

Program EVALPLOT
(Version 2018-1)

by

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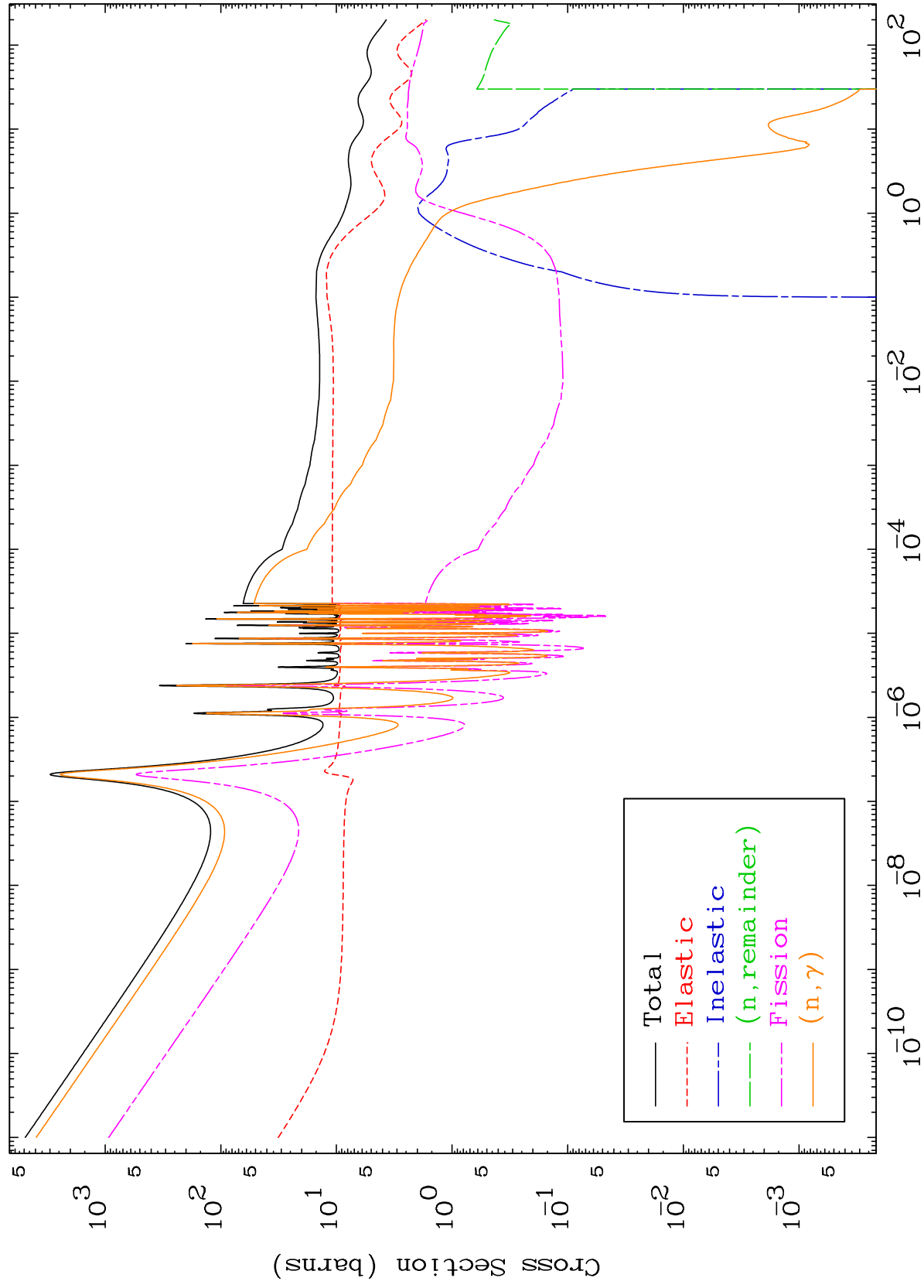
Press Mouse Button to Start

MAT 9990

Major

293 Kelvin Cross Sections

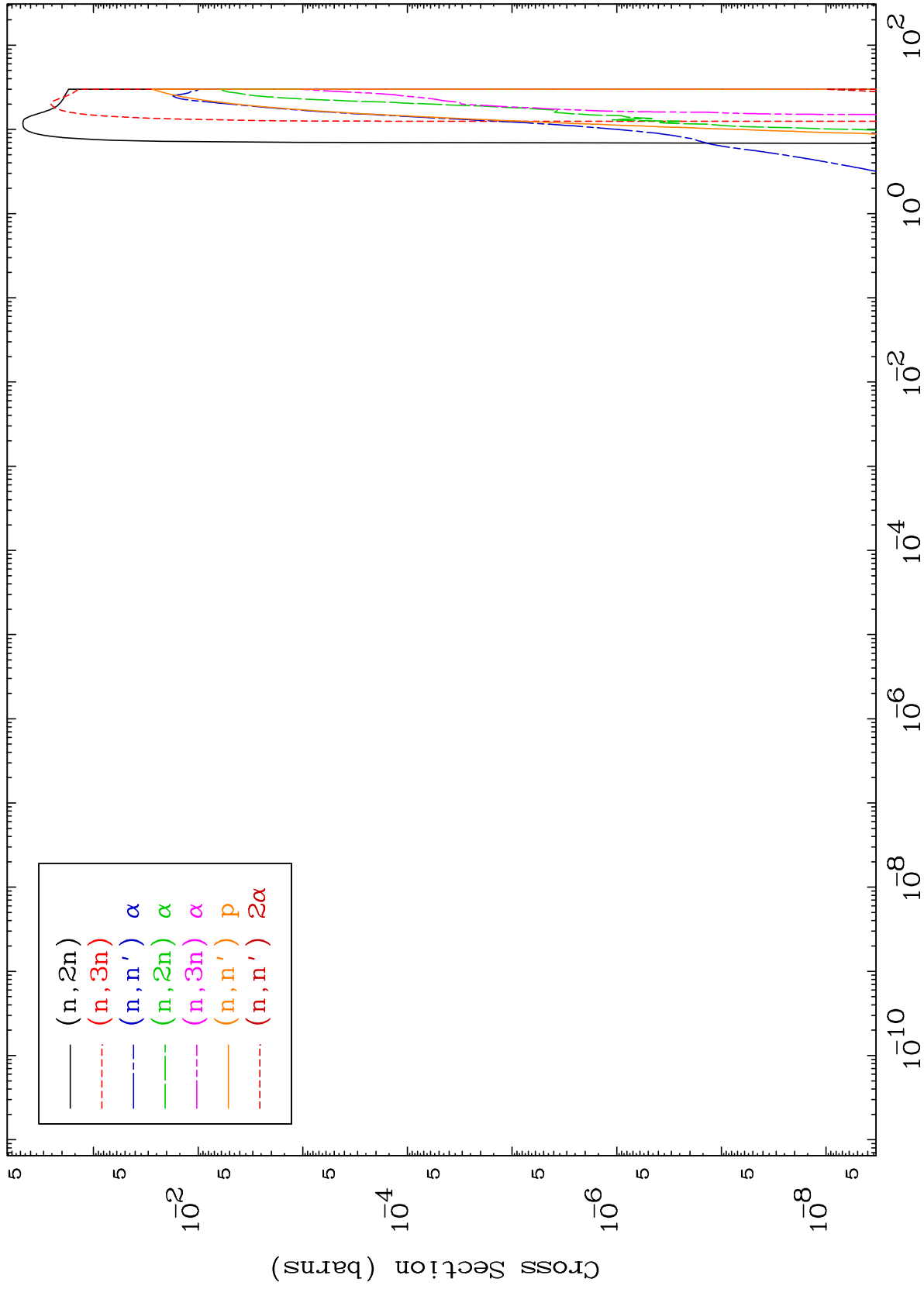
103-Lr-261



MAT 9990

Neutron Production
293 Kelvin Cross Sections

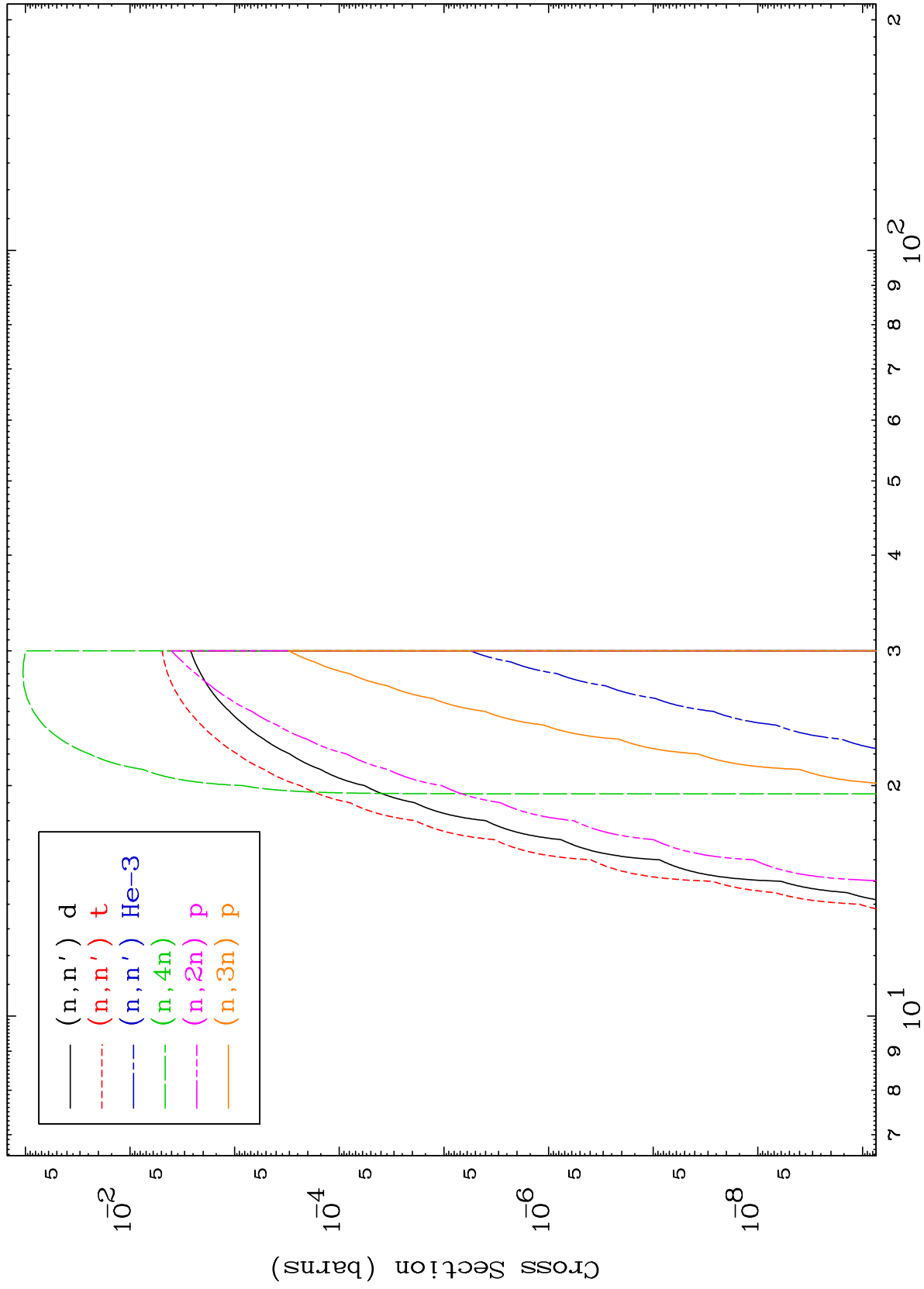
103-Lr-261



MAT 9990

Neutron Production
293 Kelvin Cross Sections

103-Lr-261



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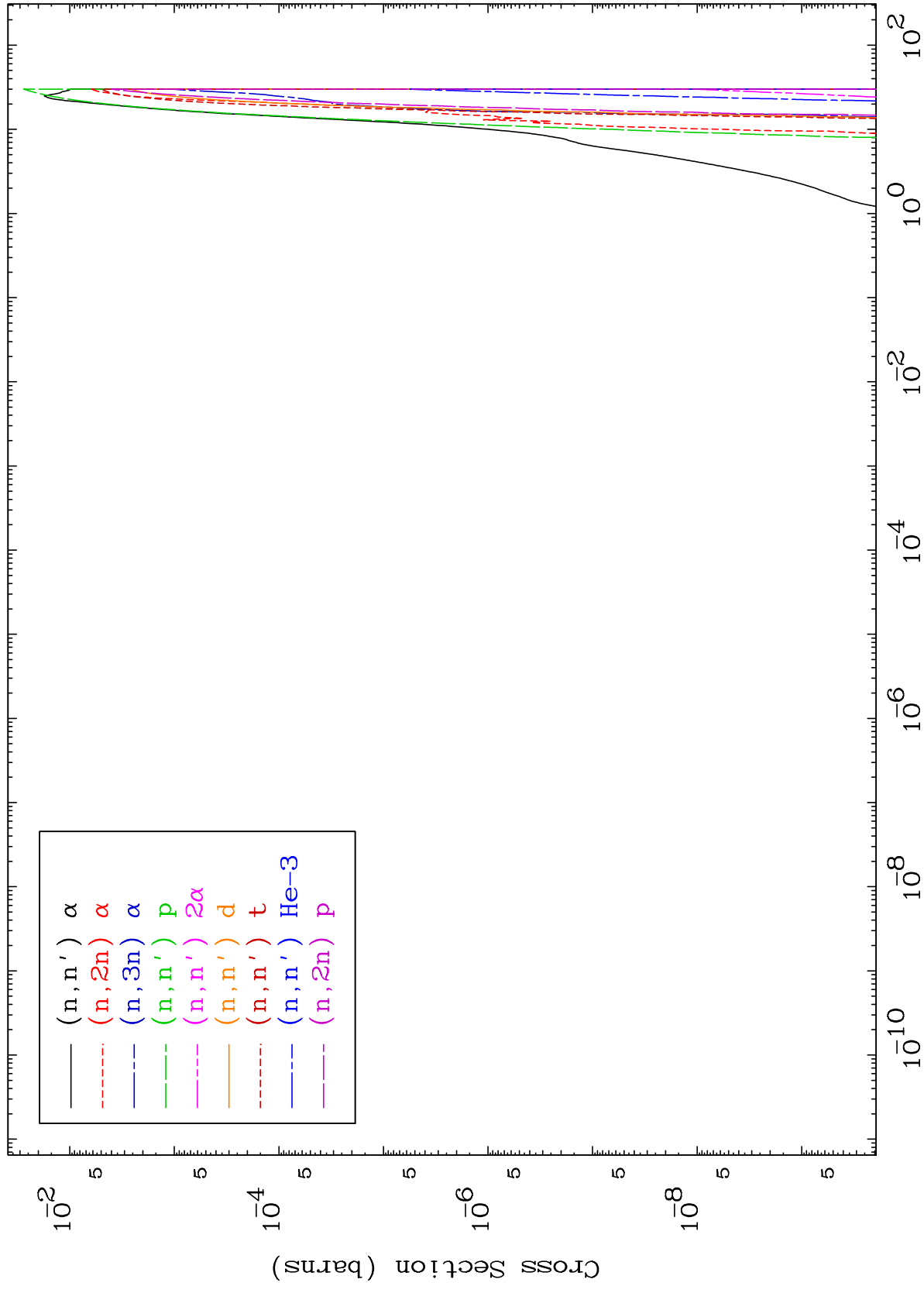
Incident Energy (MeV)

103-Lr-261

MAT 9990

Charged Particle 293 Kelvin Cross Sections

103-Lr-261

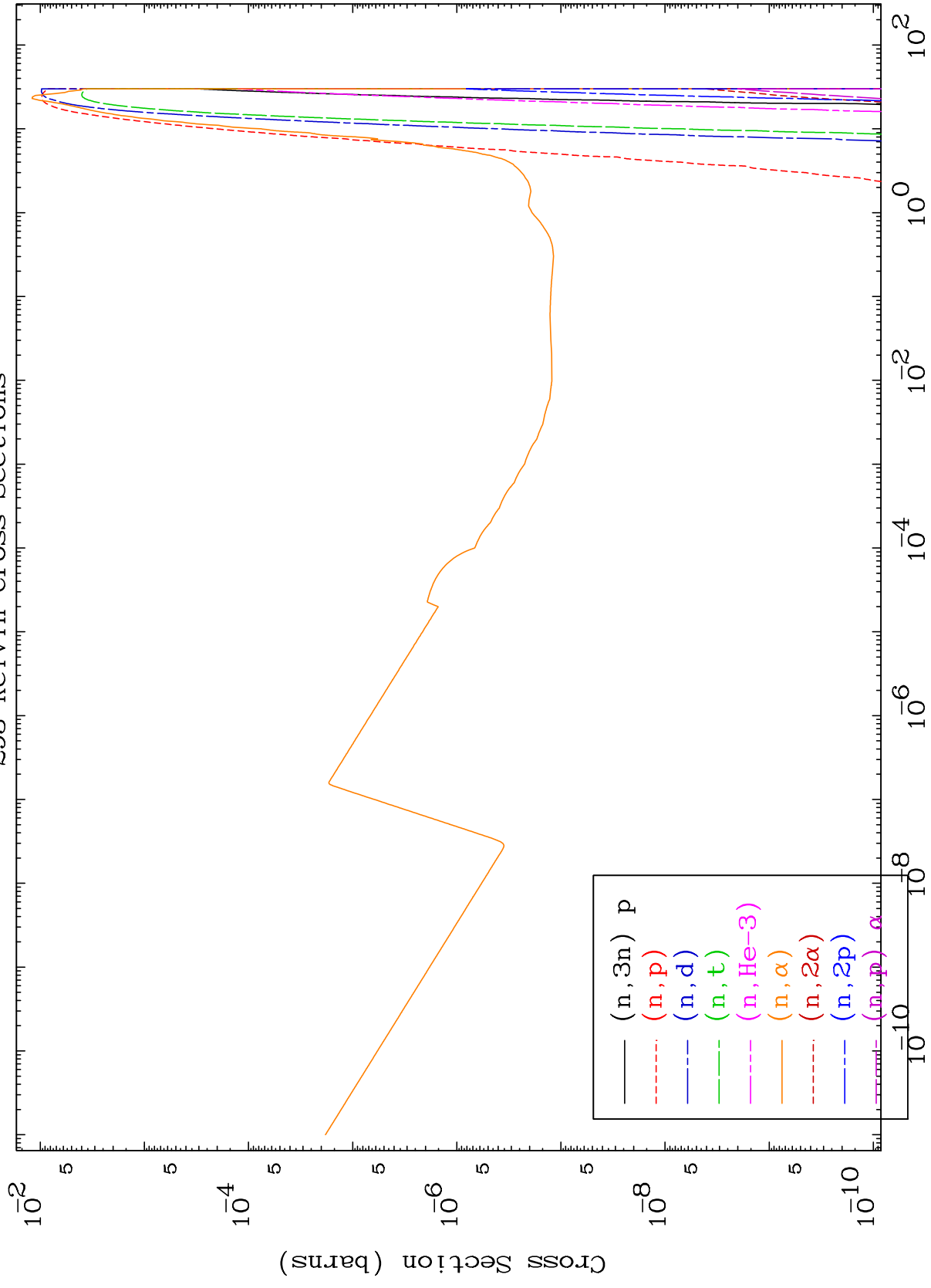


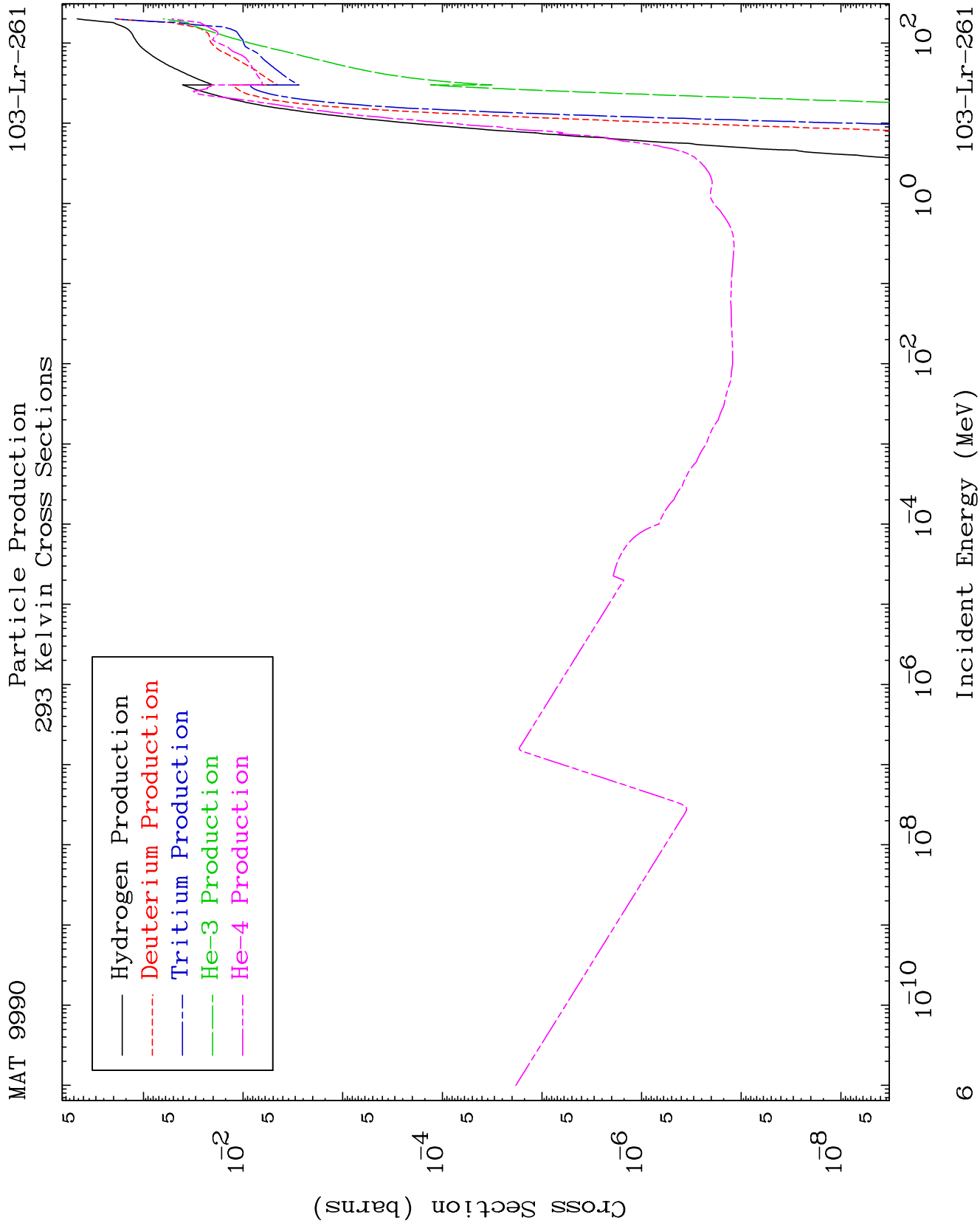
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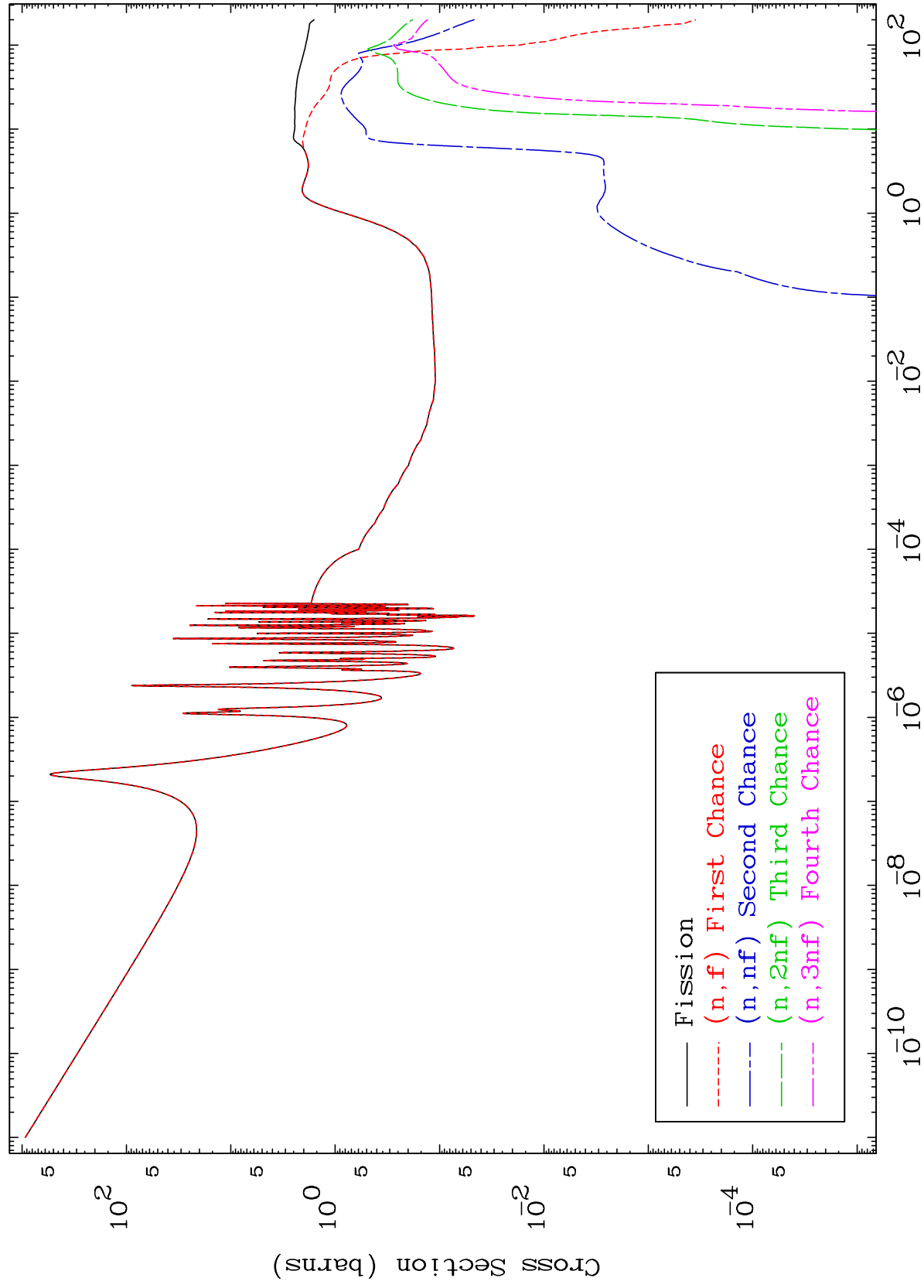
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Charged Particle
293 Kelvin Cross Sections

103-Lr-261



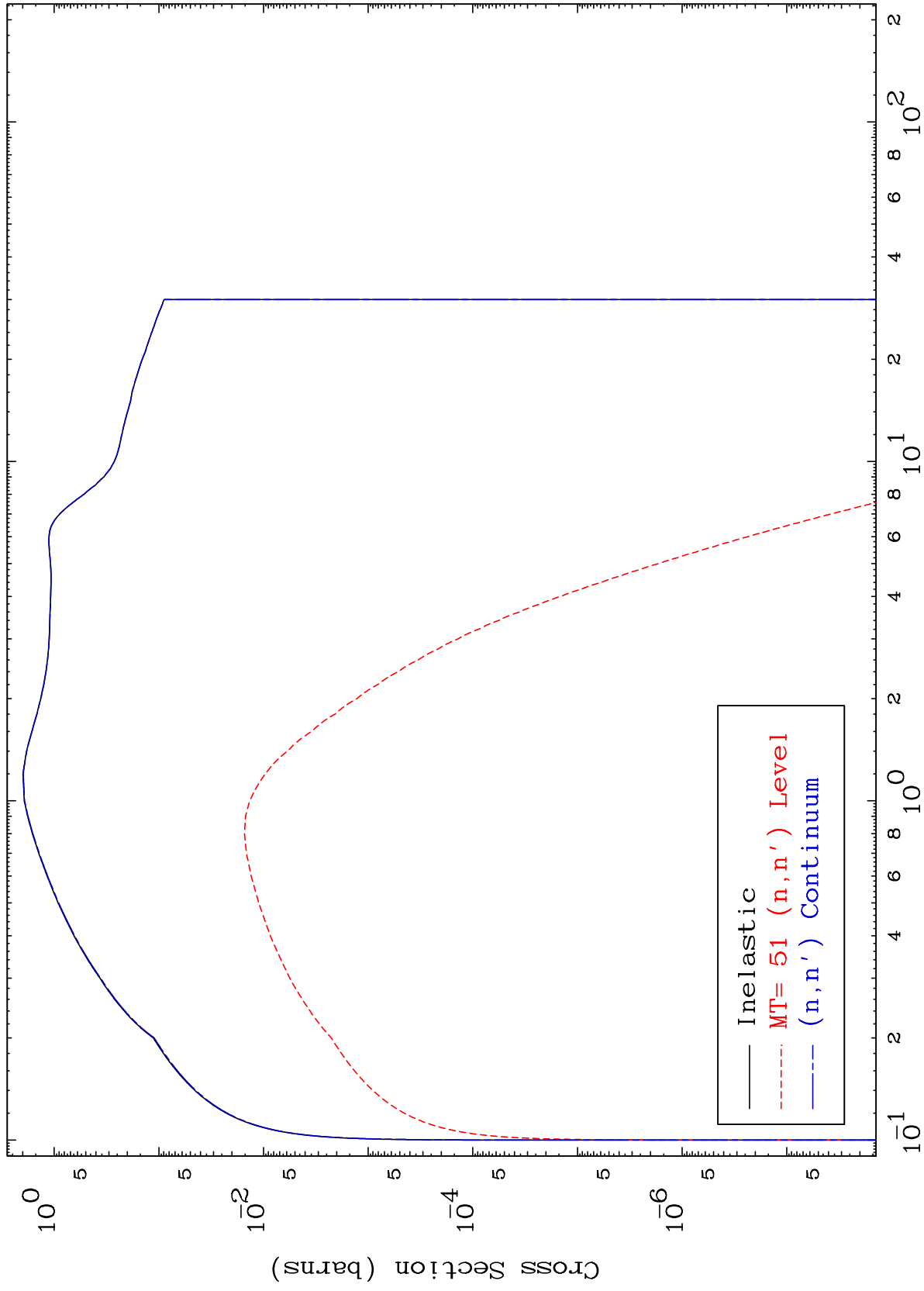




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(n,n') Level
293 Kelvin Cross Sections

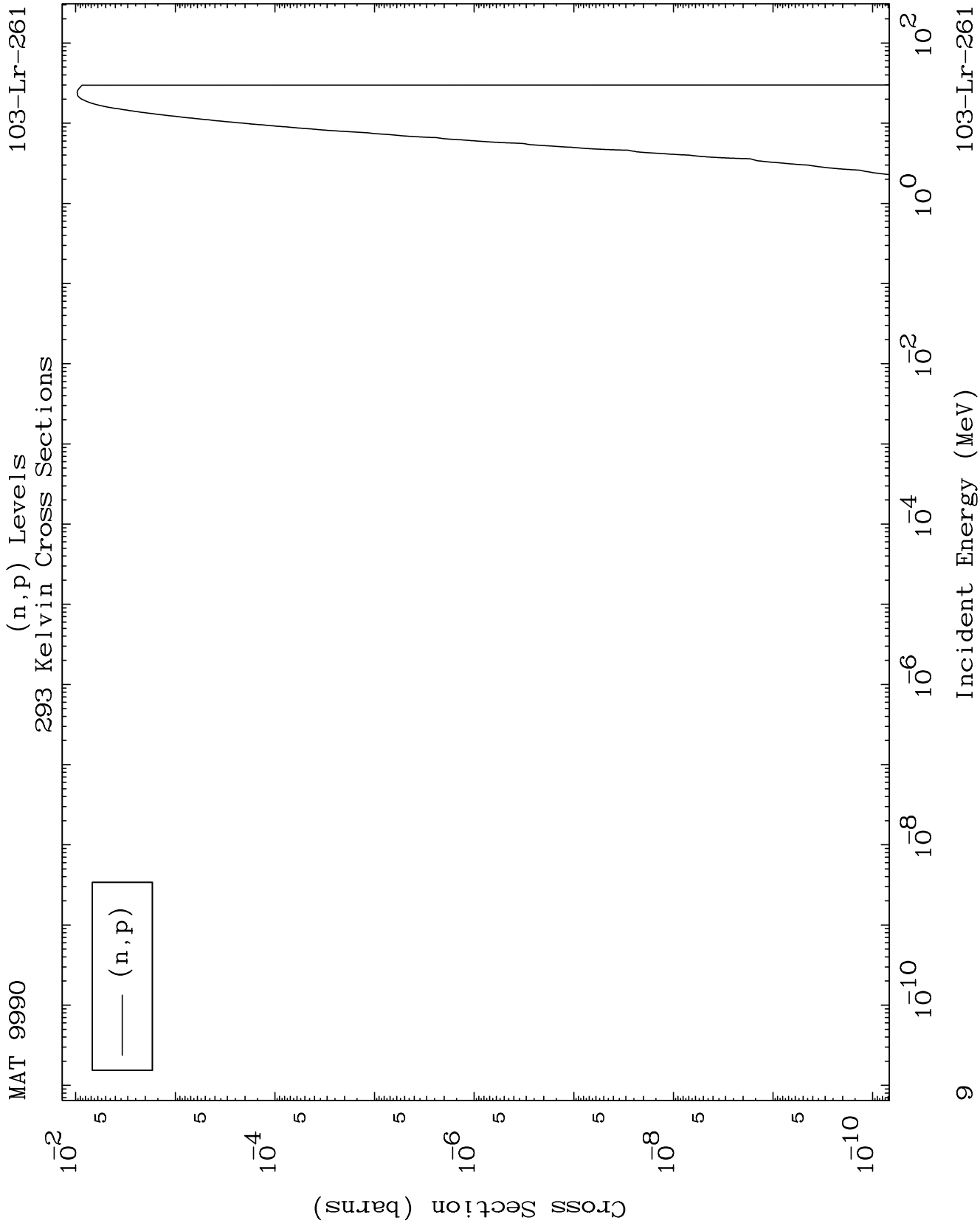
103-Lr-261

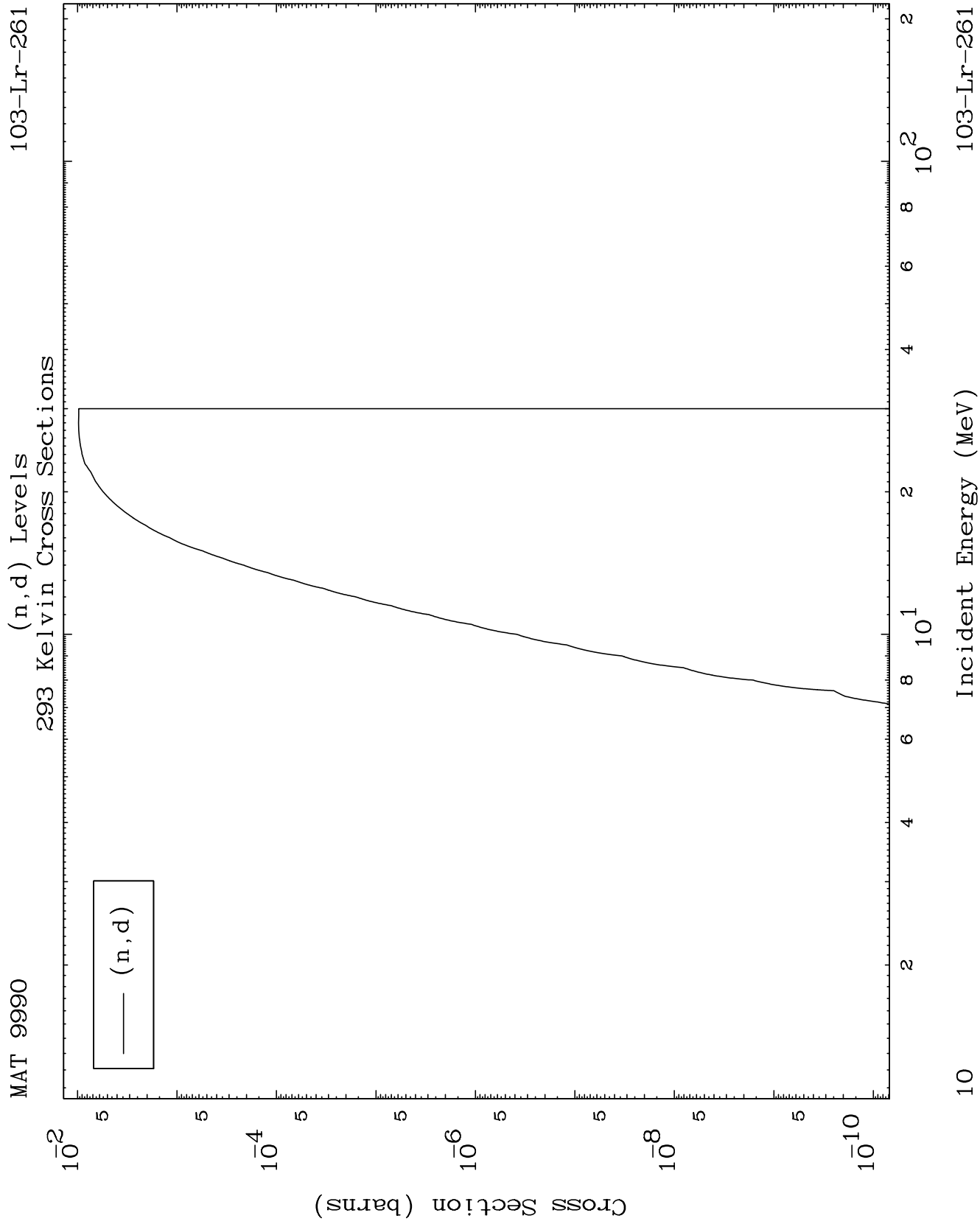


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Incident Energy (MeV)

103-Lr-261

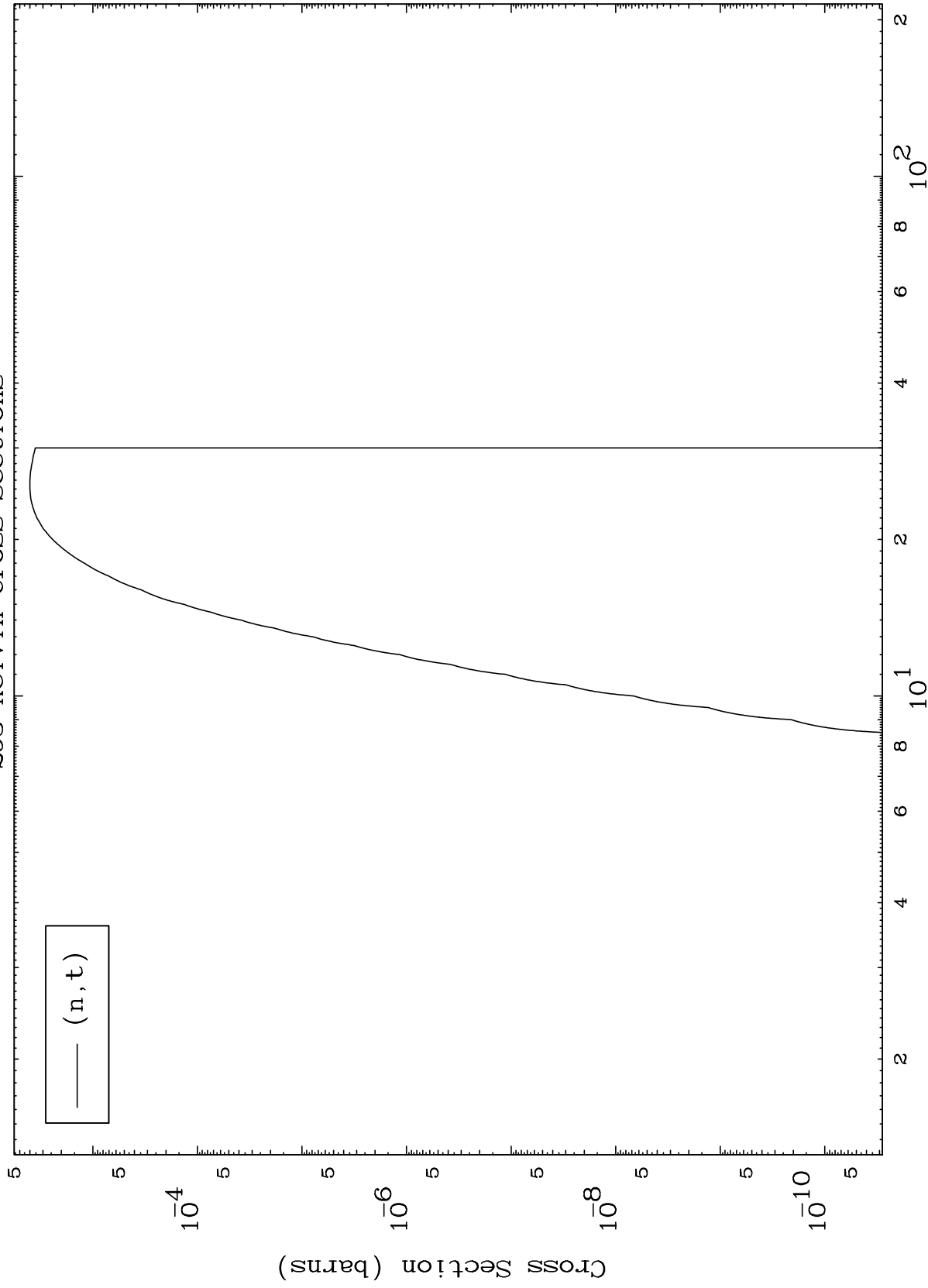




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103-Lr-261

(n,t) Levels
293 Kelvin Cross Sections



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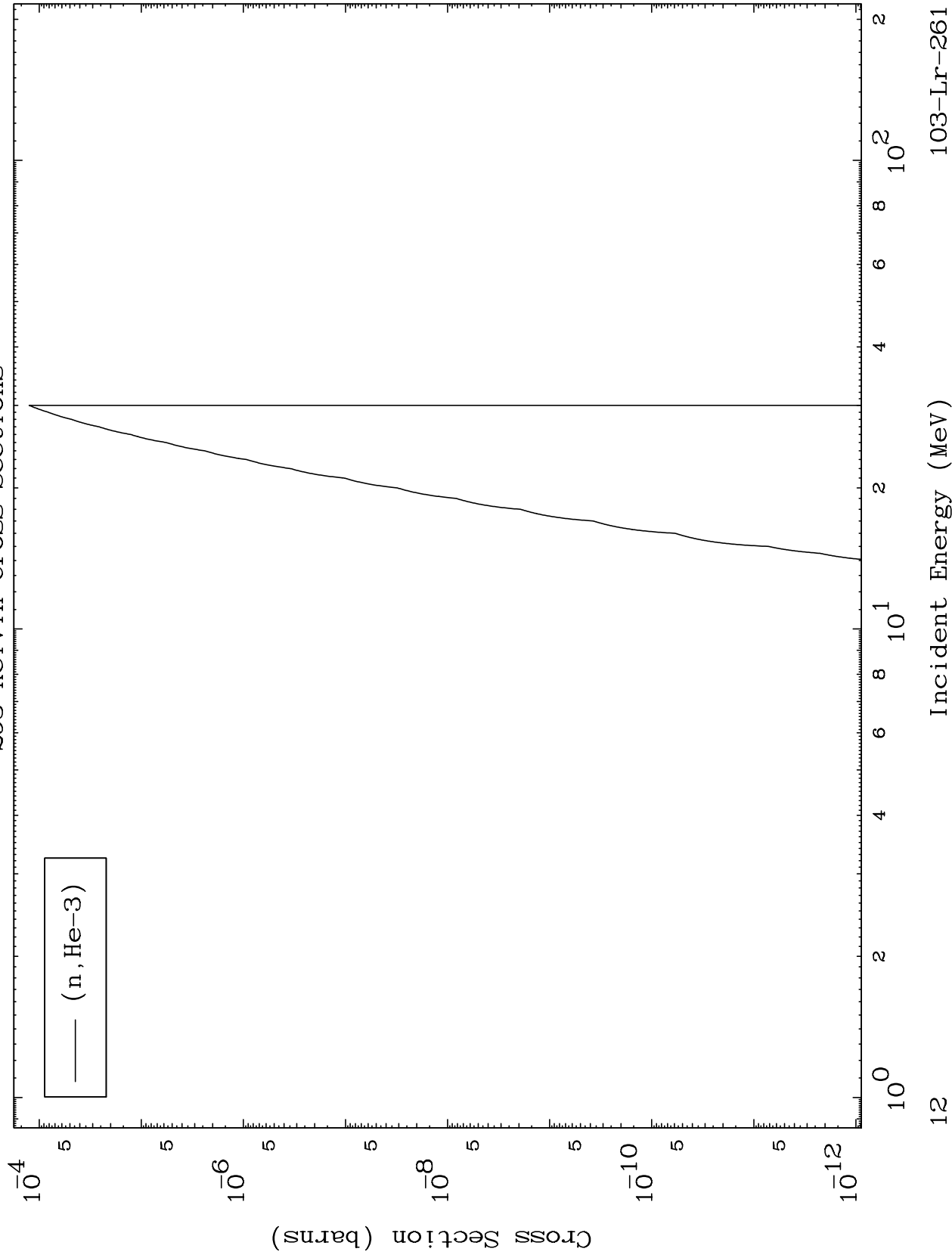
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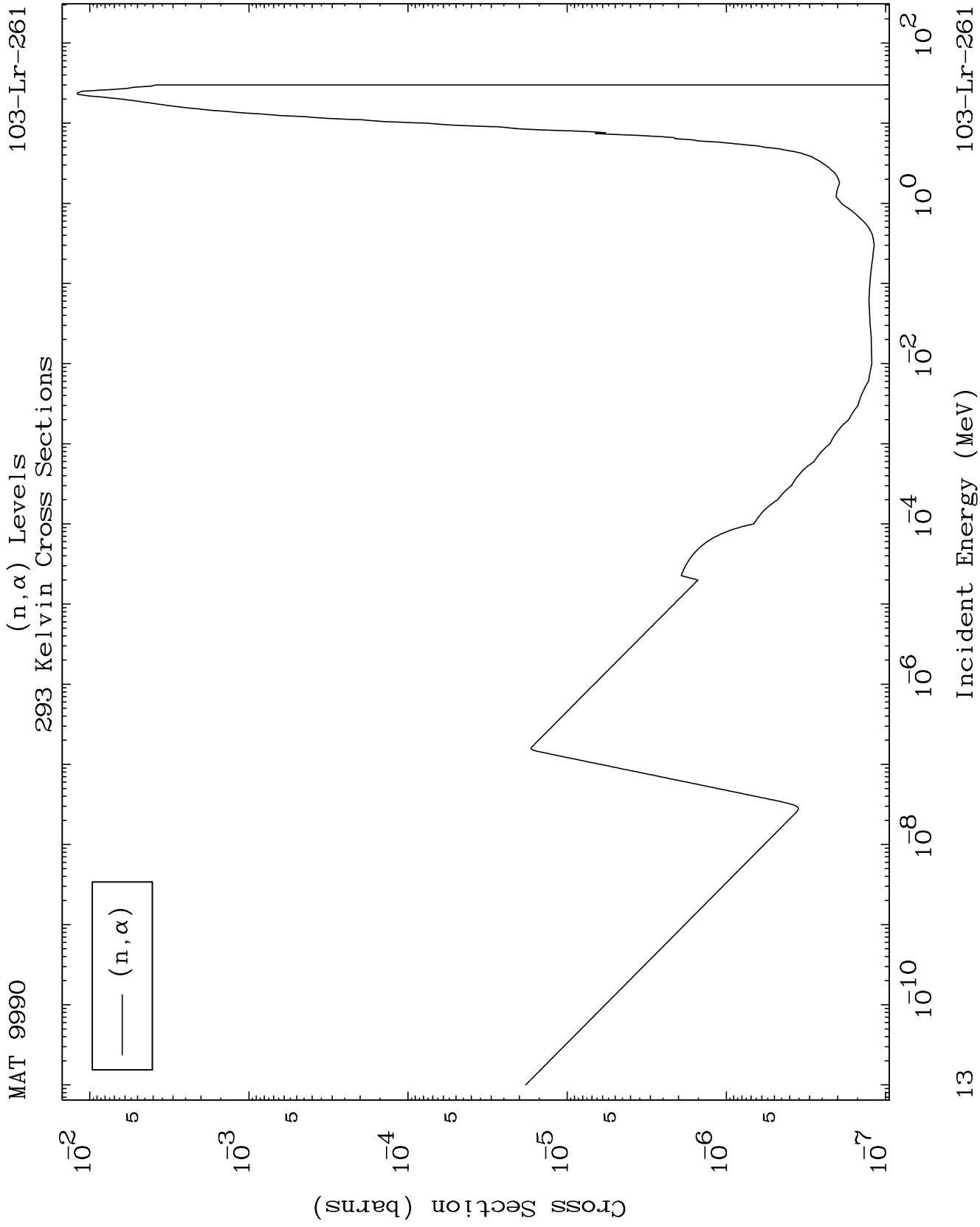
MAT 9990

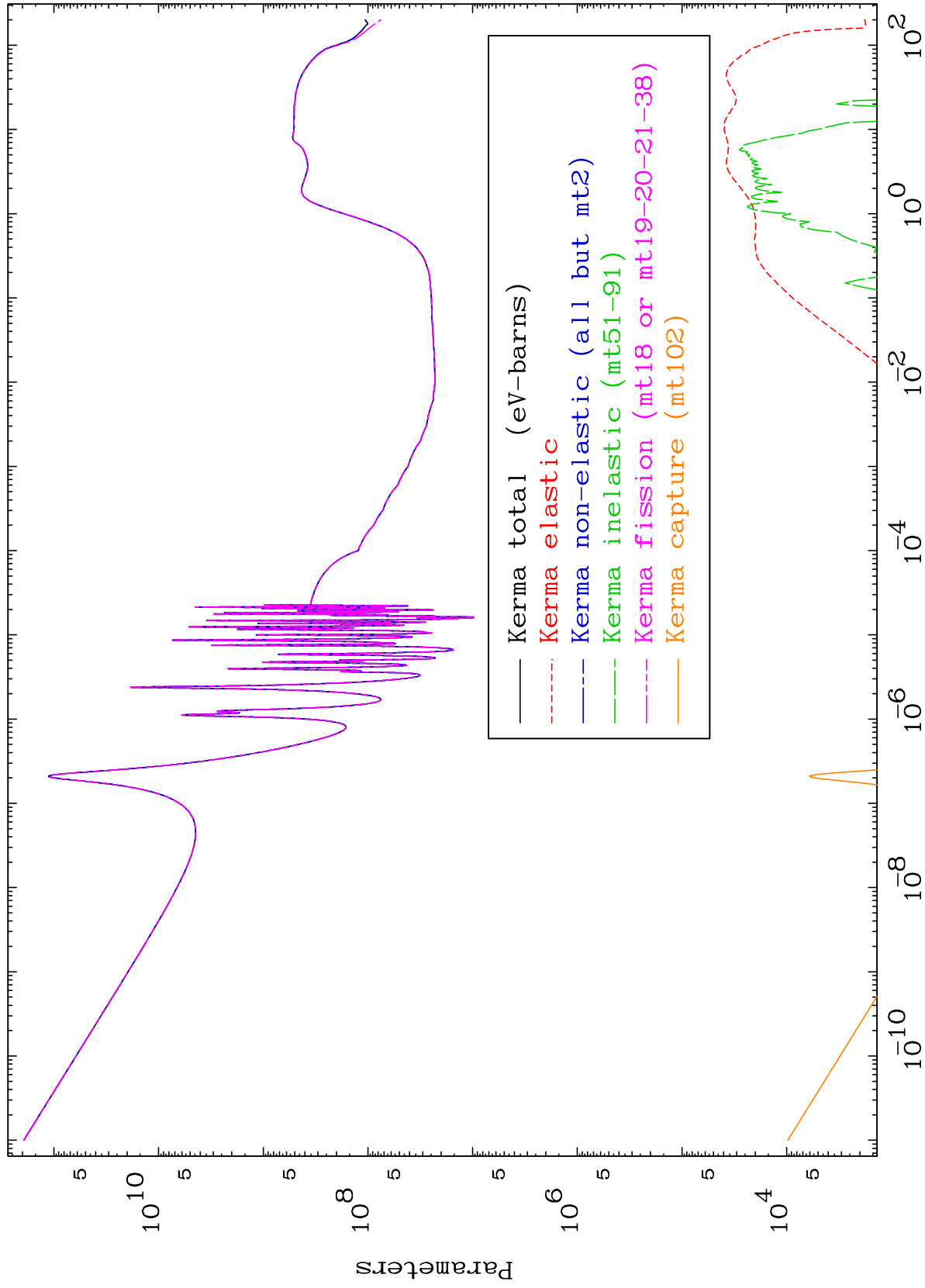
(n,He3) Levels

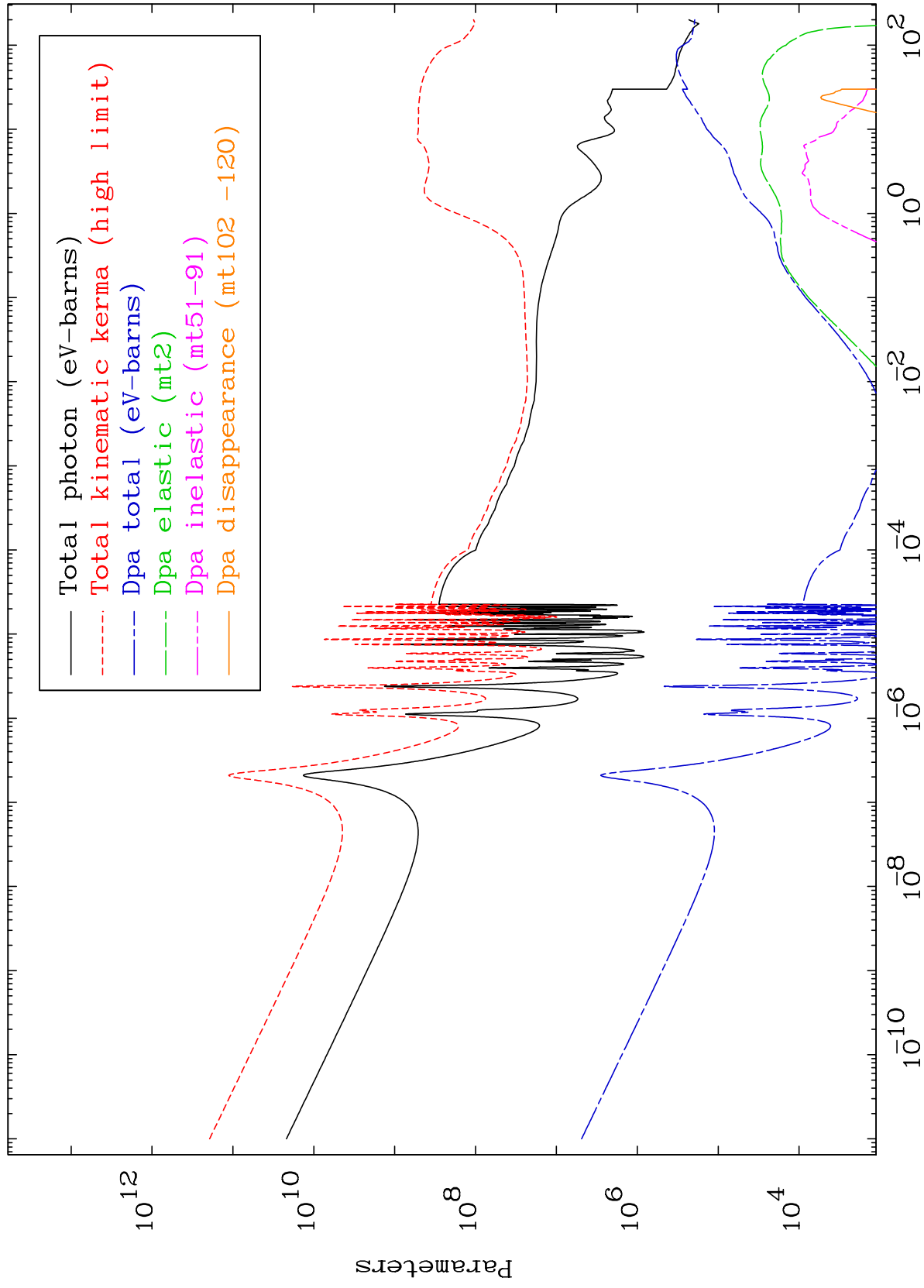
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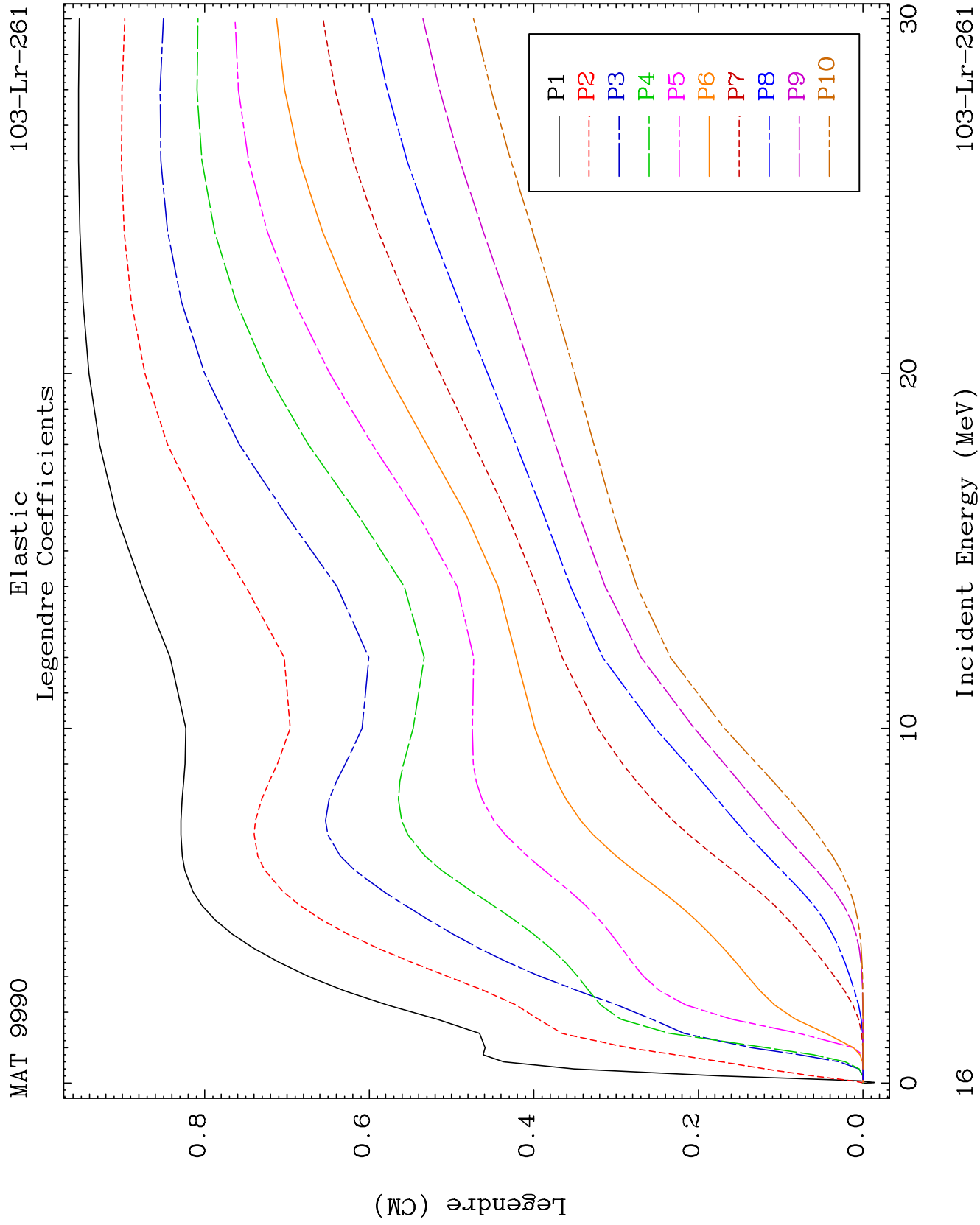
293 Kelvin Cross Sections

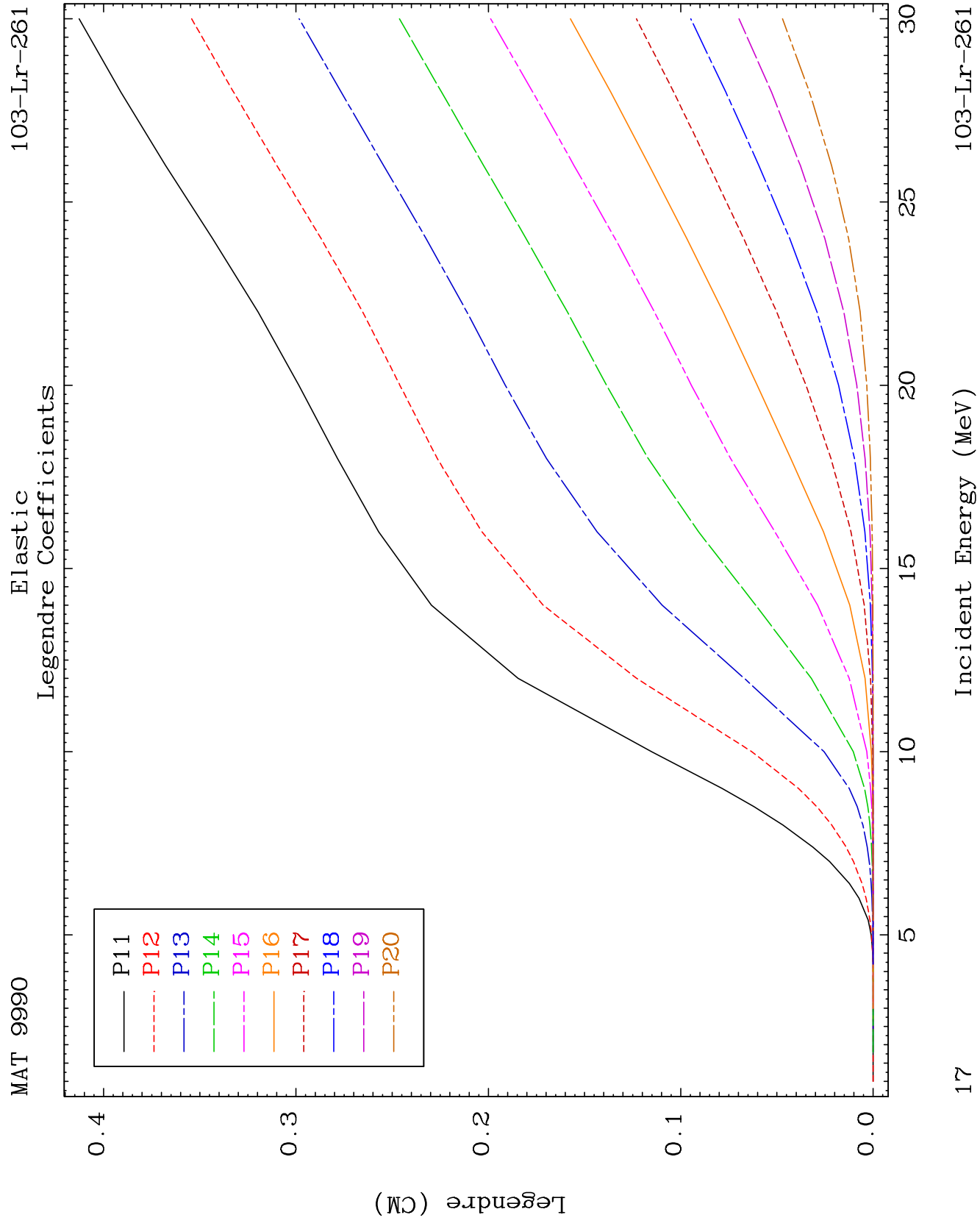


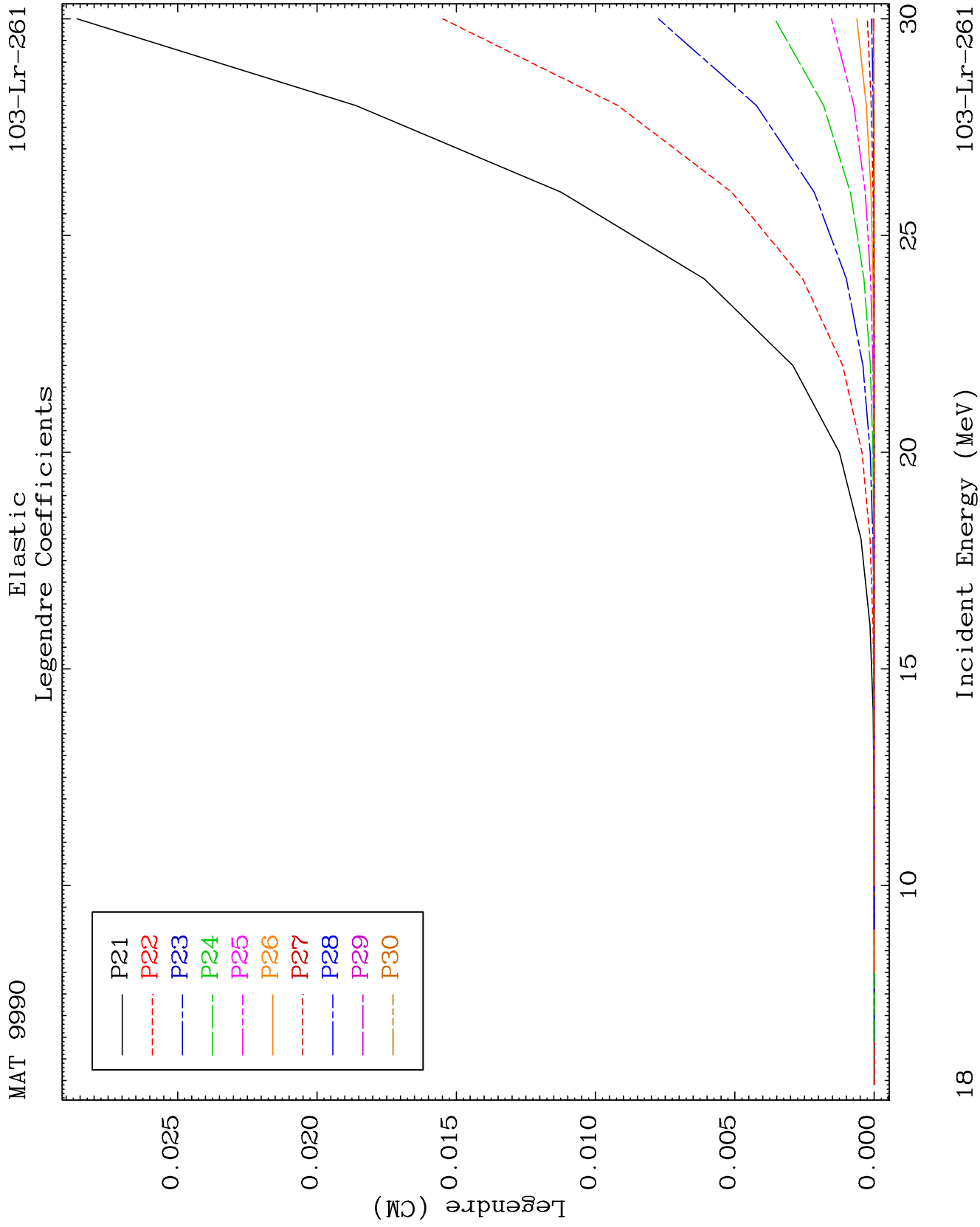


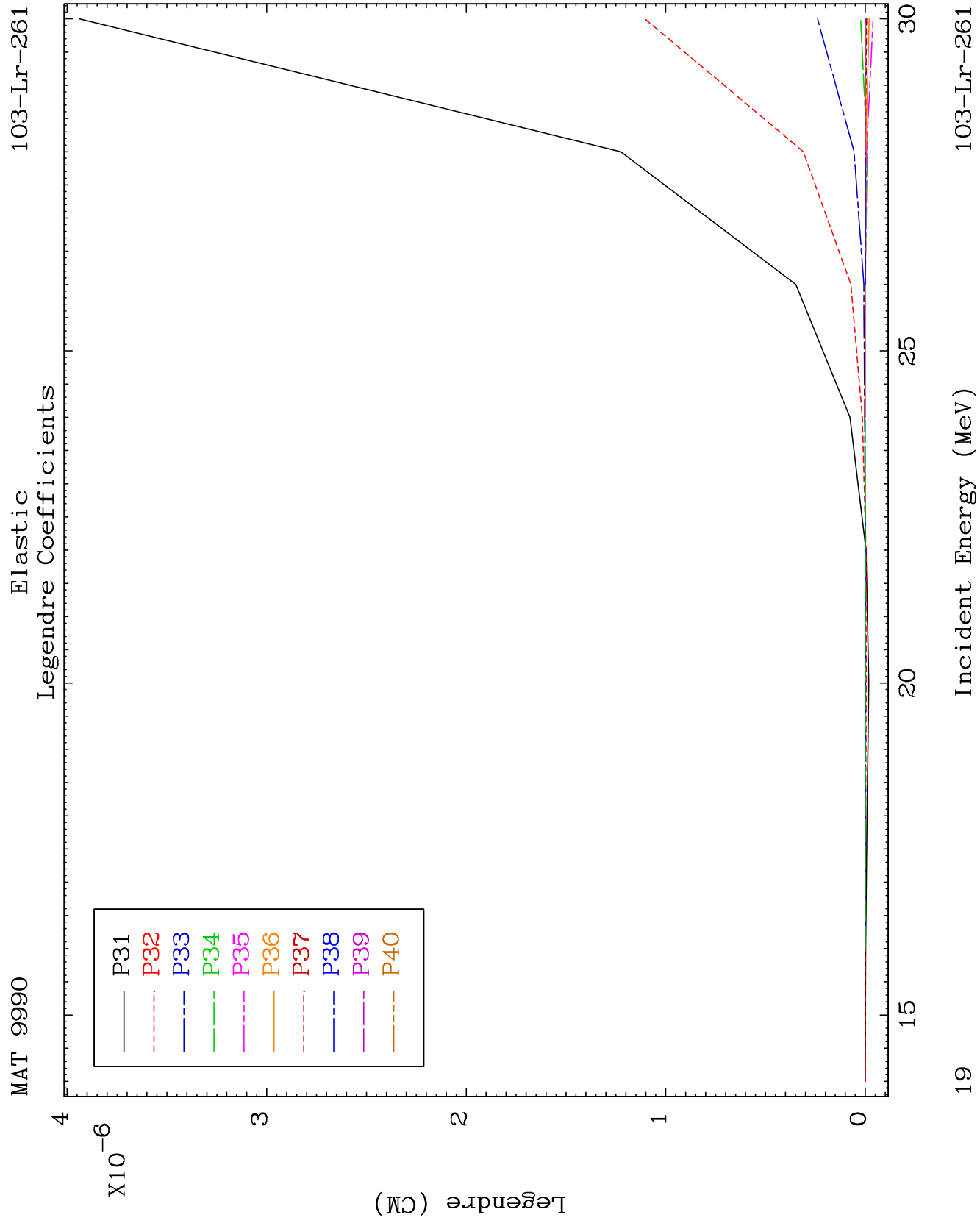








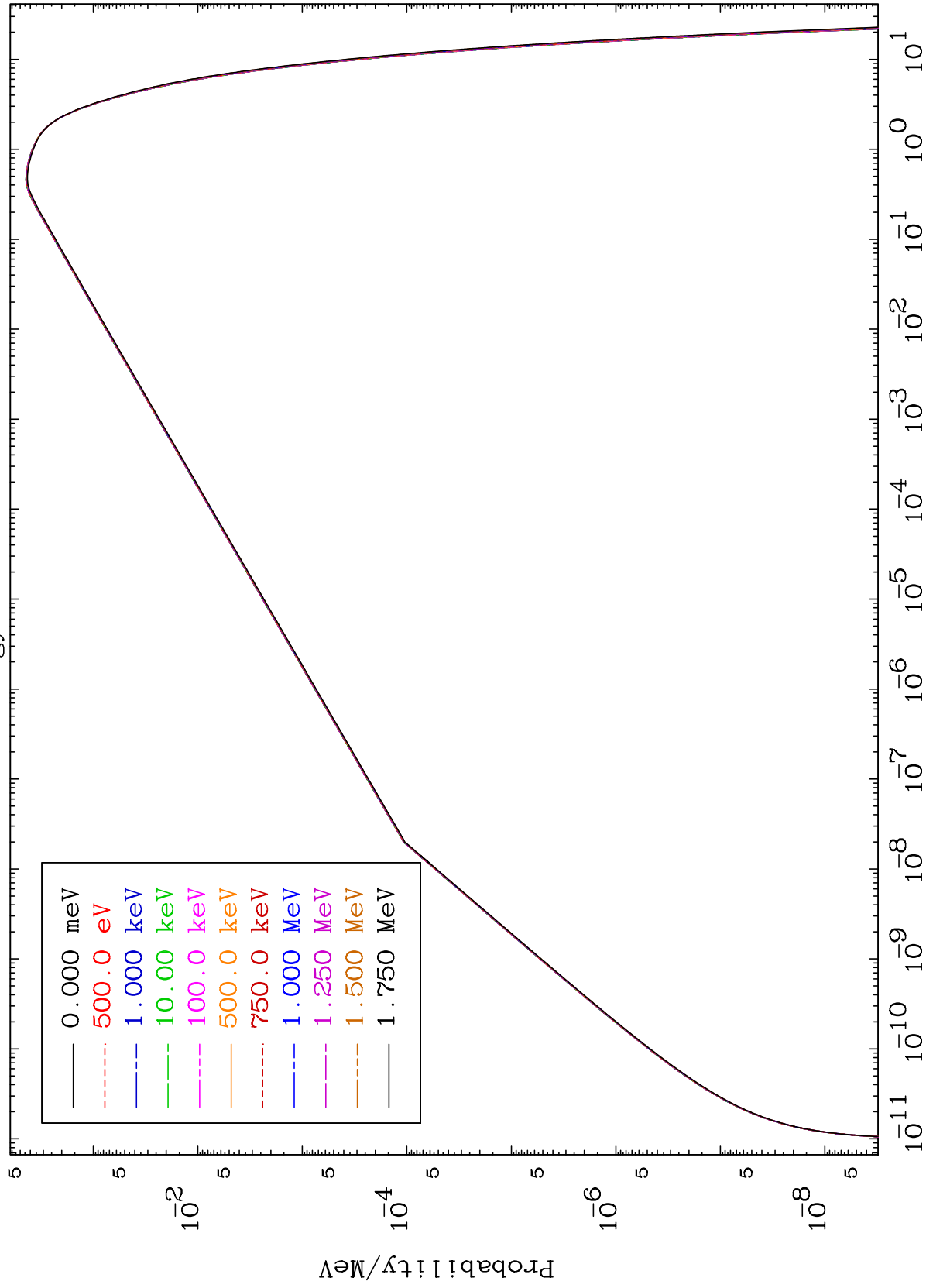




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Fission
Energy Distribution

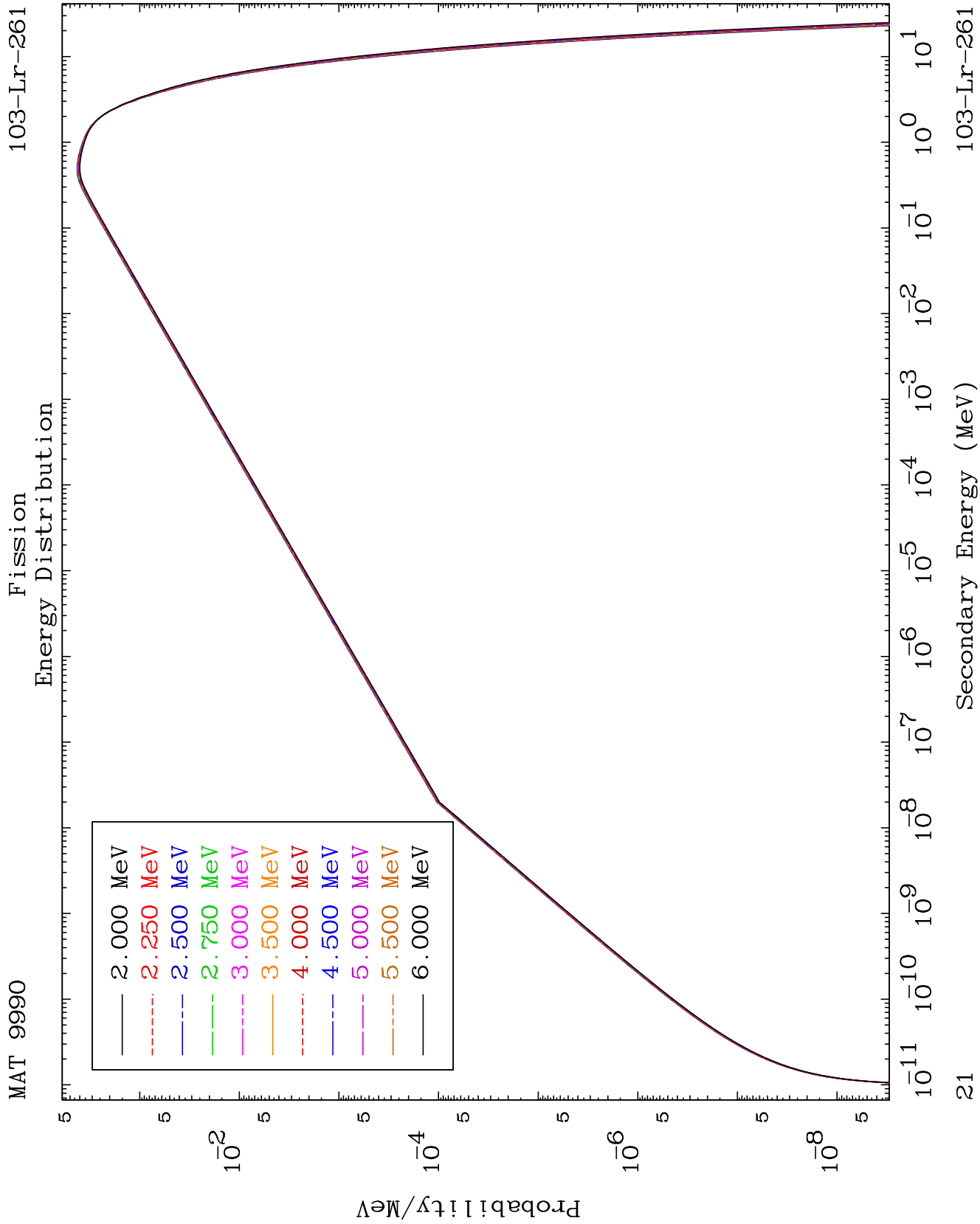
$^{103}\text{--}\text{Lr--}^{261}$



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Secondary Energy (MeV)

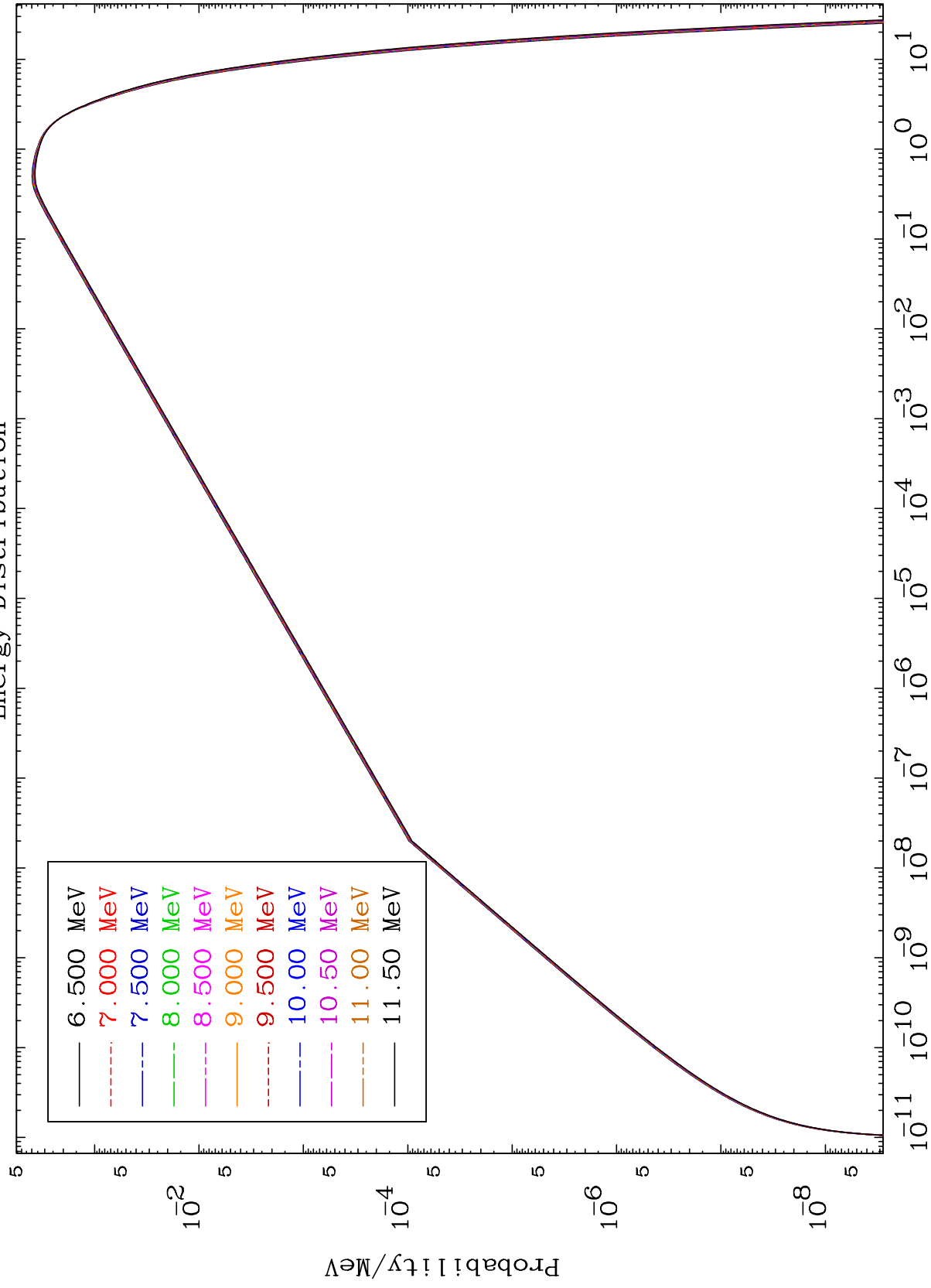
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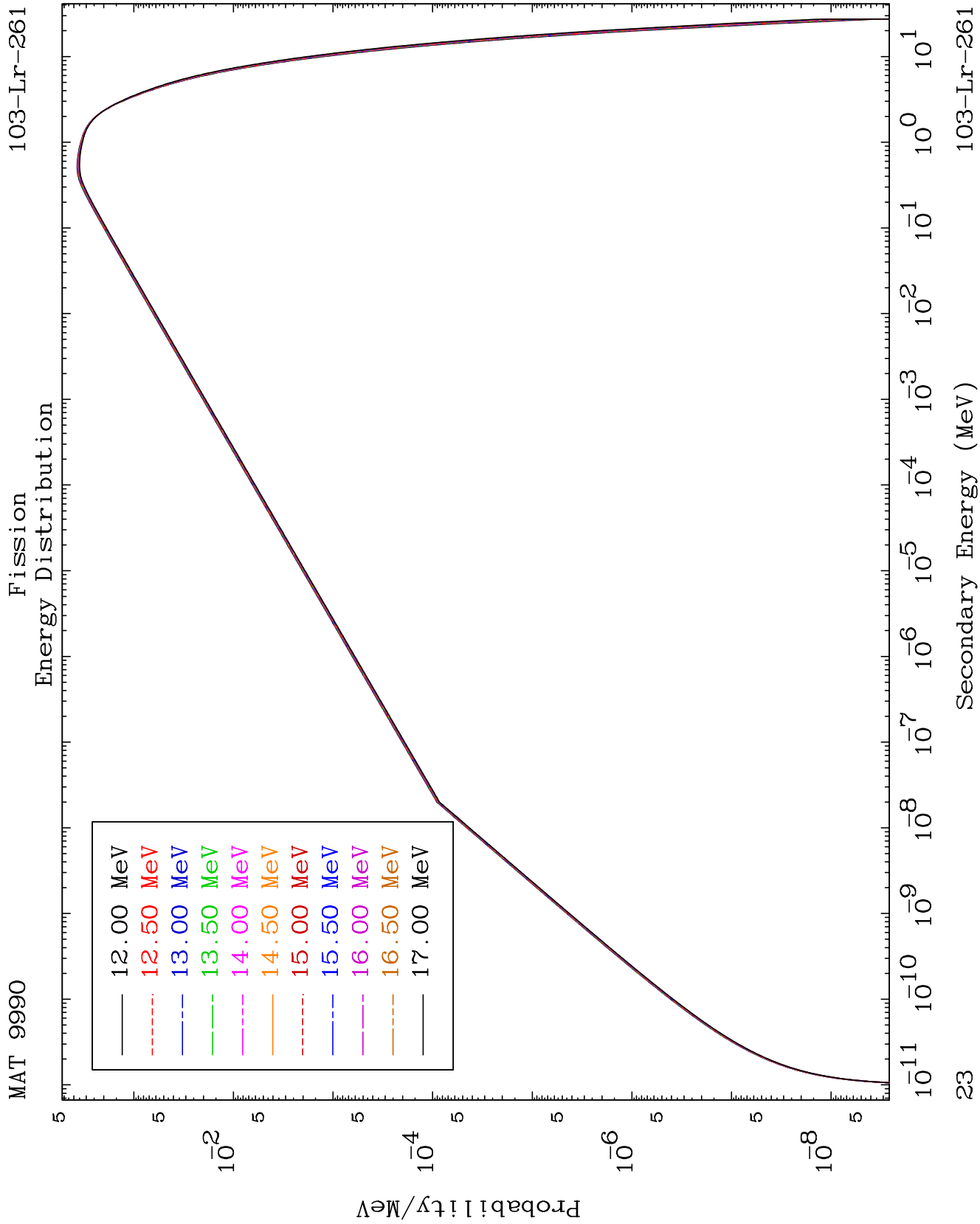


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Fission
Energy Distribution

103-Lr-261

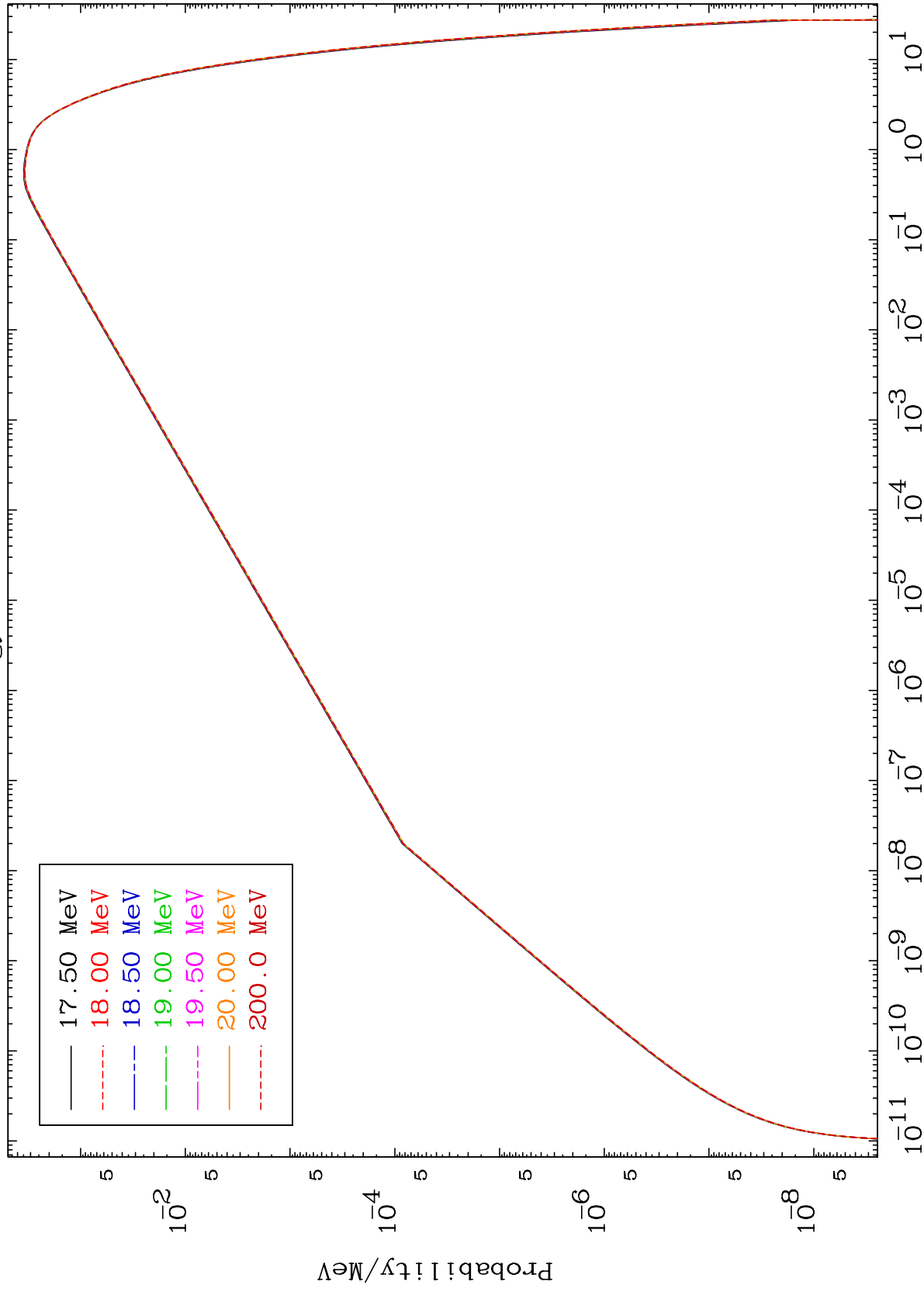




MAT 9990

Fission
Energy Distribution

103-Lr-261



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Secondary Energy (MeV)

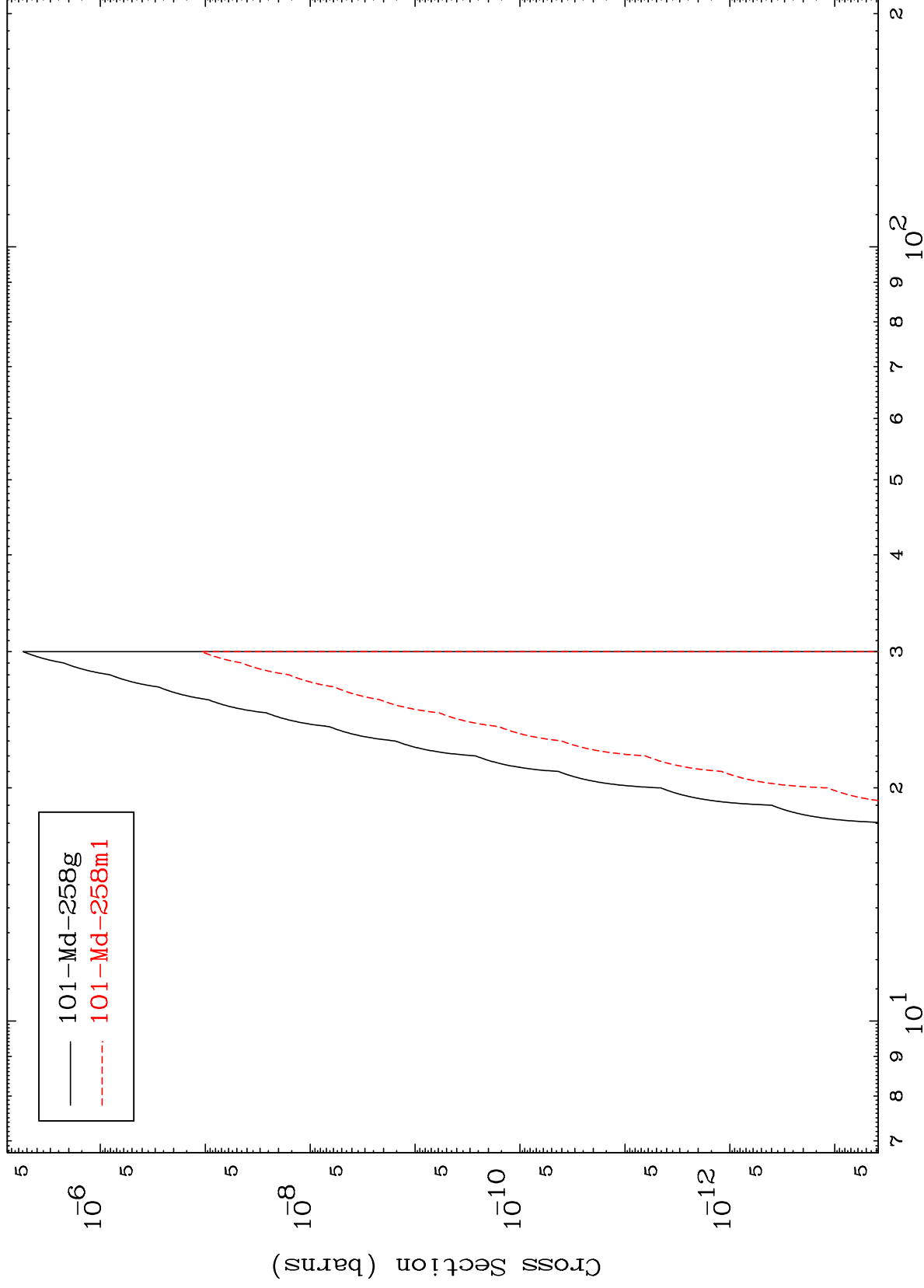
103-Lr-261

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(n,n') He-3

103-Lr-261

Radionuclide Production Cross Section



25

Incident Energy (MeV)

103-Lr-261

