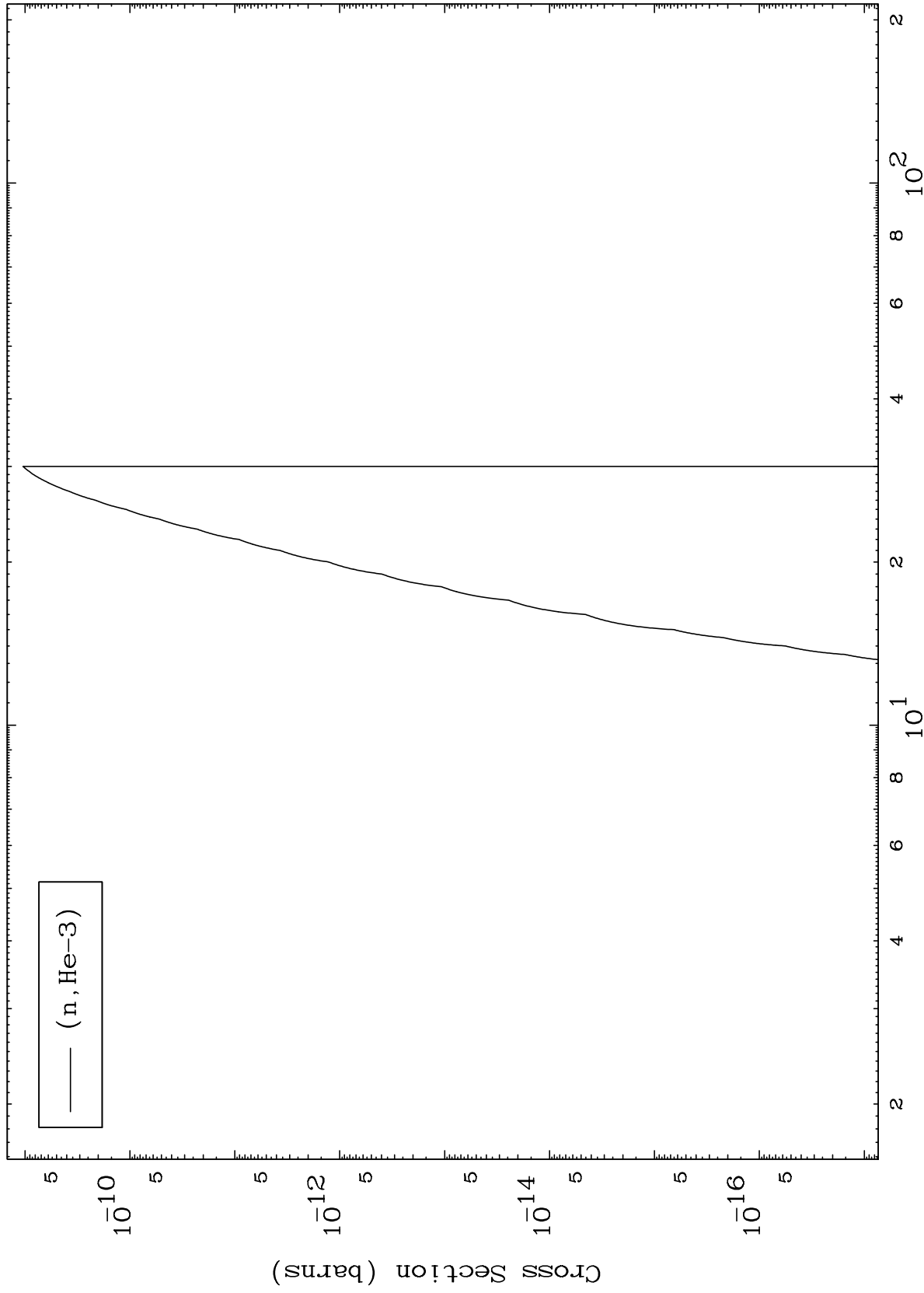
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(n,He3) Levels
293 Kelvin Cross Sections

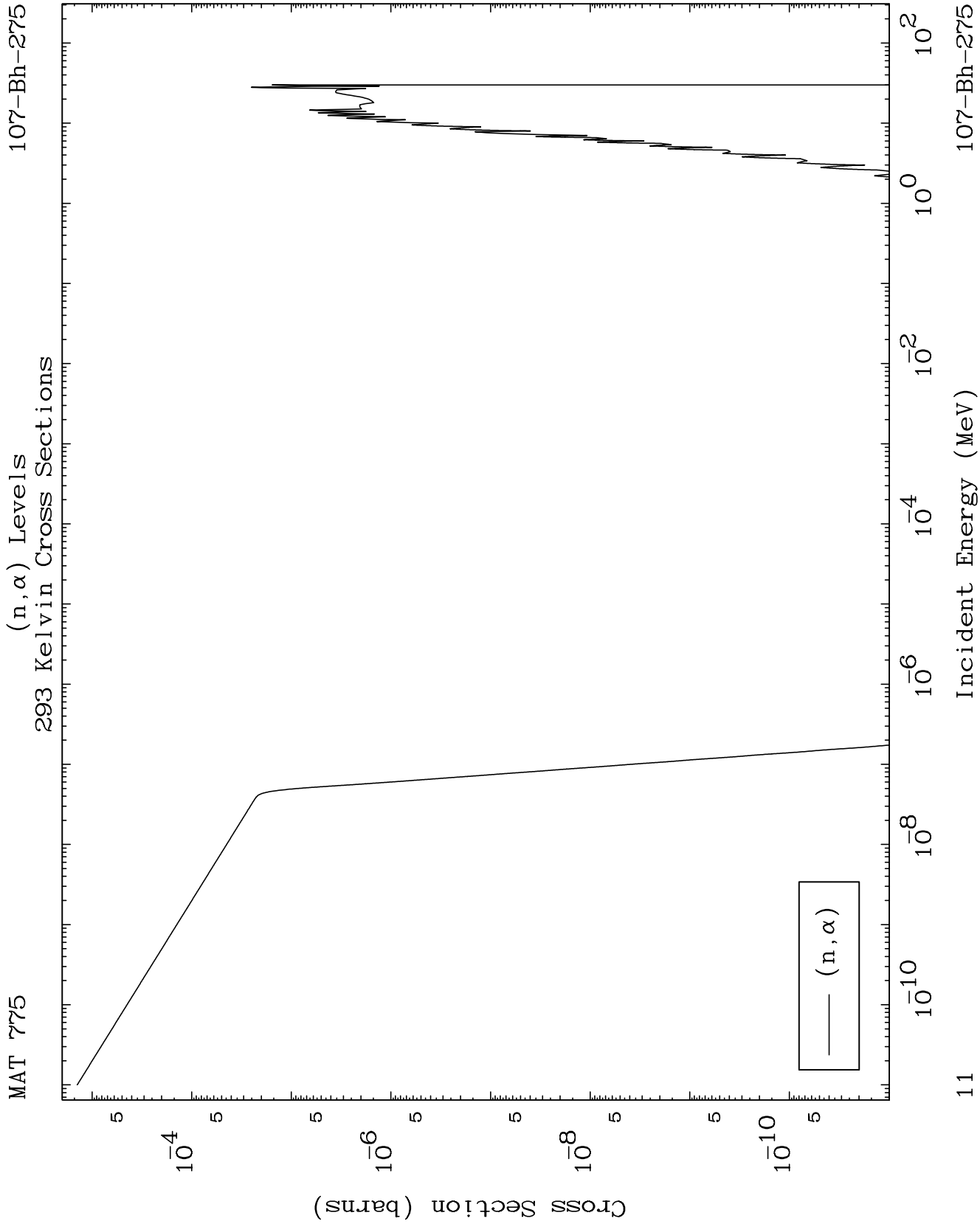
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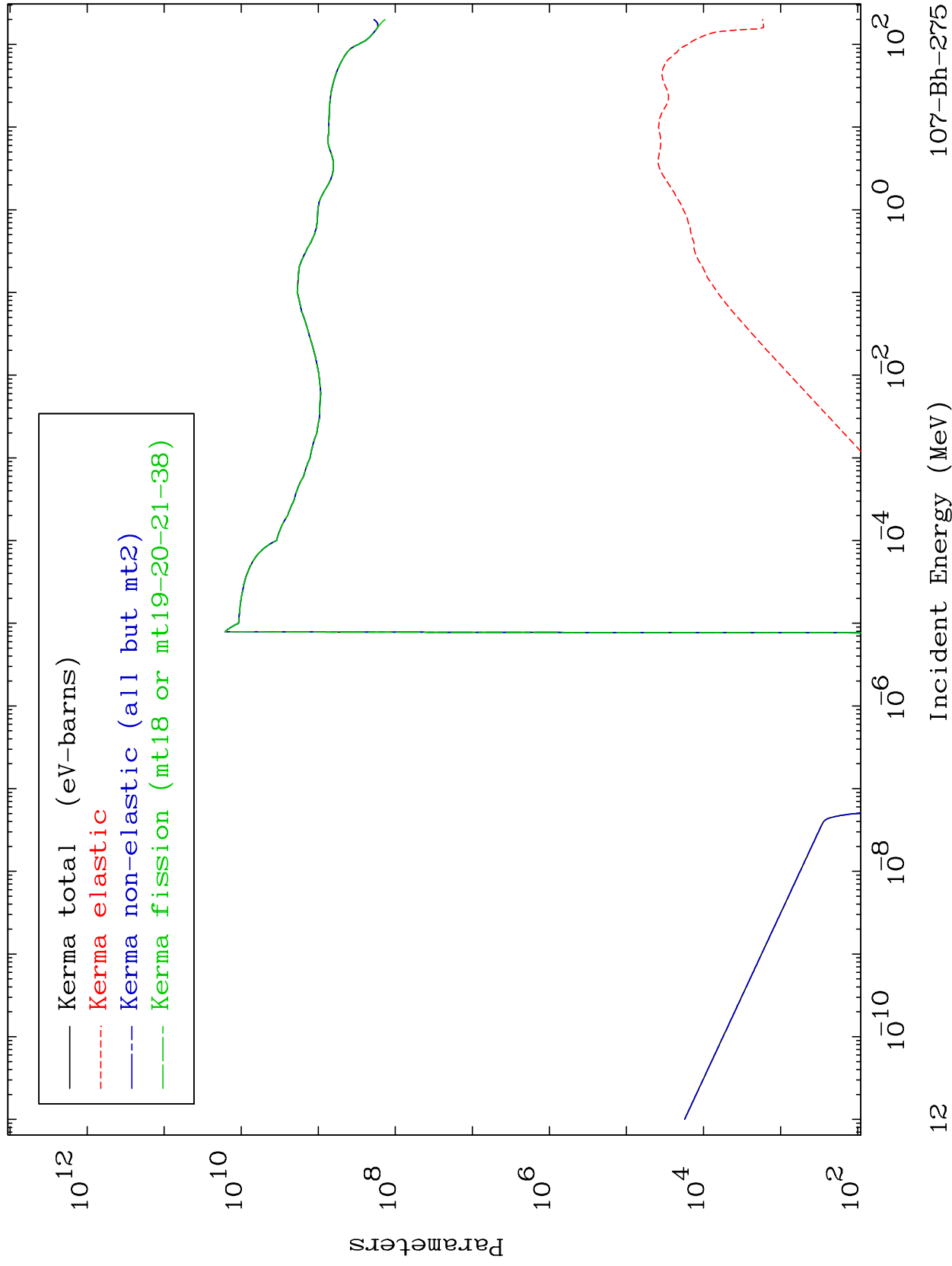


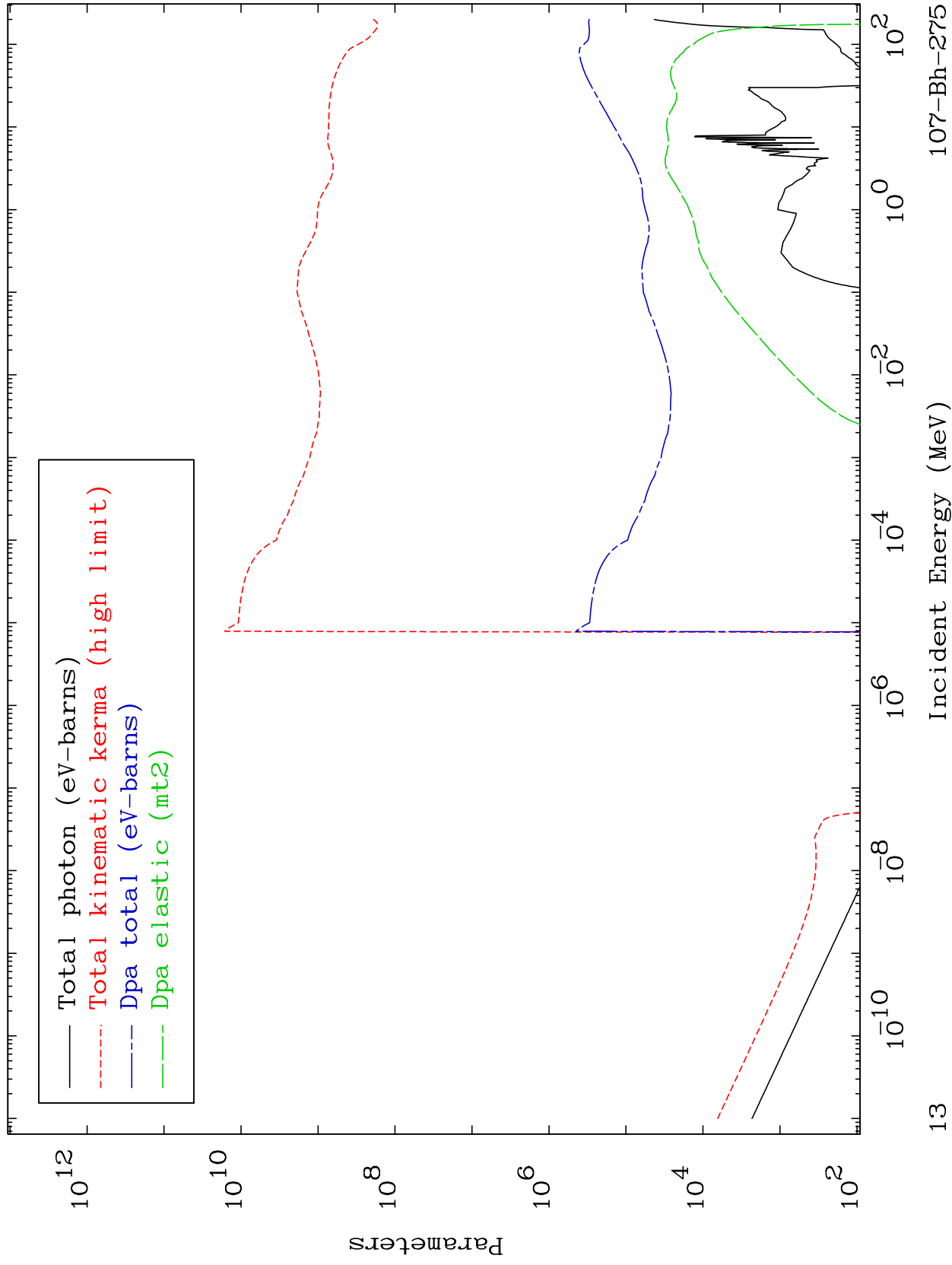
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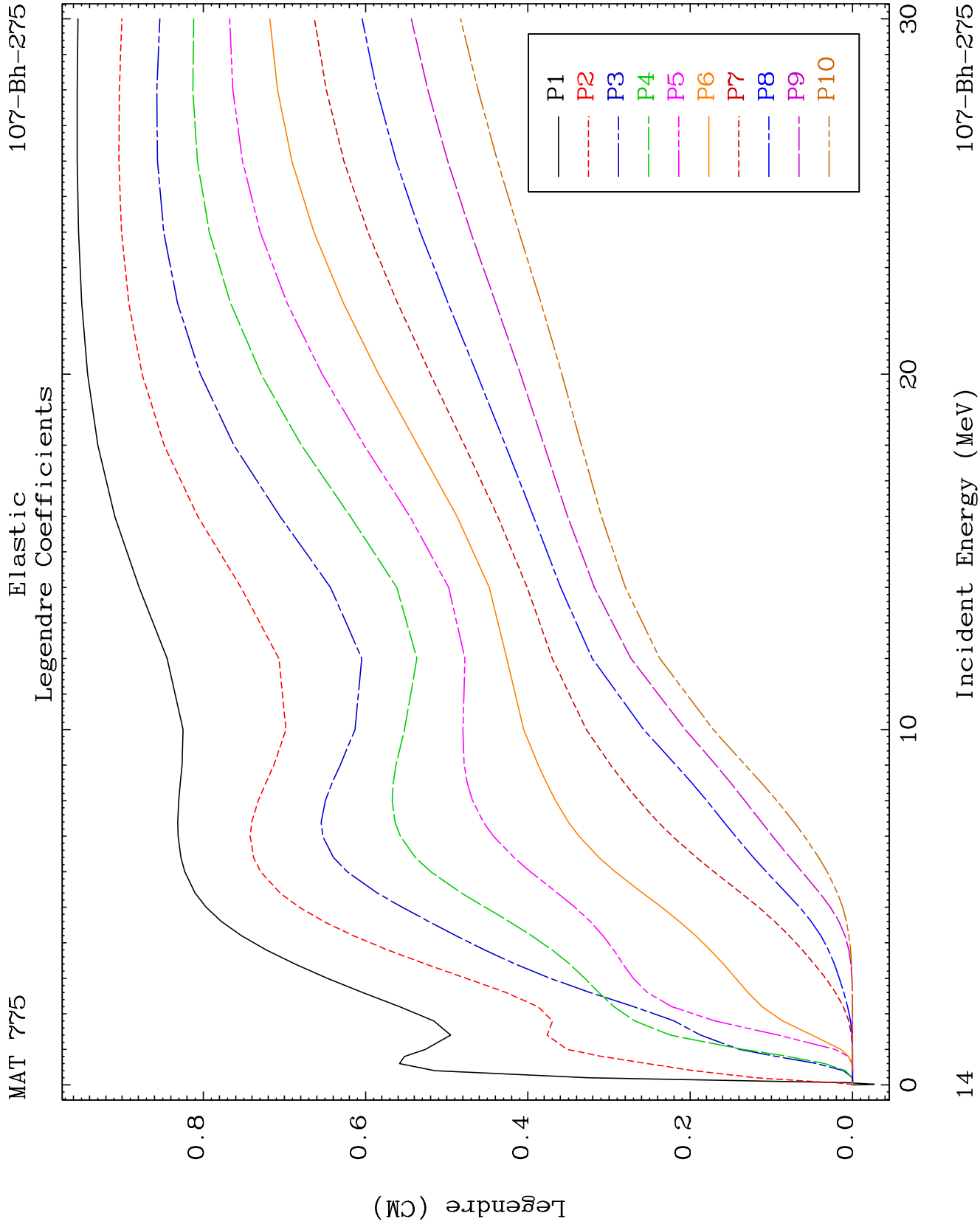
Incident Energy (MeV)

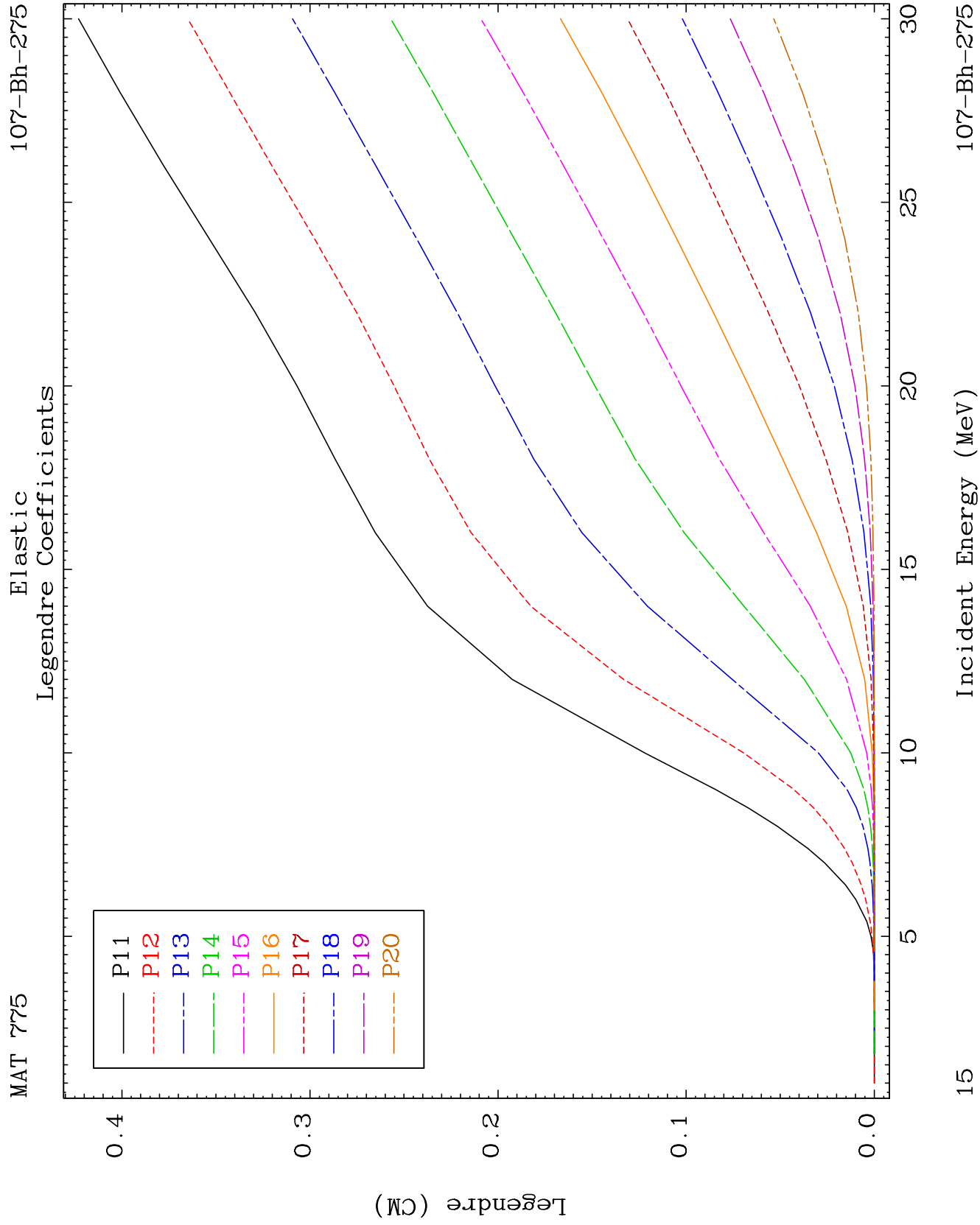
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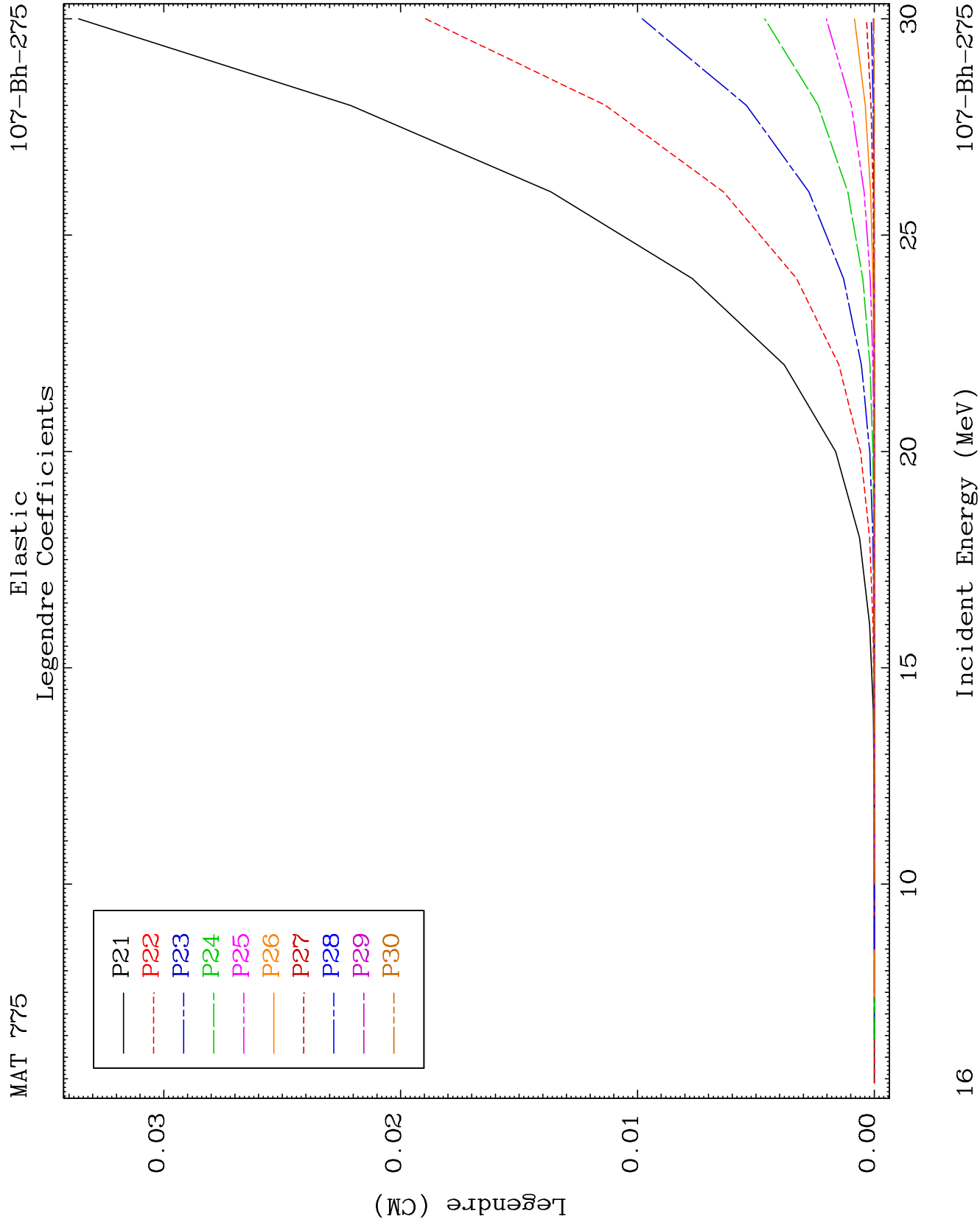


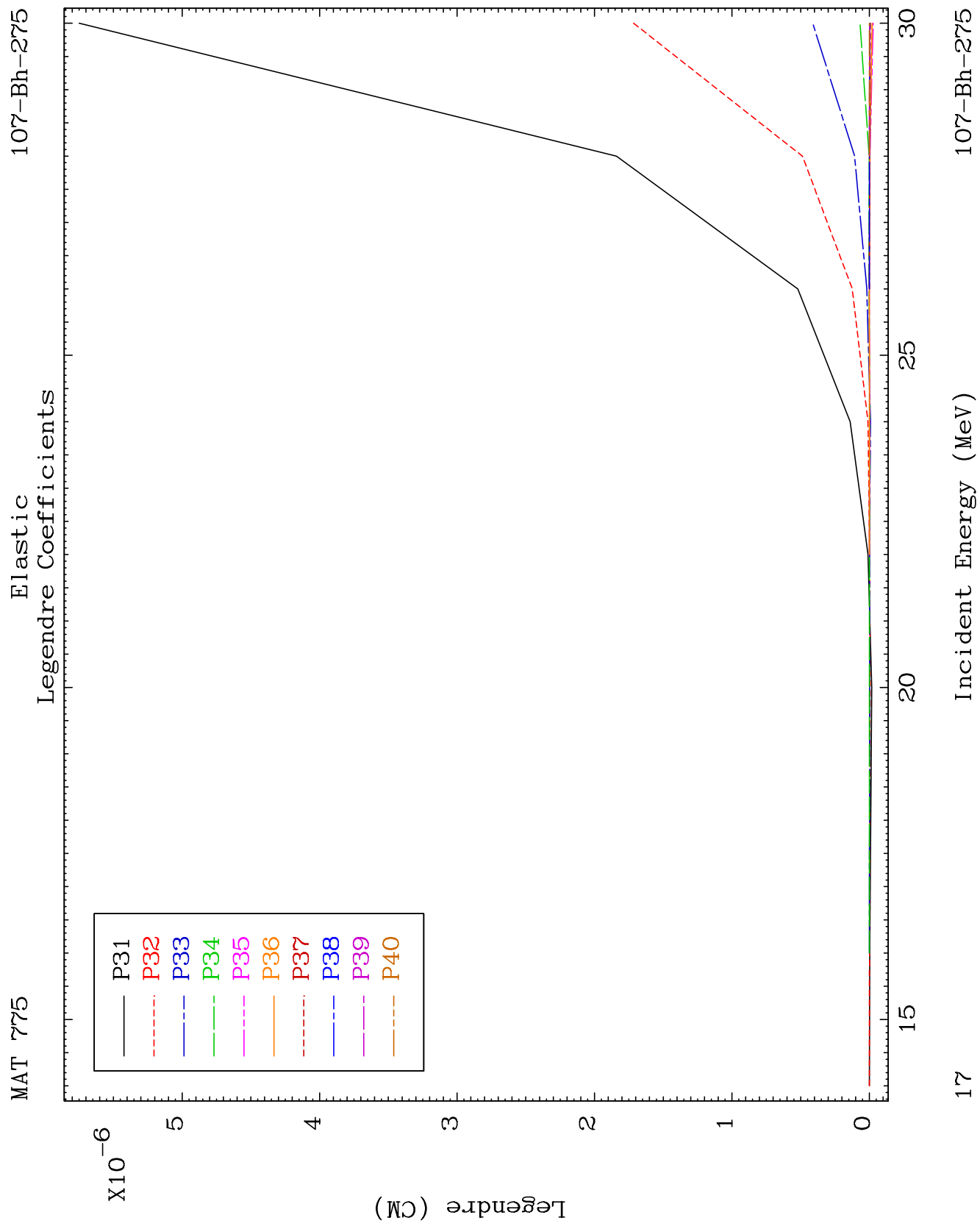








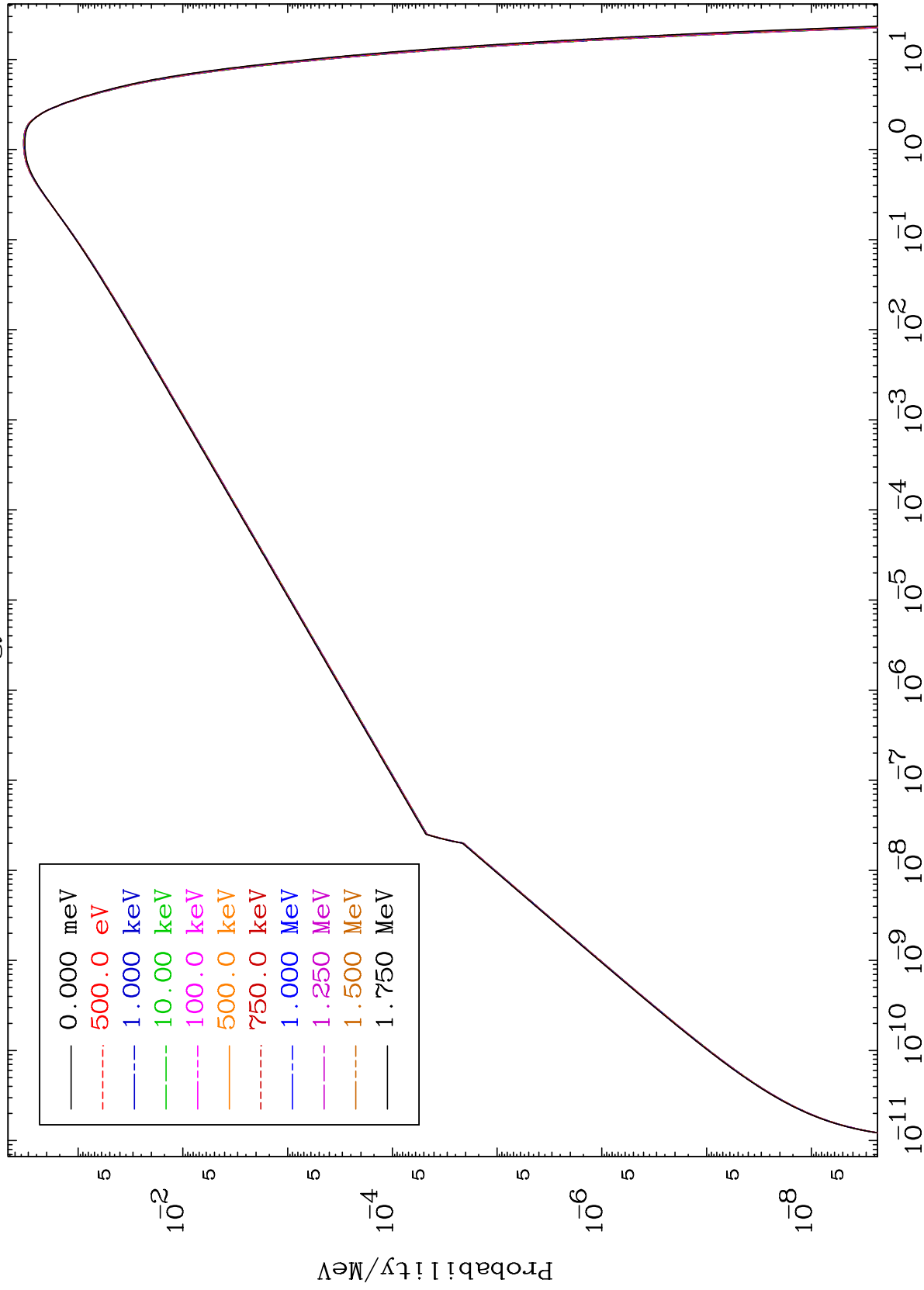




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Fission
Energy Distribution

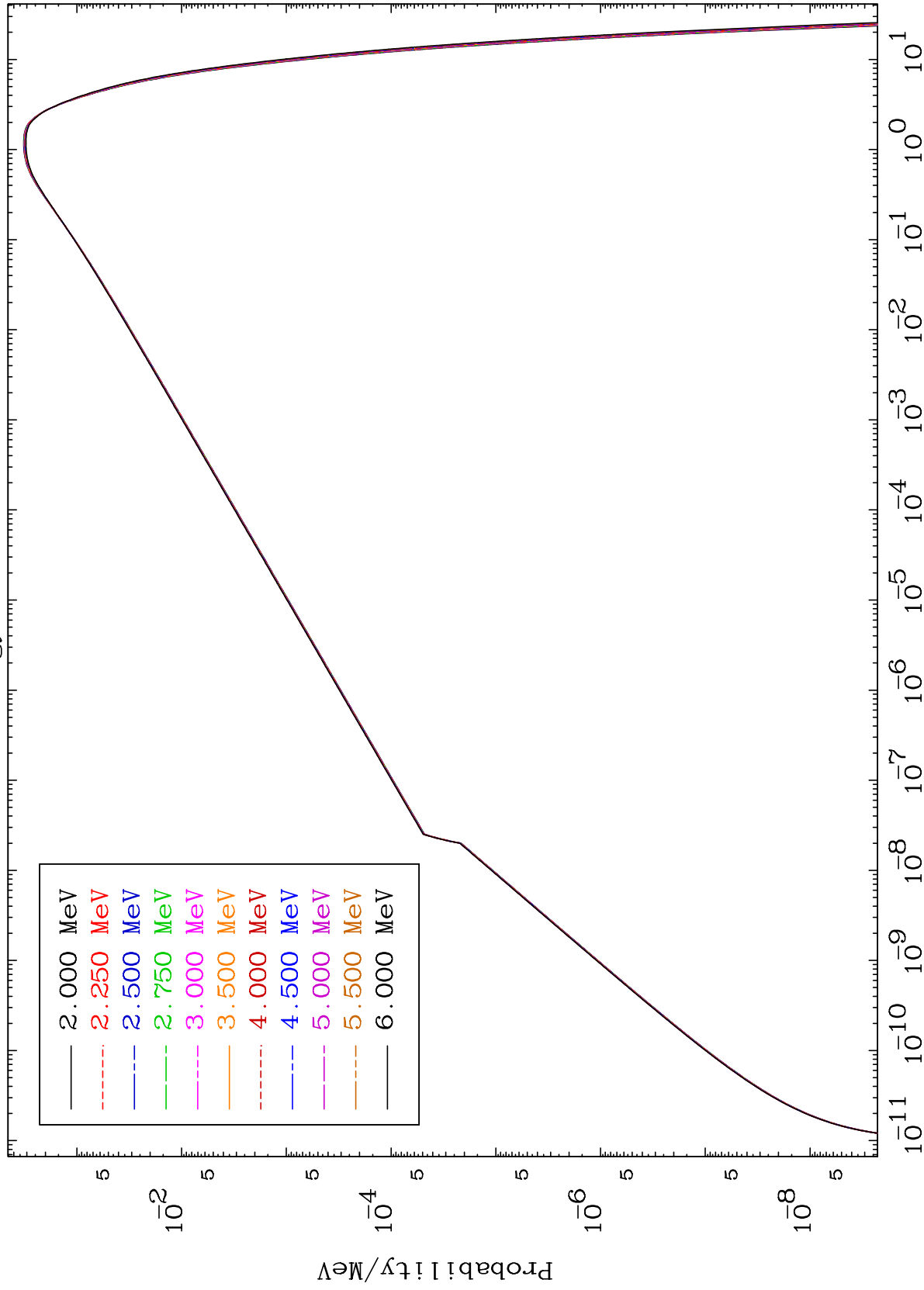
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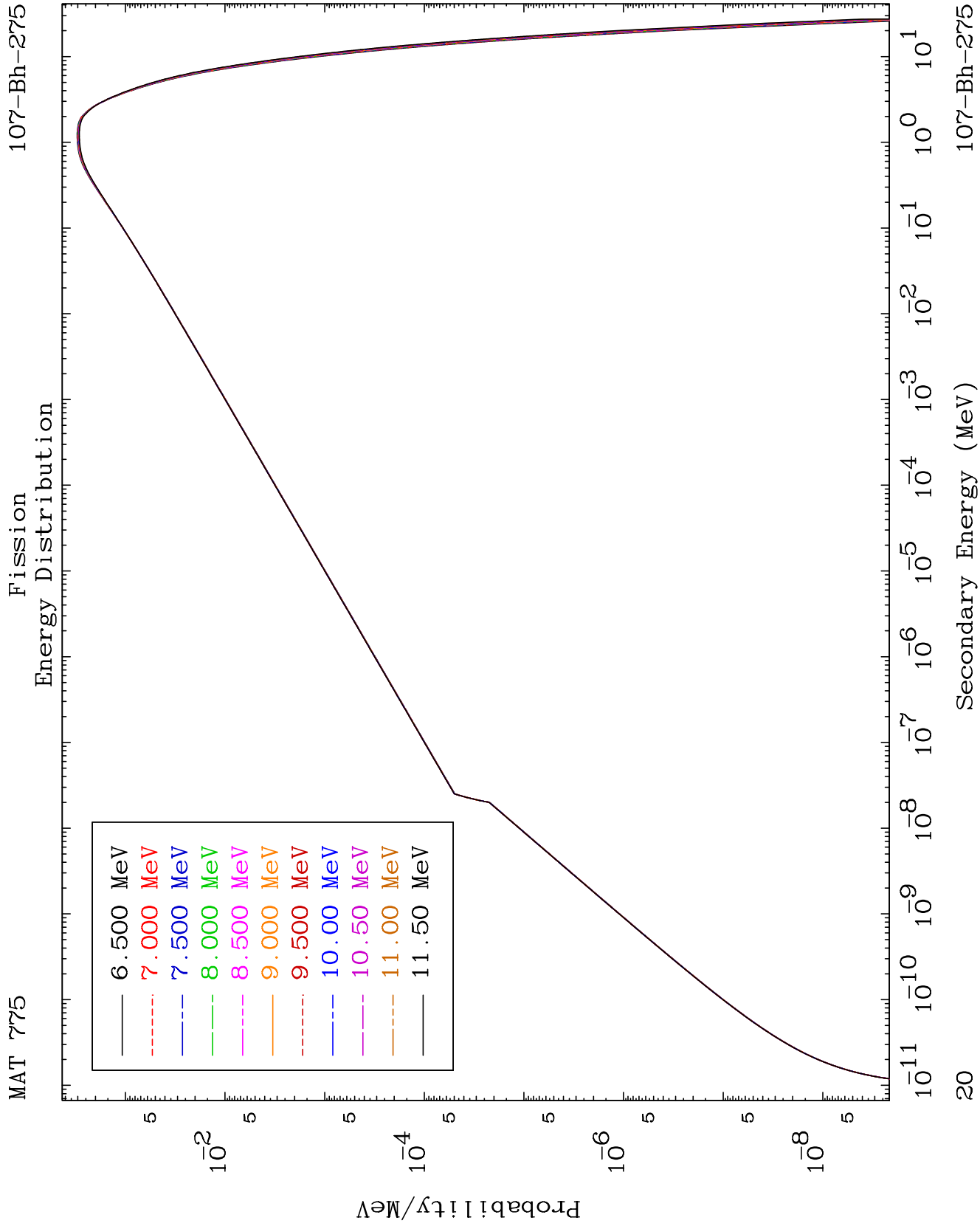


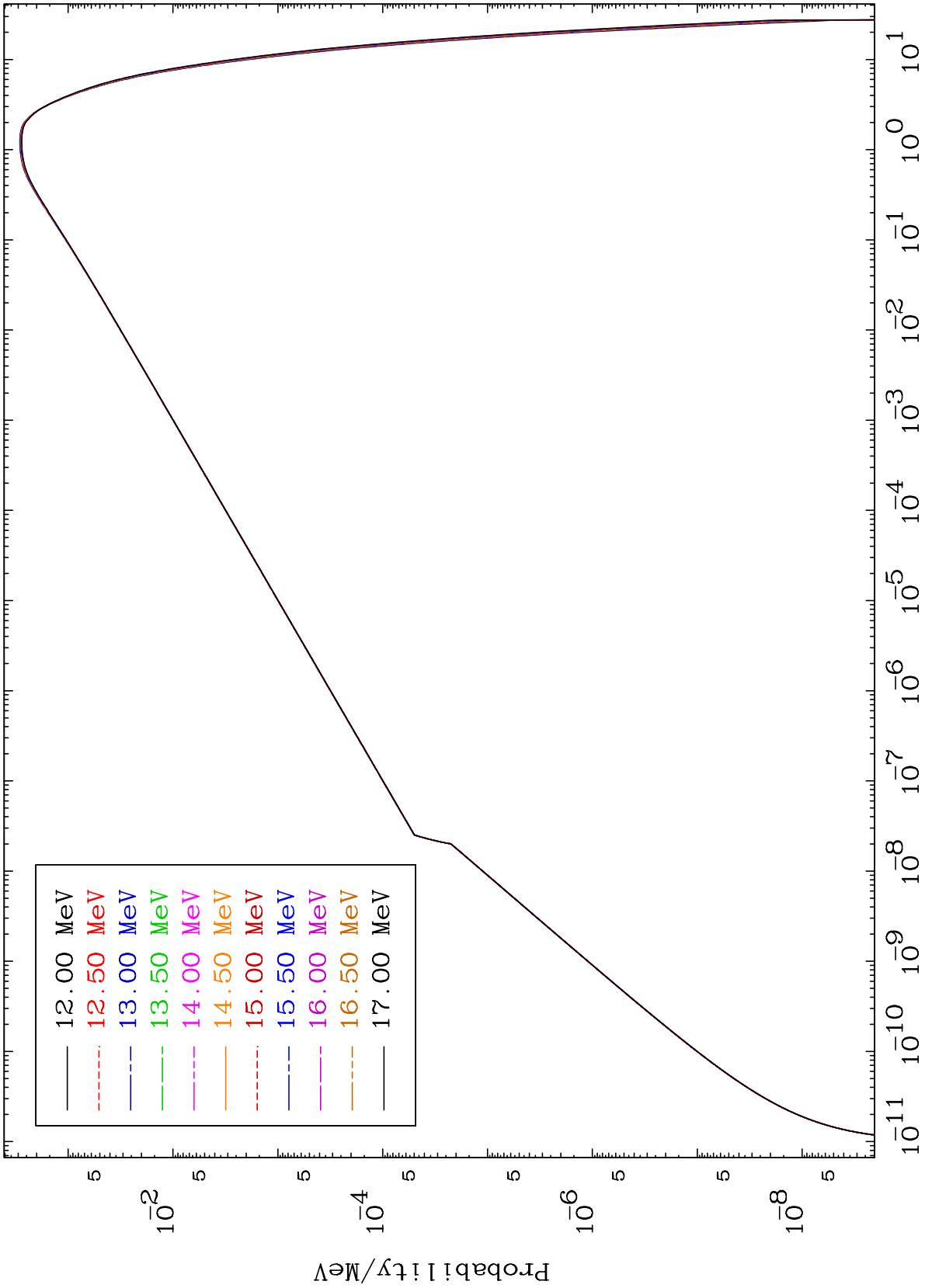
MAT 775

Fission
Energy Distribution

107-Bh-275



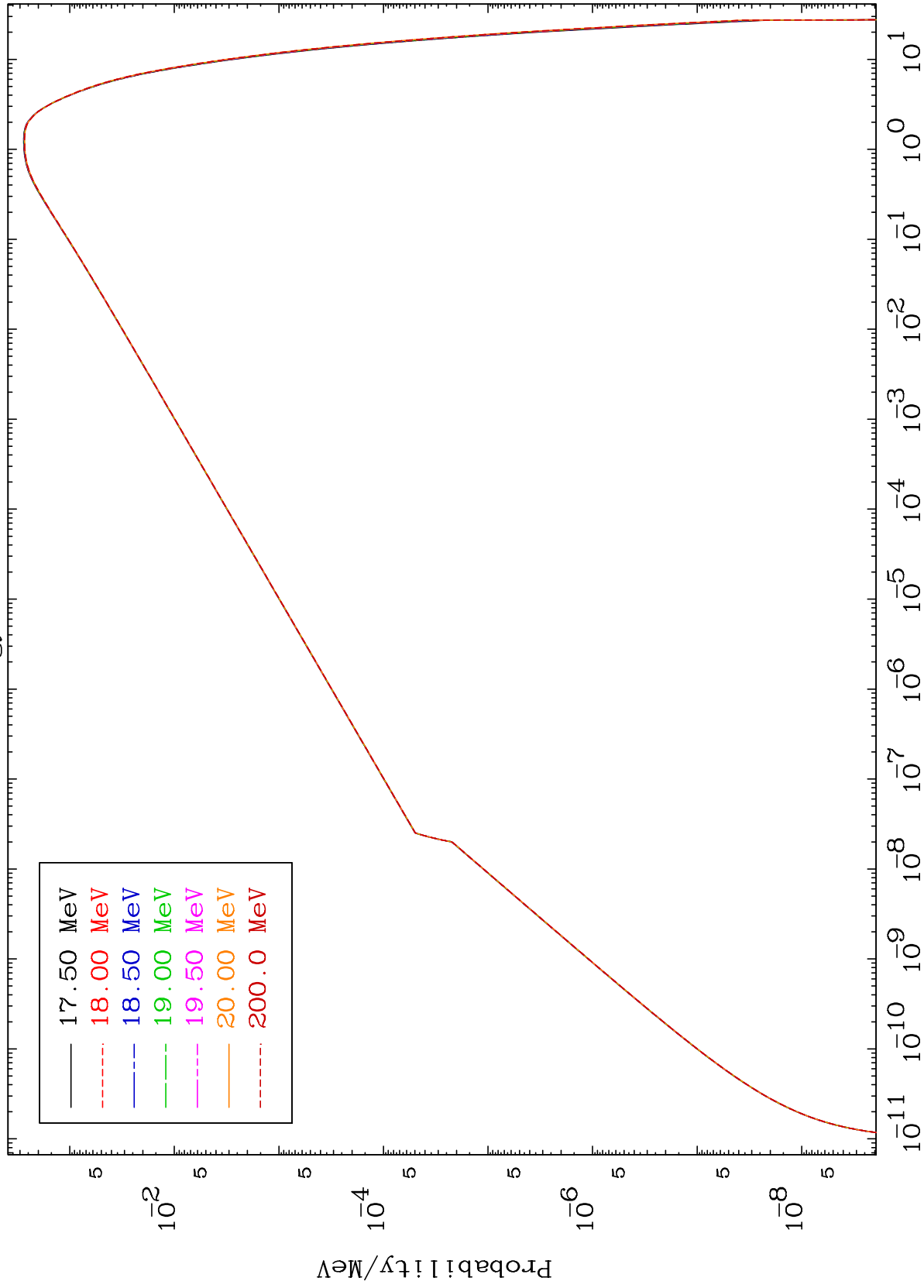




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Fission
Energy Distribution

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Secondary Energy (MeV)

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