

Program EVALPLOT  
(Version 2018-1)

by

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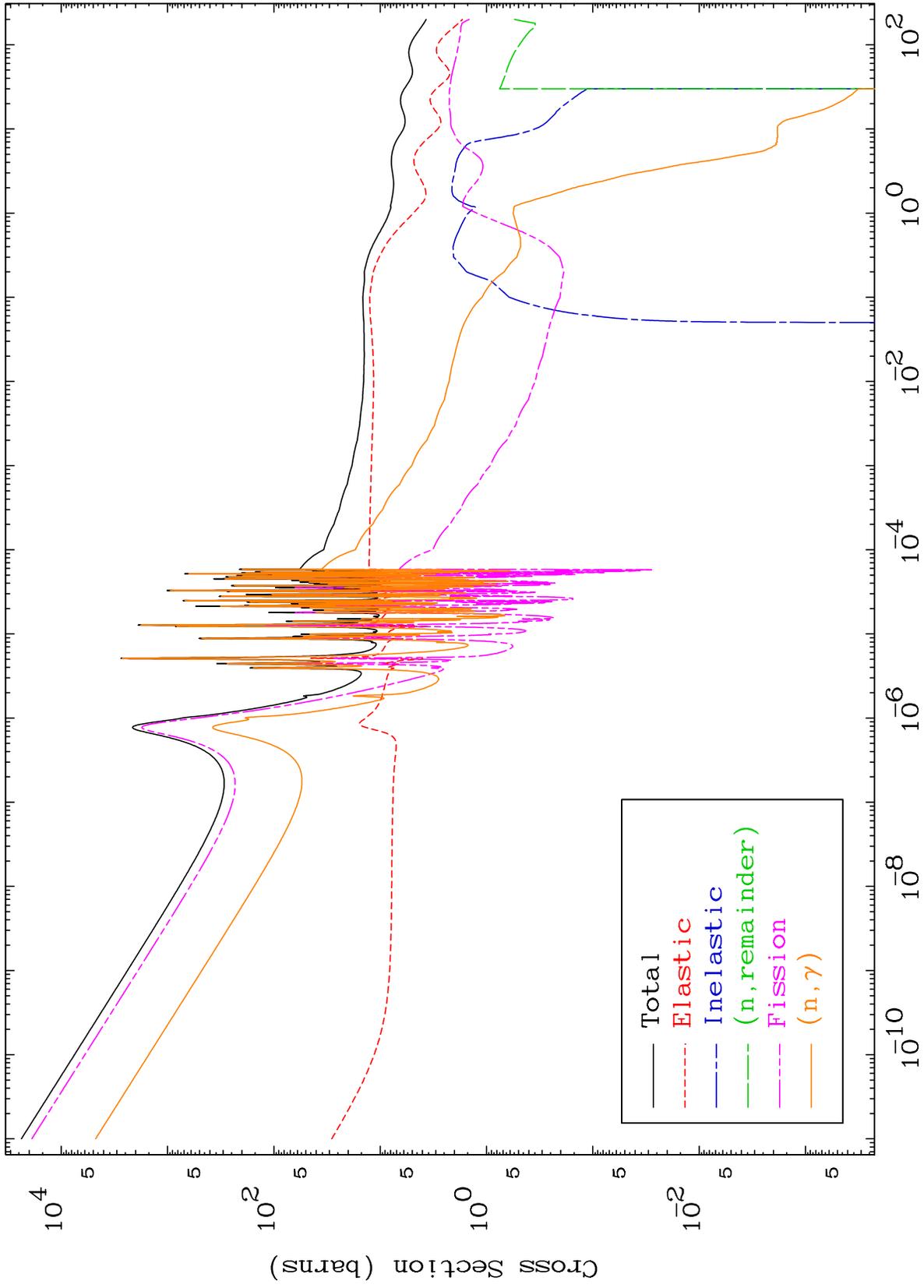
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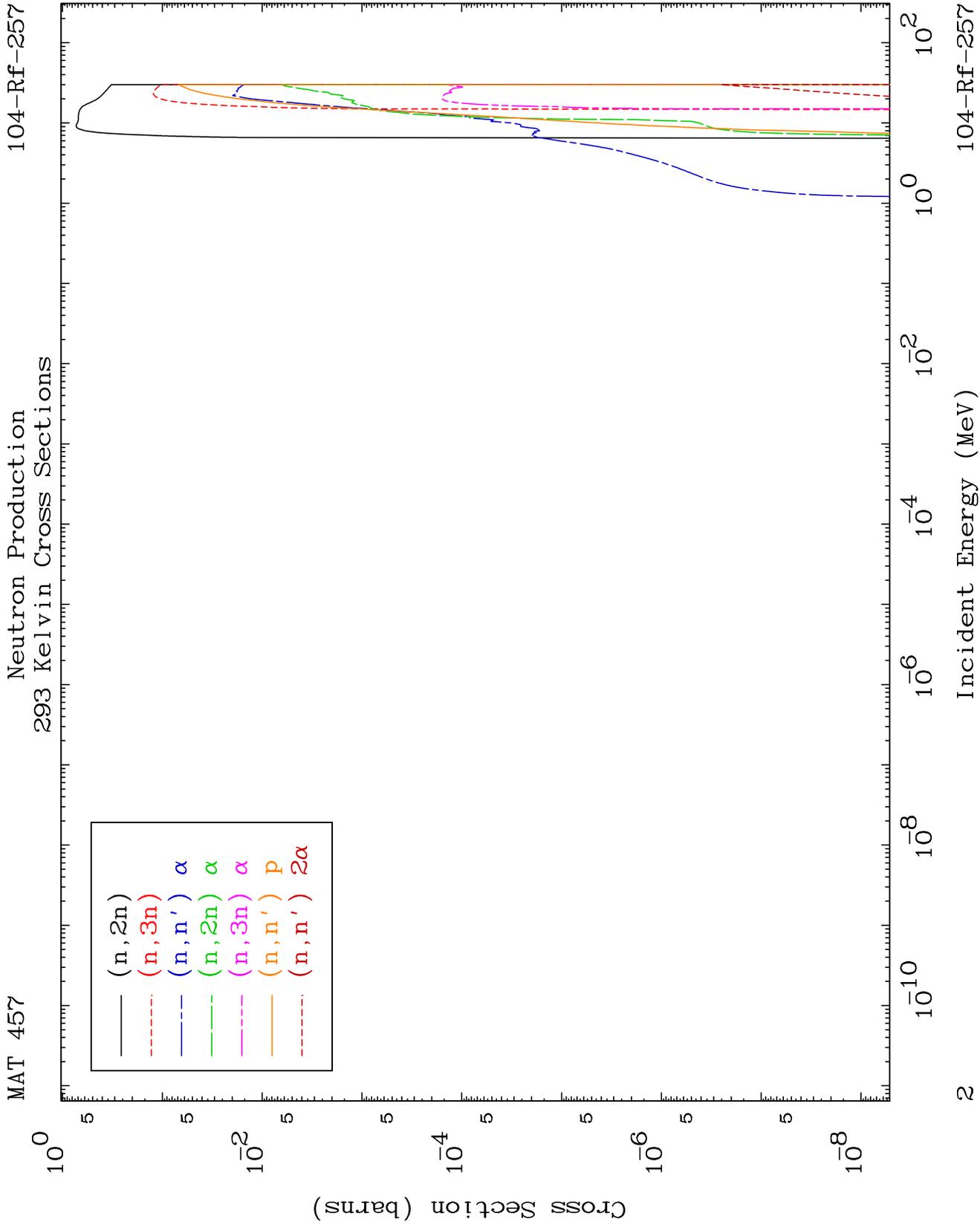
Press Mouse Button to Start

MAT 457

Major  
293 Kelvin Cross Sections

104-Rf-257

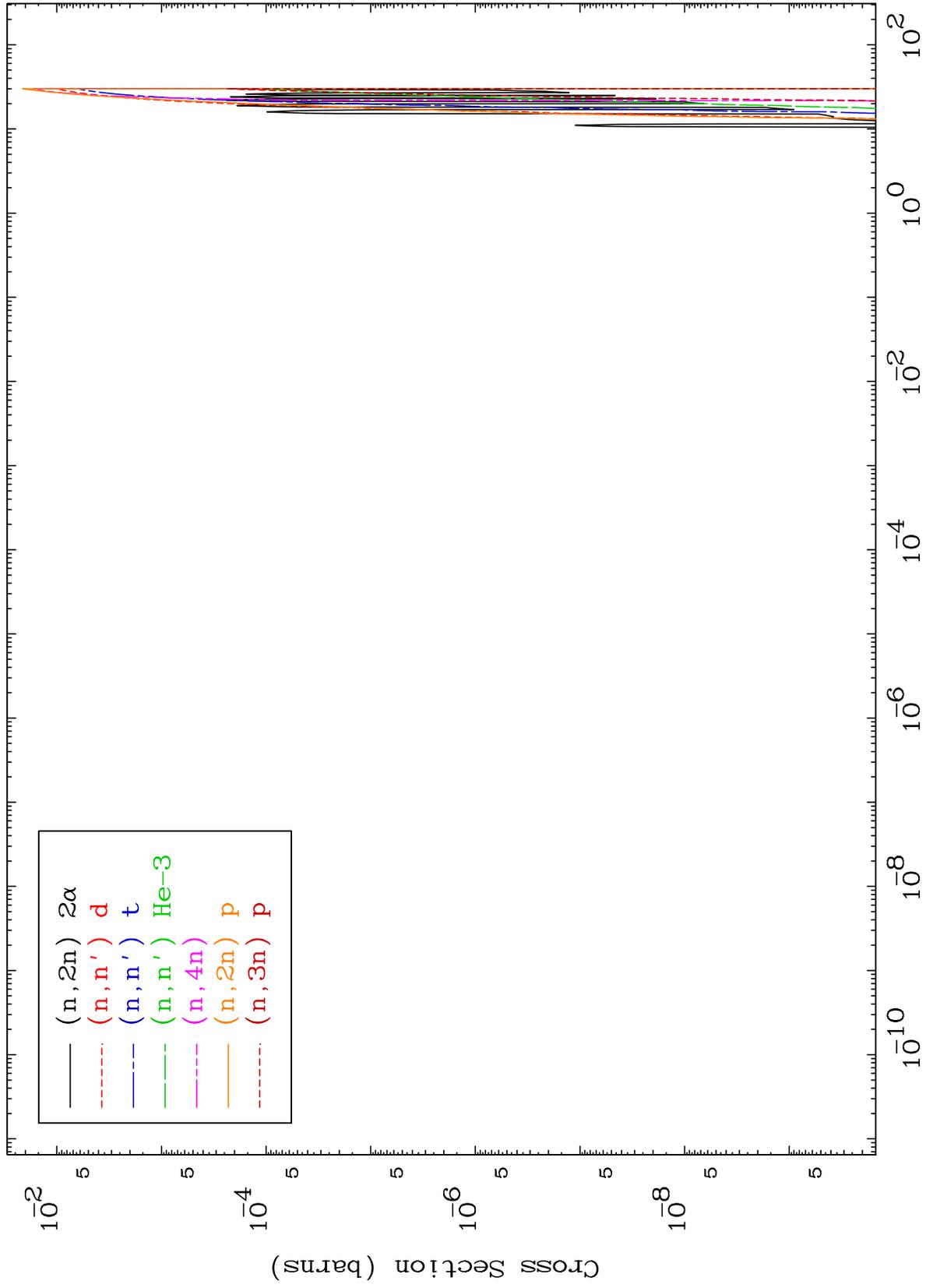




MAT 457

Neutron Production  
293 Kelvin Cross Sections

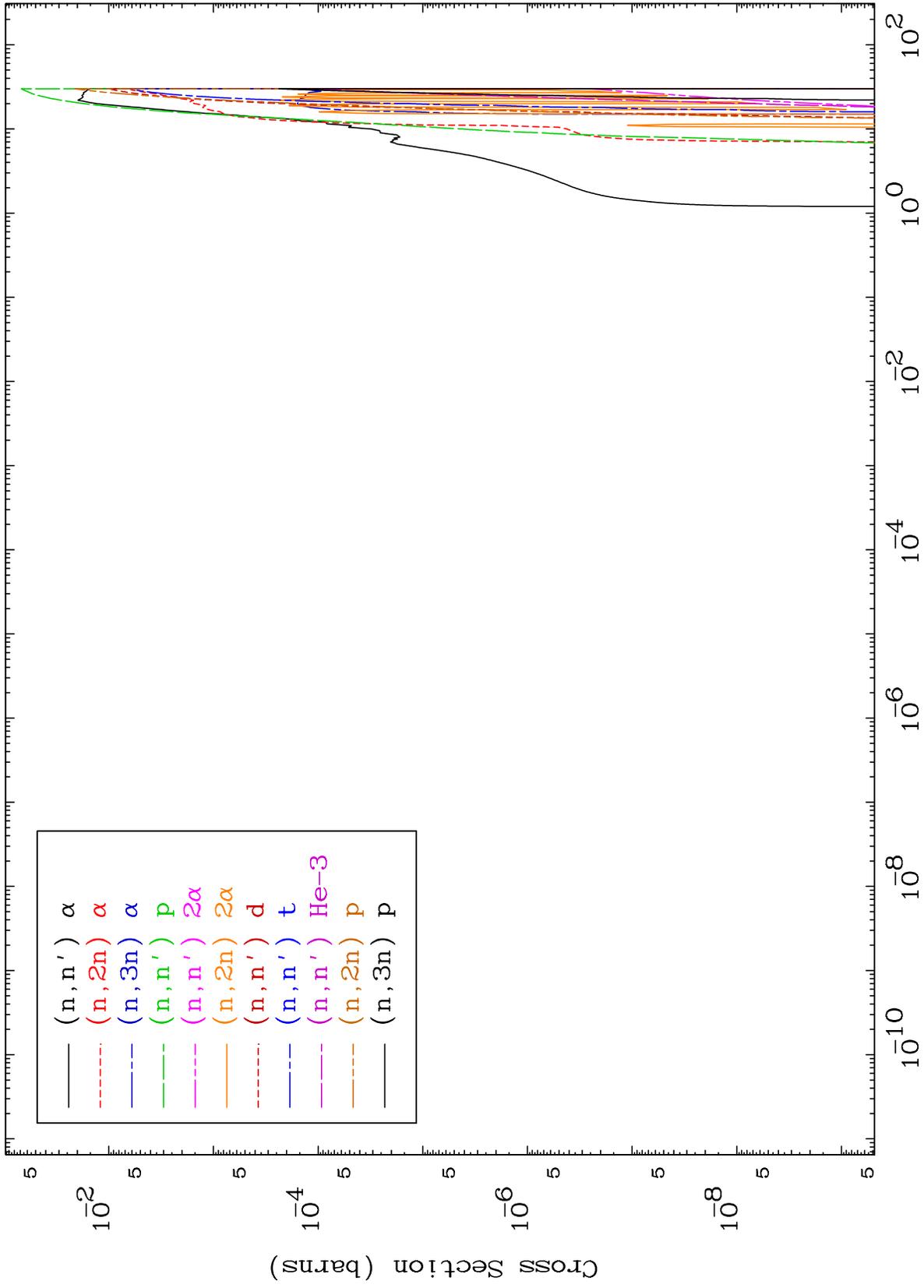
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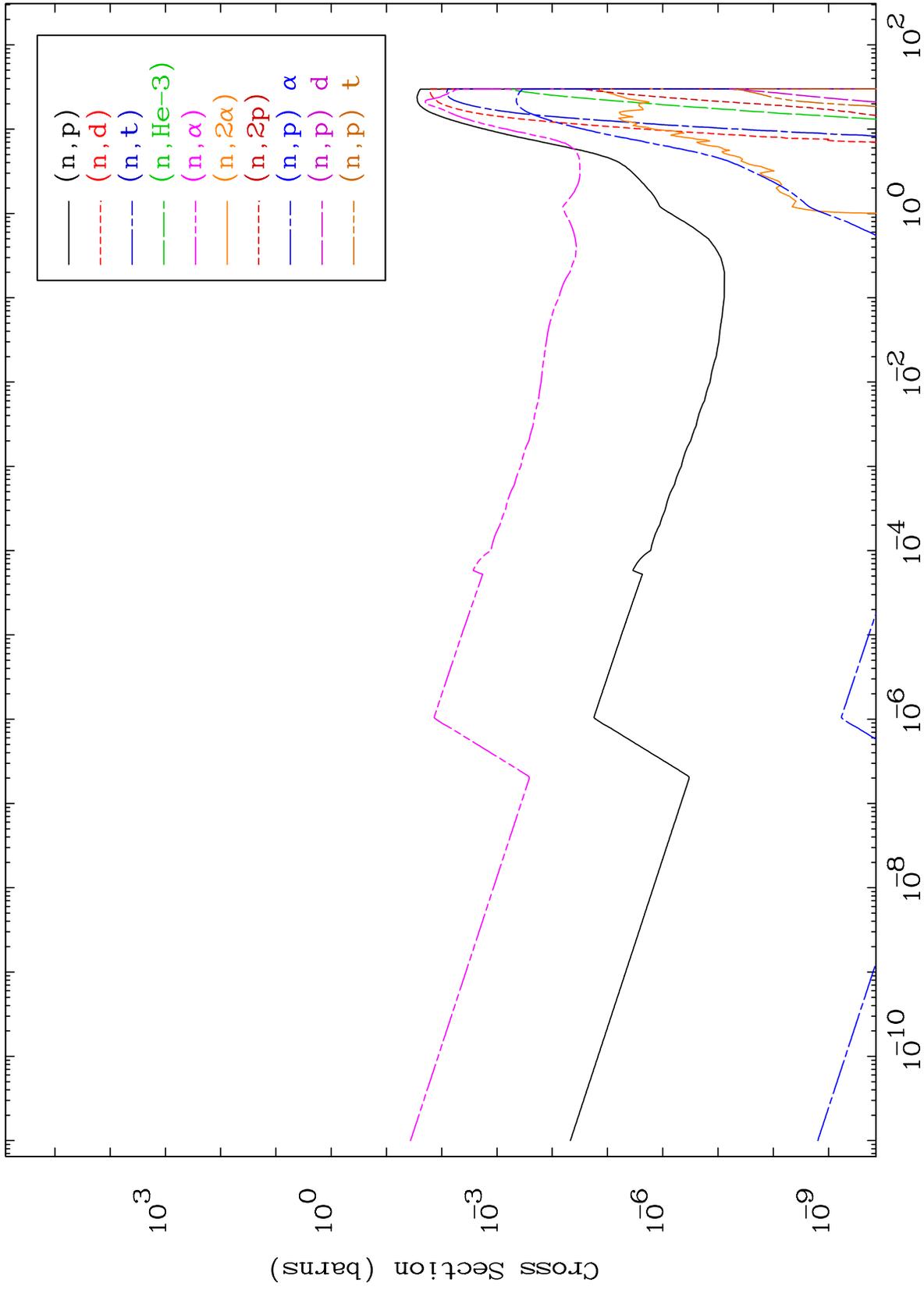


MAT 457

Charged Particle  
293 Kelvin Cross Sections

104-Rf-257

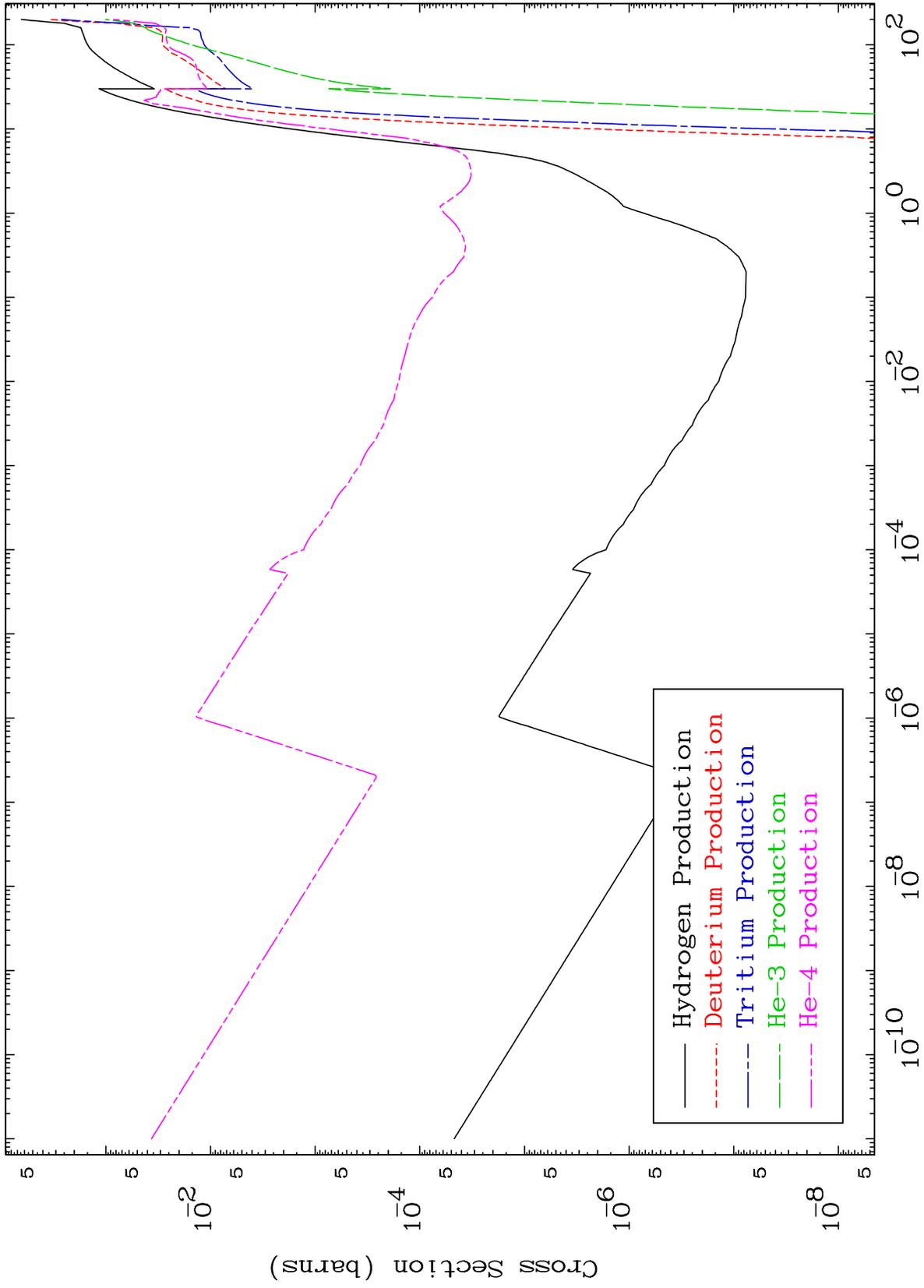




MAT 457

Particle Production  
293 Kelvin Cross Sections

104-Rf-257

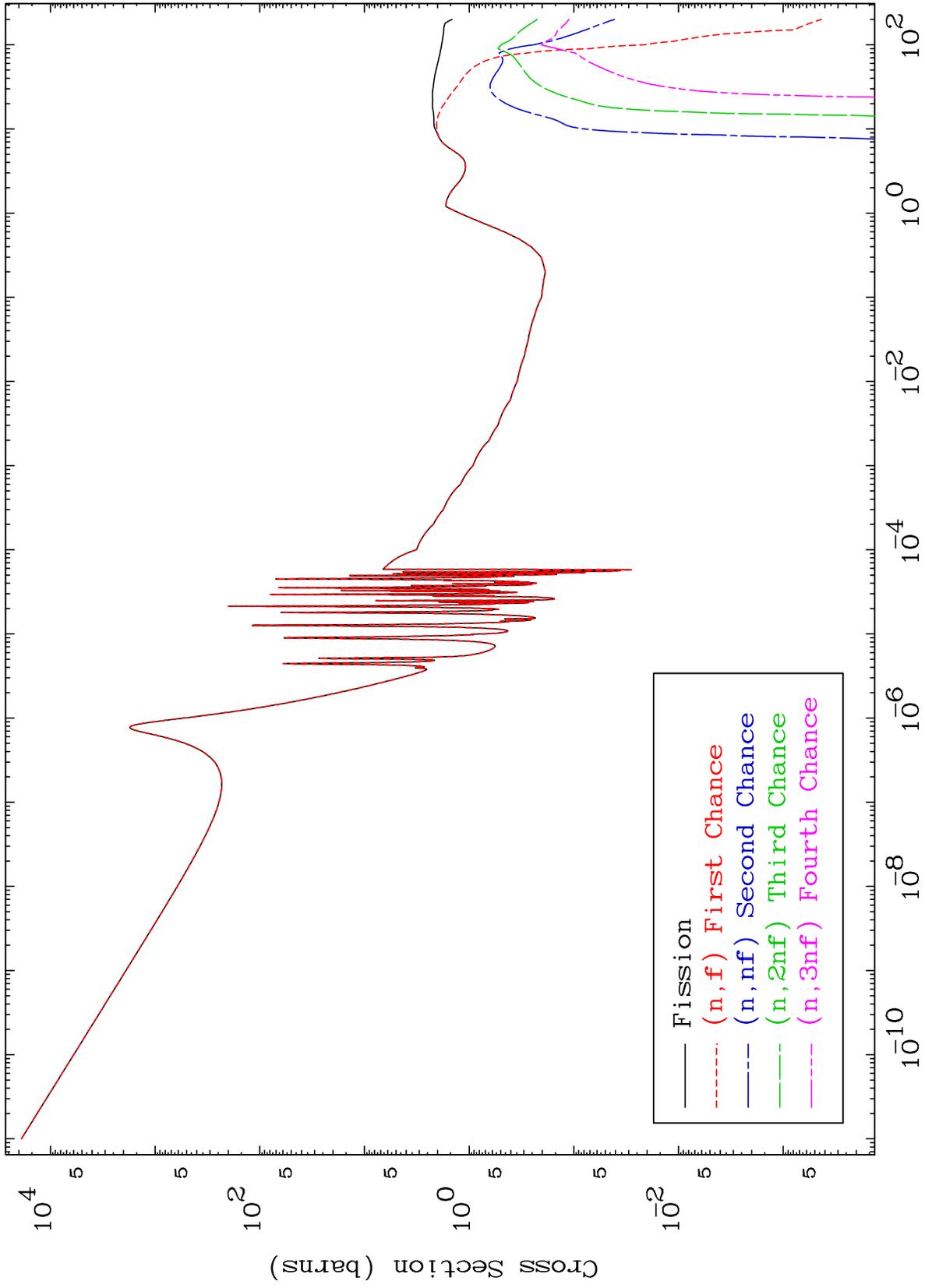


104-Rf-257

MAT 457

Fission  
293 Kelvin Cross Sections

104-Rf-257



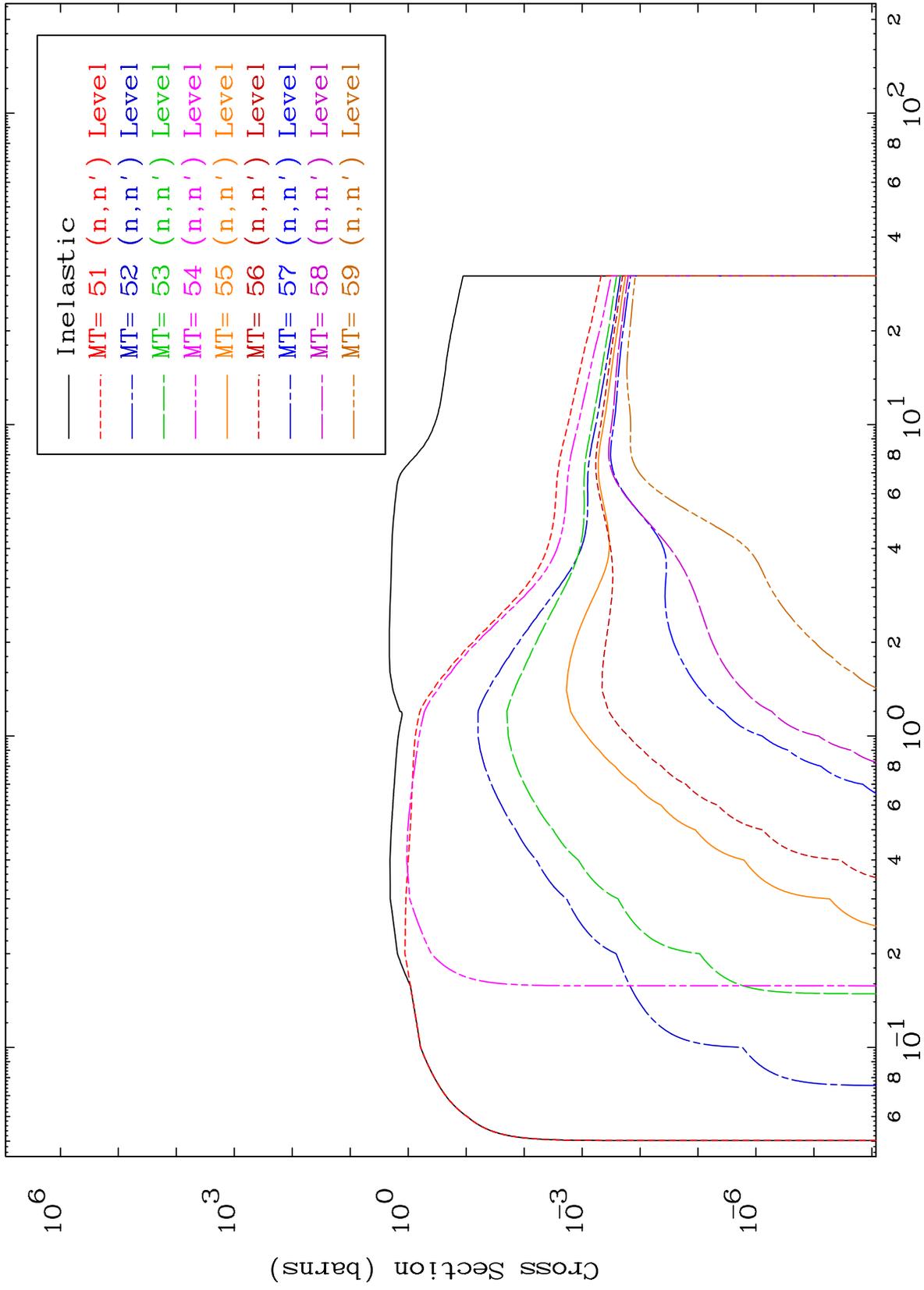
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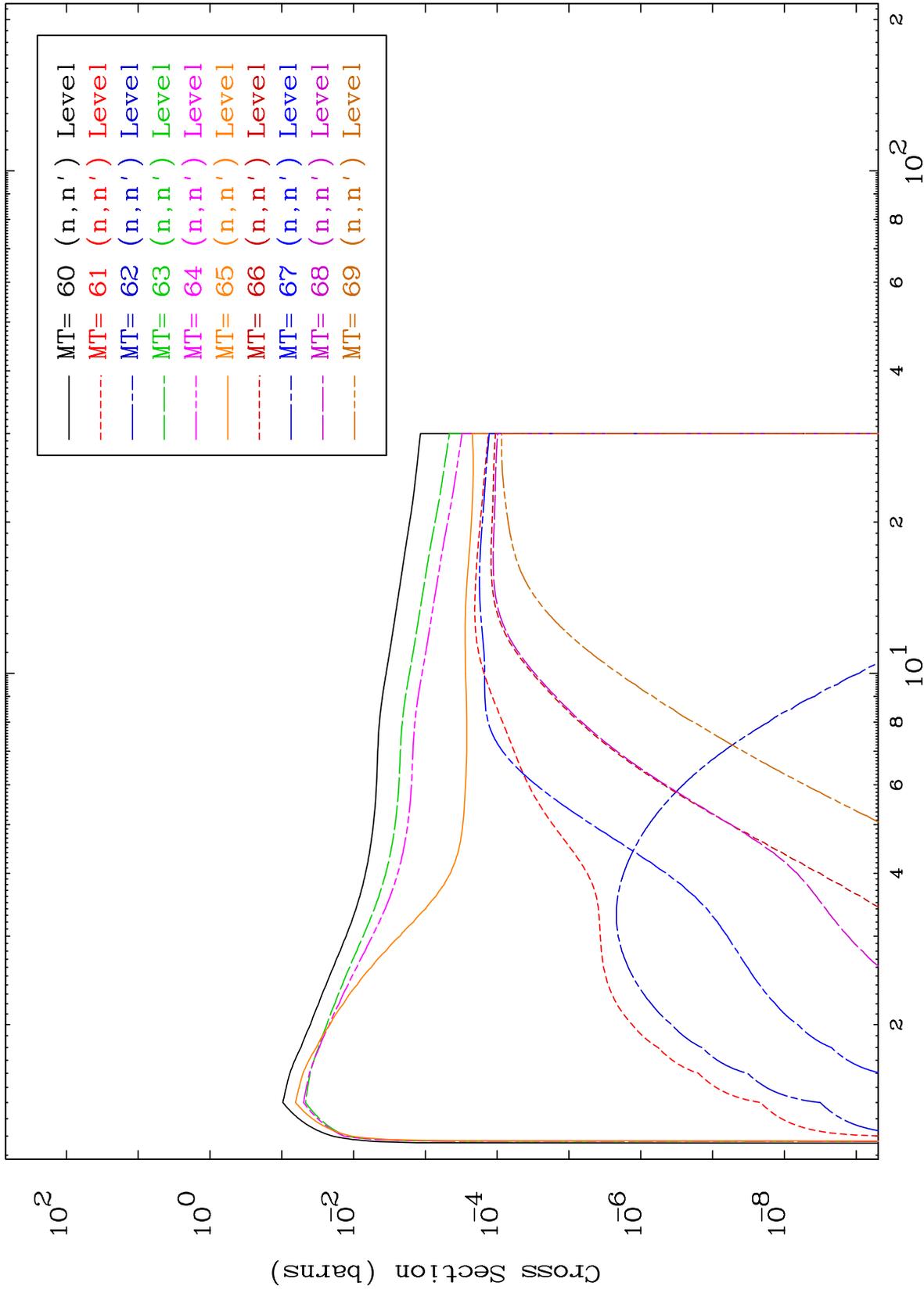
MAT 457

(n,n') Level

104-Rf-257

293 Kelvin Cross Sections



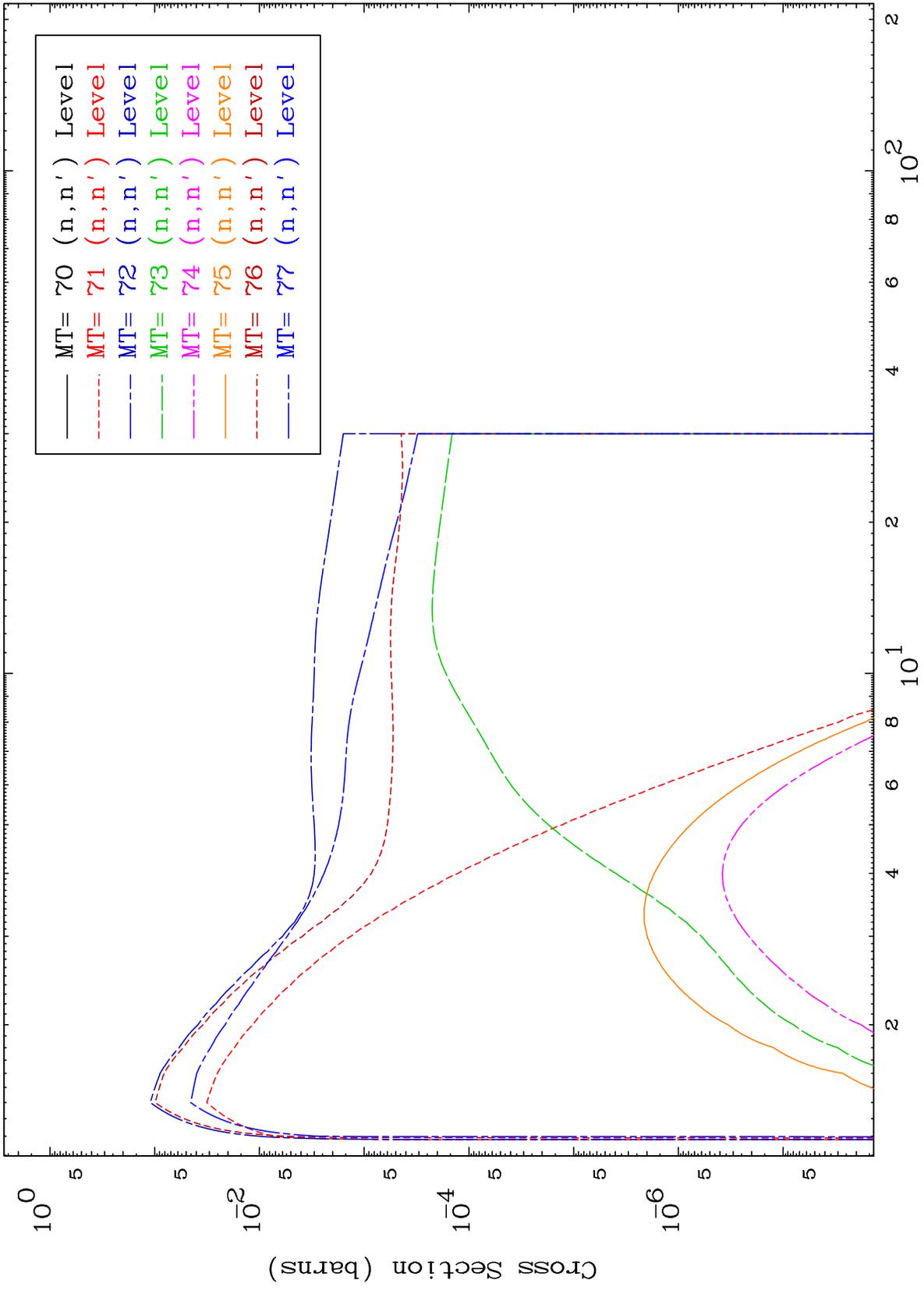


MAT 457

(n,n') Level

104-Rf-257

293 Kelvin Cross Sections



10

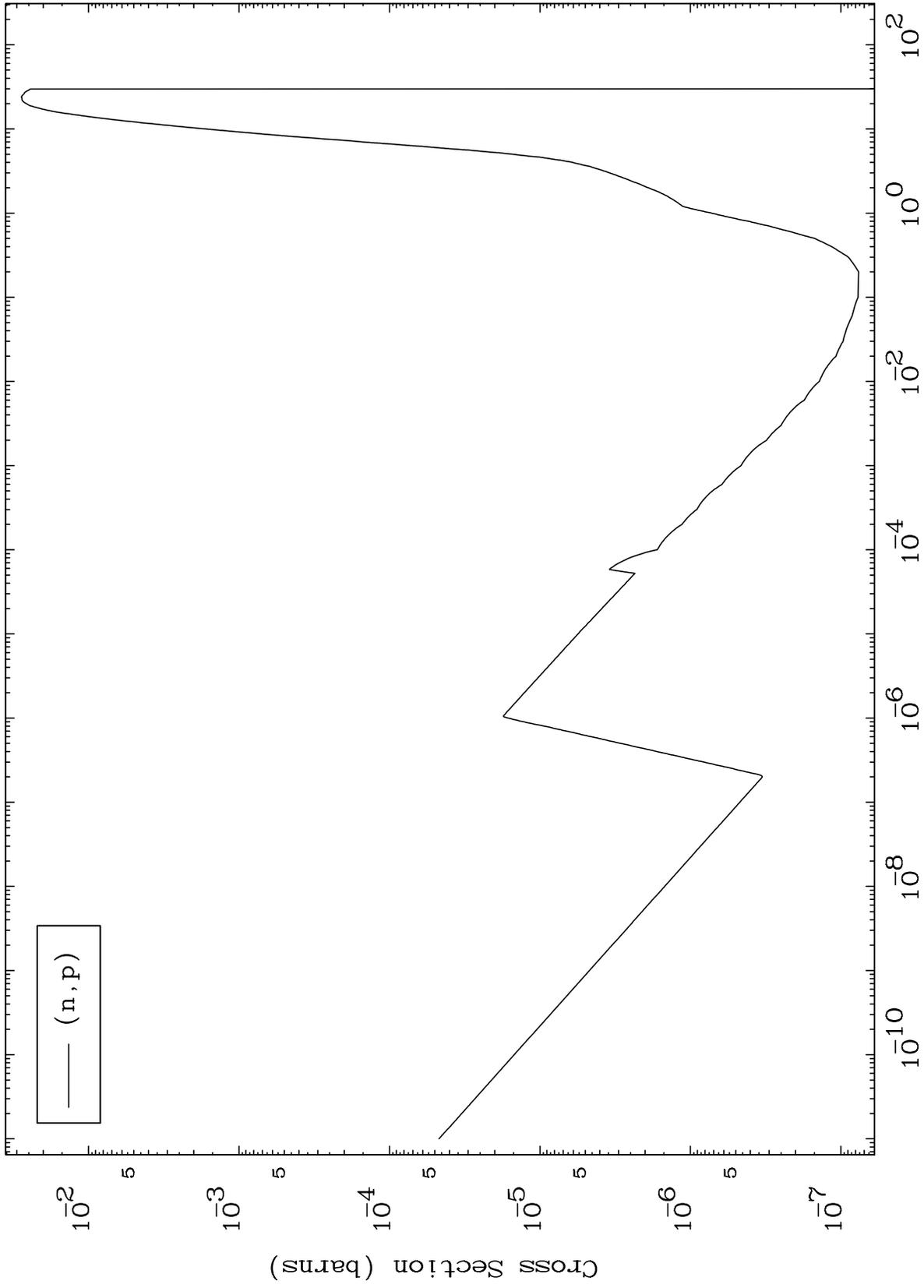
Incident Energy (MeV)

104-Rf-257

MAT 457

(n,p) Levels  
293 Kelvin Cross Sections

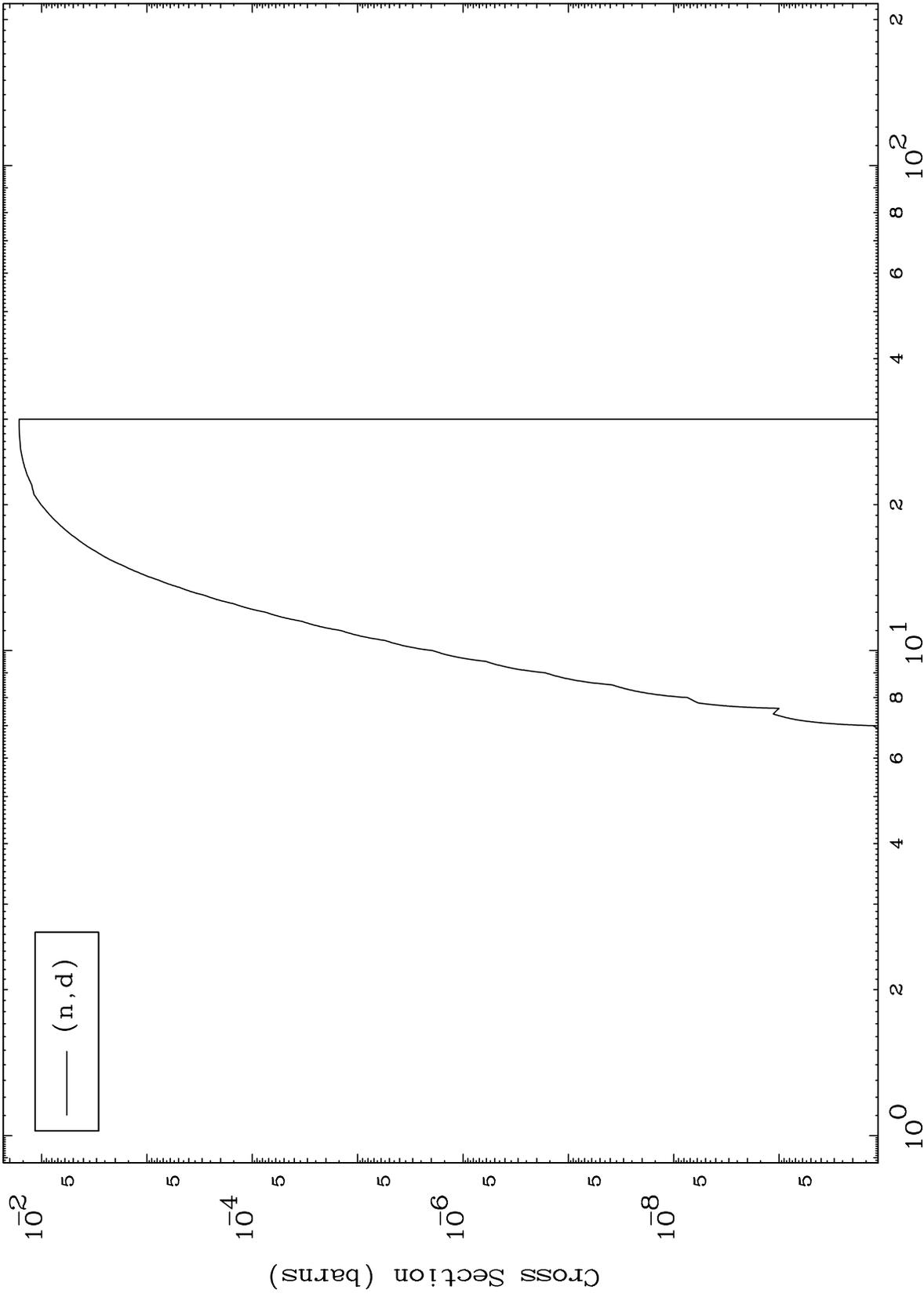
104-Rf-257



MAT 457

(n,d) Levels  
293 Kelvin Cross Sections

104-Rf-257



12

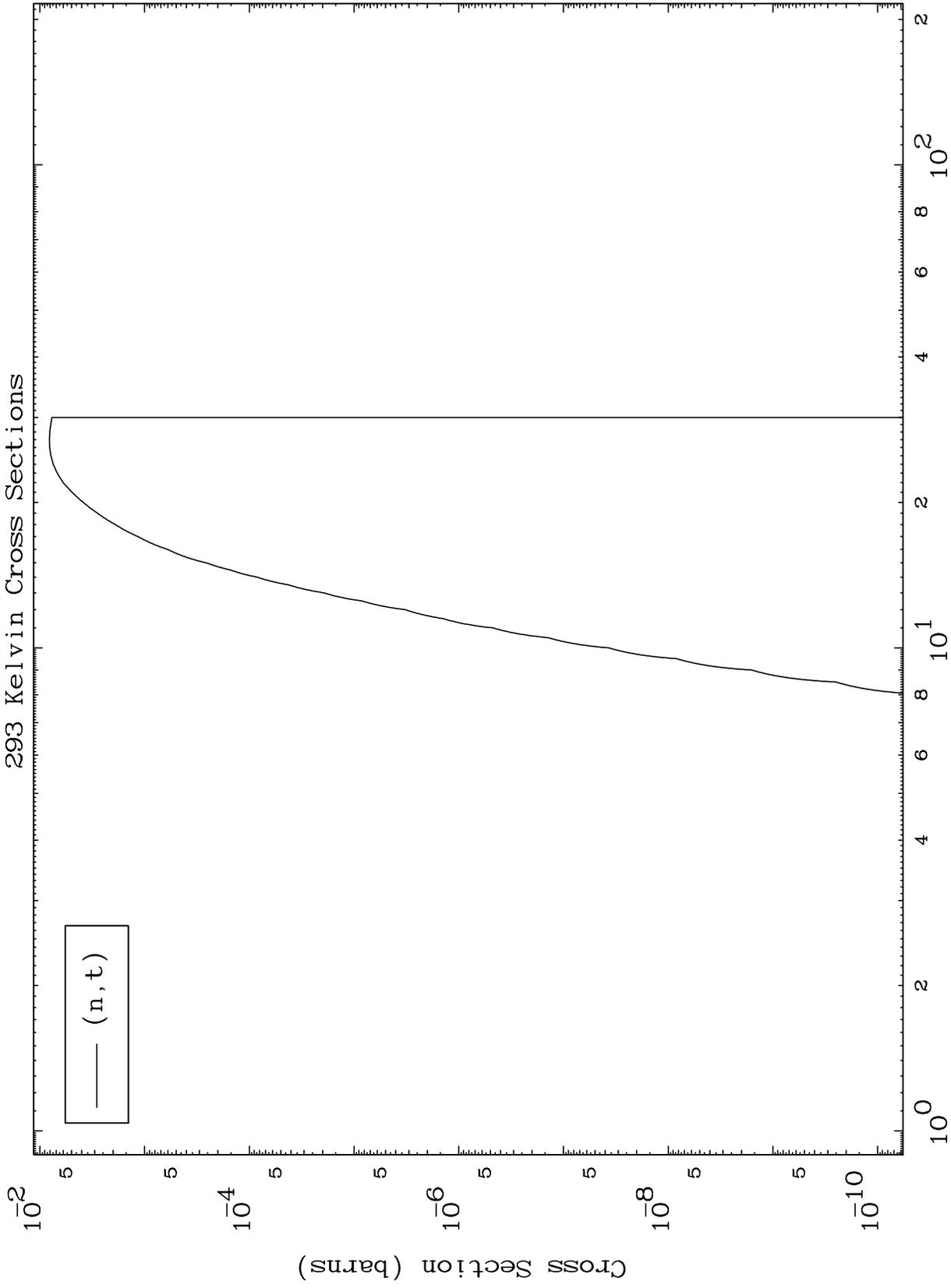
Incident Energy (MeV)

104-Rf-257

MAT 457

(n,t) Levels  
293 Kelvin Cross Sections

104-Rf-257



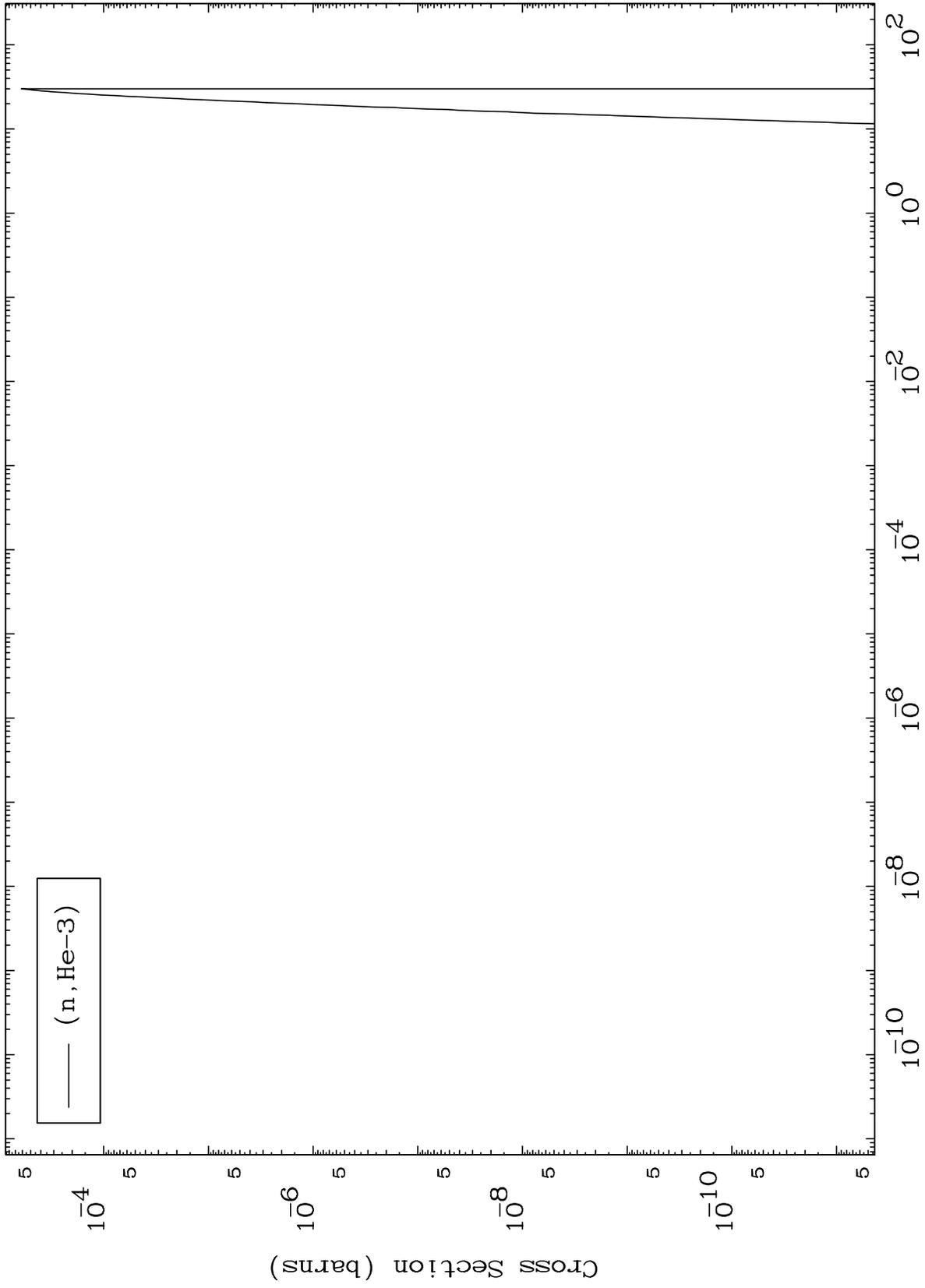
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104-Rf-257

MAT 457

(n,He3) Levels  
293 Kelvin Cross Sections

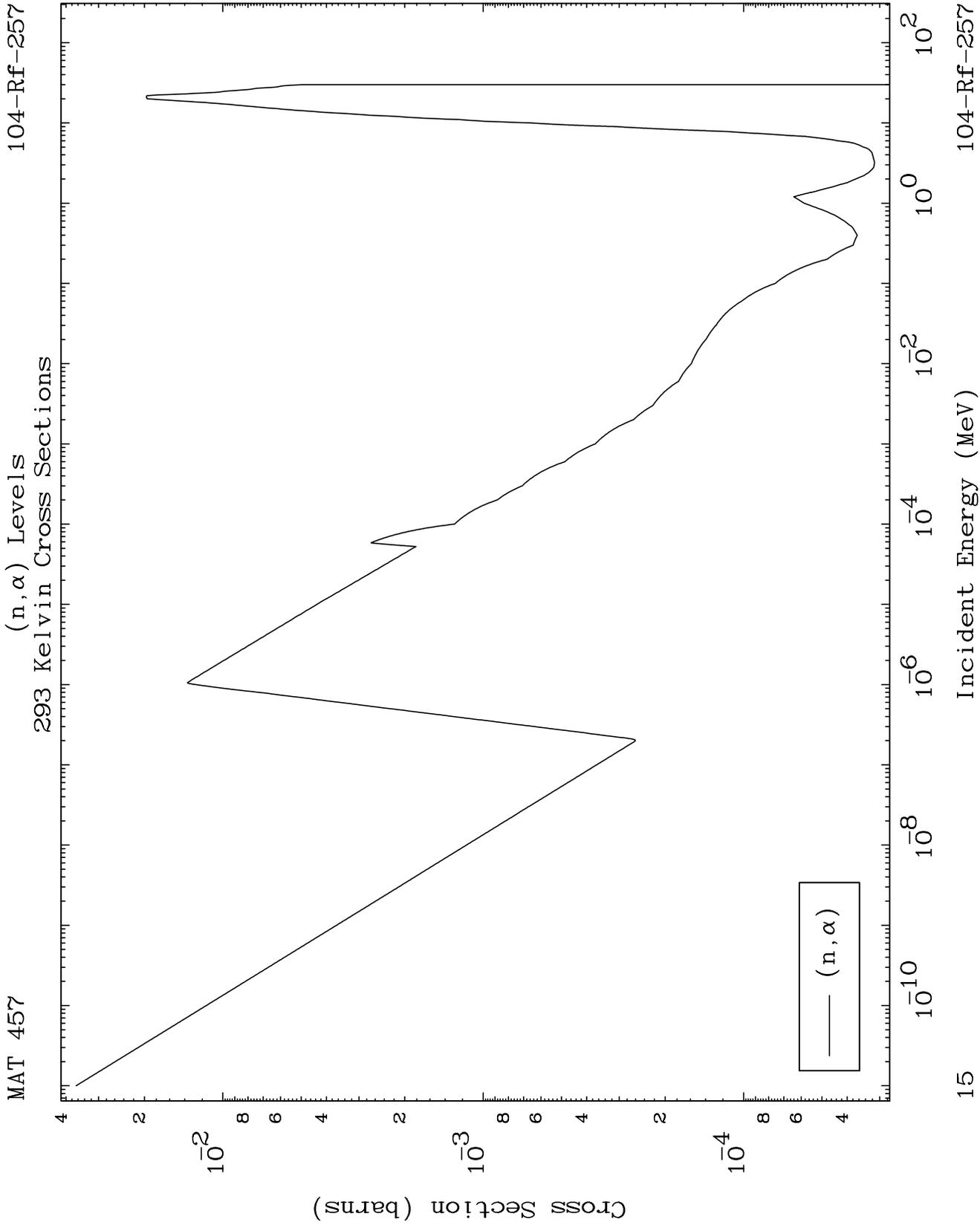
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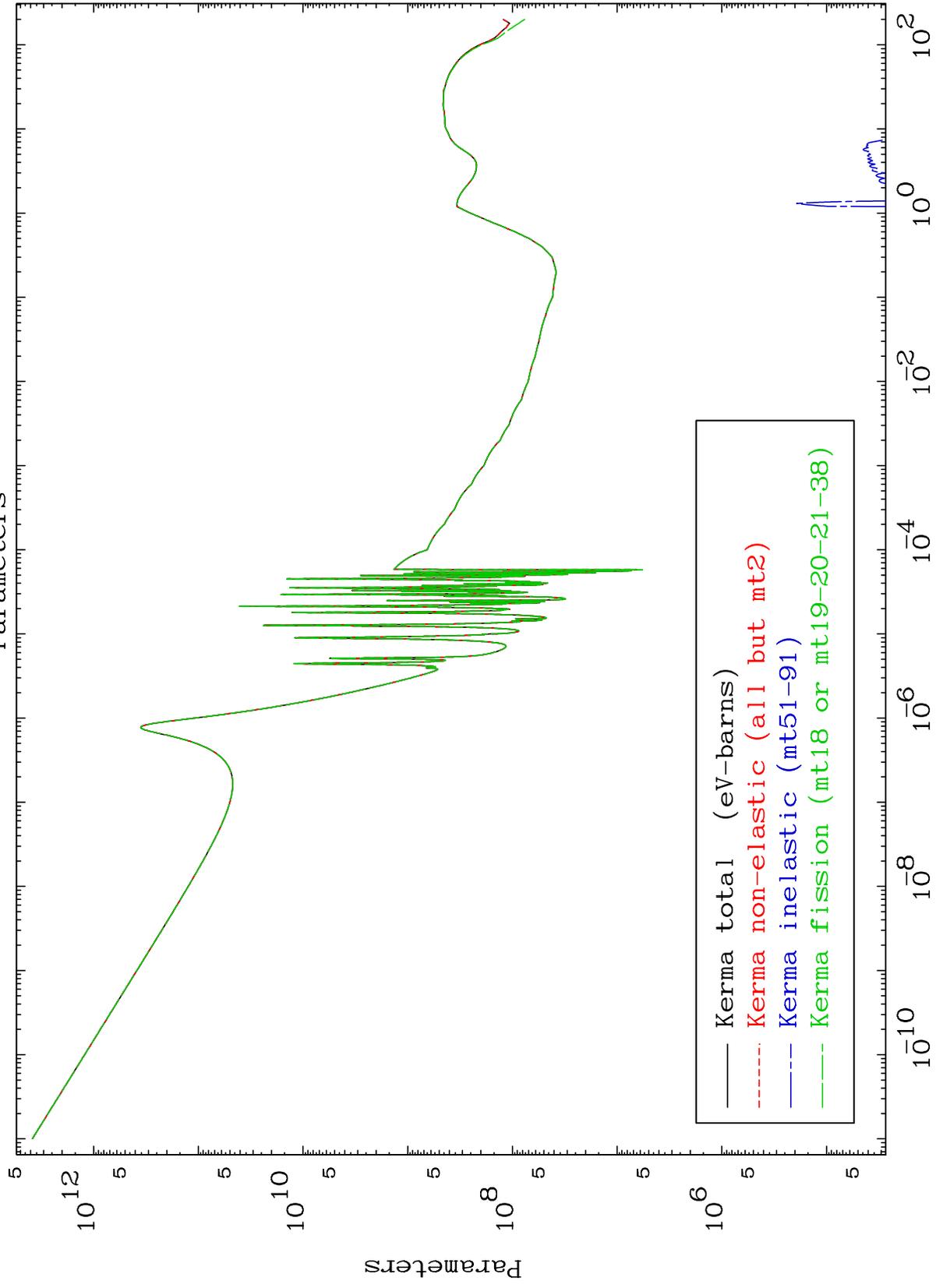


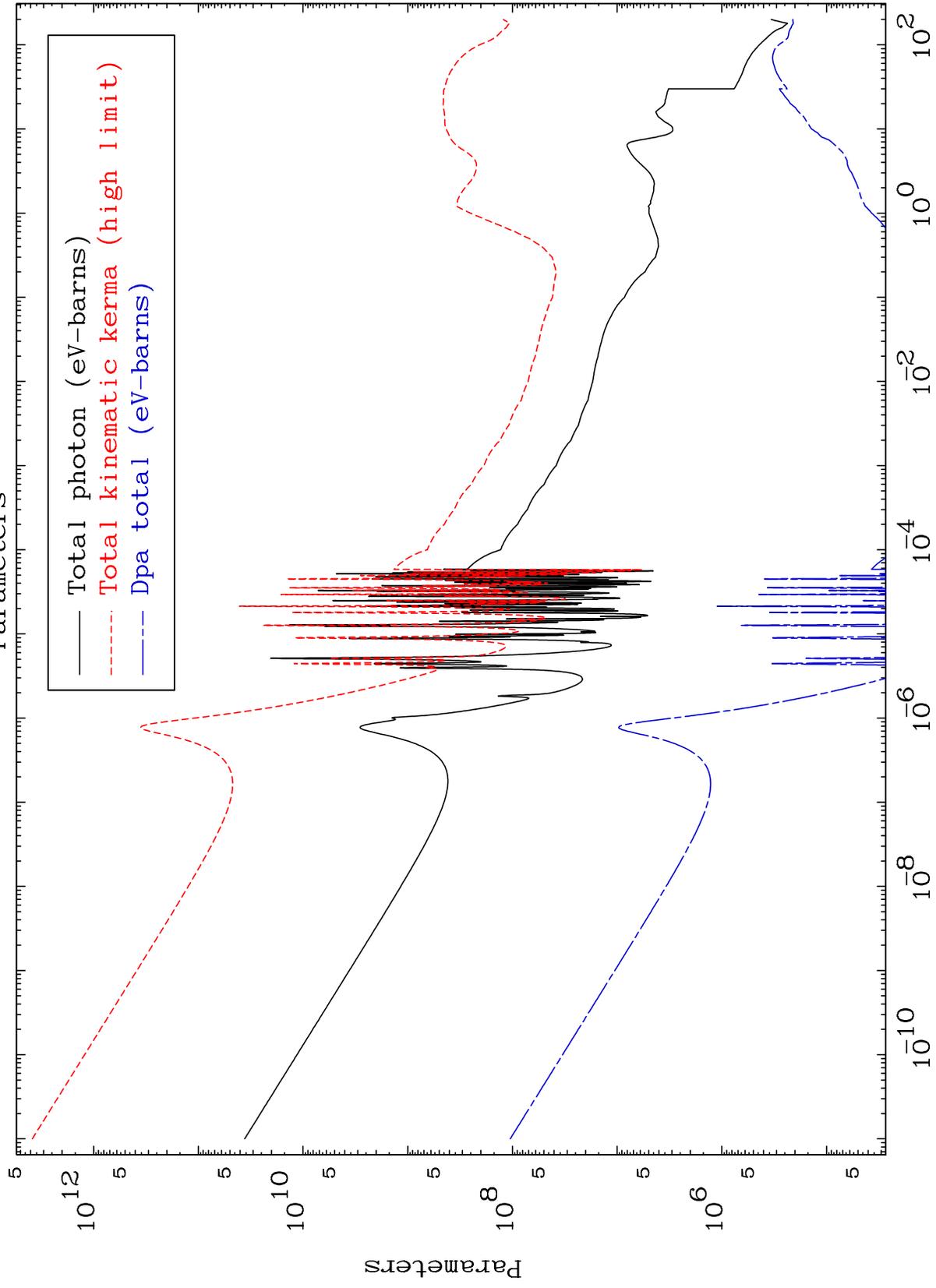
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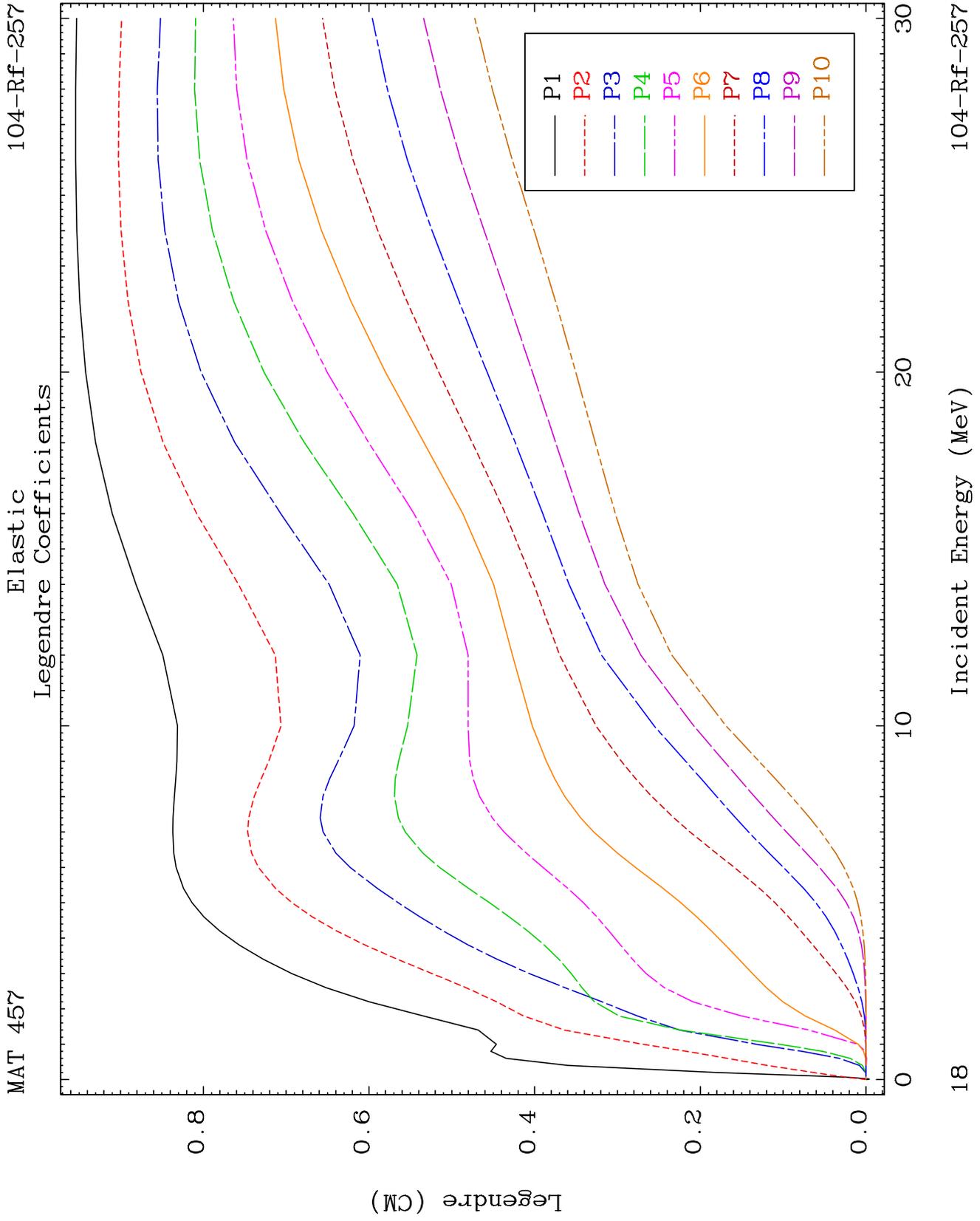
Incident Energy (MeV)

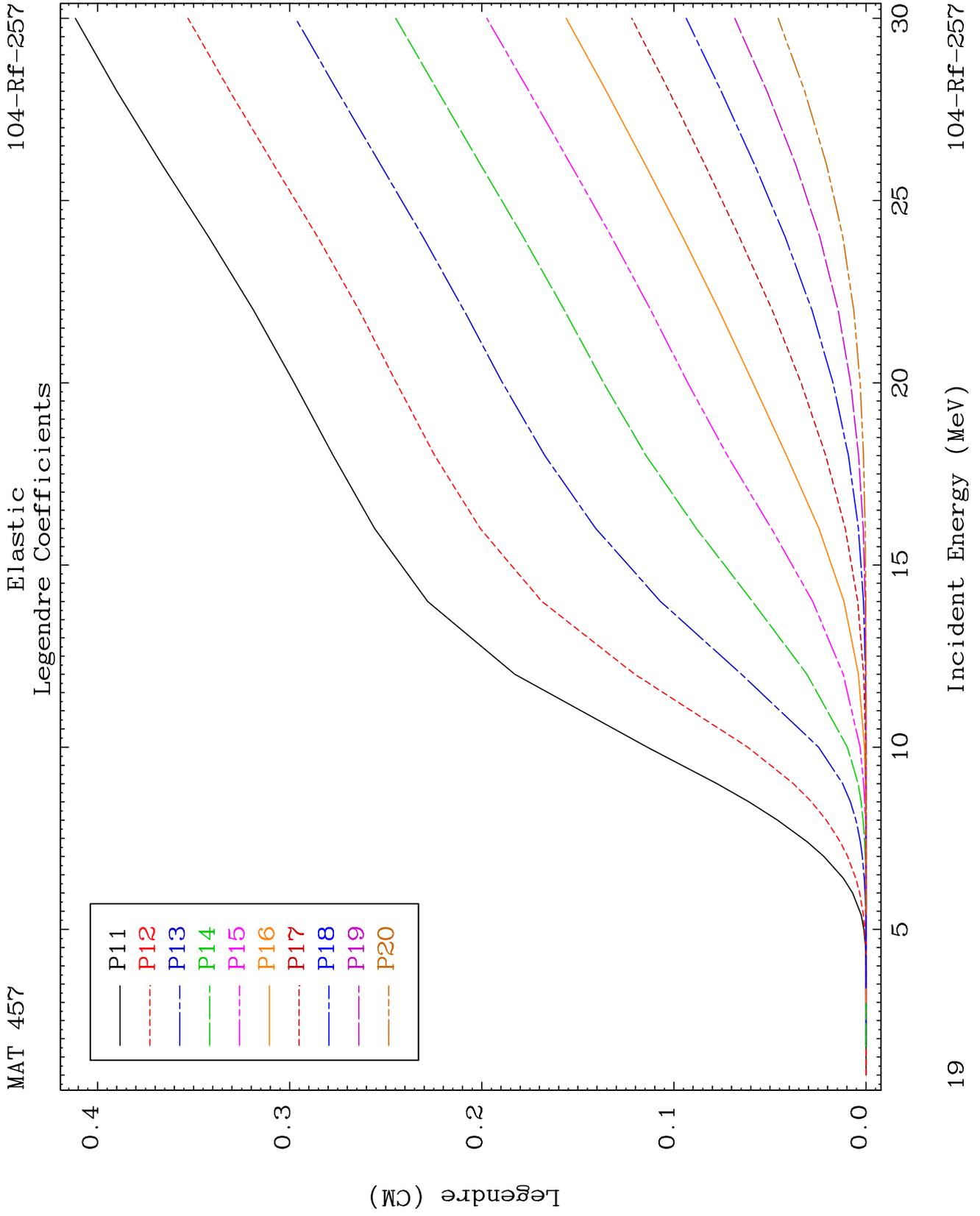
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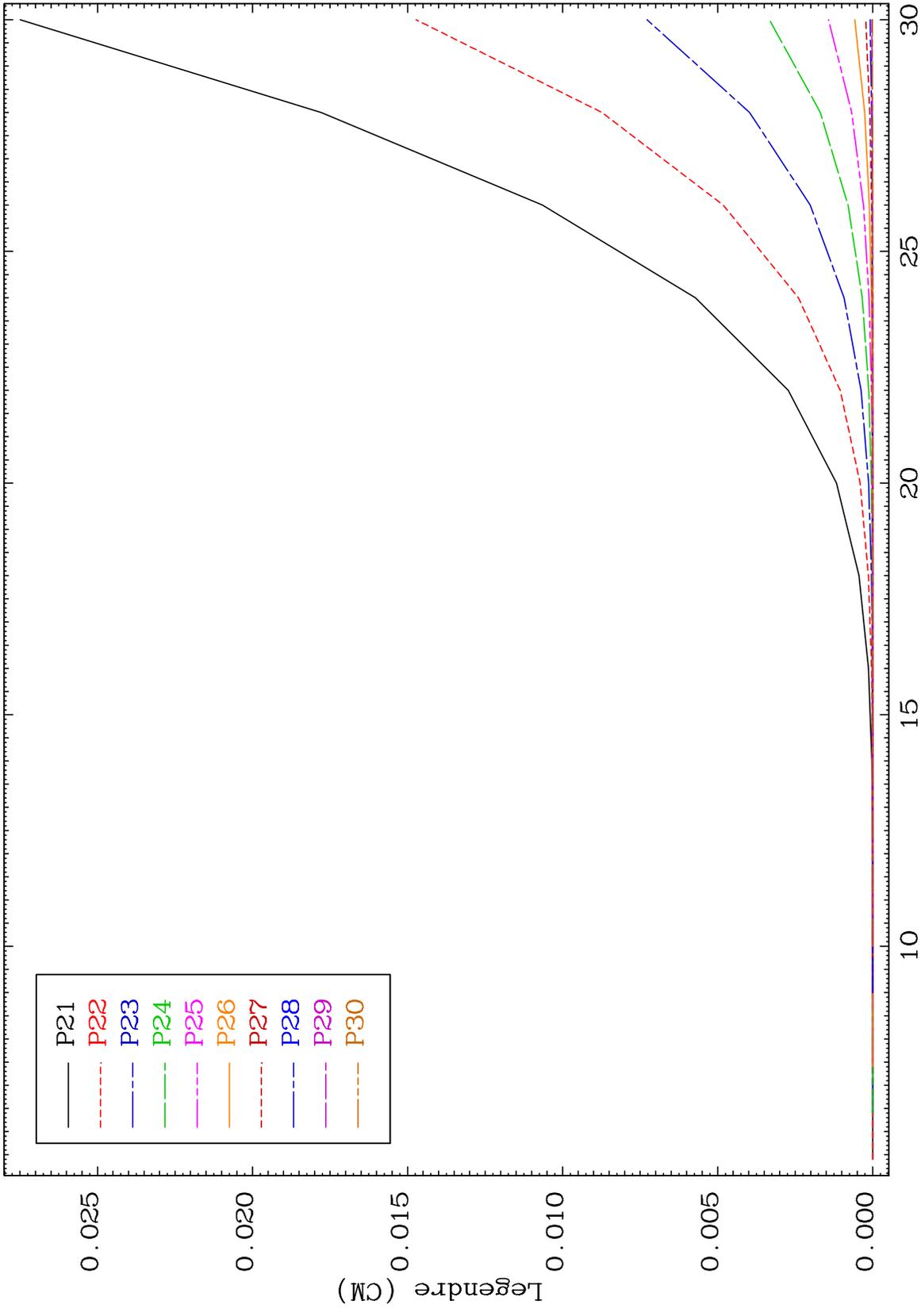




MAT 457

Elastic  
Legendre Coefficients

104-Rf-257



20

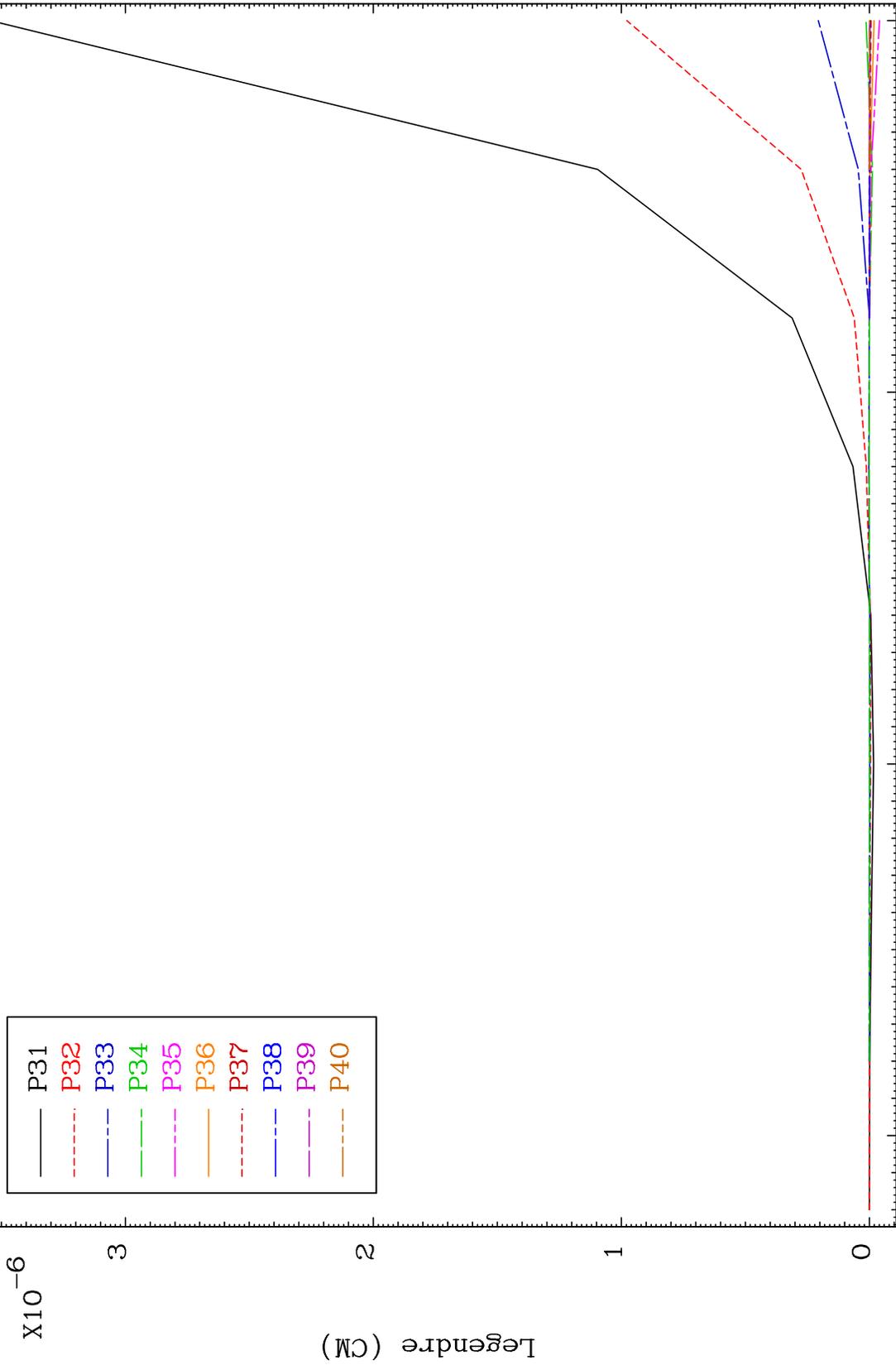
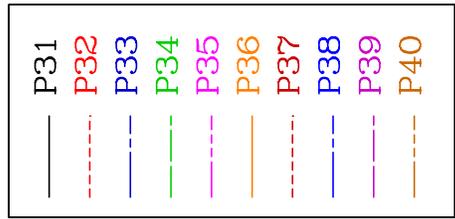
Incident Energy (MeV)

104-Rf-257

MAT 457

Elastic Legendre Coefficients

104-Rf-257



21

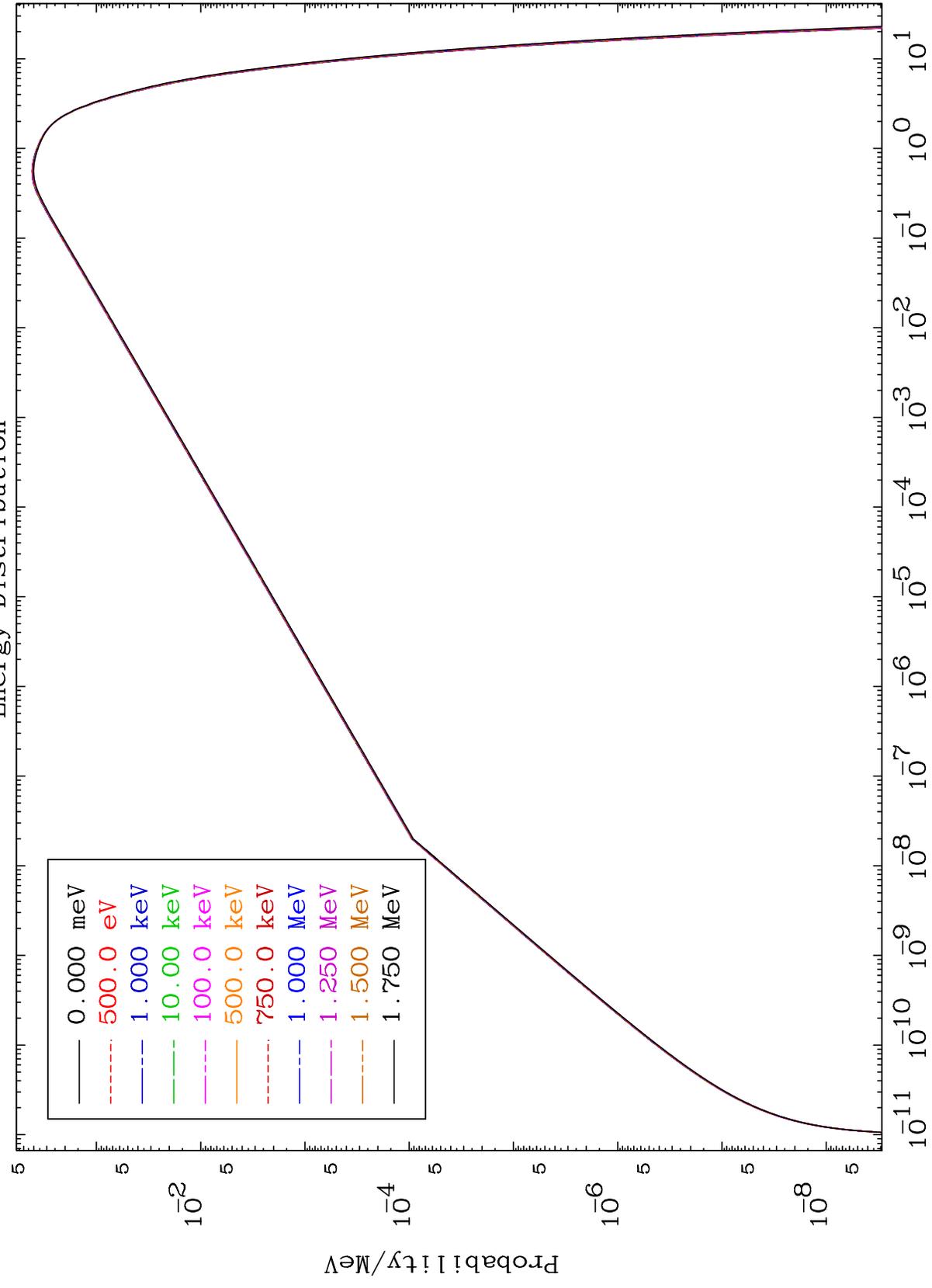
Incident Energy (MeV)

104-Rf-257

MAT 457

### Fission Energy Distribution

104-Rf-257

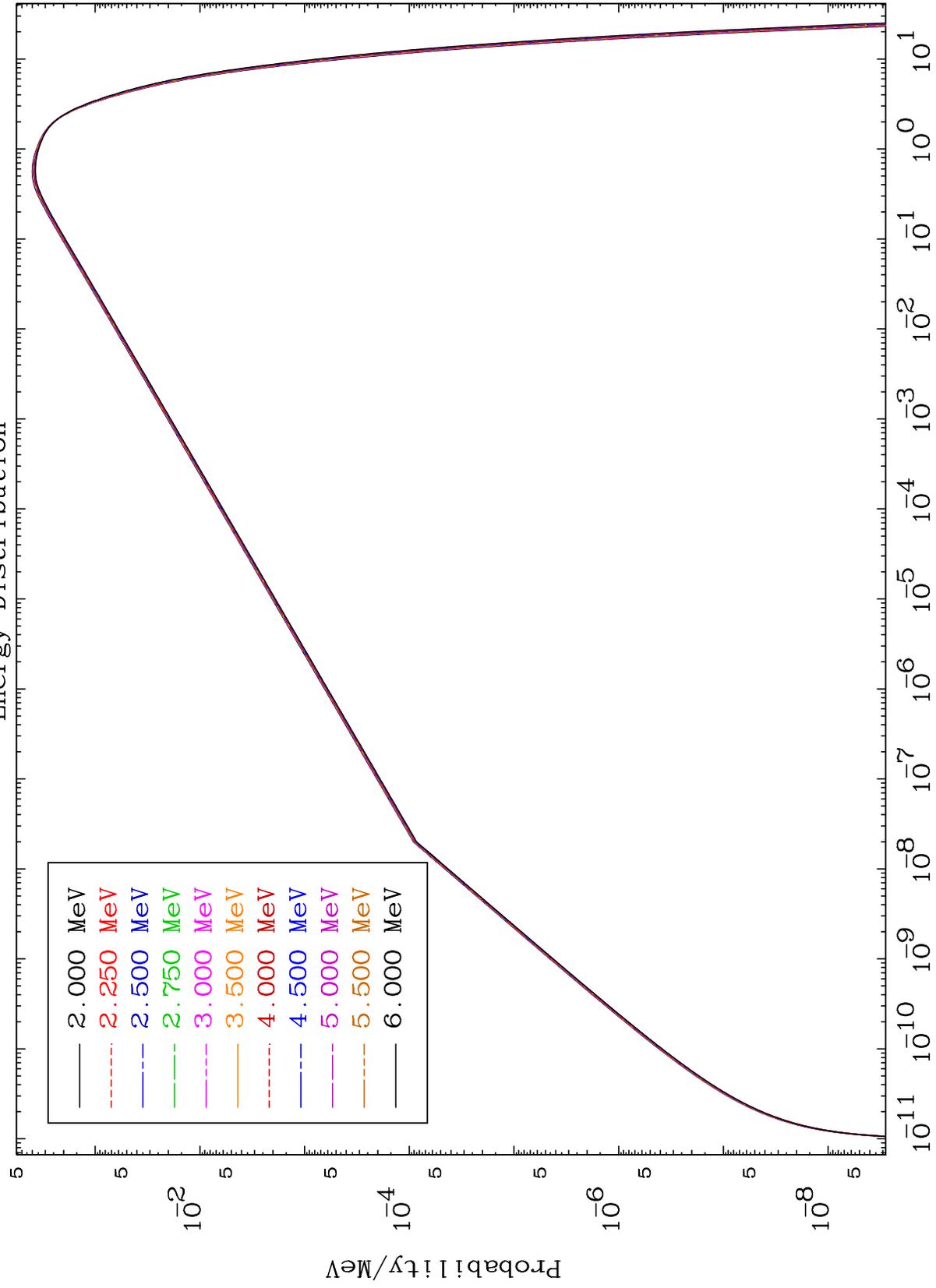


104-Rf-257

MAT 457

# Fission Energy Distribution

104-Rf-257

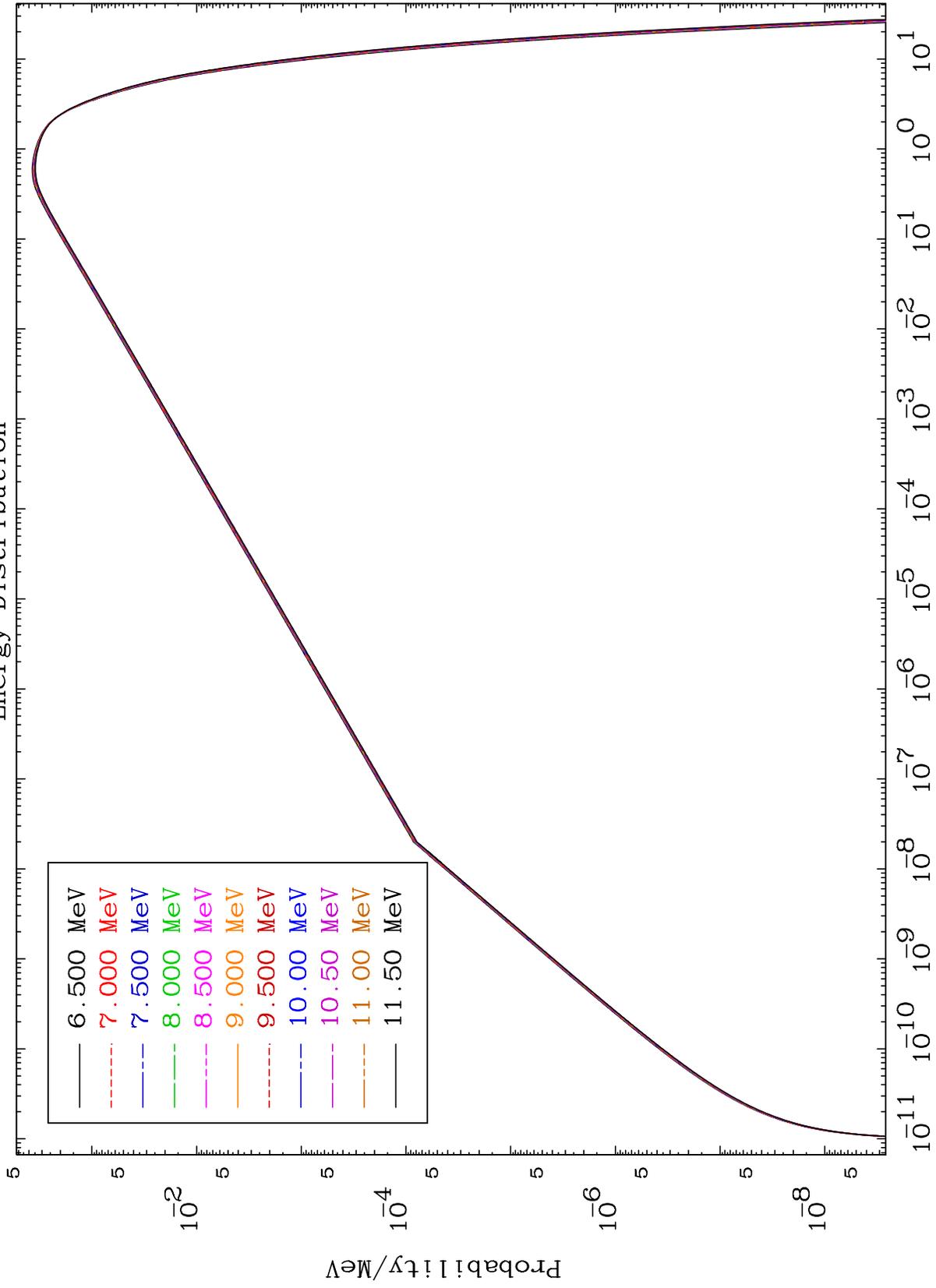


104-Rf-257

MAT 457

### Fission Energy Distribution

104-Rf-257

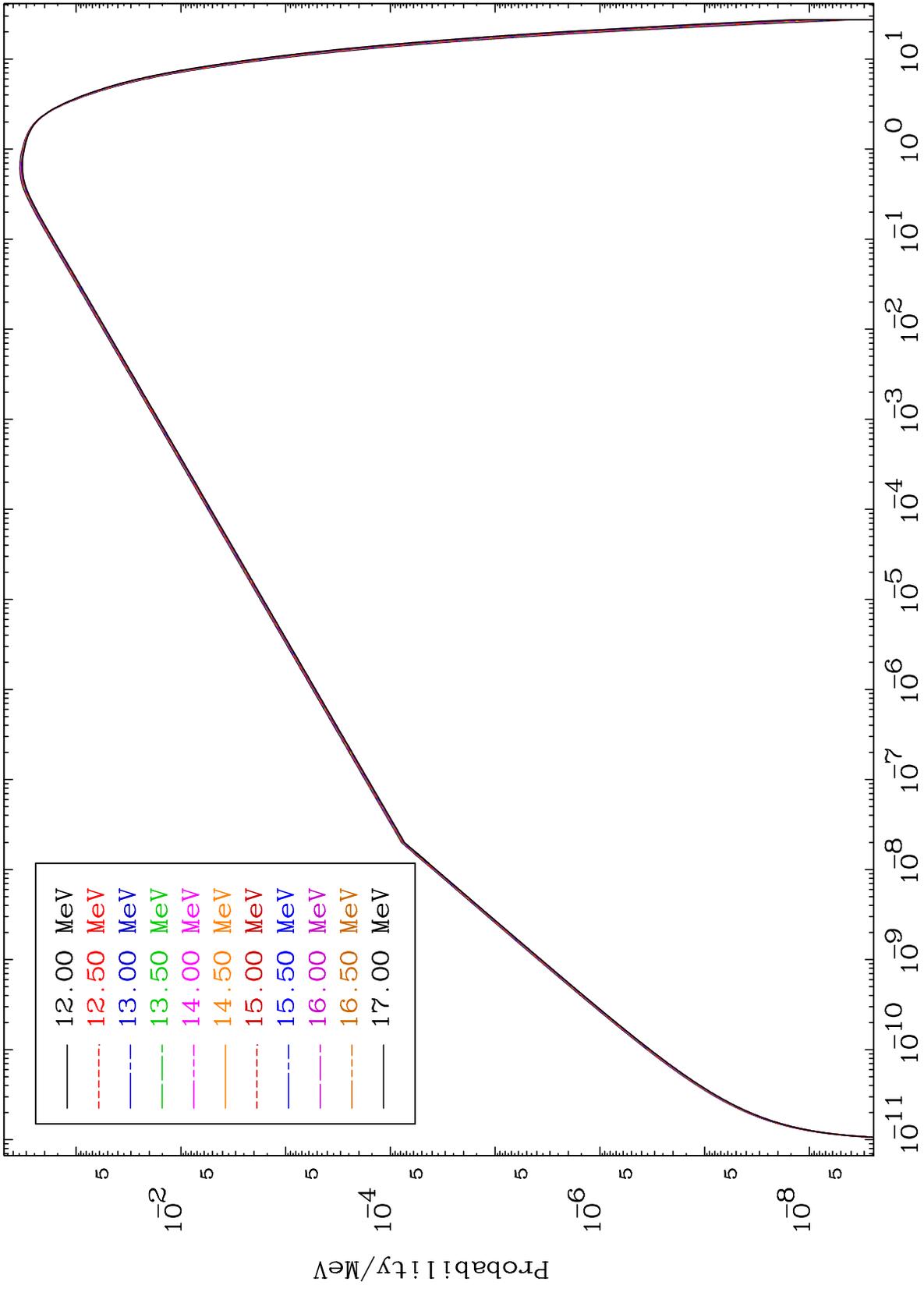


104-Rf-257

MAT 457

# Fission Energy Distribution

104-Rf-257



25

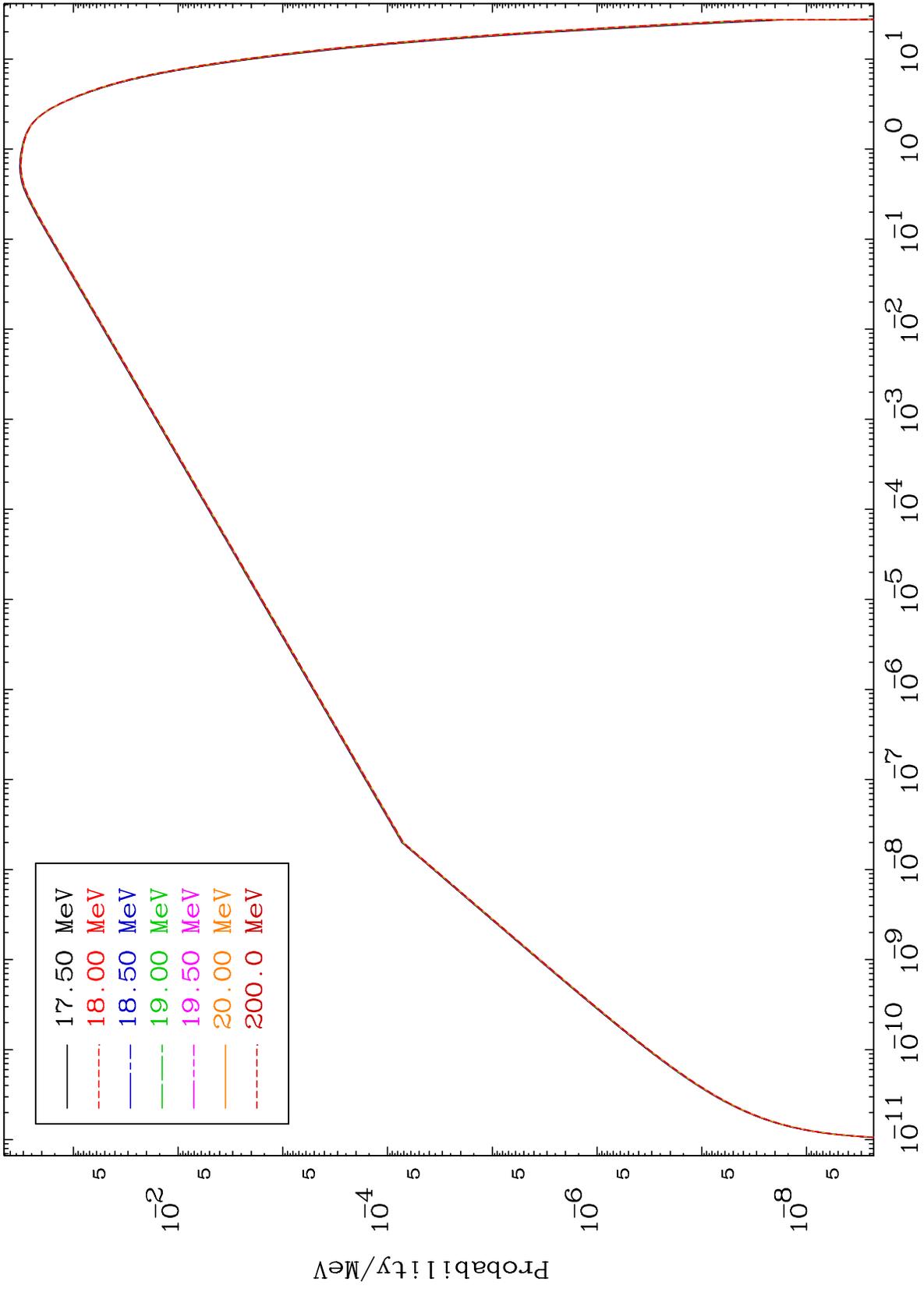
Secondary Energy (MeV)

104-Rf-257

MAT 457

# Fission Energy Distribution

104-Rf-257

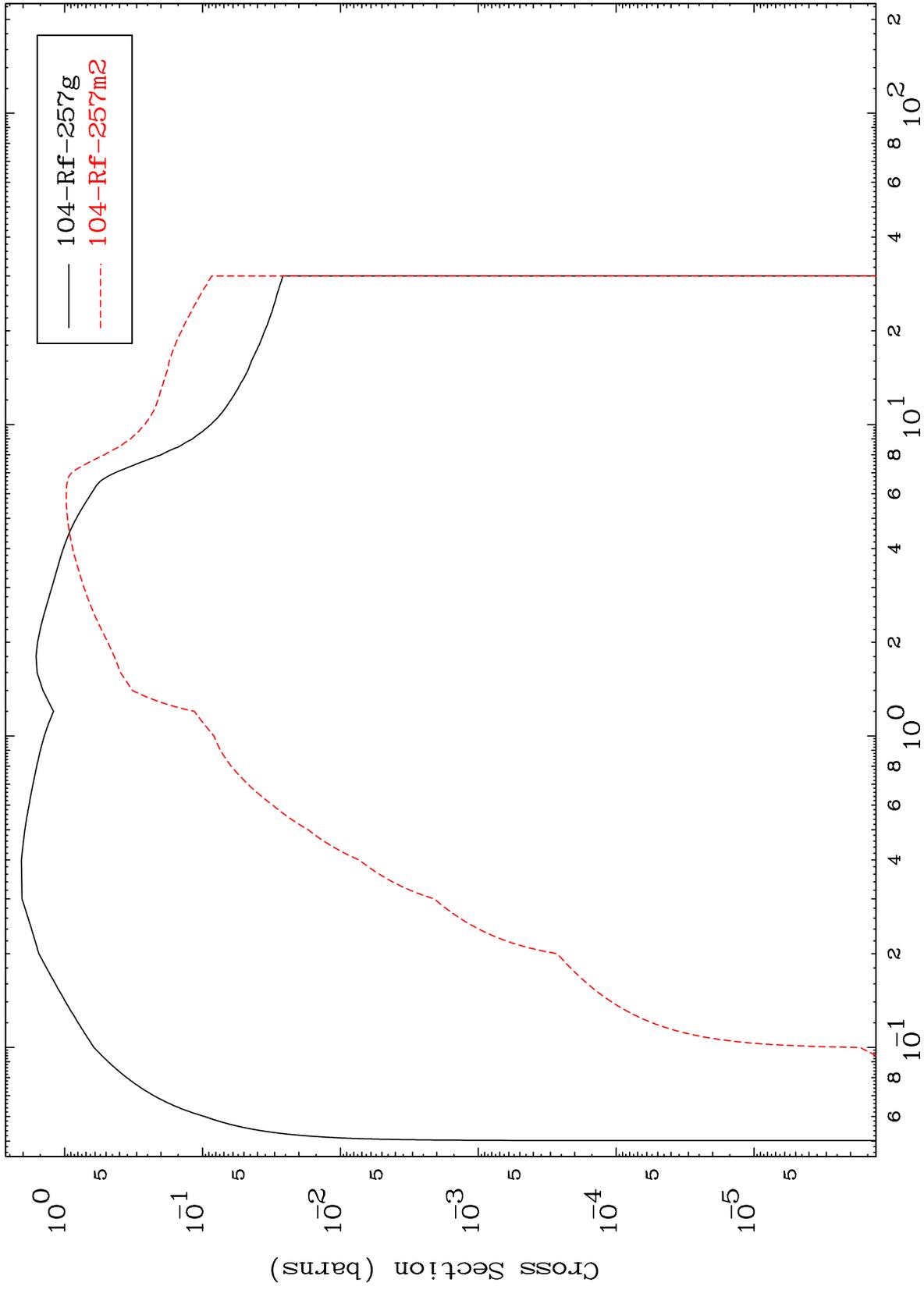


104-Rf-257

MAT 457

104-Rf-257

### Inelastic Radionuclide Production Cross Section



27

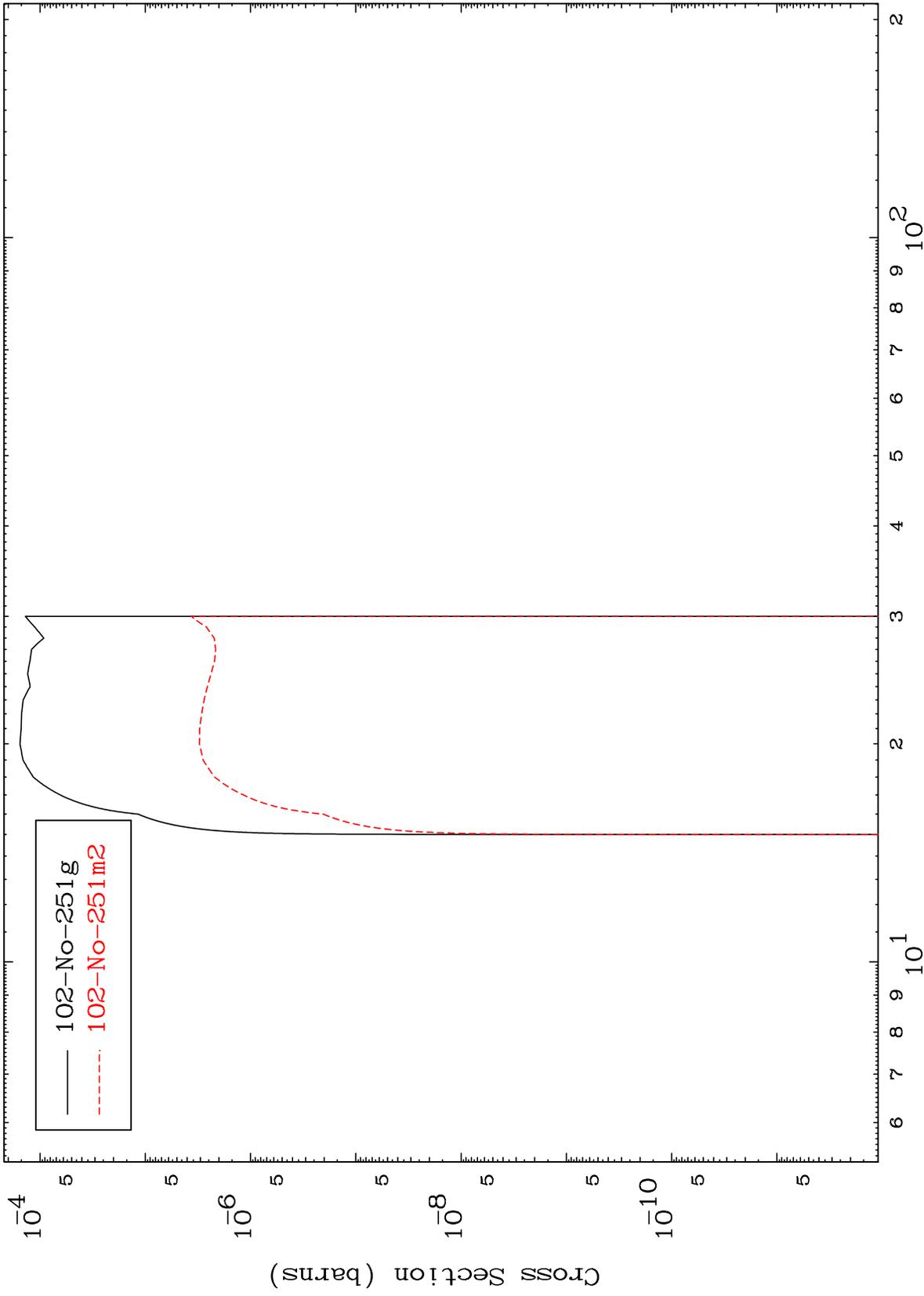
104-Rf-257

MAT 457

(n,3n)  $\alpha$

104-Rf-257

Radionuclide Production Cross Section



28

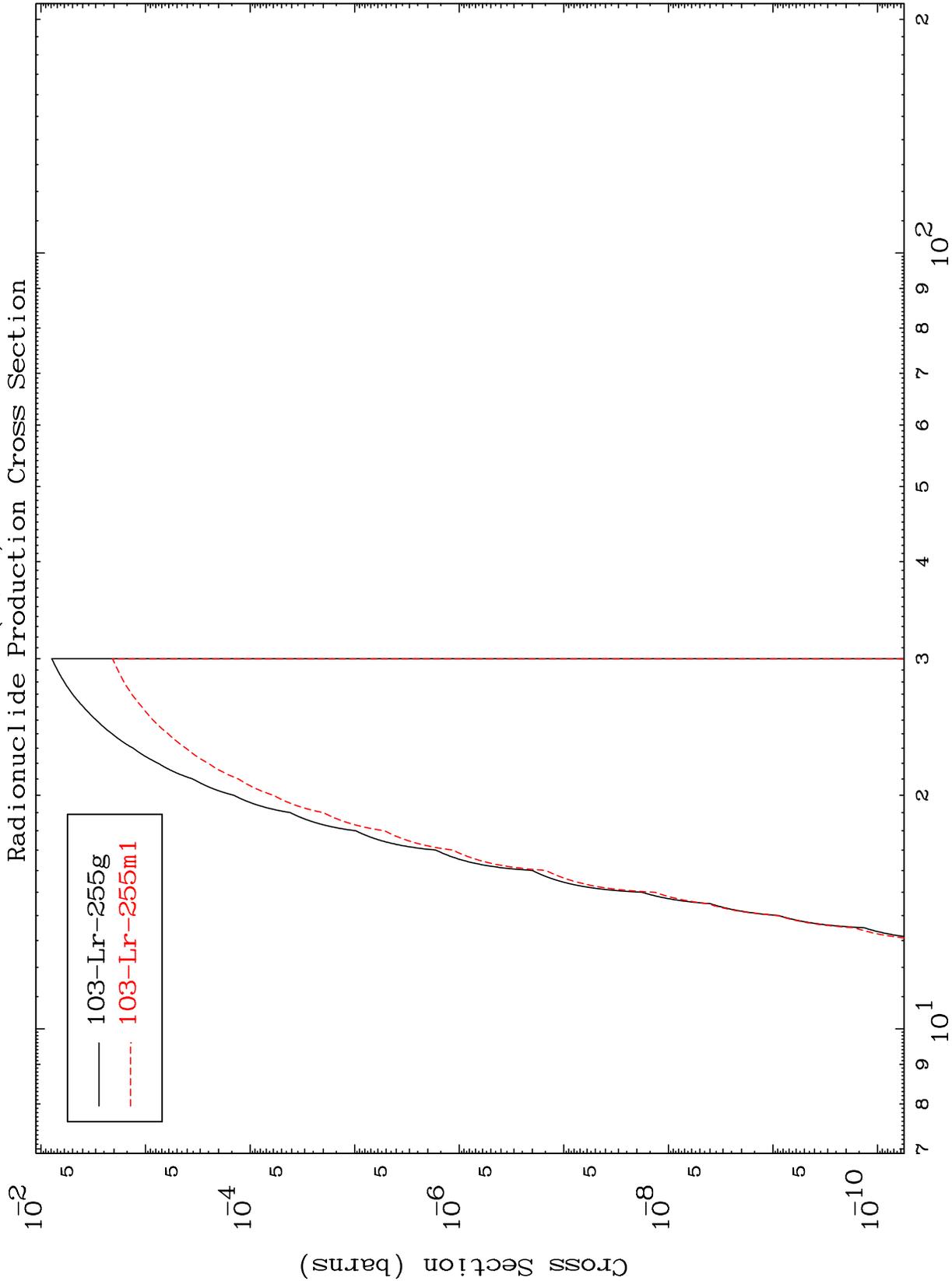
Incident Energy (MeV)

104-Rf-257

MAT 457

(n,n') d

104-Rf-257



29

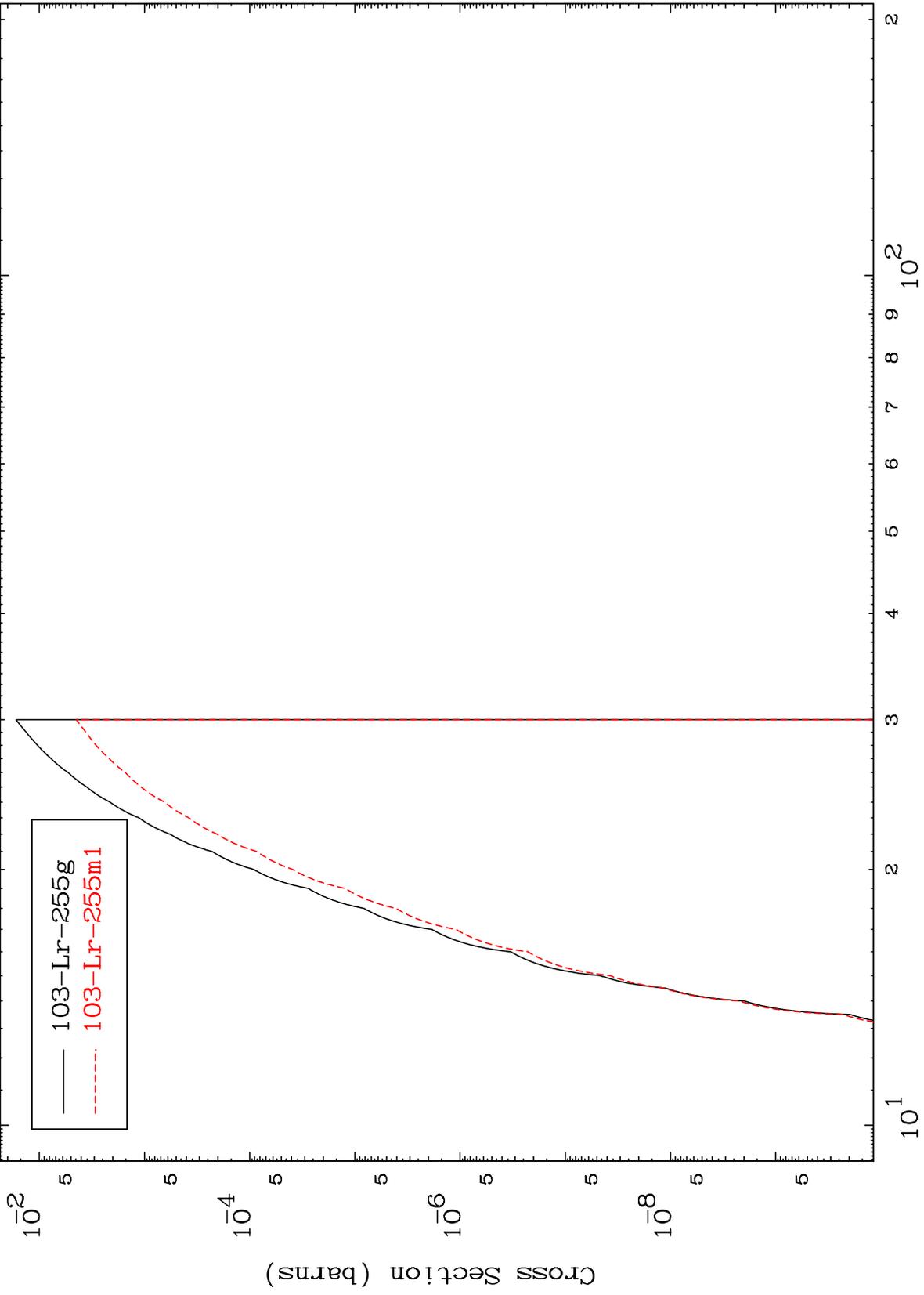
104-Rf-257

MAT 457

(n,2n) p

104-Rf-257

Radionuclide Production Cross Section



Incident Energy (MeV)

104-Rf-257

30

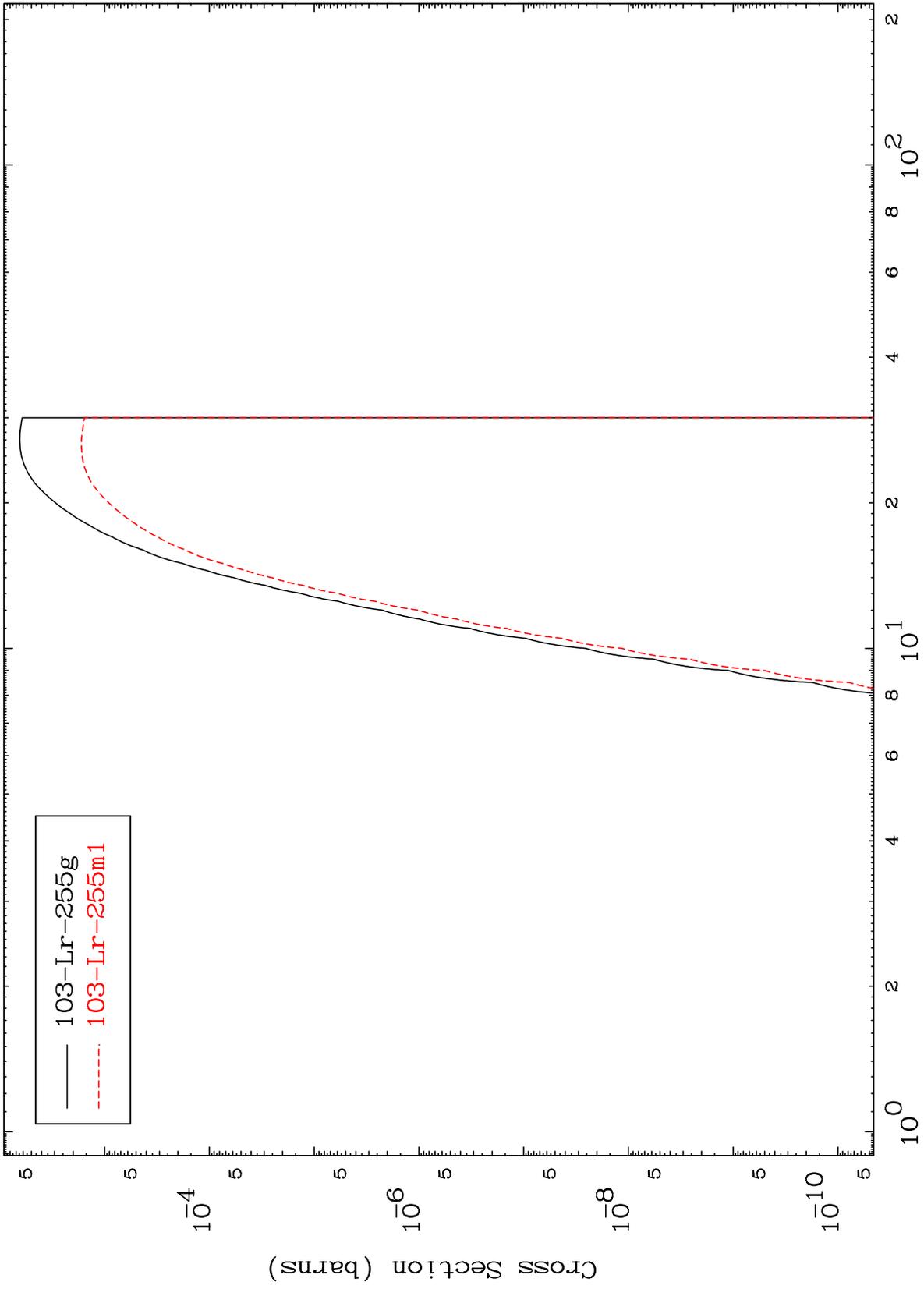
10<sup>2</sup>

10<sup>1</sup>

MAT 457

(n, t)  
Radionuclide Production Cross Section

104-Rf-257



31

Incident Energy (MeV)

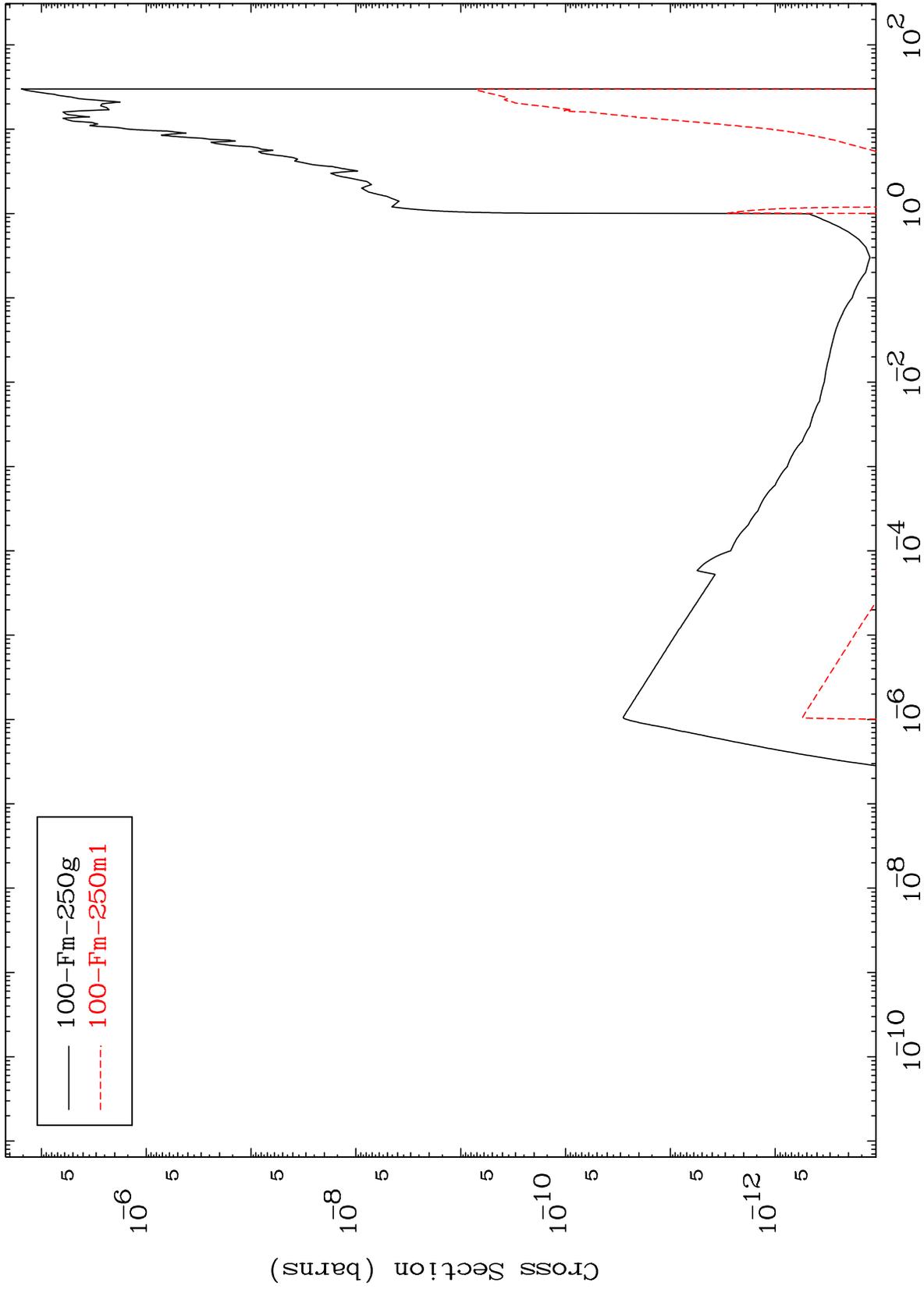
104-Rf-257

MAT 457

(n,2α)

104-Rf-257

Radionuclide Production Cross Section



32

Incident Energy (MeV)

104-Rf-257