

Program EVALPLOT
(Version 2018-1)

by

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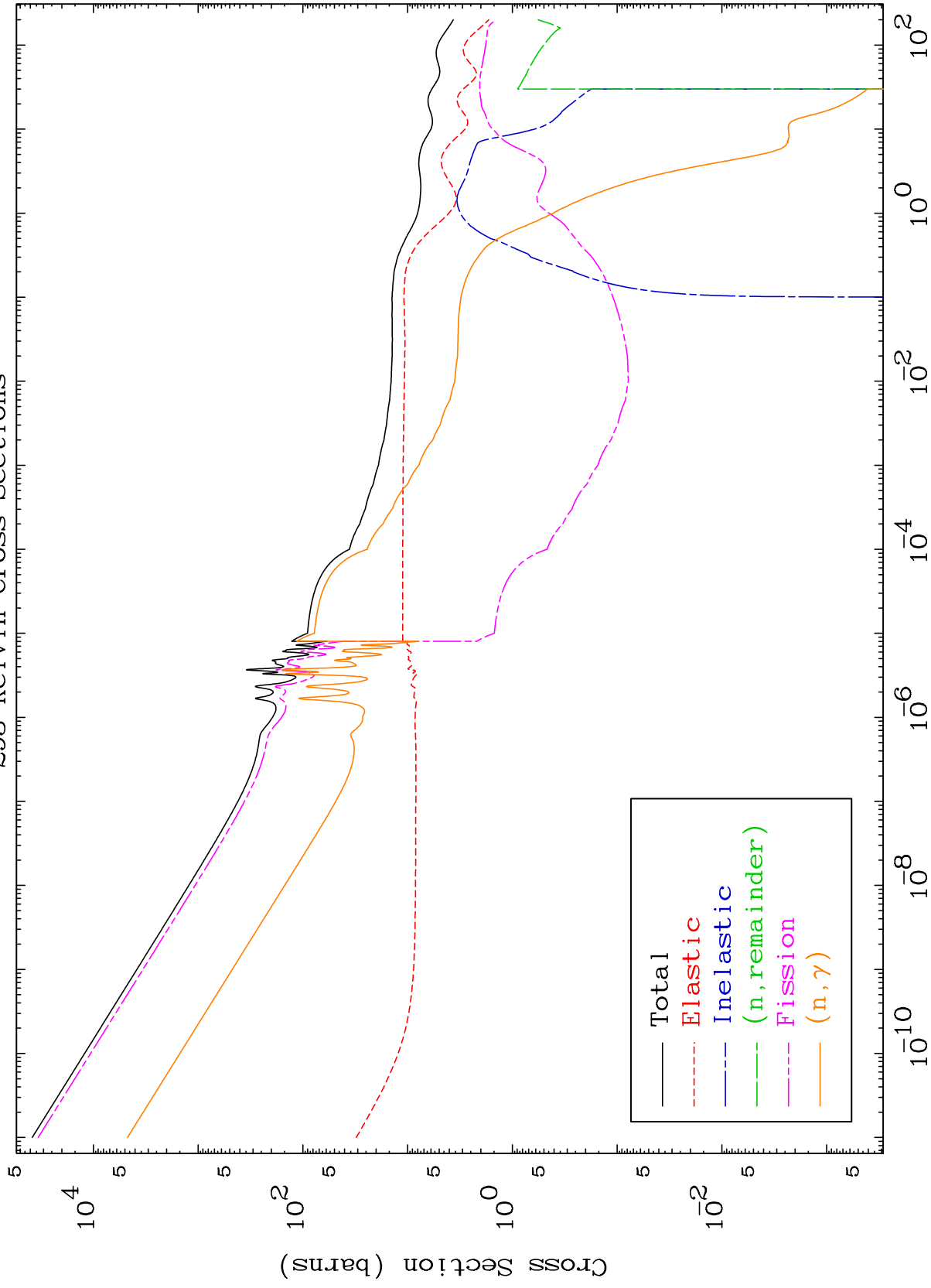
Press Mouse Button to Start

MAT 9983

Major

293 Kelvin Cross Sections

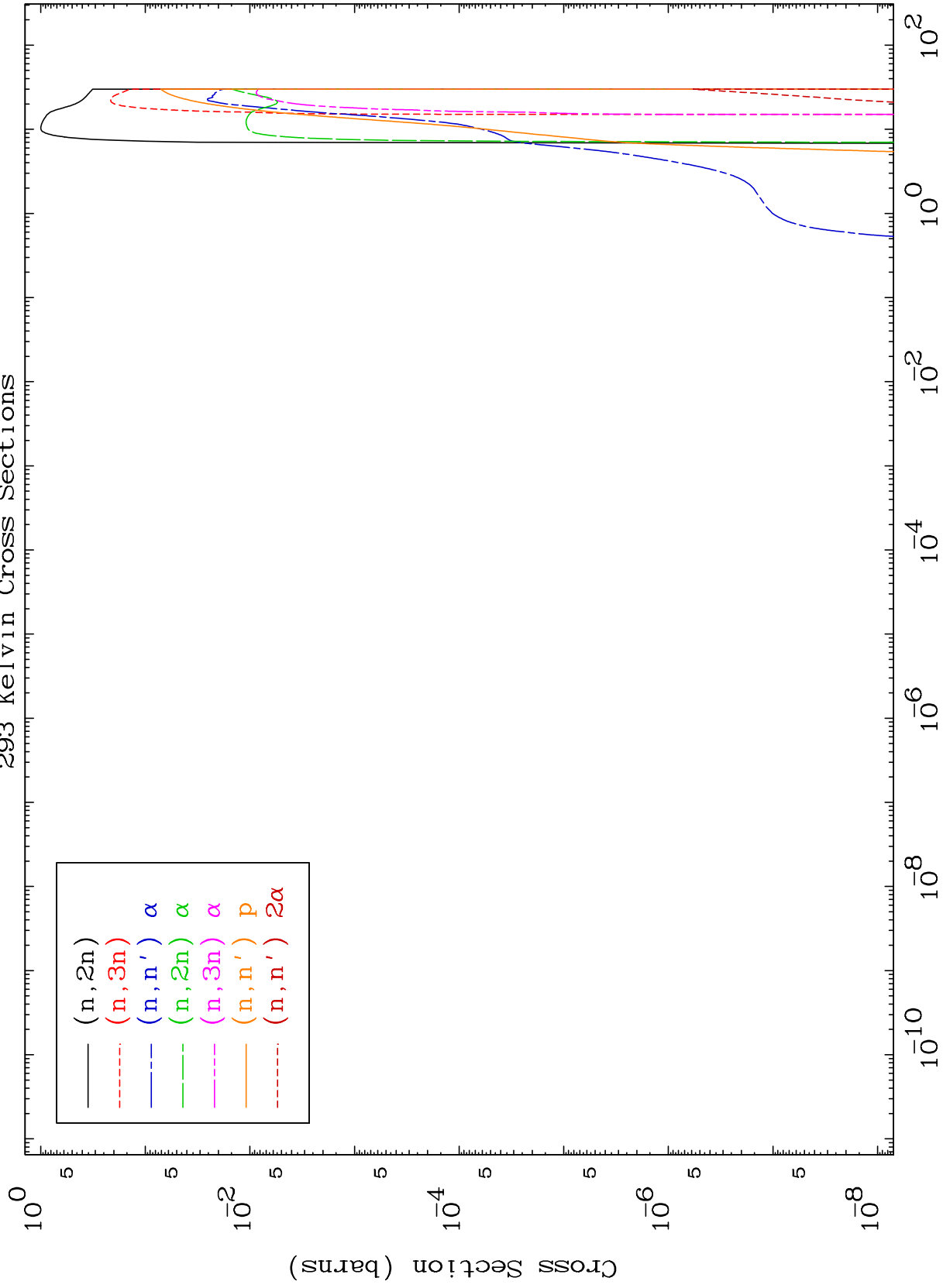
103-Lr-254



MAT 9983

Neutron Production
293 Kelvin Cross Sections

103-Lr-254



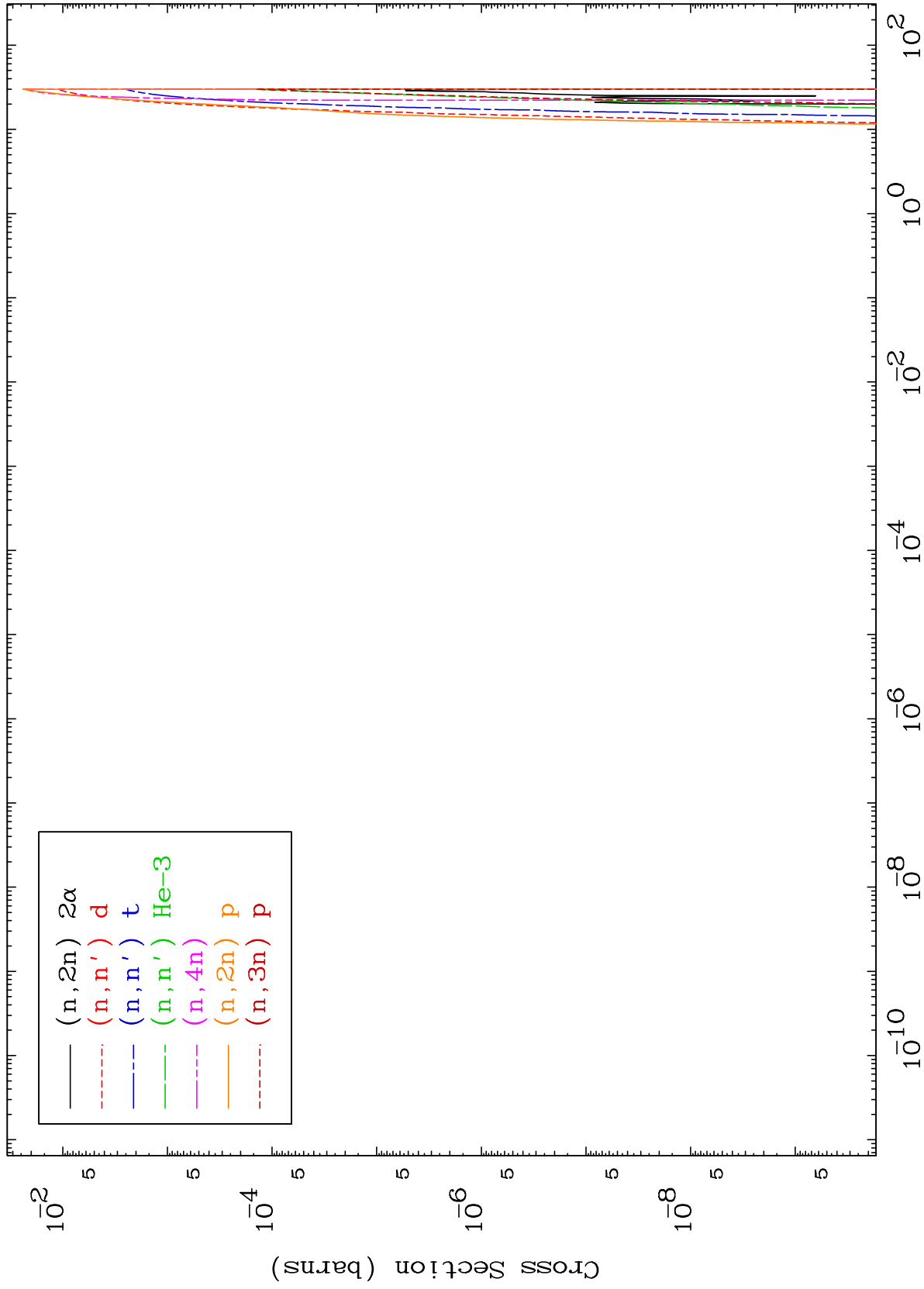
2

103-Lr-254

MAT 9983

Neutron Production
293 Kelvin Cross Sections

103-Lr-254



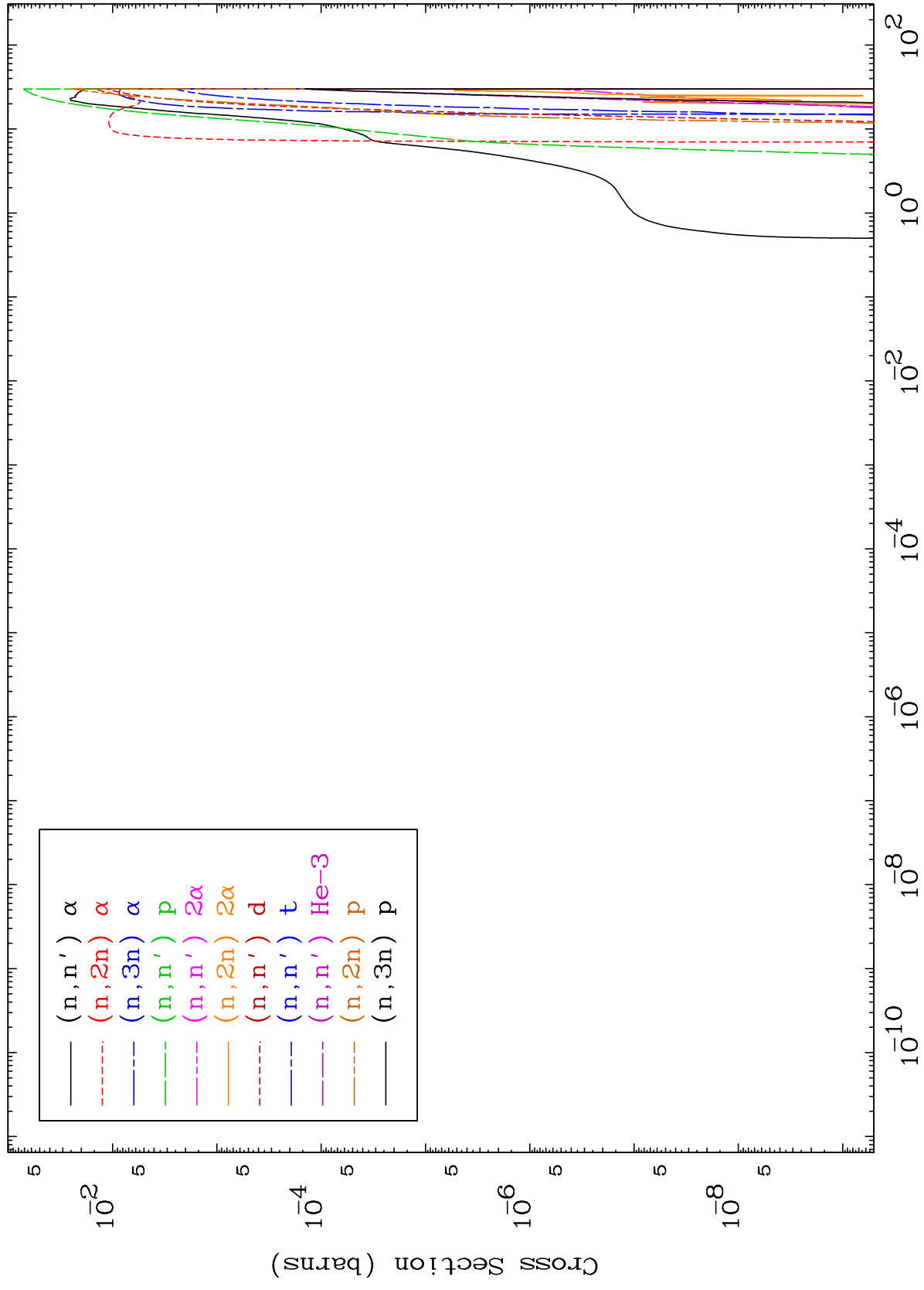
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103-Lr-254

MAT 9983

Charged Particle
293 Kelvin Cross Sections

103-Lr-254



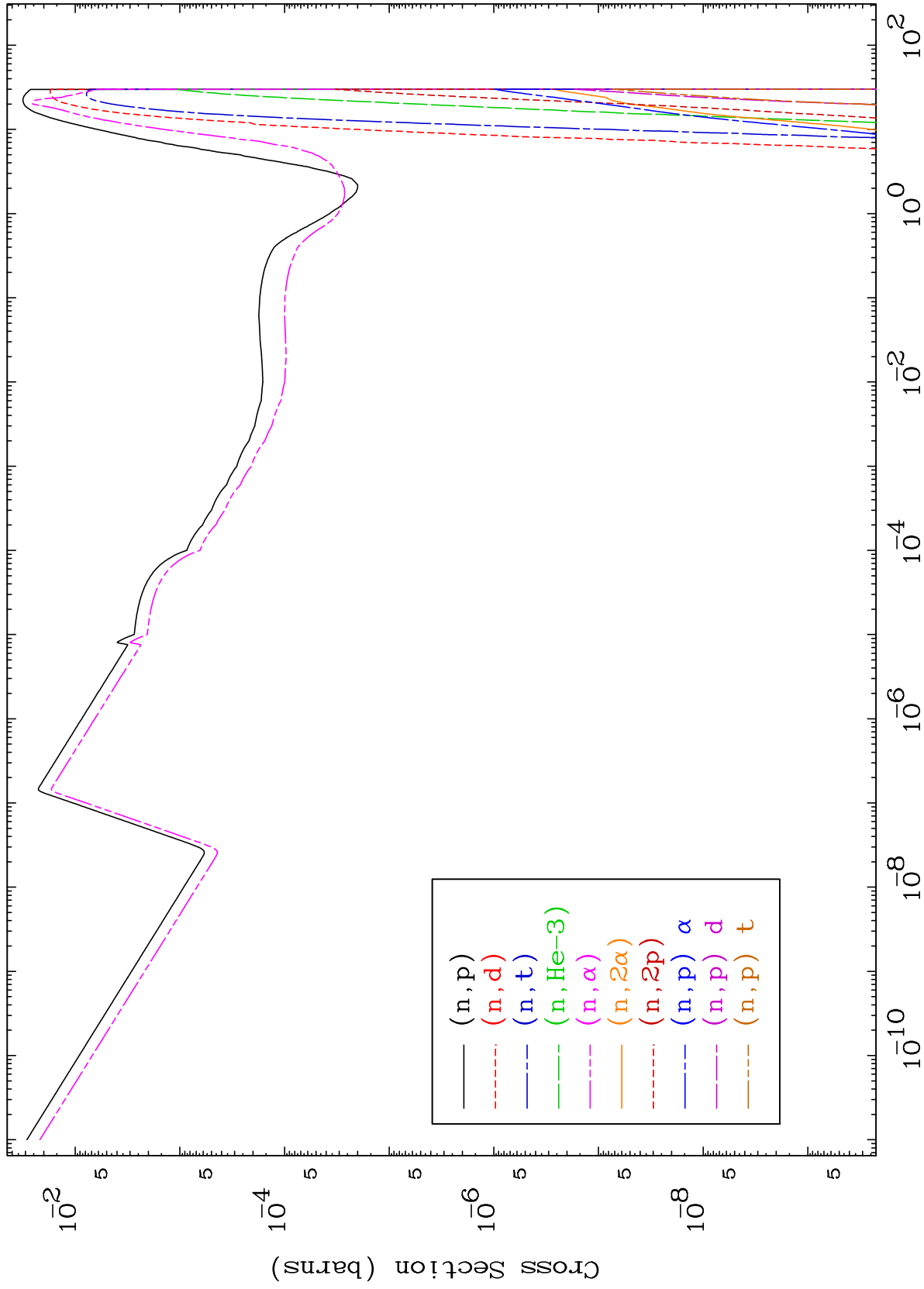
103-Lr-254

Incident Energy (MeV)

MAT 9983

293 Charged Particle
Kelvin Cross Sections

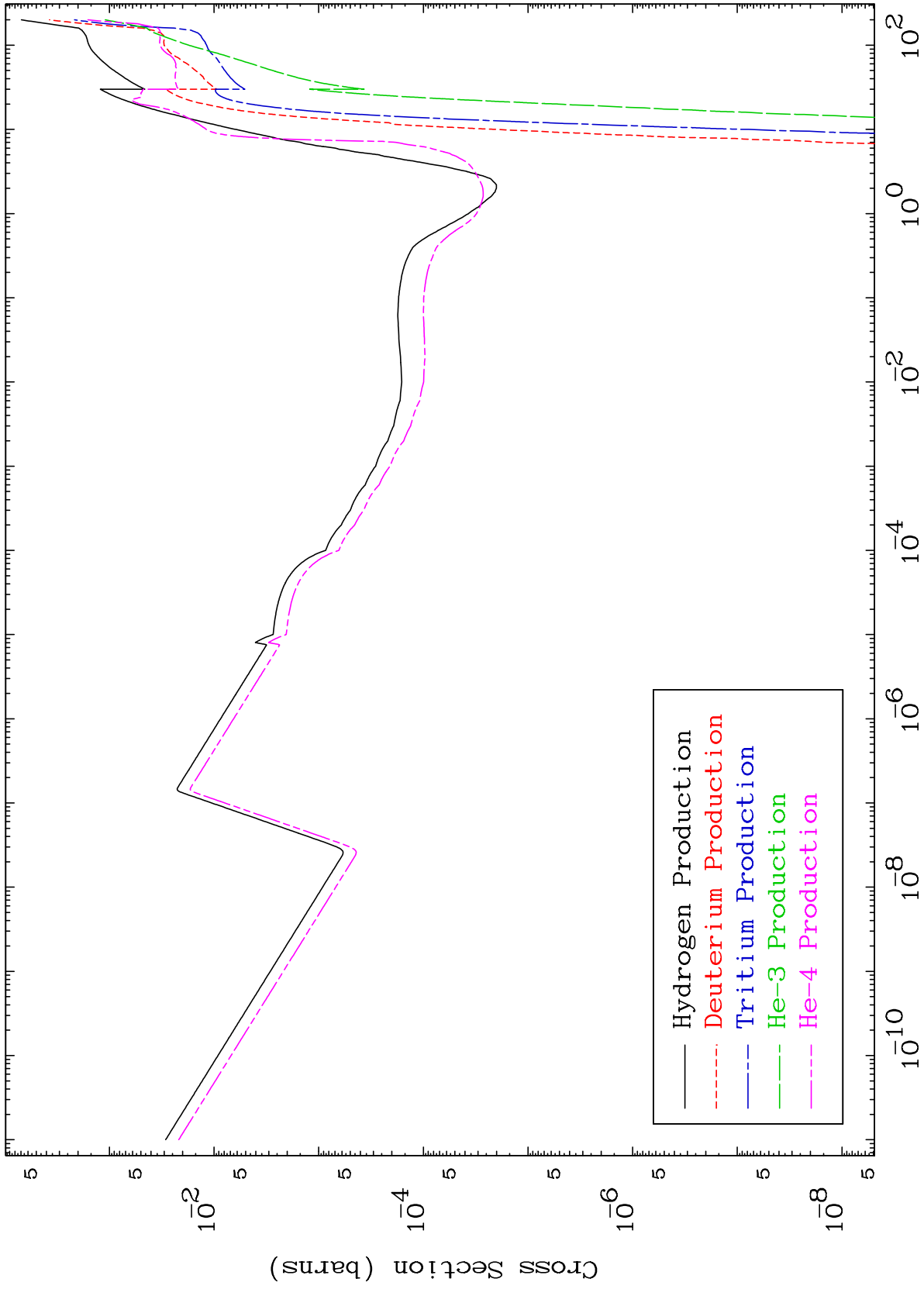
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MAT 9983

Particle Production
293 Kelvin Cross Sections

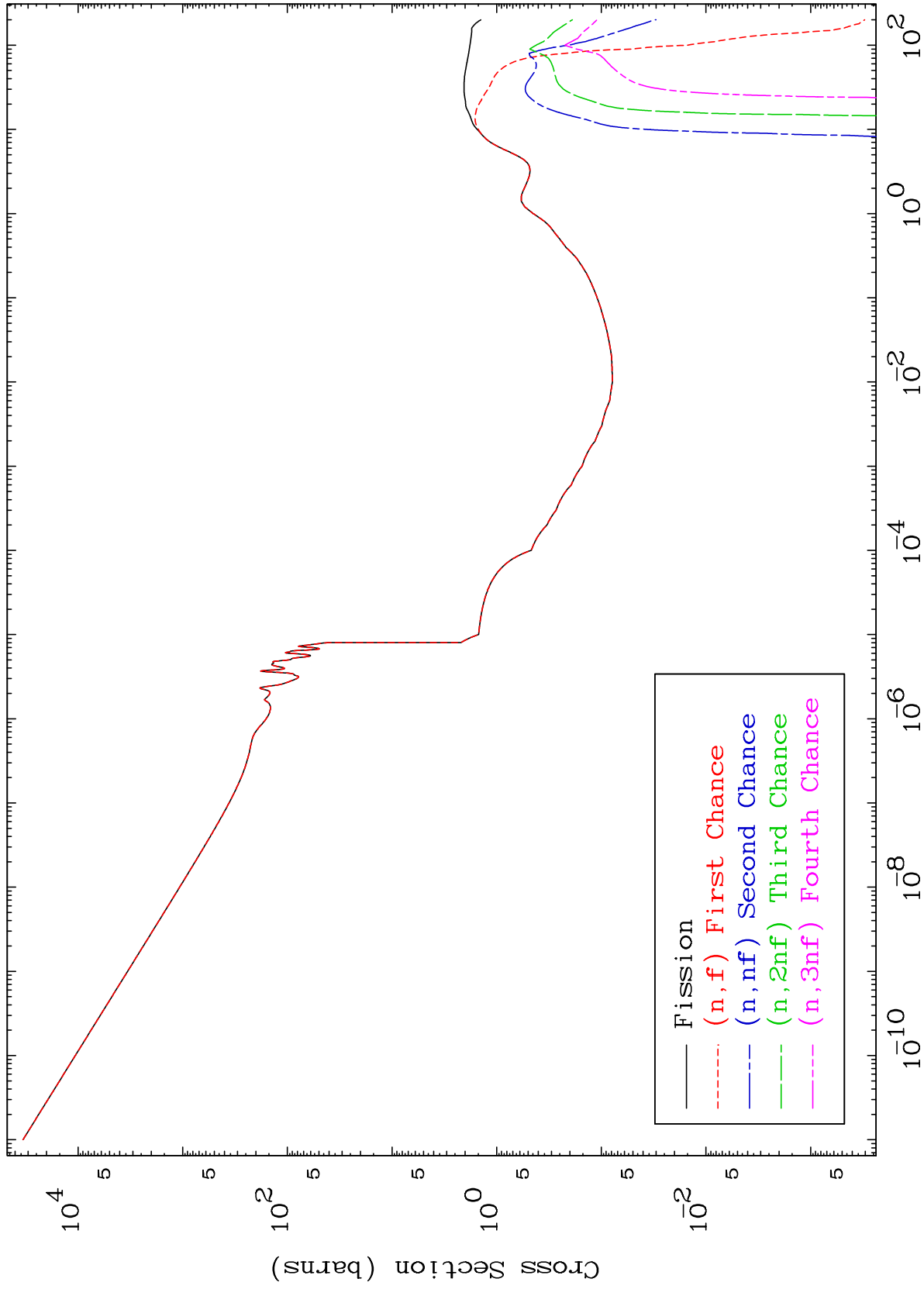
103-Lr-254



MAT 9983

Fission
293 Kelvin Cross Sections

103-Lr-254



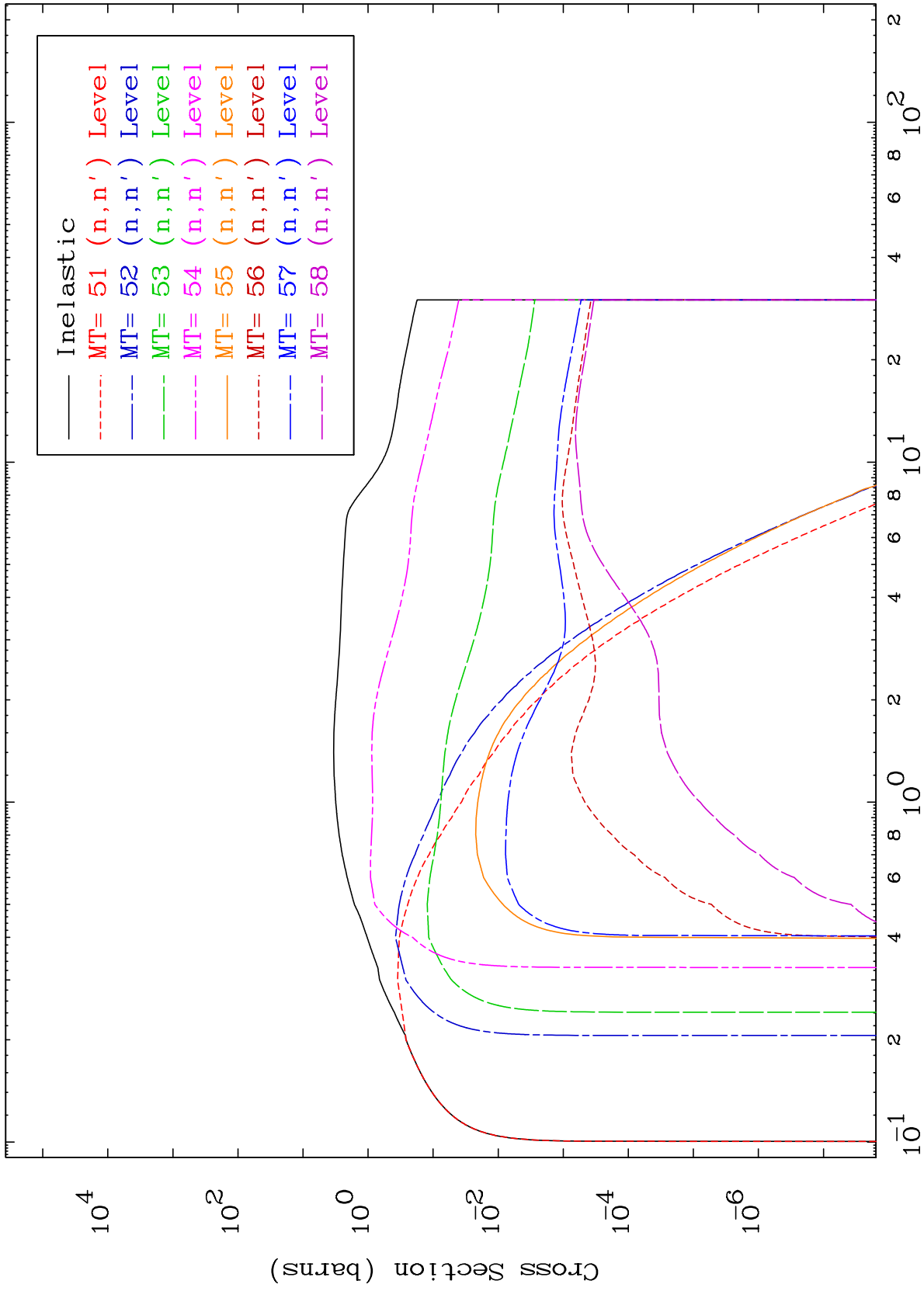
103-Lr-254

MAT 9983

(n,n') Level

103-Lr-254

293 Kelvin Cross Sections



8

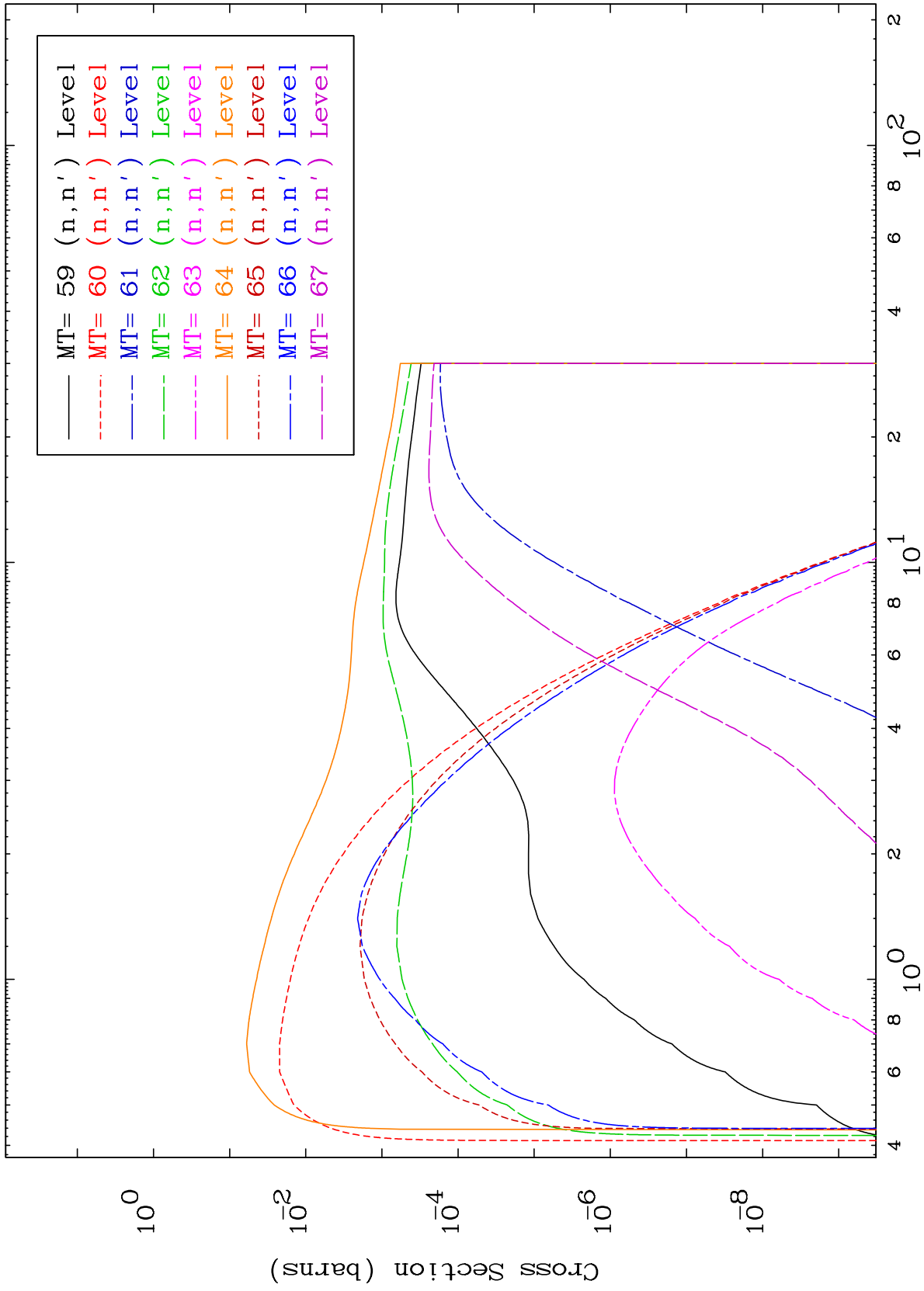
Incident Energy (MeV)

103-Lr-254

MAT 9983

(n,n') Level
293 Kelvin Cross Sections

103-Lr-254



9

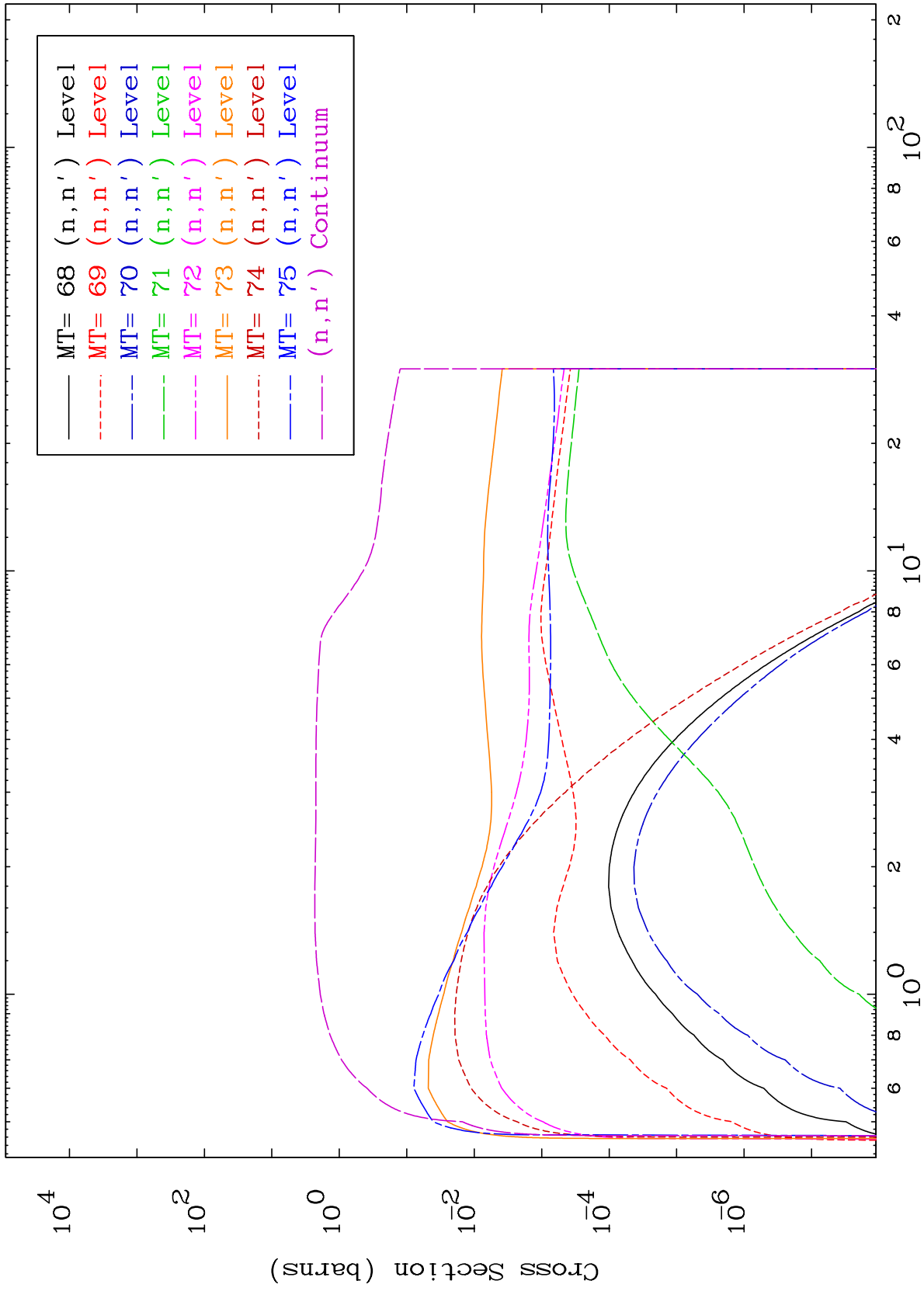
Incident Energy (MeV)

103-Lr-254

MAT 9983

(n,n') Level
293 Kelvin Cross Sections

103-Lr-254



10

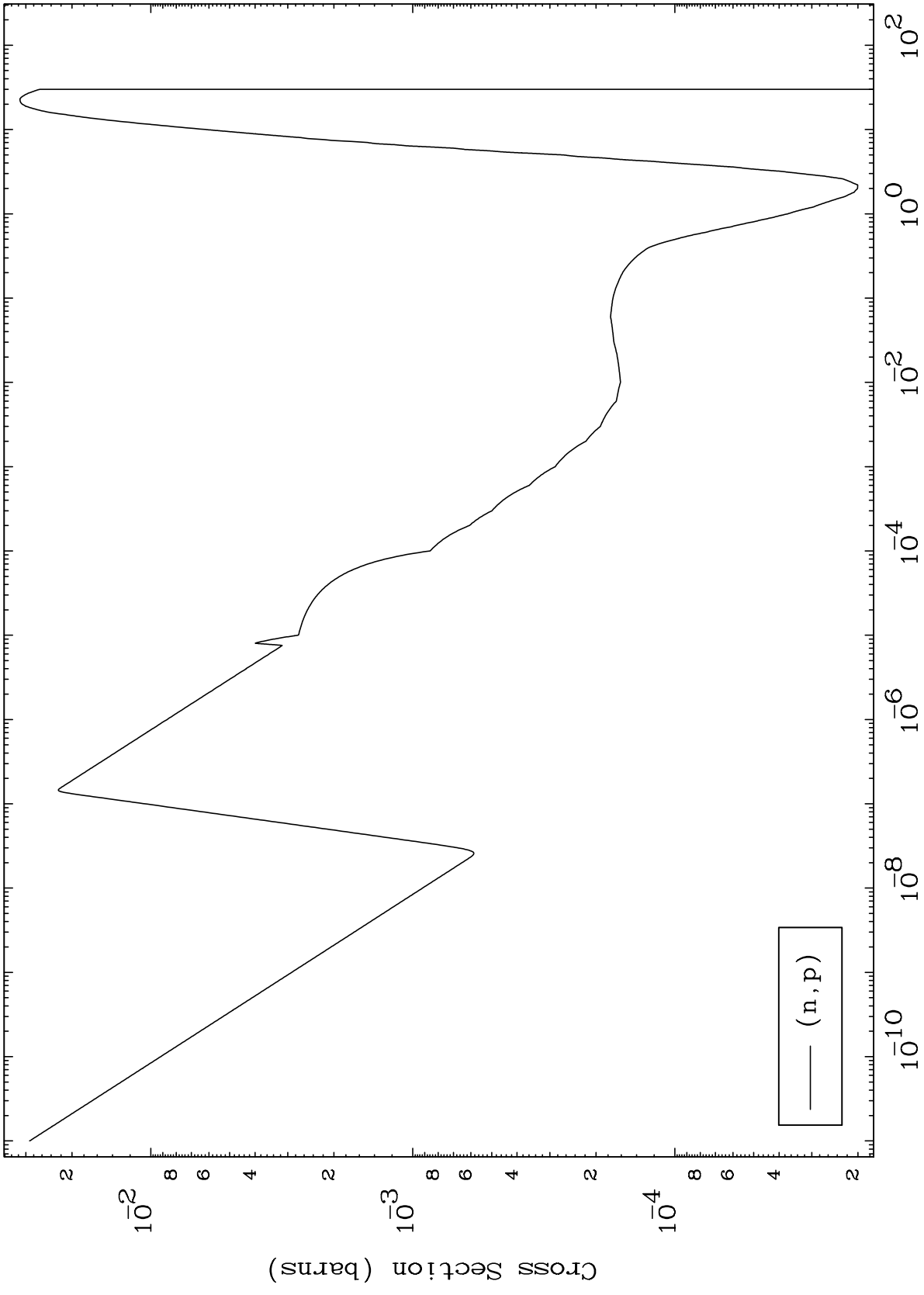
Incident Energy (MeV)

103-Lr-254

MAT 9983

(n,p) Levels
293 Kelvin Cross Sections

103-Lr-254

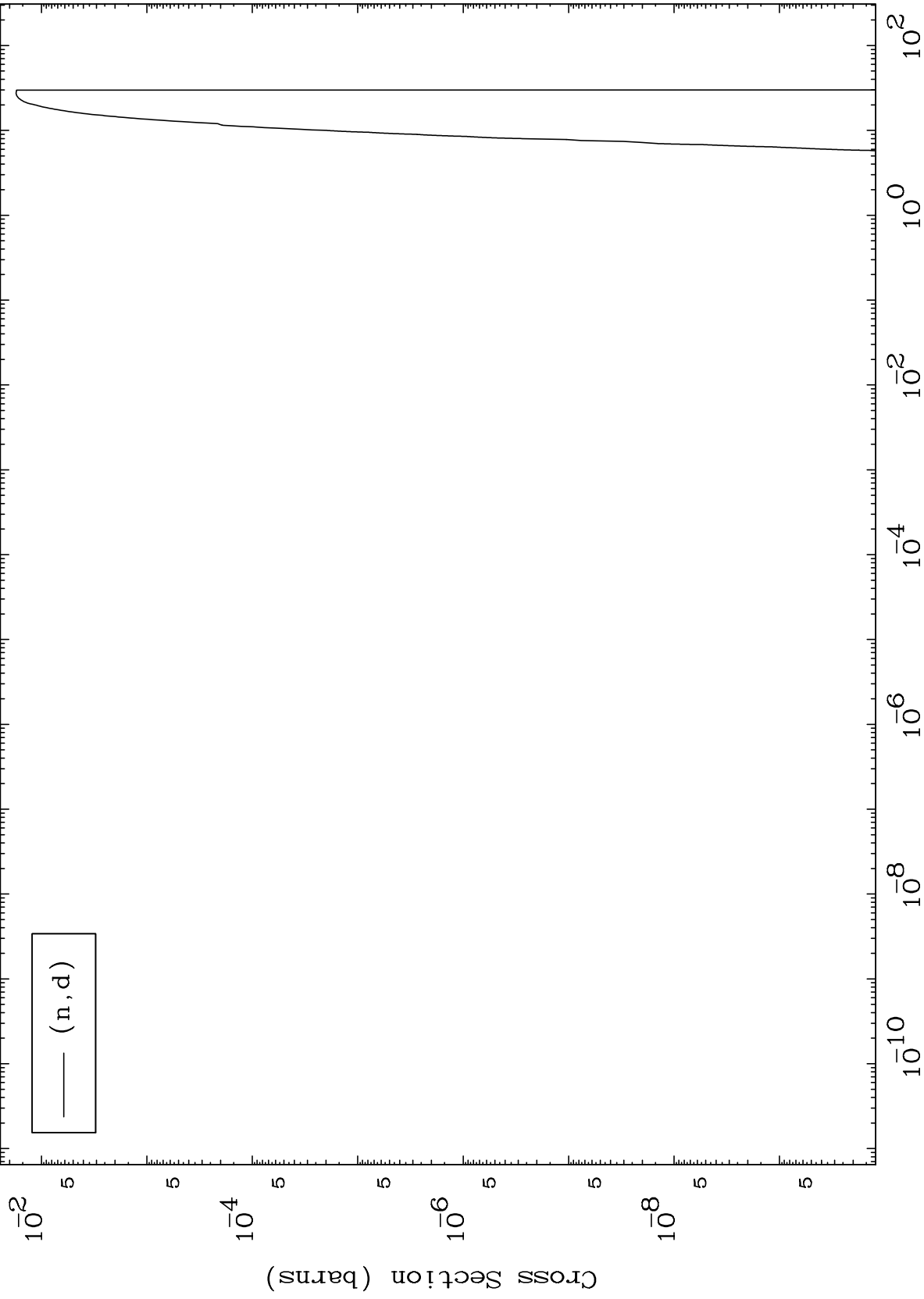


103-Lr-254

MAT 9983

(n,d) Levels
293 Kelvin Cross Sections

103-Lr-254



12

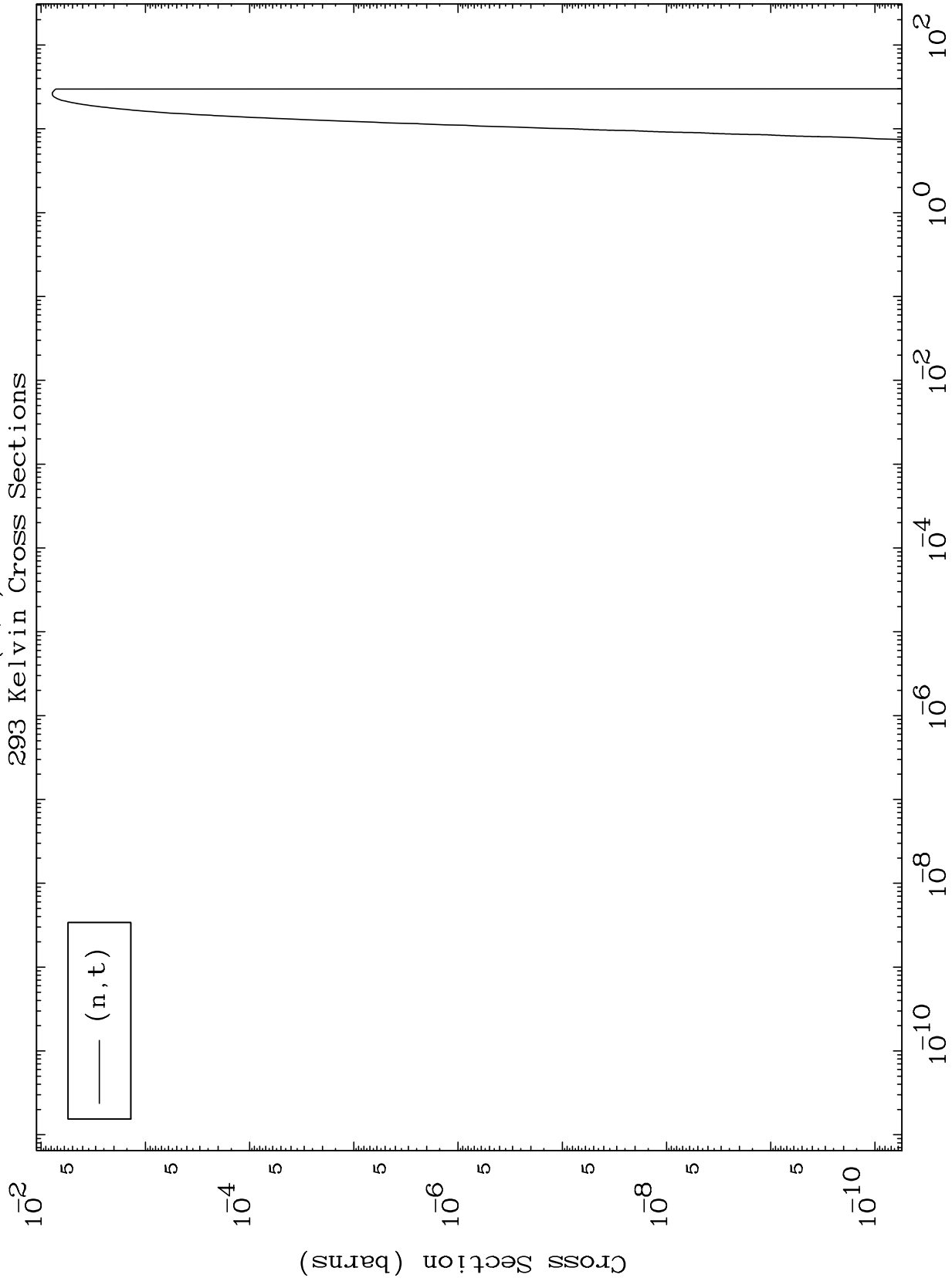
Incident Energy (MeV)

103-Lr-254

MAT 9983

(n,t) Levels
293 Kelvin Cross Sections

103-Lr-254



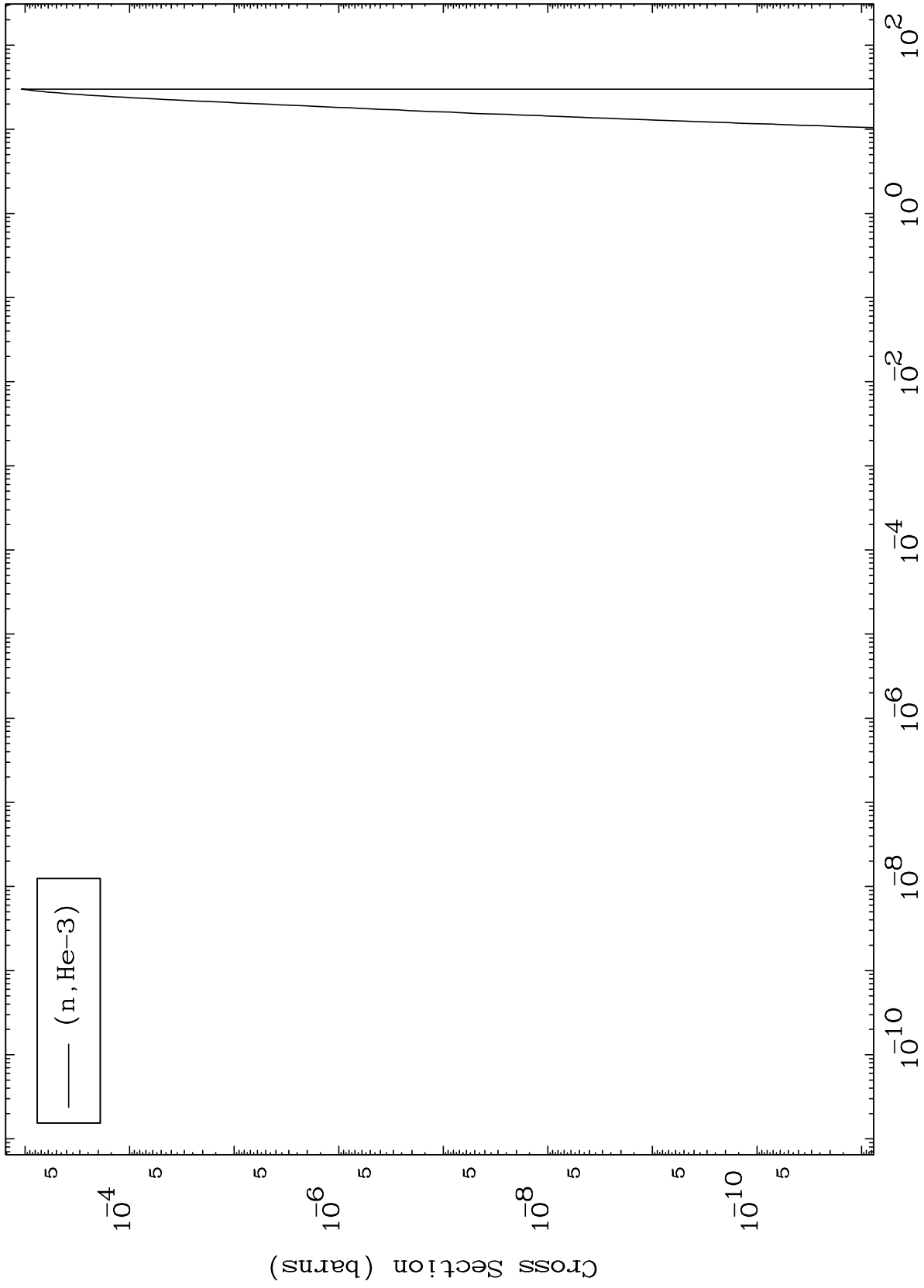
13

103-Lr-254

MAT 9983

(n,He3) Levels
293 Kelvin Cross Sections

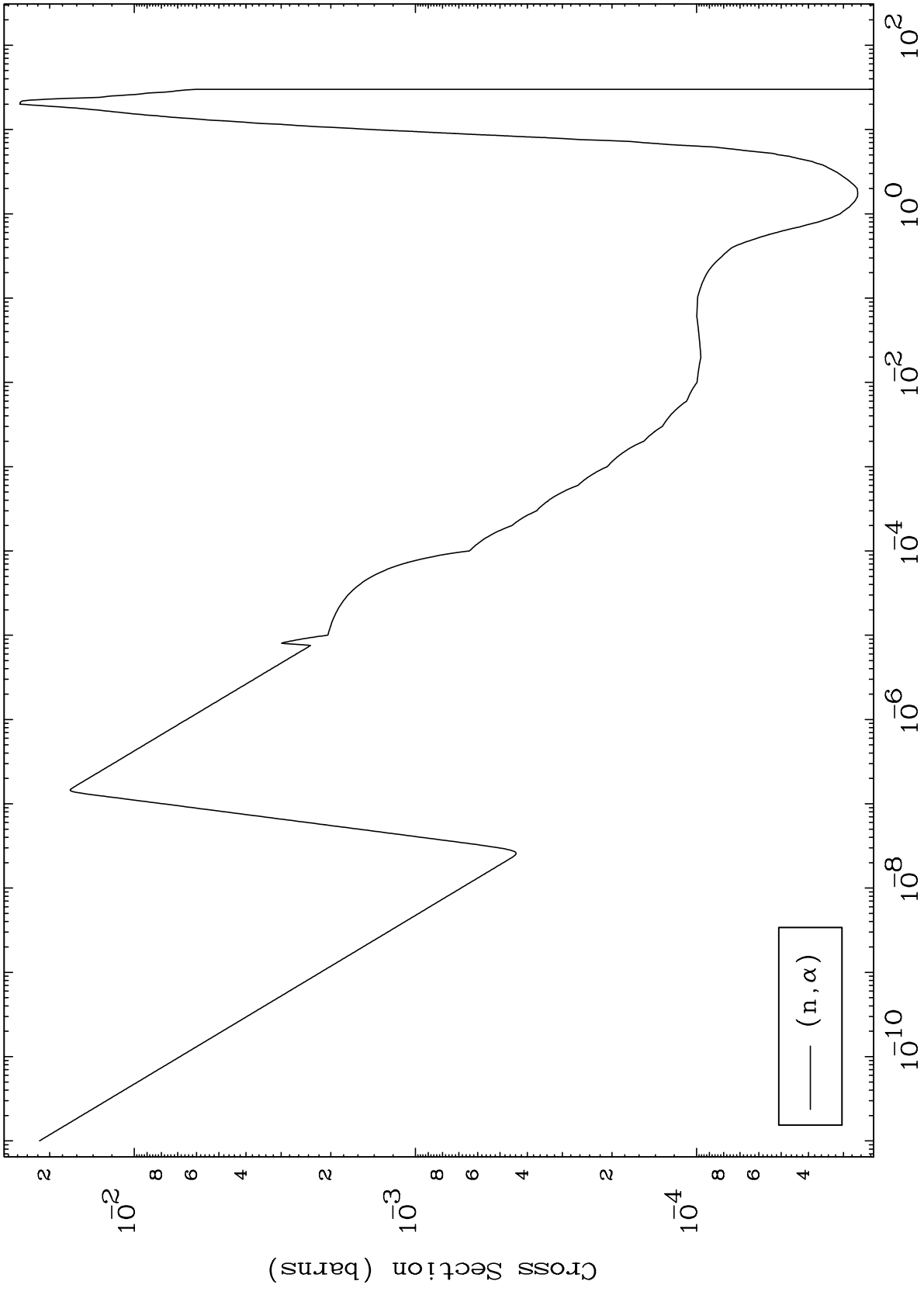
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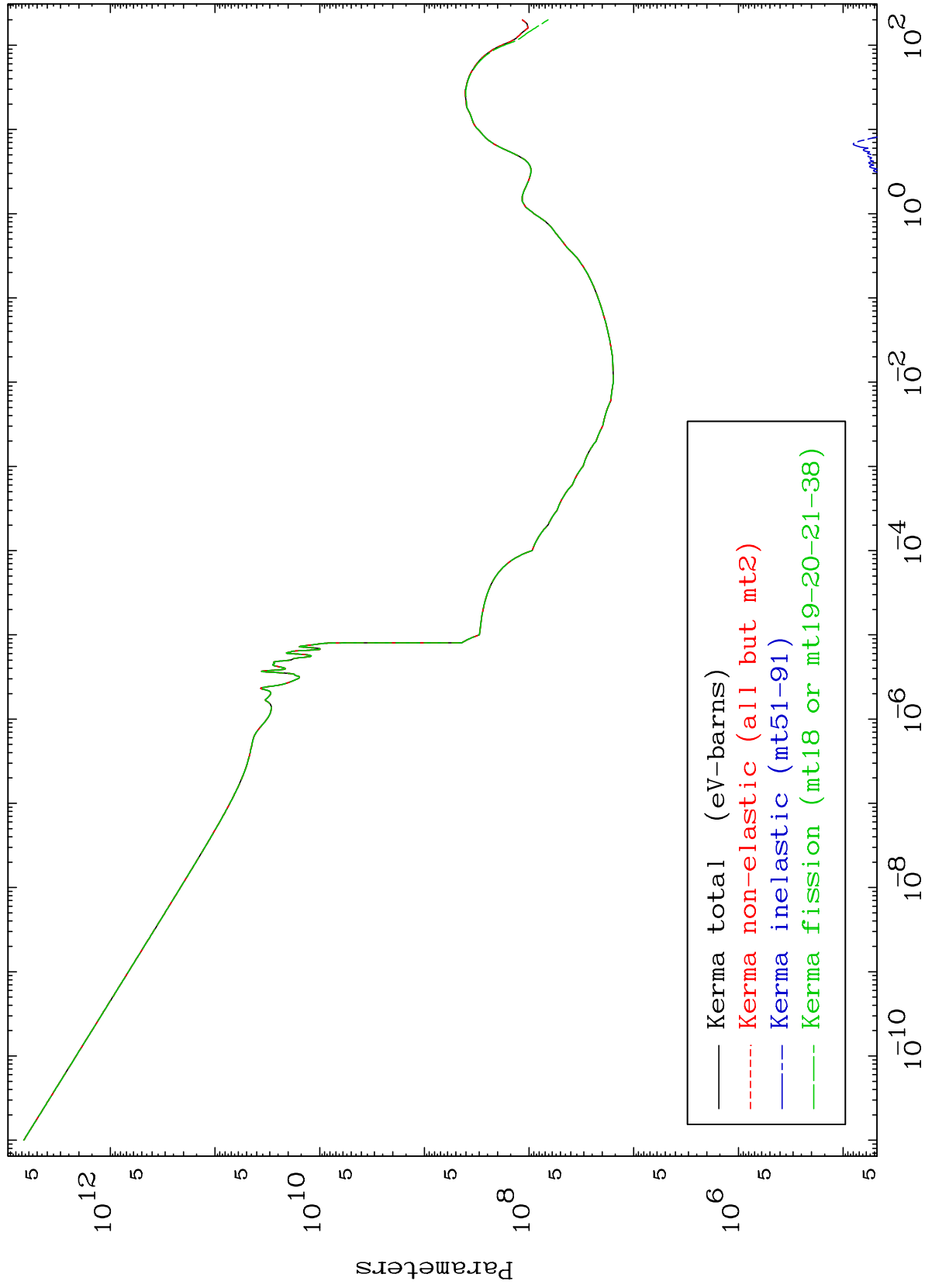


MAT 9983

(n, α) Levels
293 Kelvin Cross Sections

103-Lr-254

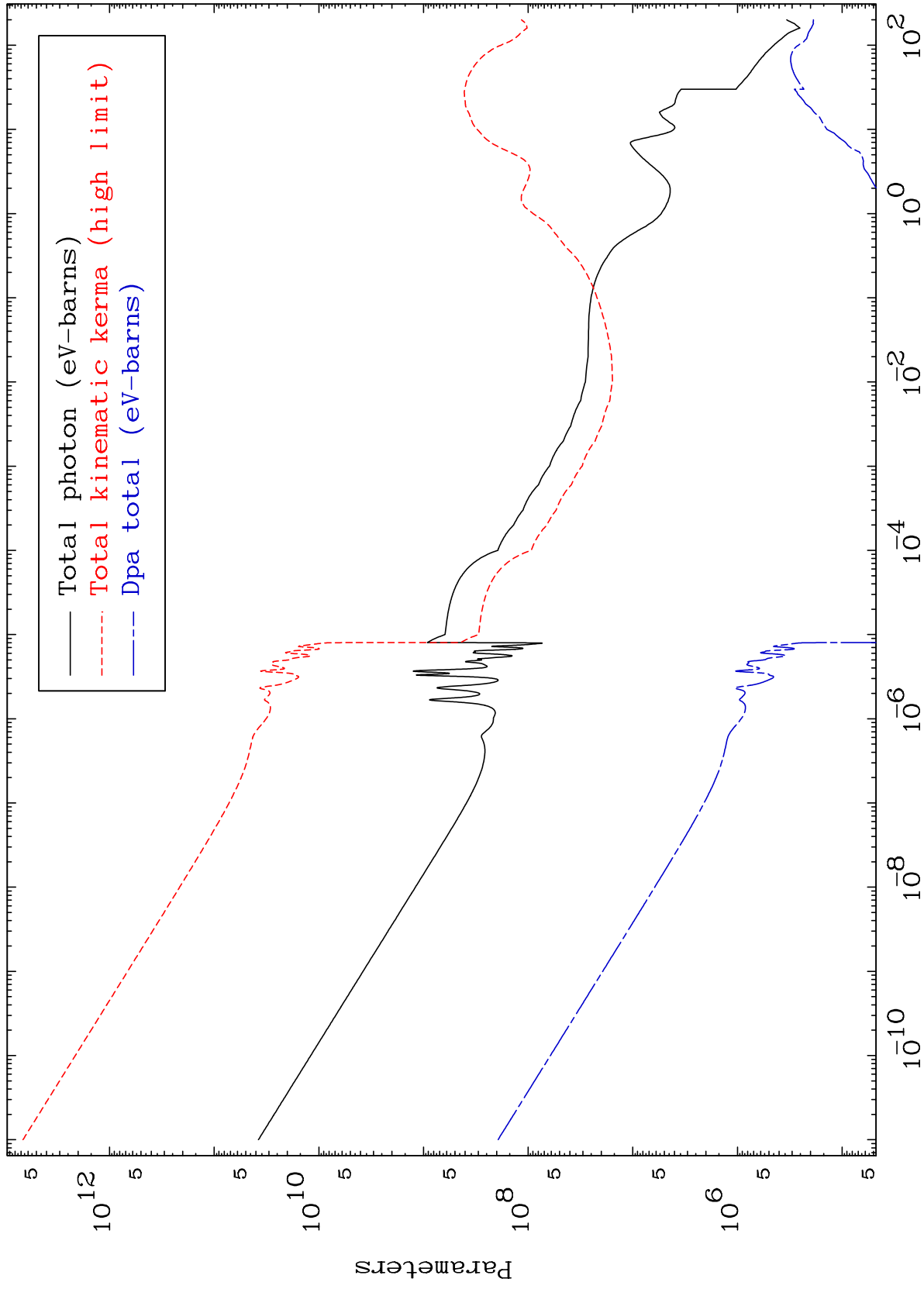




MAT 9983

Energy Release
Parameters

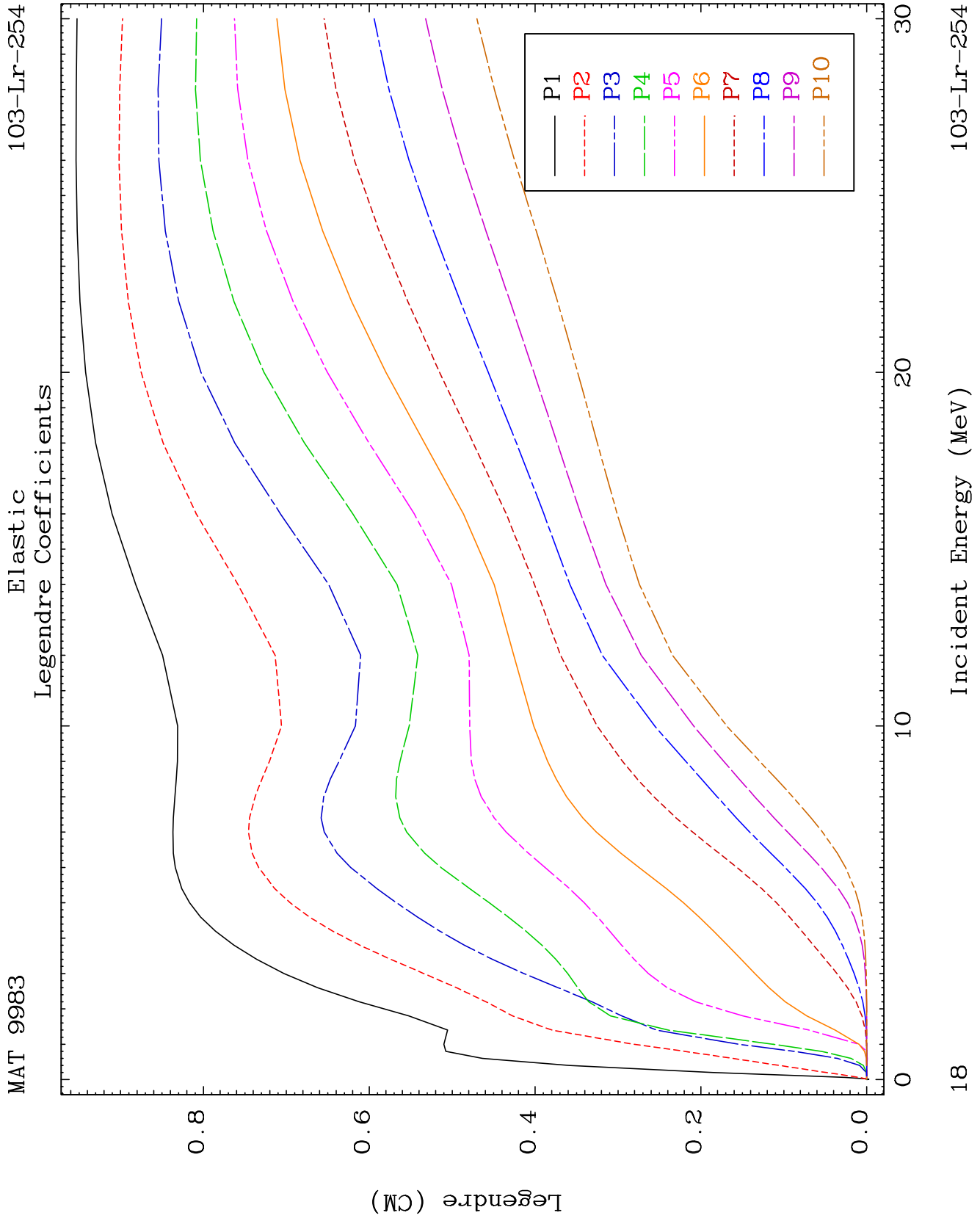
103-Lr-254



17

Incident Energy (MeV)

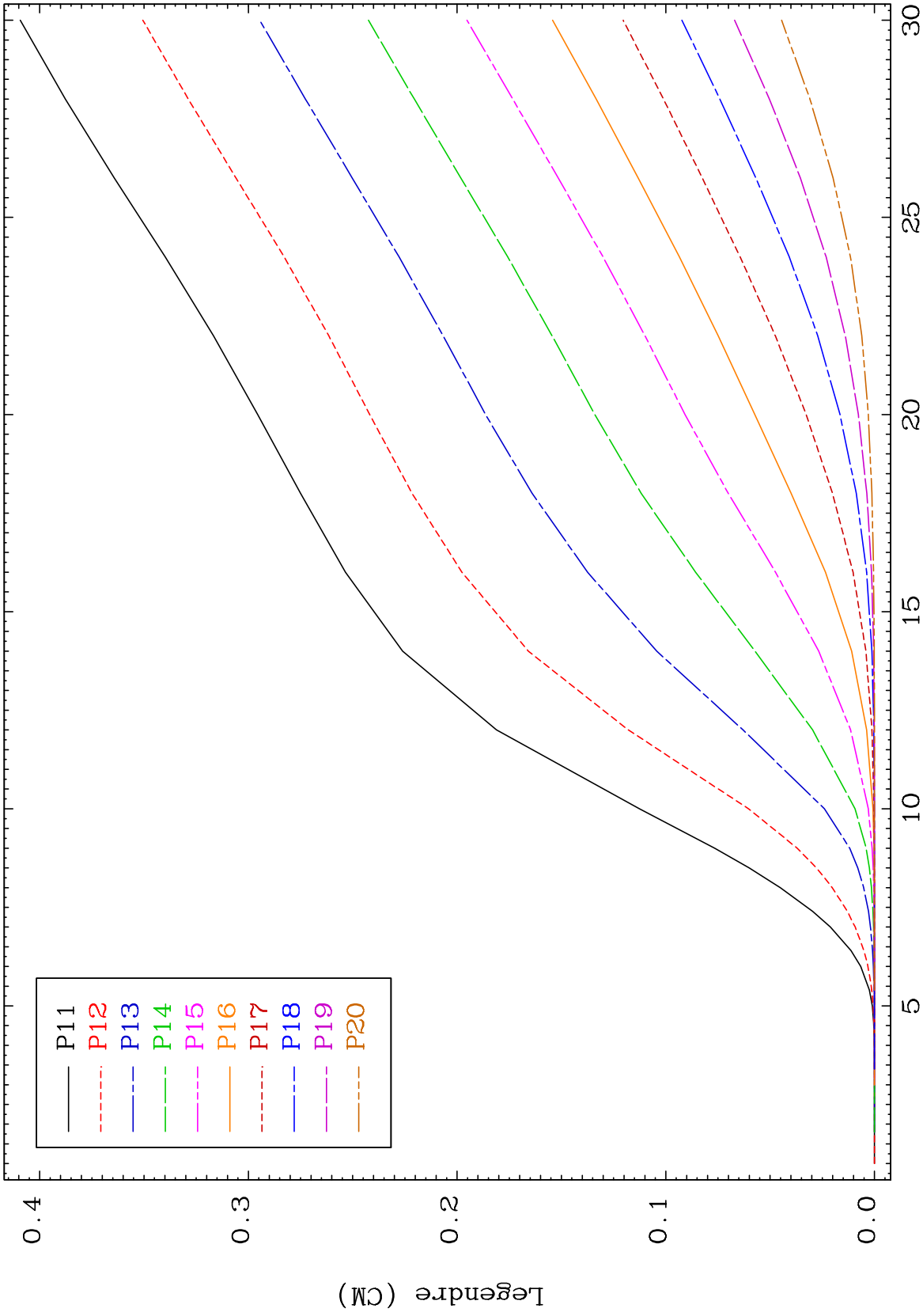
103-Lr-254



MAT 9983

Elastic Legendre Coefficients

103-Lr-254



19

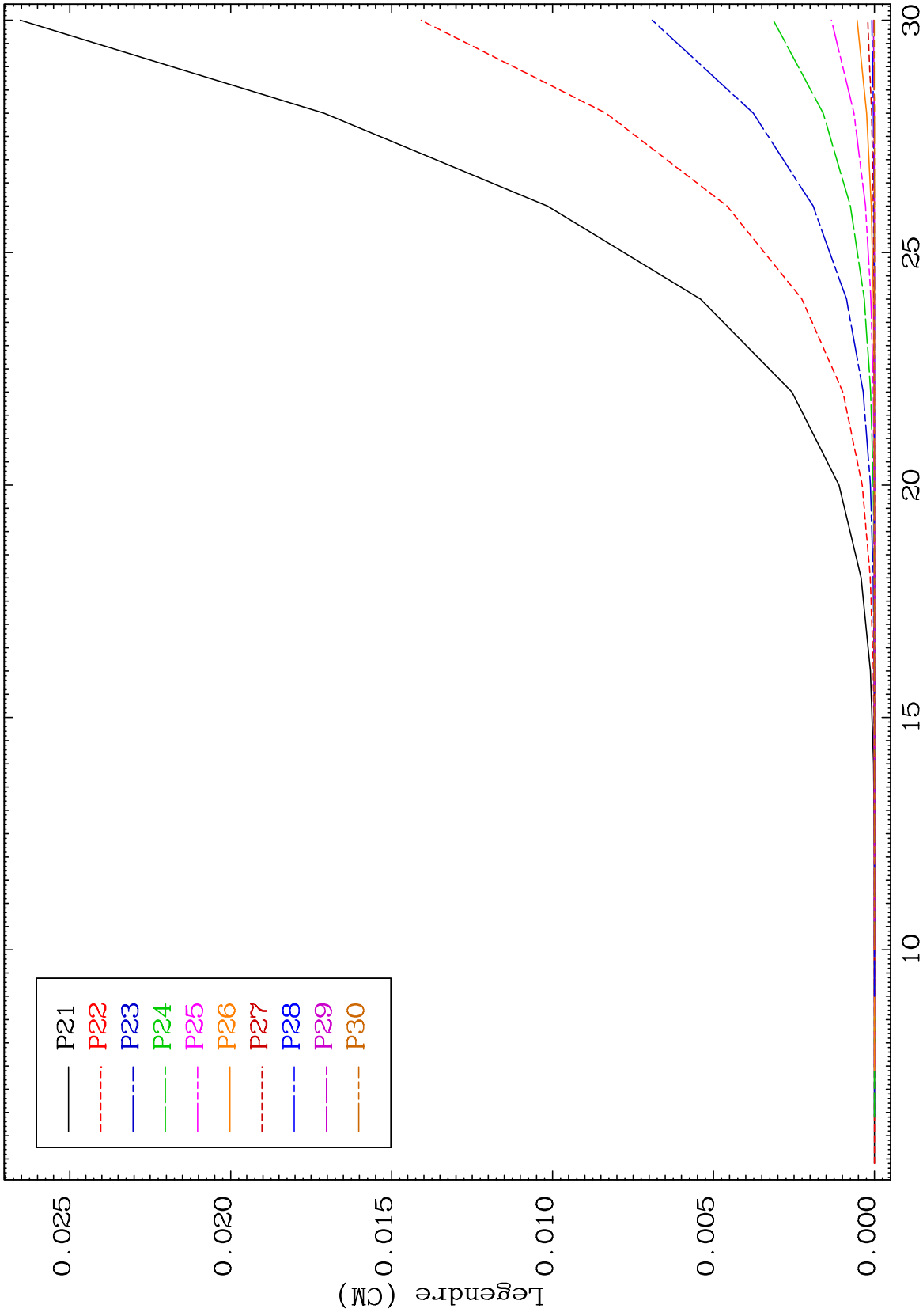
Incident Energy (MeV)

103-Lr-254

MAT 9983

Elastic Legendre Coefficients

103-Lr-254



103-Lr-254

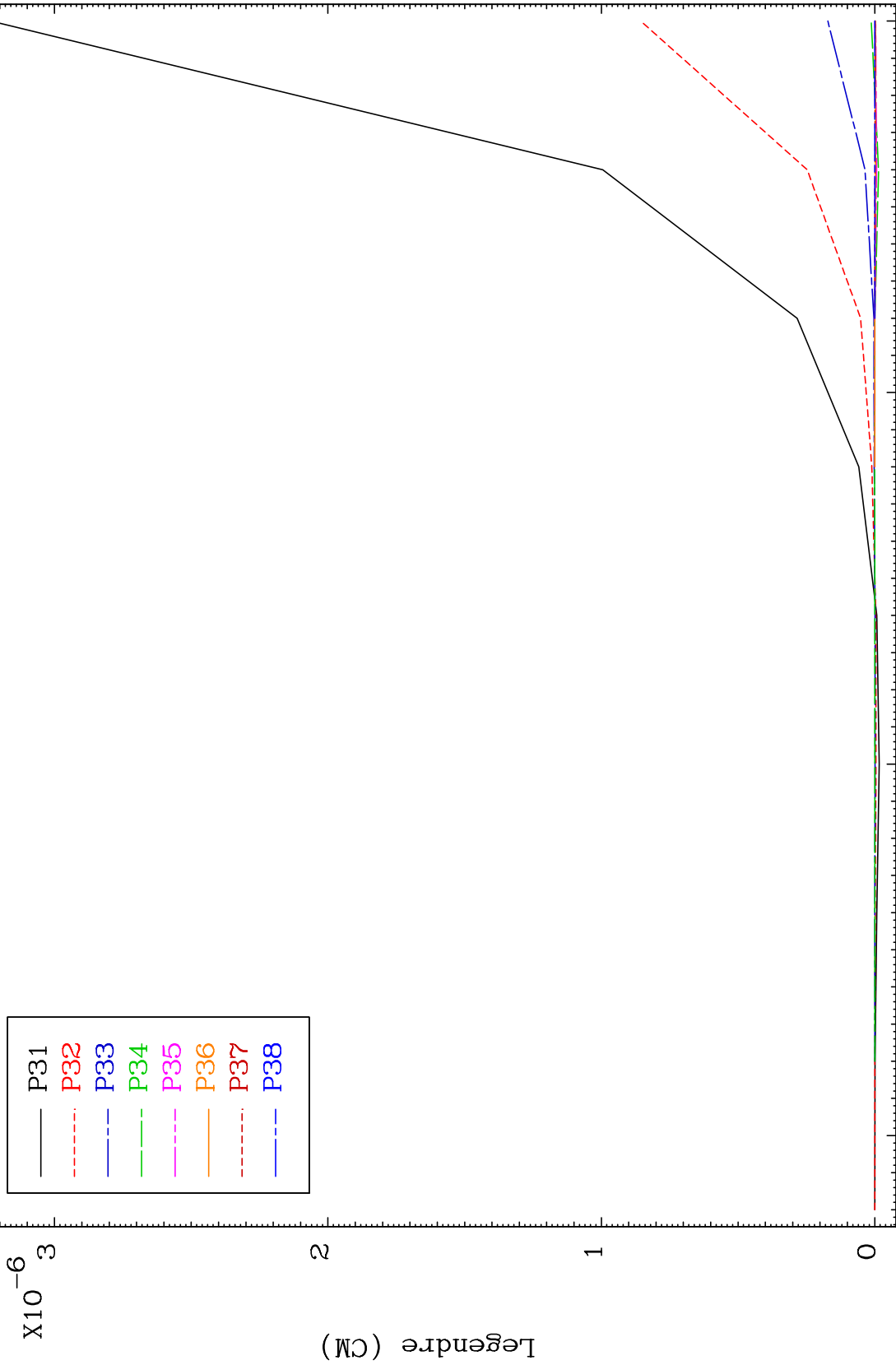
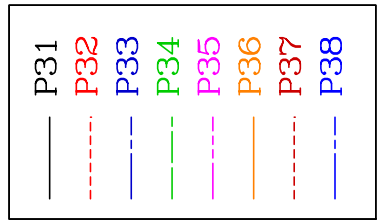
Incident Energy (MeV)

20

MAT 9983

Elastic Legendre Coefficients

103-Lr-254



21

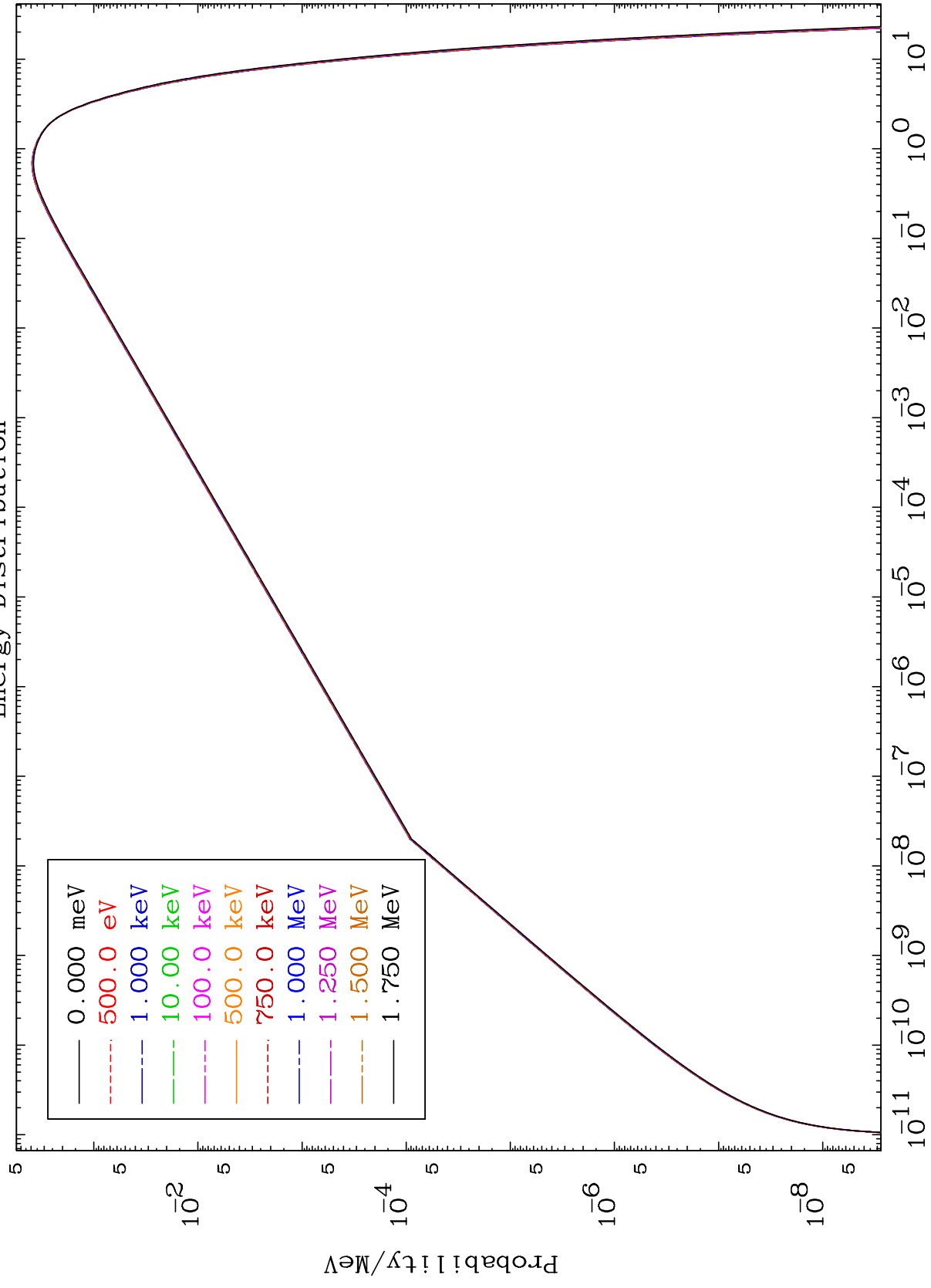
Incident Energy (MeV)

103-Lr-254

MAT 9983

Fission Energy Distribution

103-Lr-254

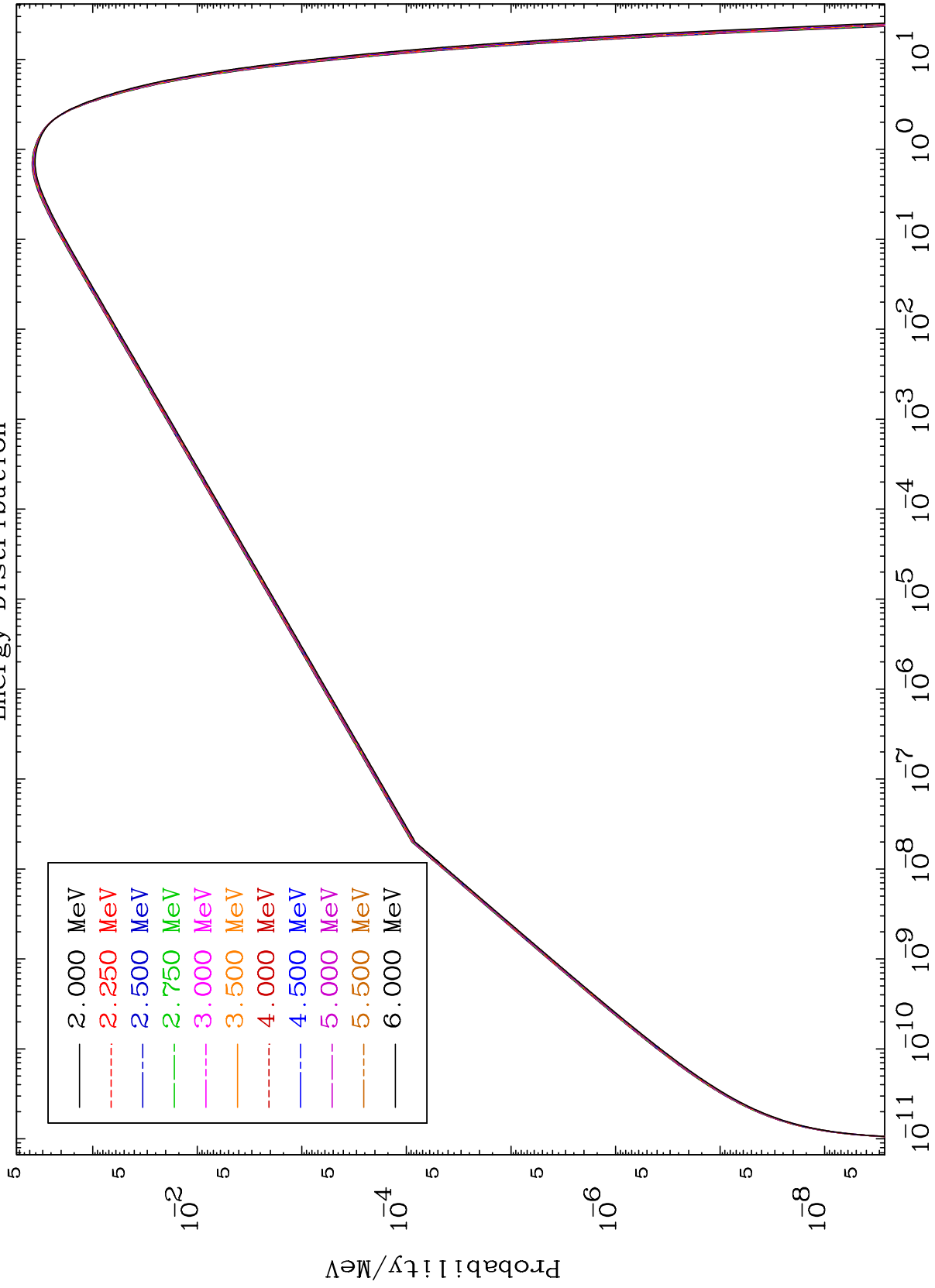


103-Lr-254

MAT 9983

Fission Energy Distribution

103-Lr-254

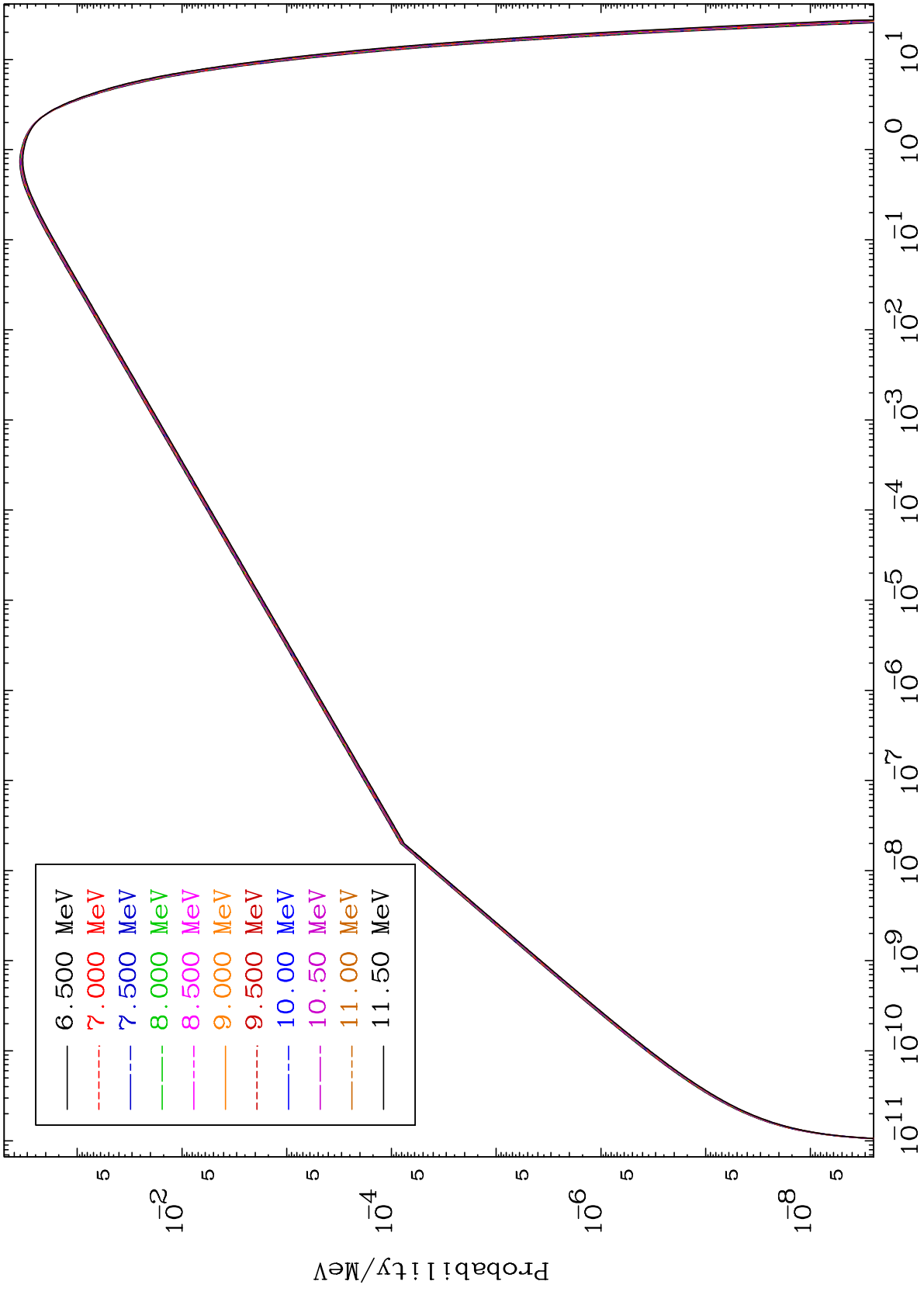


103-Lr-254

MAT 9983

Fission Energy Distribution

103-Lr-254

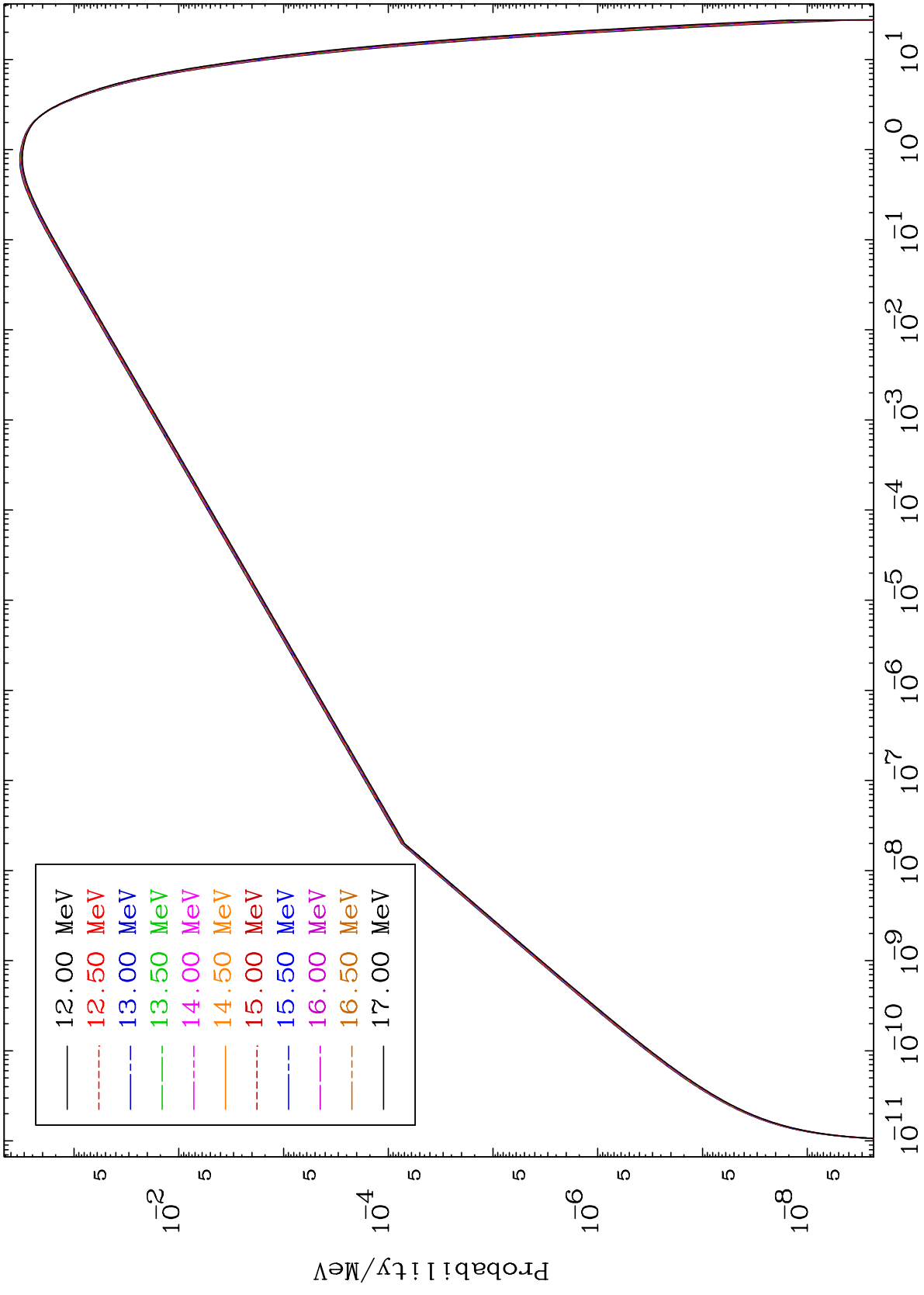


103-Lr-254

MAT 9983

Fission
Energy Distribution

103-Lr-254

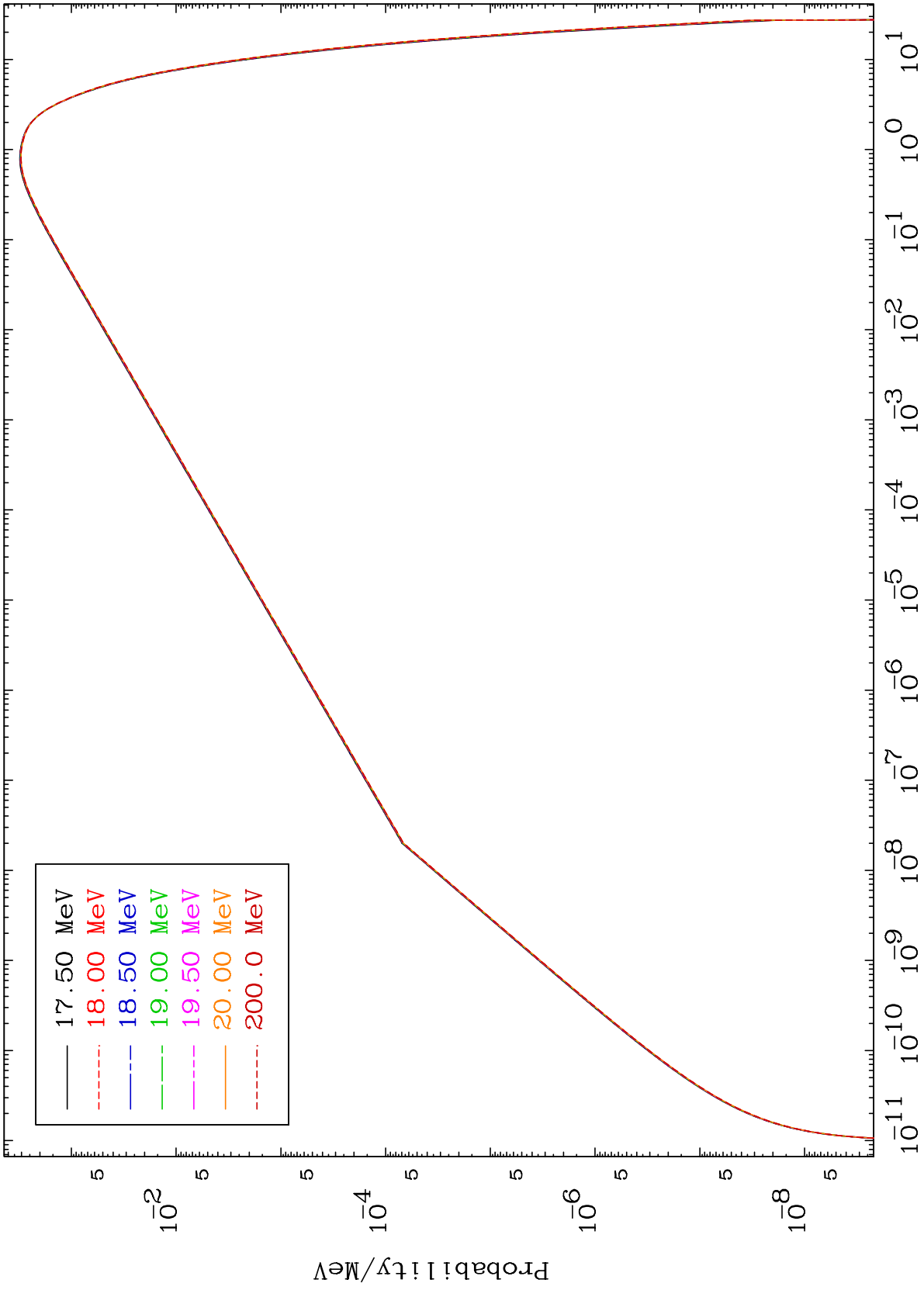


103-Lr-254

MAT 9983

Fission Energy Distribution

103-Lr-254

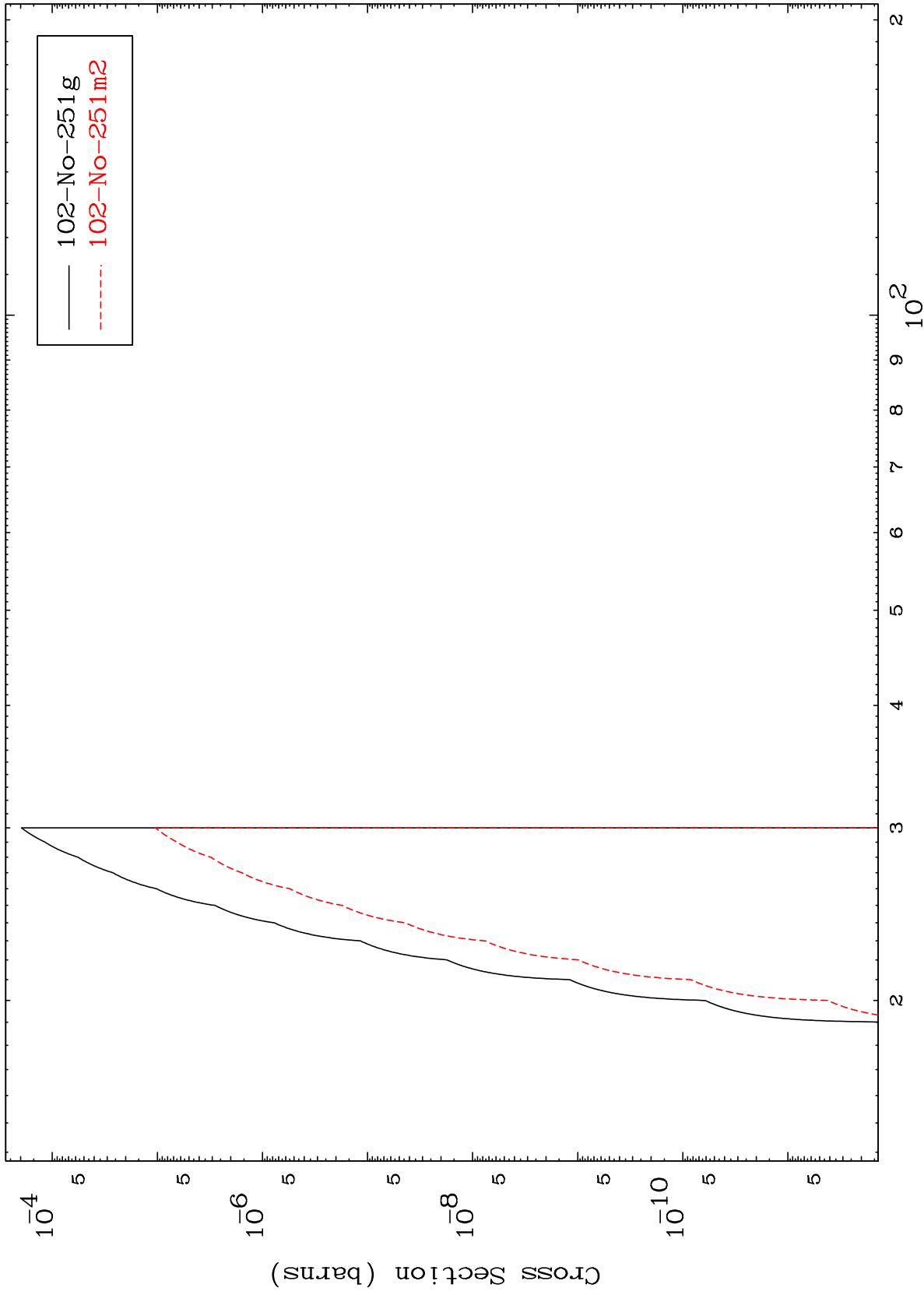


MAT 9983

(n,2n) d

103-Lr-254

Radionuclide Production Cross Section



27

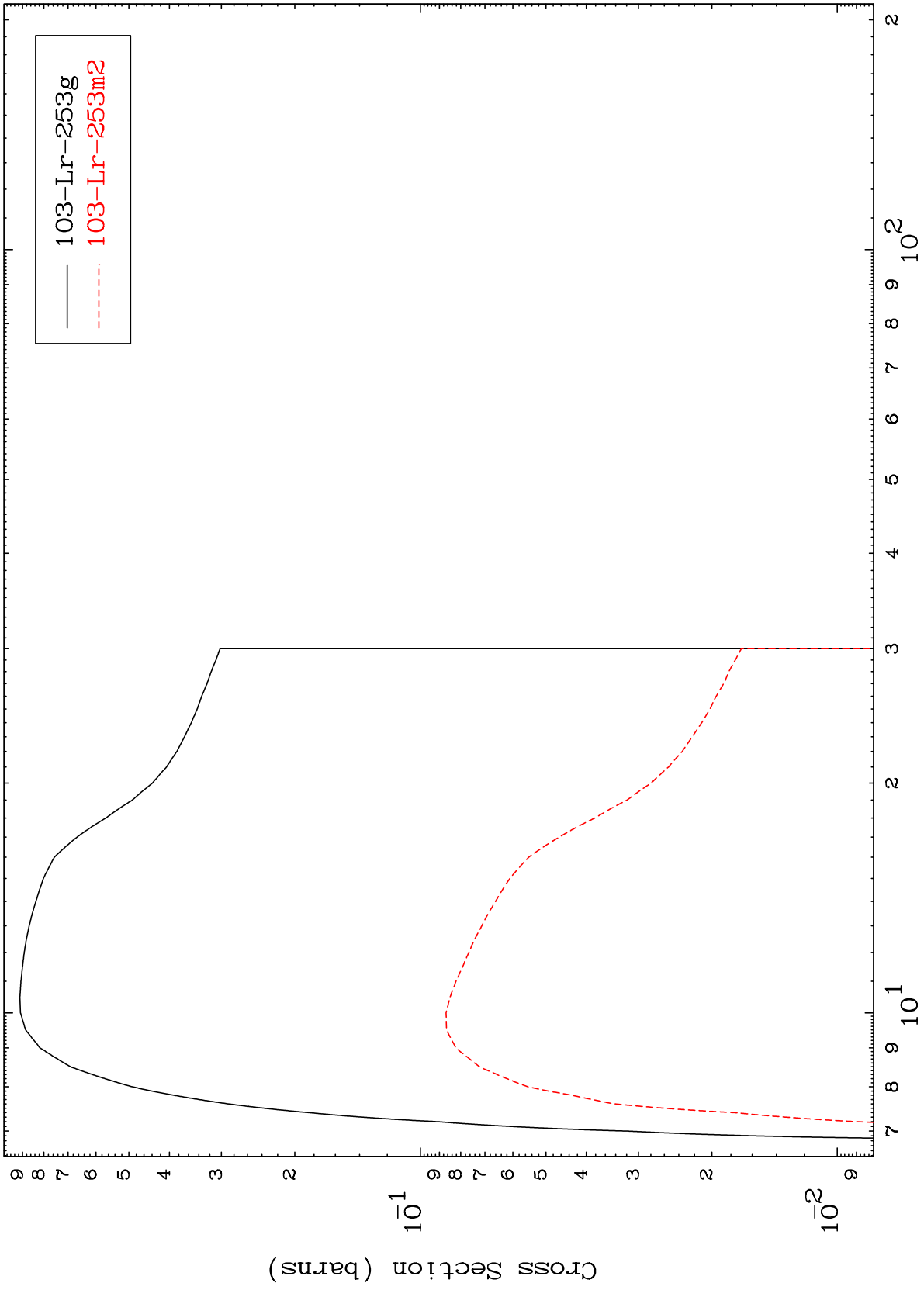
Incident Energy (MeV)

103-Lr-254

MAT 9983

103-Lr-254

(n,2n)
Radionuclide Production Cross Section



28

Incident Energy (MeV)

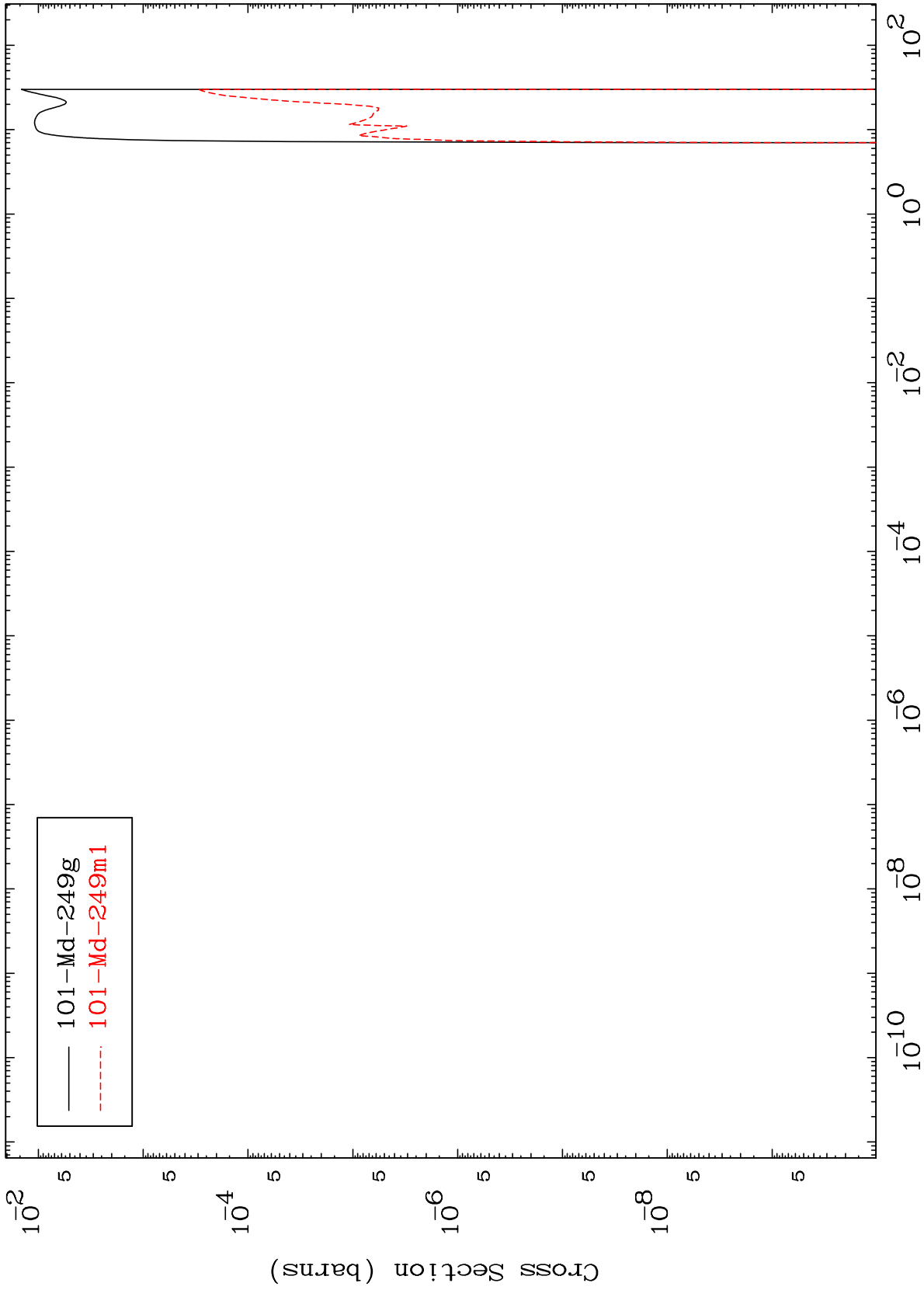
103-Lr-254

MAT 9983

(n,2n) α

103-Lr-254

Radionuclide Production Cross Section



29

Incident Energy (MeV)

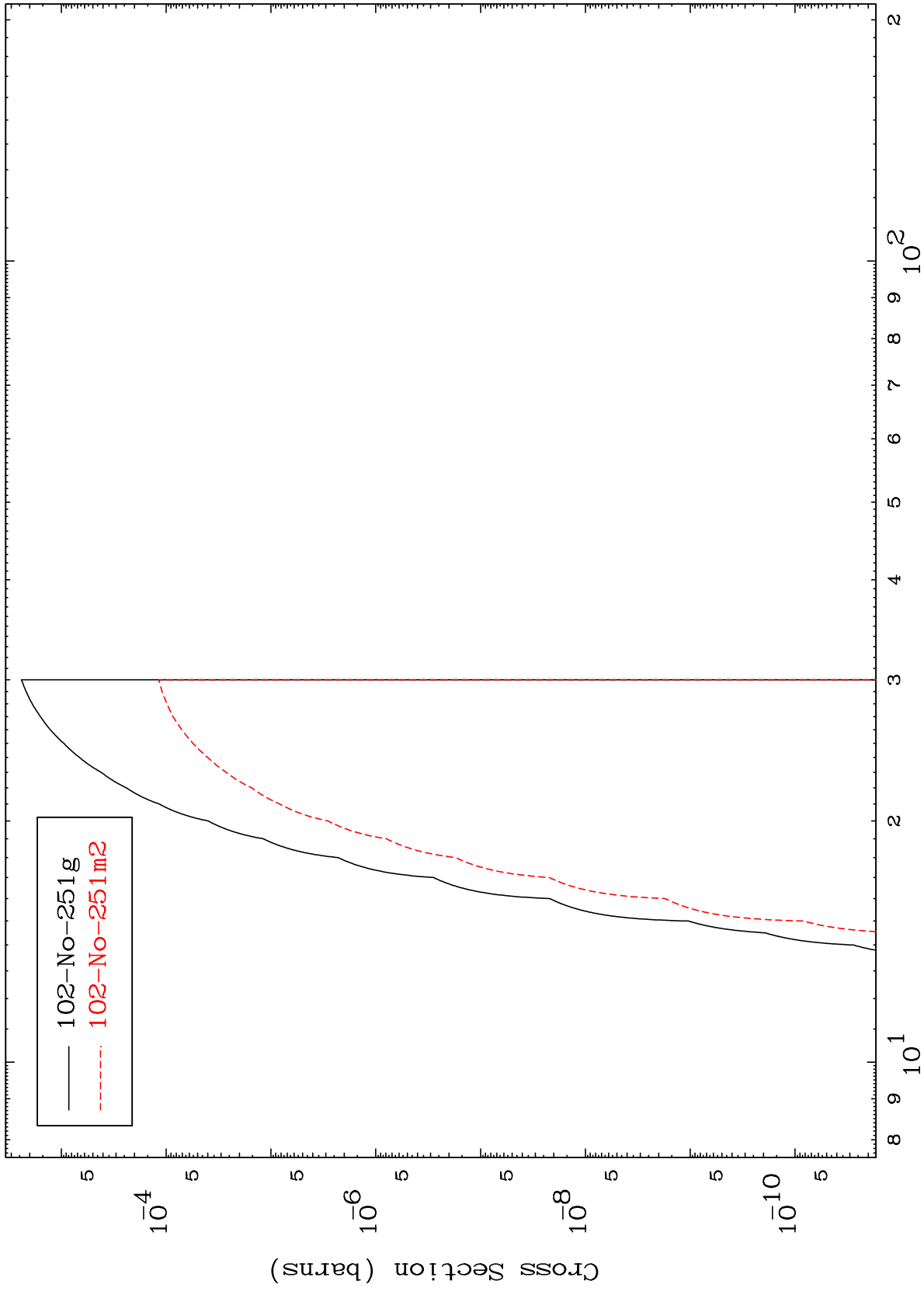
103-Lr-254

MAT 9983

(n,n') t

103-Lr-254

Radionuclide Production Cross Section



30

Incident Energy (MeV)

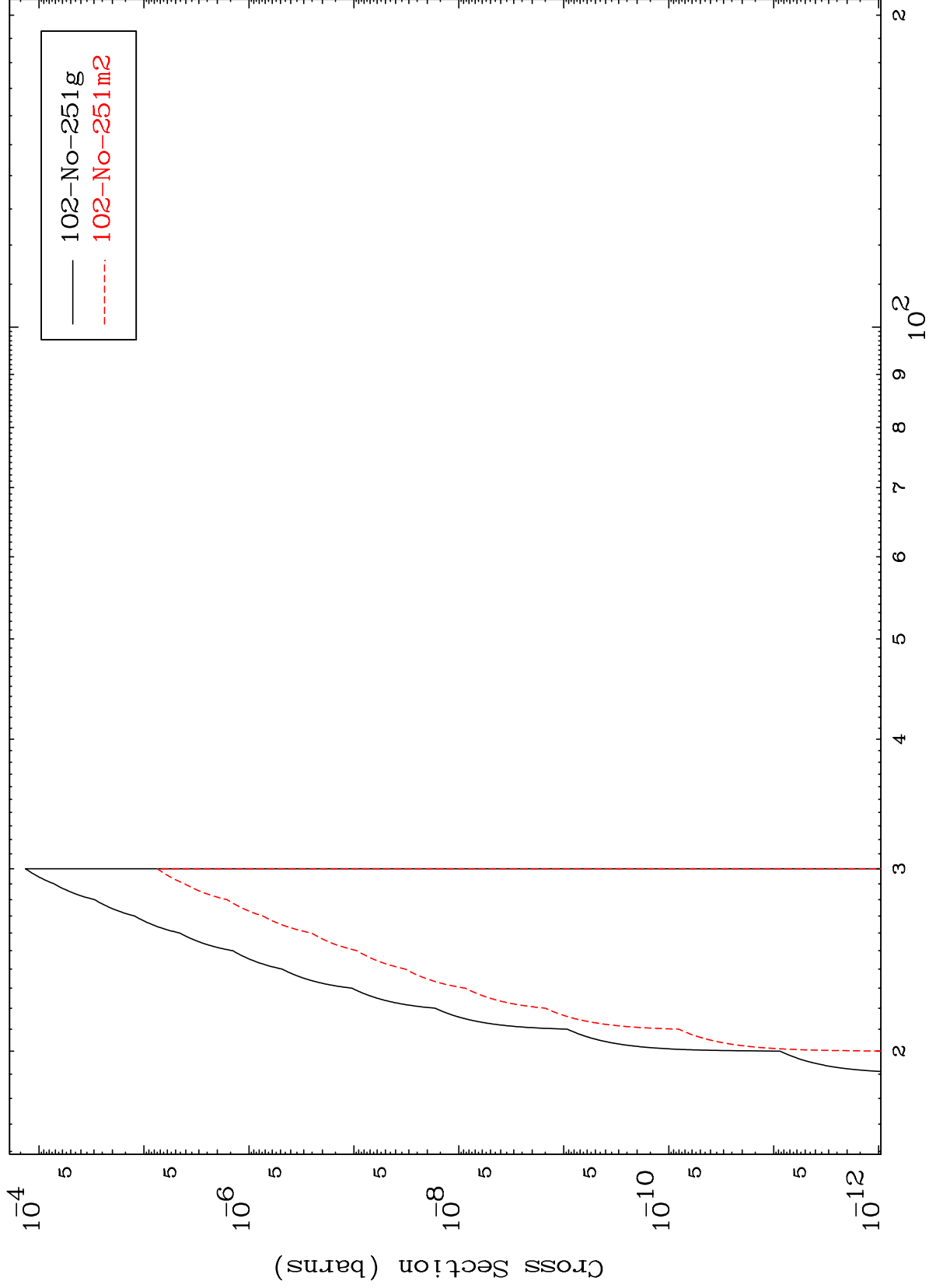
103-Lr-254

MAT 9983

(n,3n) p

103-Lr-254

Radionuclide Production Cross Section



31

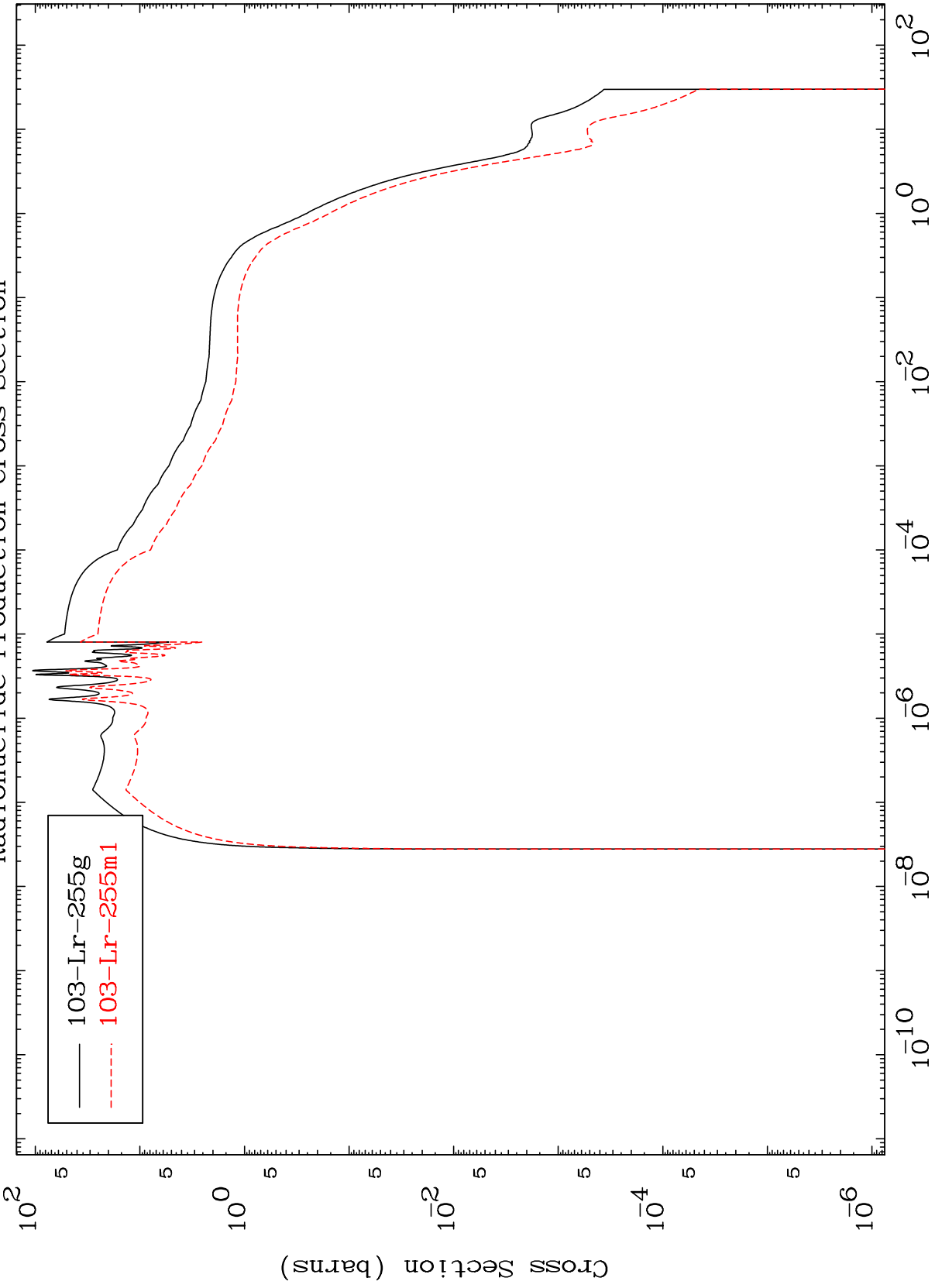
Incident Energy (MeV)

103-Lr-254

MAT 9983

103-Lr-254

Radionuclide Production Cross Section



32

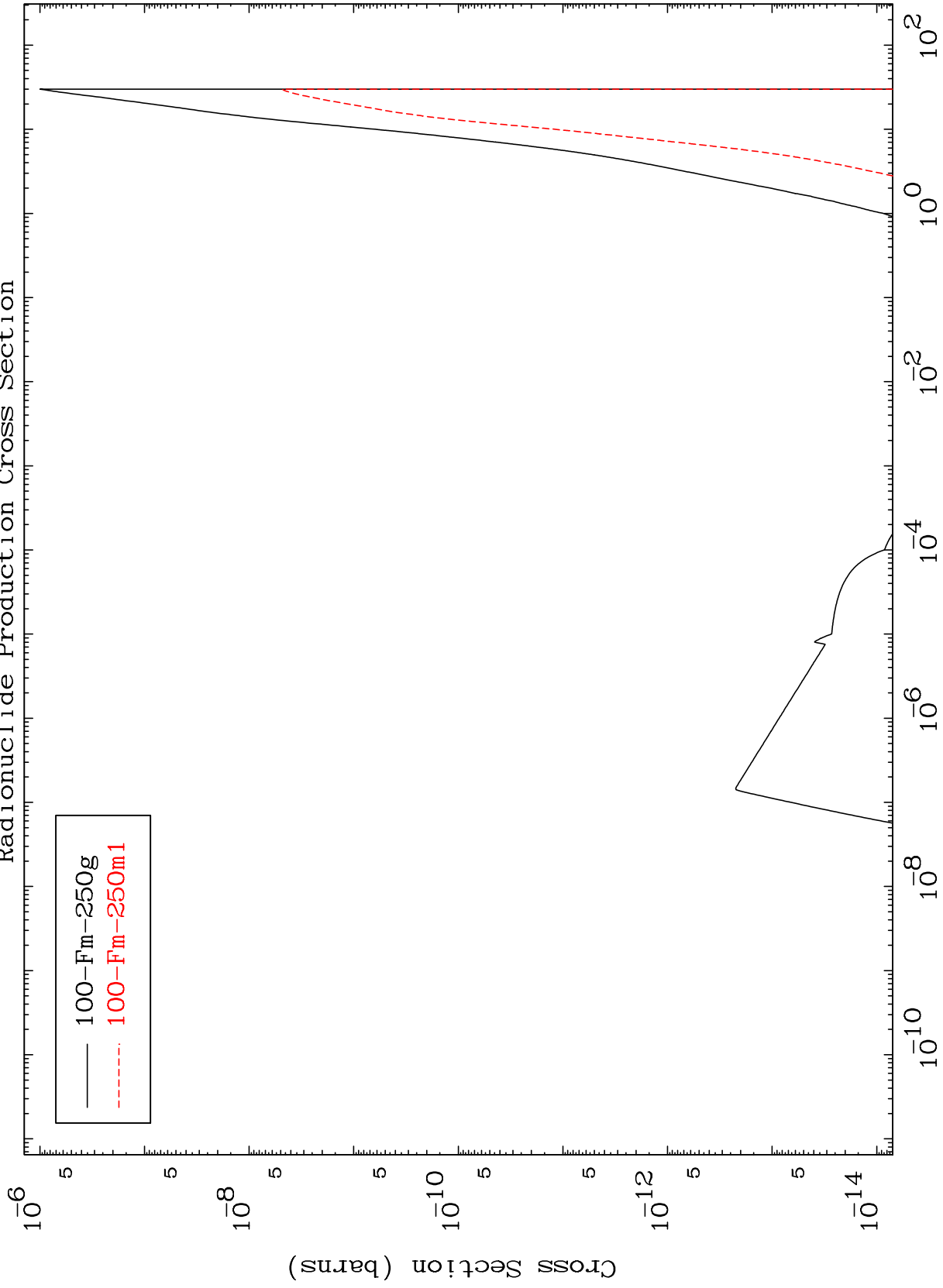
103-Lr-254

MAT 9983

(n,p) α

103-Lr-254

Radionuclide Production Cross Section



33

Incident Energy (MeV)

103-Lr-254