

Program EVALPLOT  
(Version 2018-1)

by

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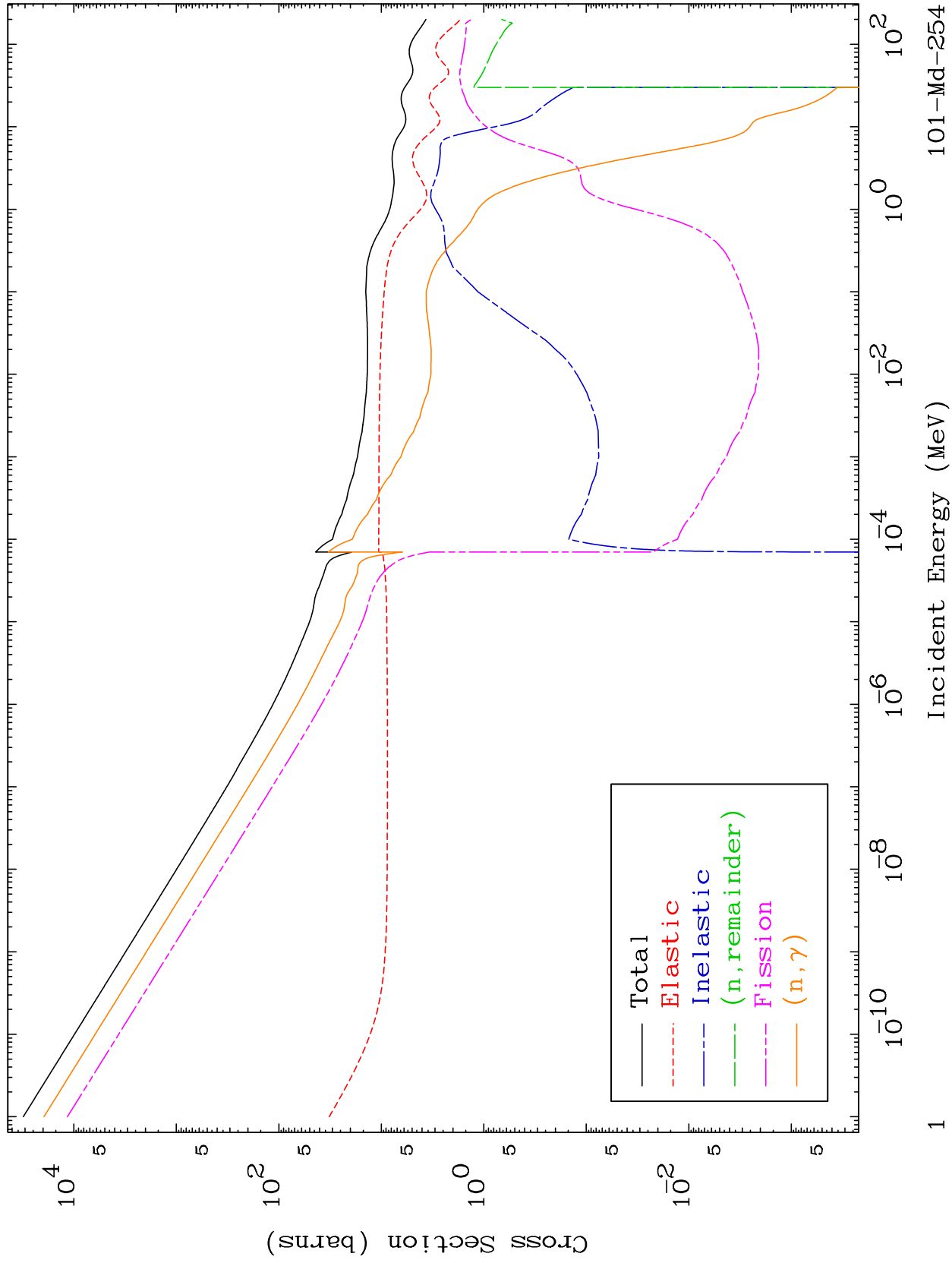
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9960

Major  
293 Kelvin Cross Sections

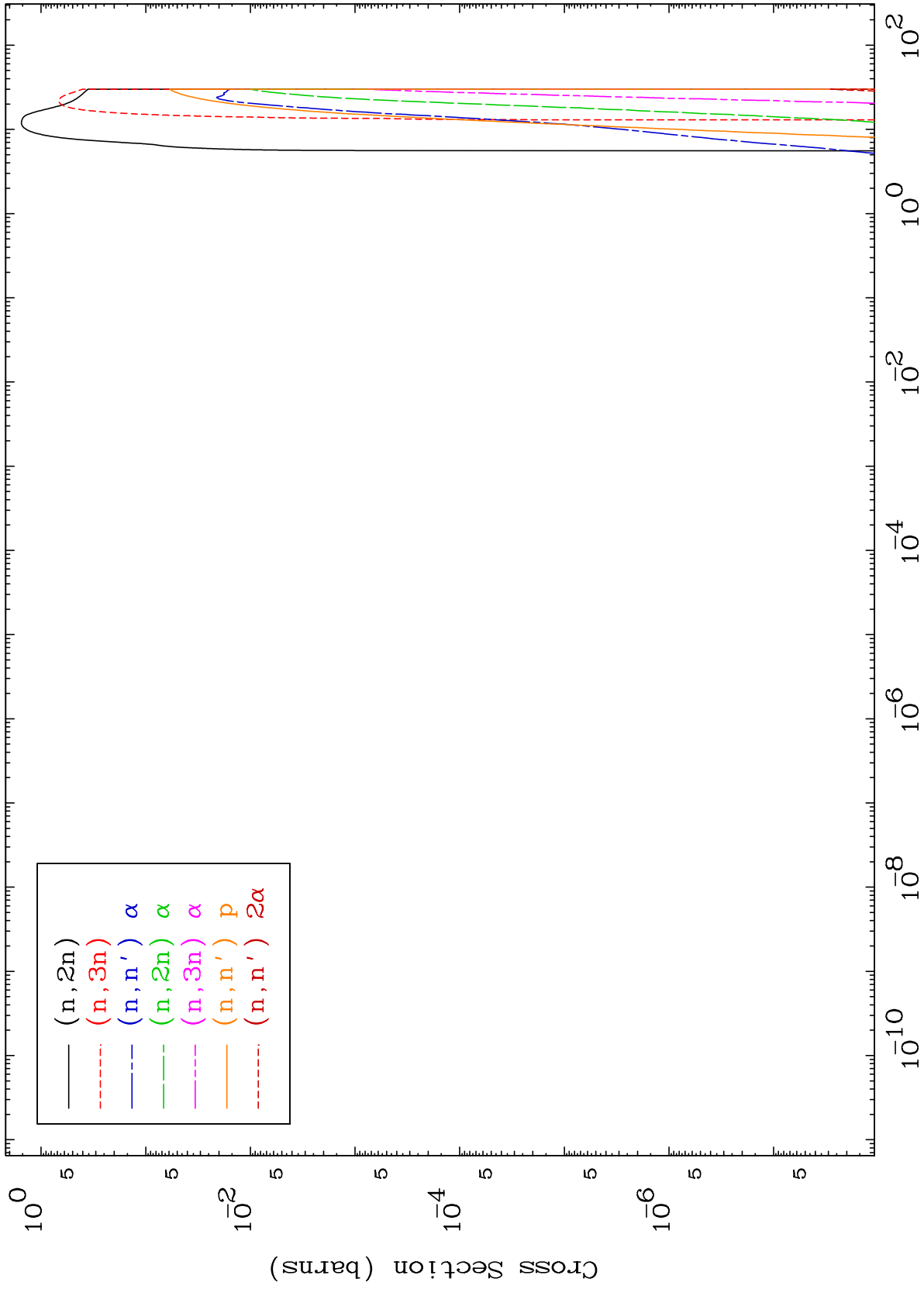
101-Md-254



MAT 9960

Neutron Production  
293 Kelvin Cross Sections

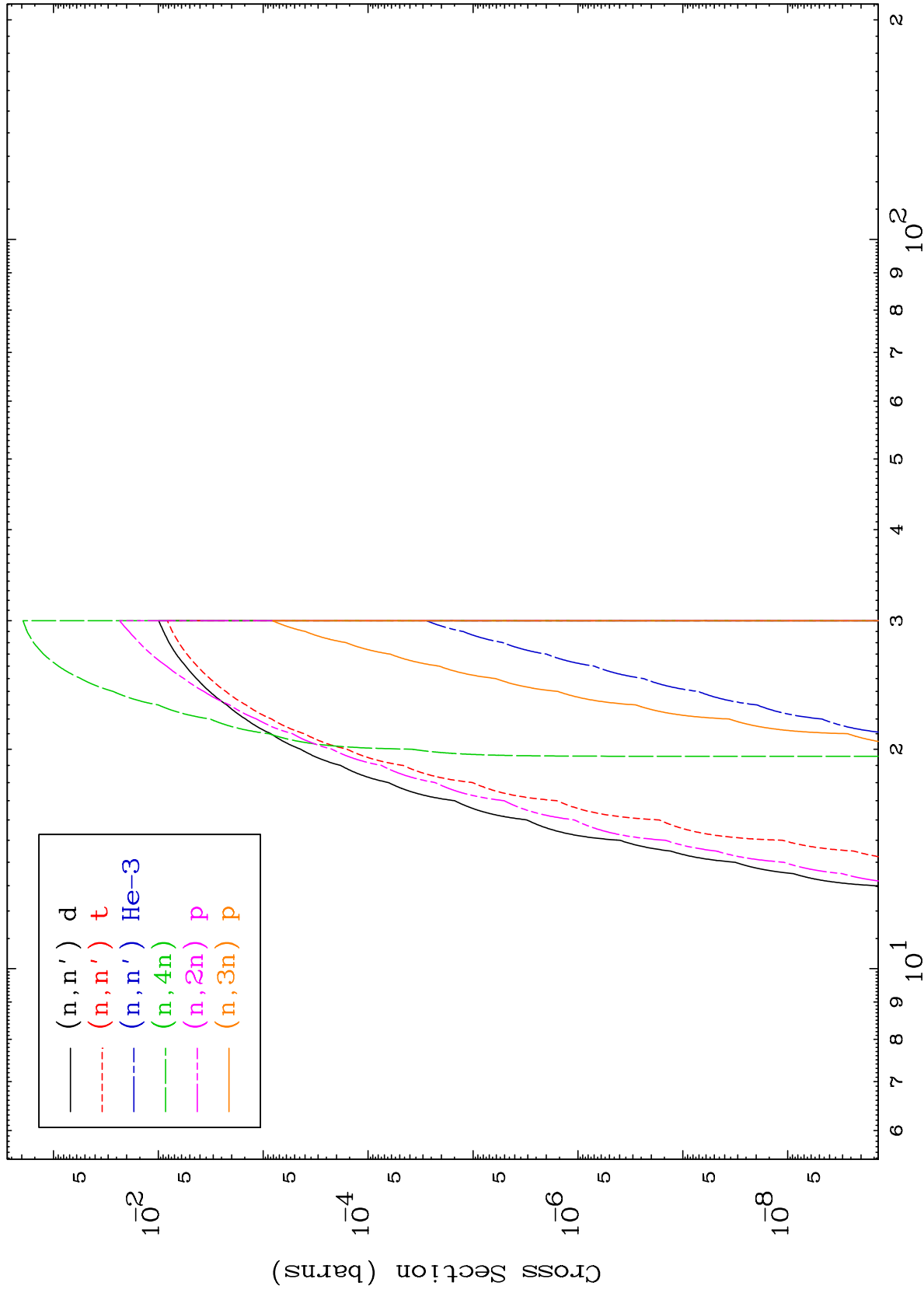
101-Md-254



MAT 9960

Neutron Production  
293 Kelvin Cross Sections

101-Md-254



3

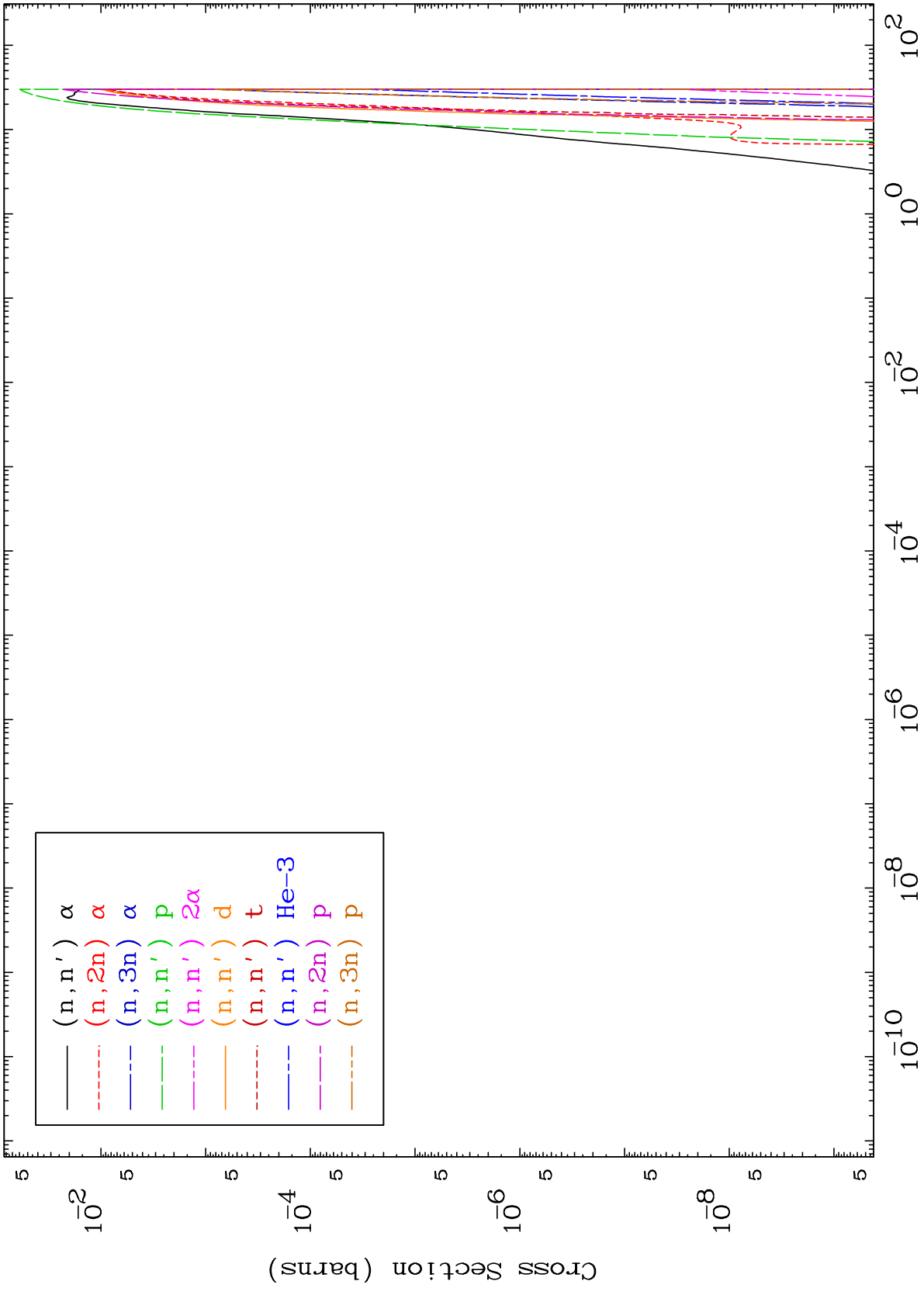
Incident Energy (MeV)

101-Md-254

MAT 9960

293 Charged Particle  
Kelvin Cross Sections

101-Md-254

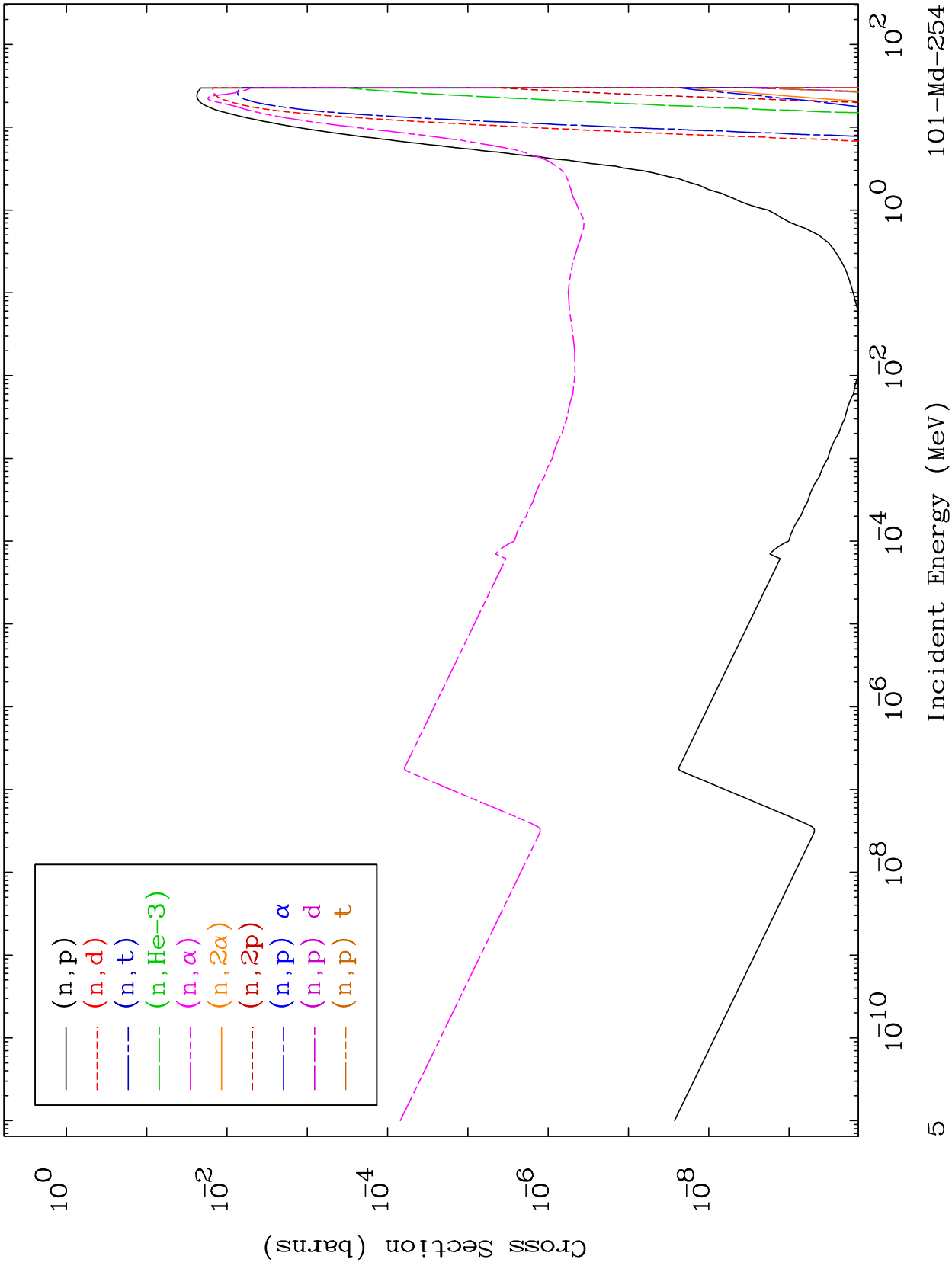


101-Md-254

MAT 9960

293 Kelvin Cross Sections

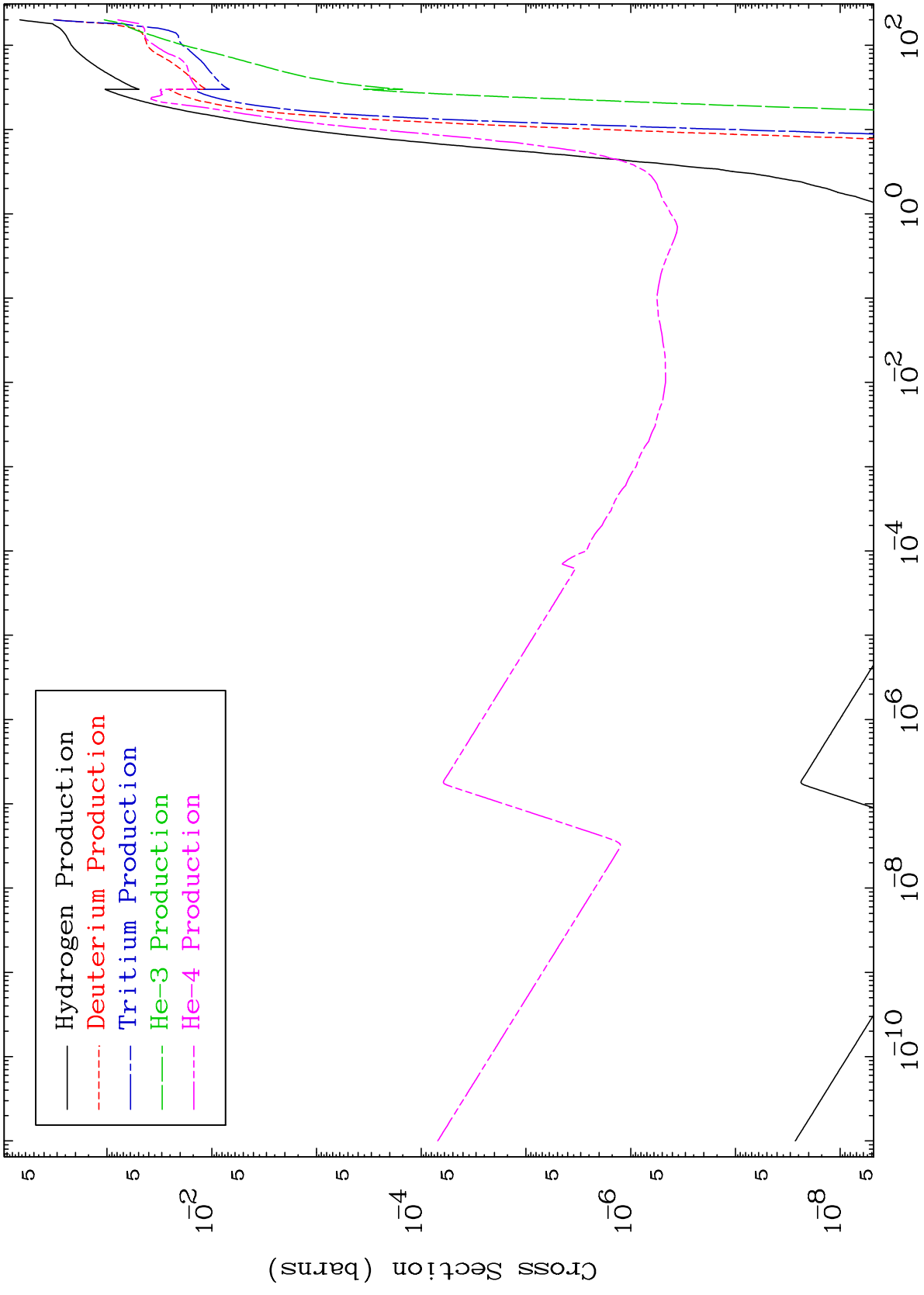
101-Md-254



MAT 9960

Particle Production  
293 Kelvin Cross Sections

101-Md-254



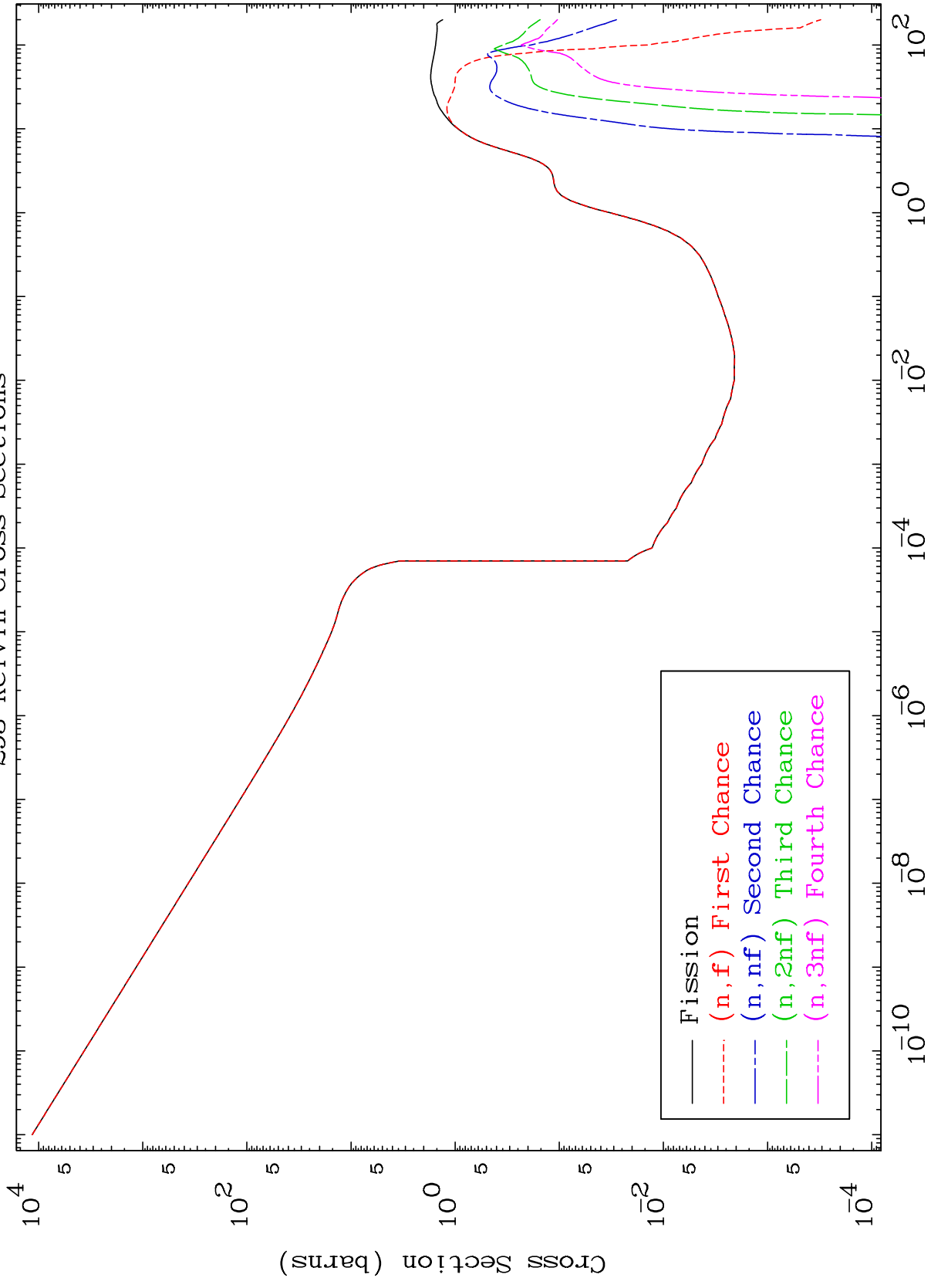
6

101-Md-254

MAT 9960

Fission  
293 Kelvin Cross Sections

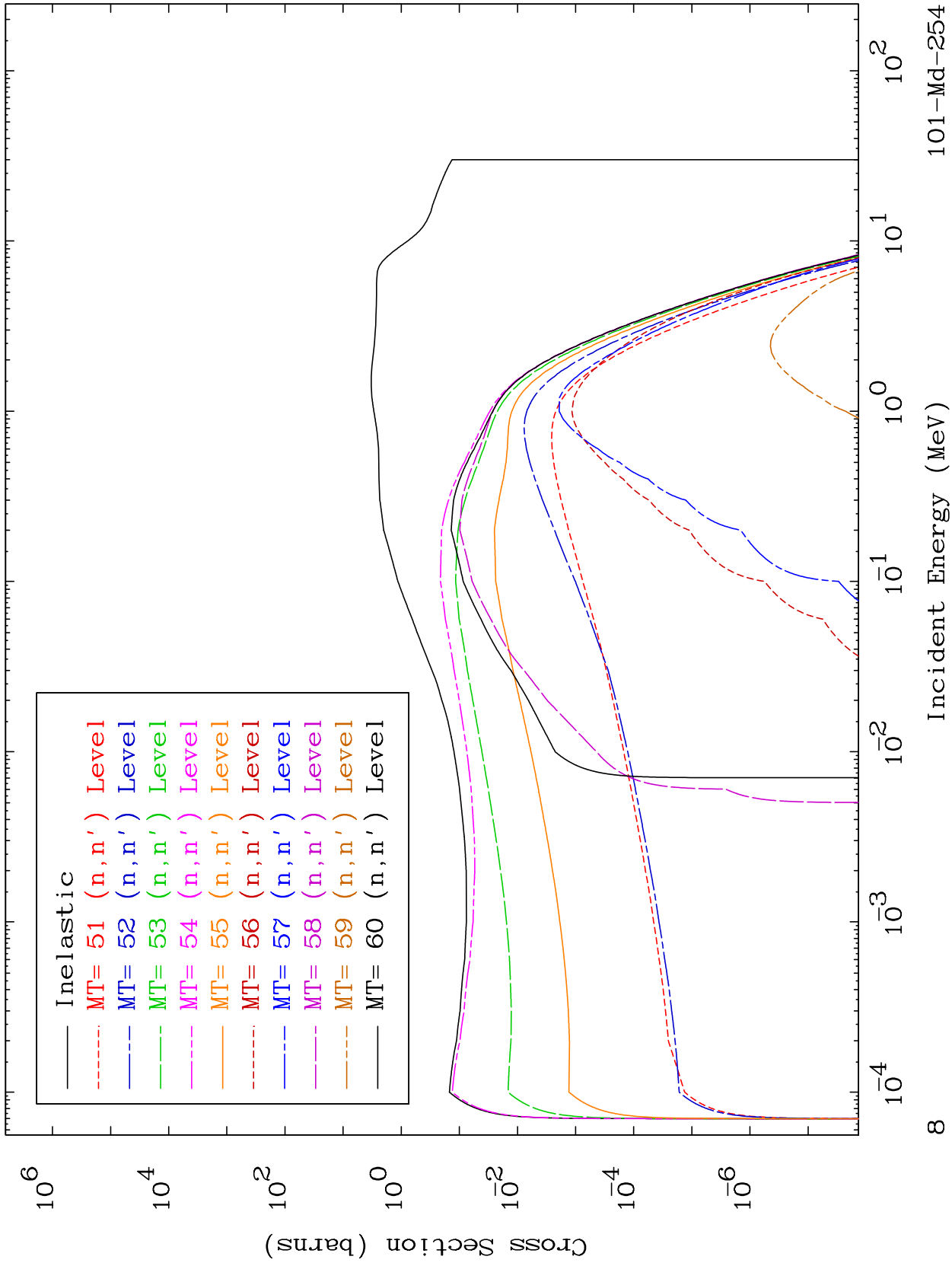
101-Md-254



MAT 9960

(n,n') Level  
293 Kelvin Cross Sections

101-Md-254

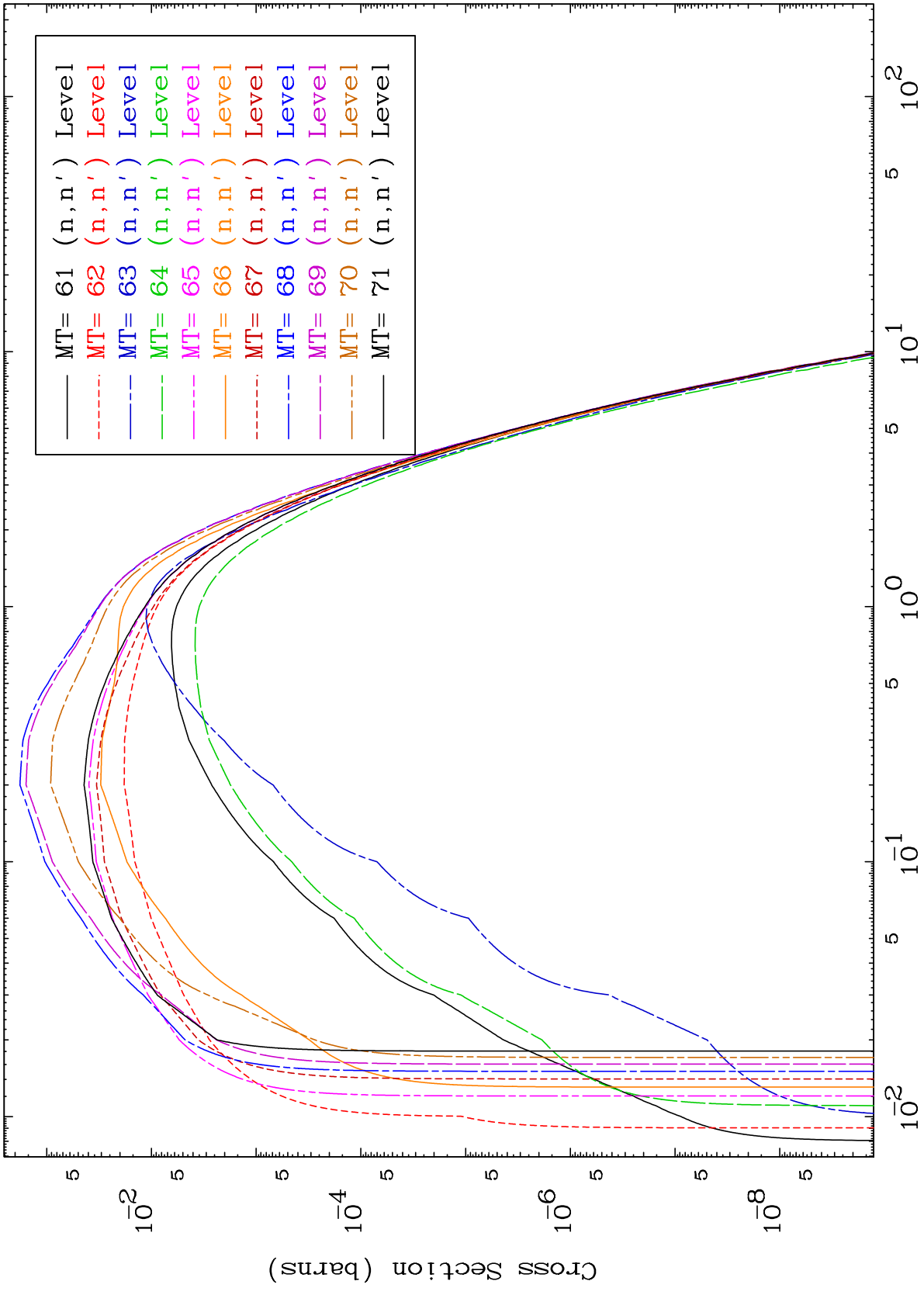


MAT 9960

(n,n') Level

101-Md-254

293 Kelvin Cross Sections



9

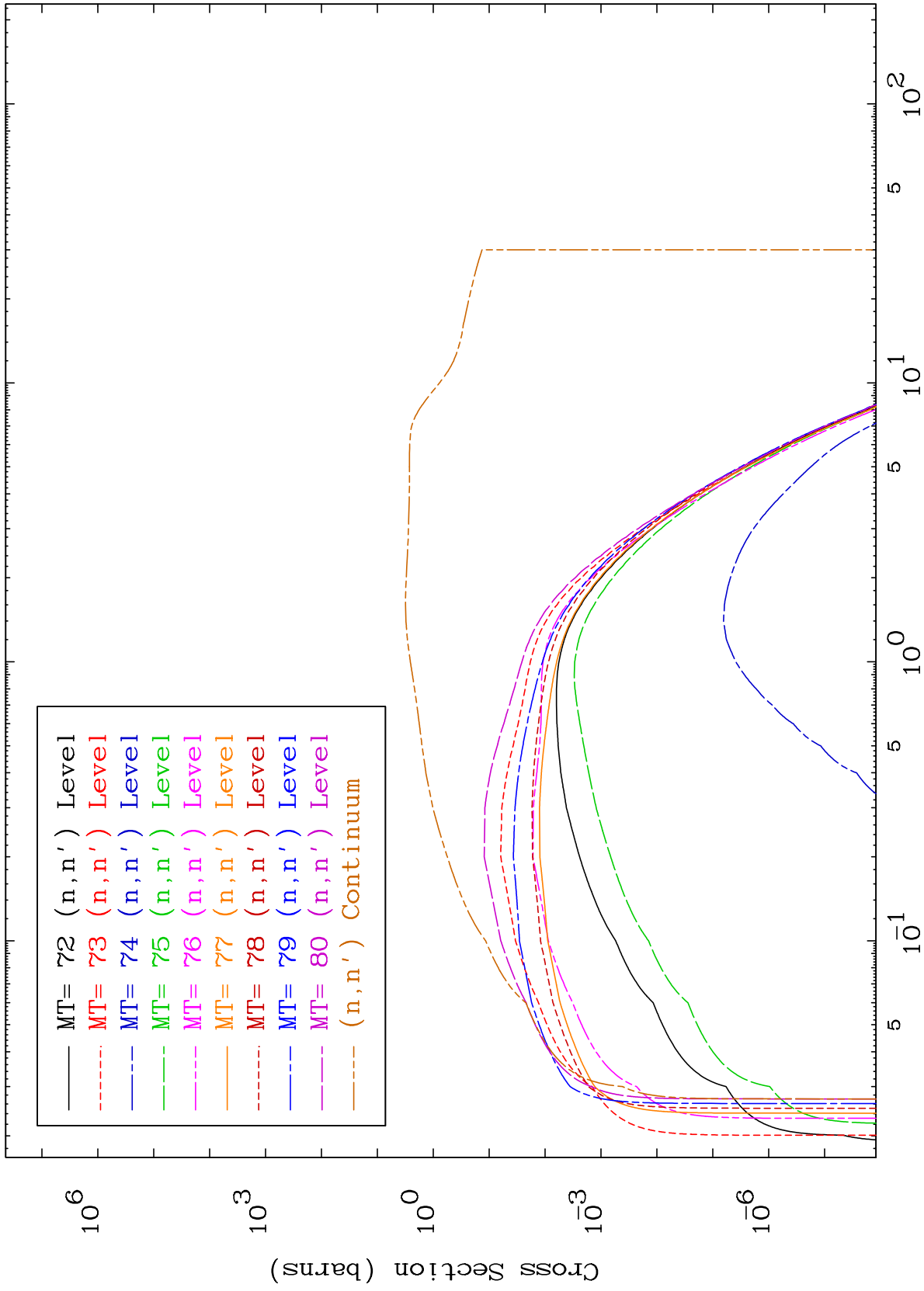
Incident Energy (MeV)

101-Md-254

MAT 9960

(n,n') Level  
293 Kelvin Cross Sections

101-Md-254



10

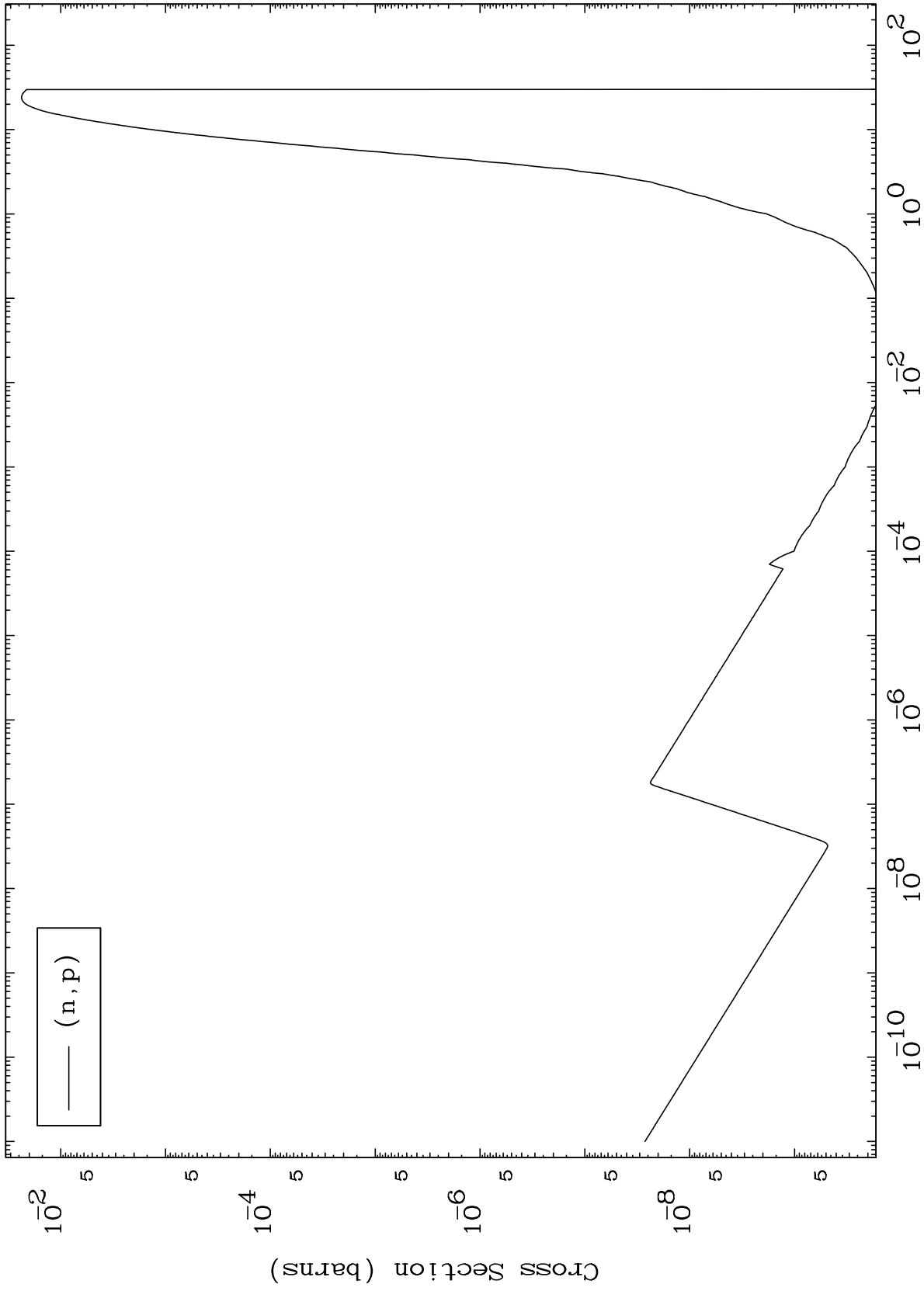
Incident Energy (MeV)

101-Md-254

MAT 9960

(n,p) Levels  
293 Kelvin Cross Sections

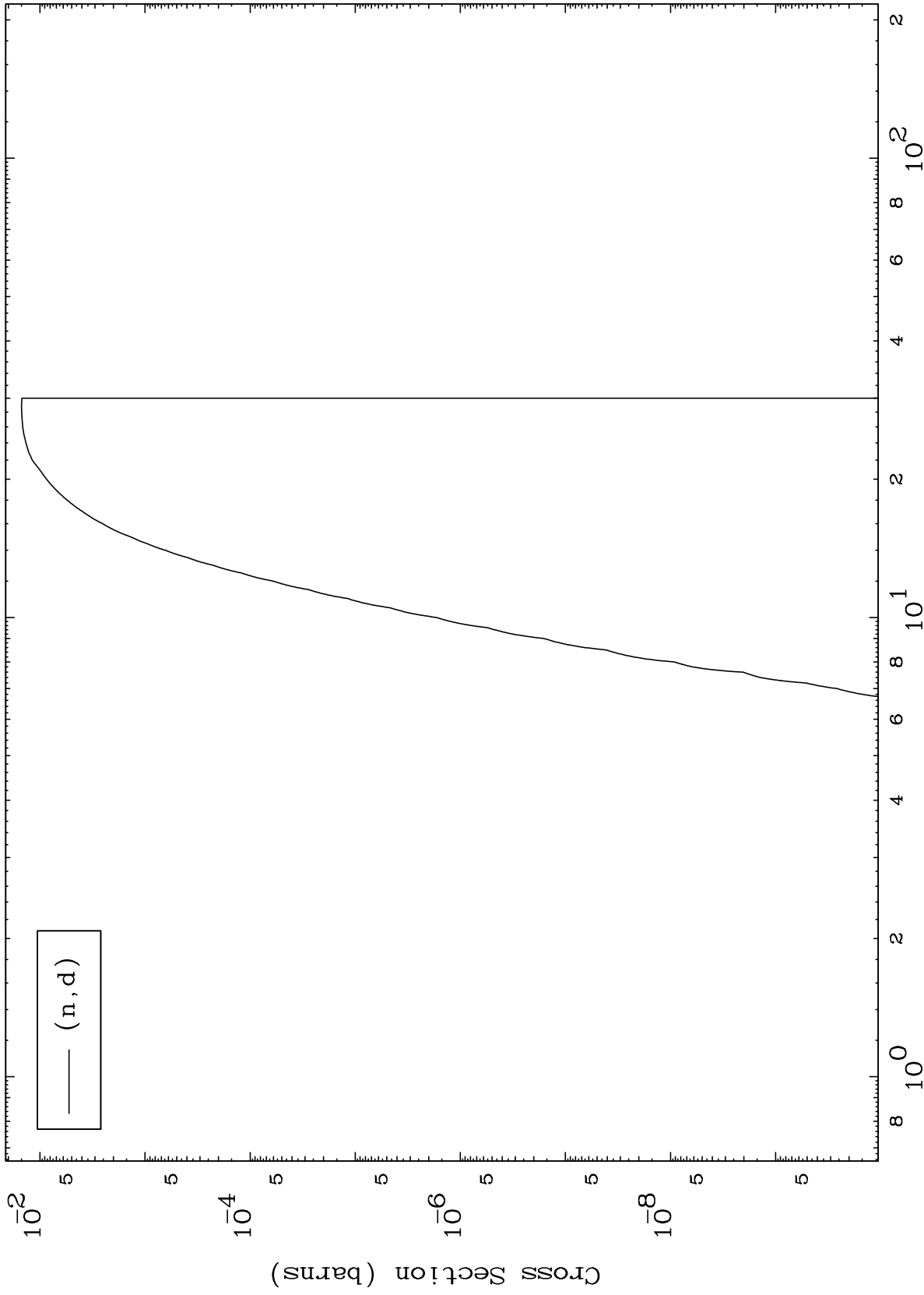
101-Md-254



MAT 9960

(n,d) Levels  
293 Kelvin Cross Sections

101-Md-254



12

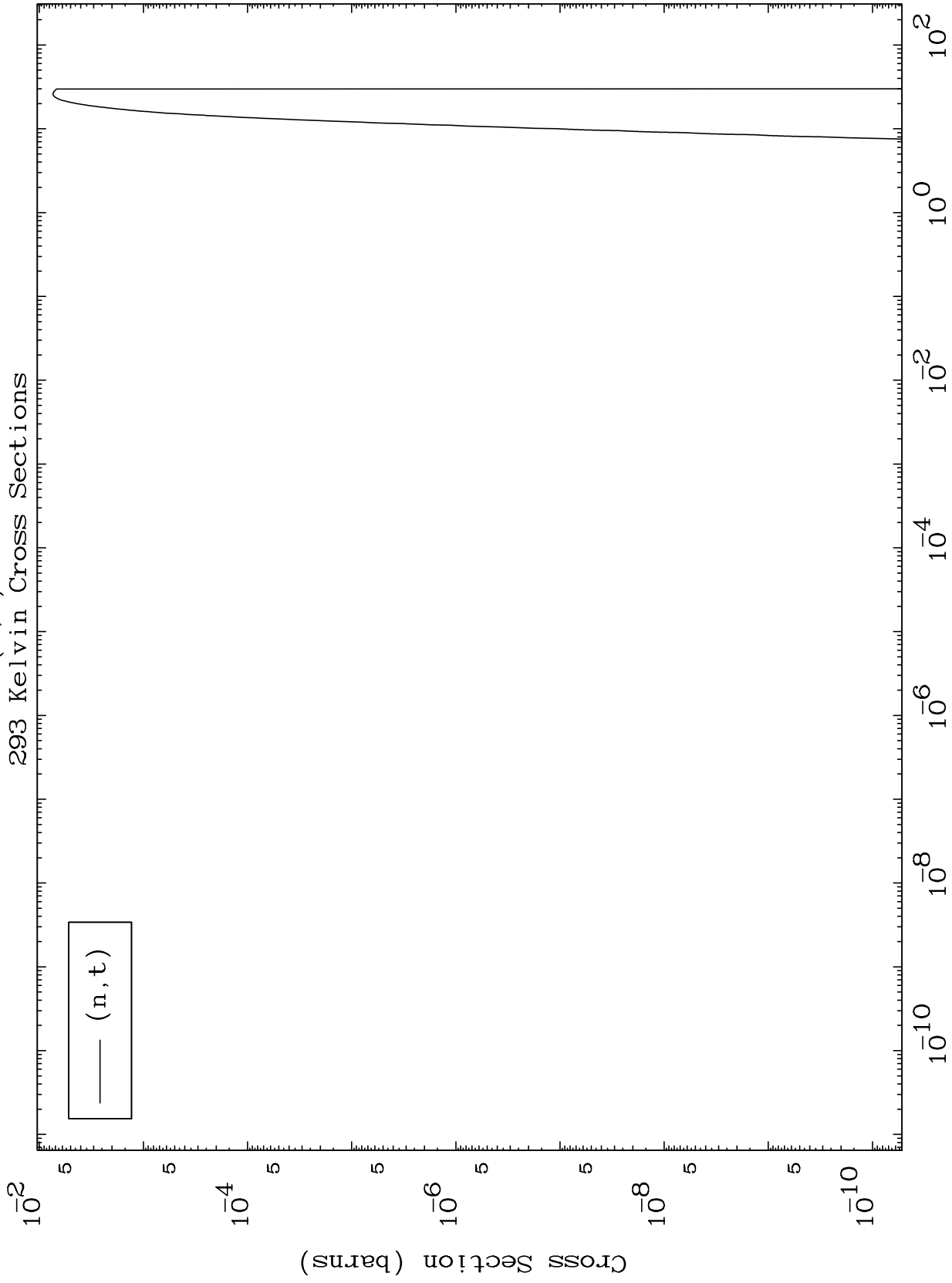
Incident Energy (MeV)

101-Md-254

MAT 9960

(n,t) Levels  
293 Kelvin Cross Sections

101-Md-254



13

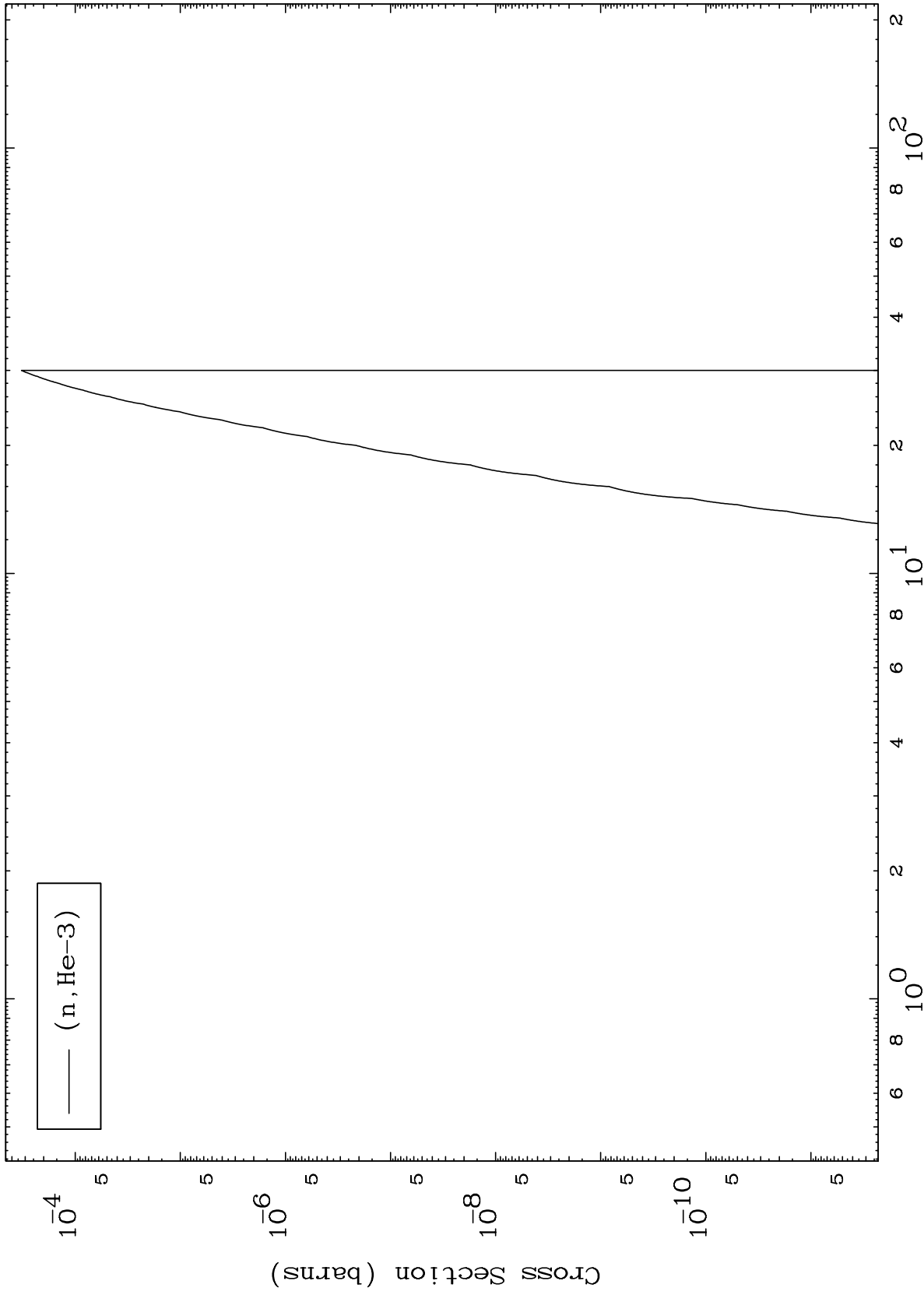
Incident Energy (MeV)

101-Md-254

MAT 9960

(n,He3) Levels  
293 Kelvin Cross Sections

101-Md-254



14

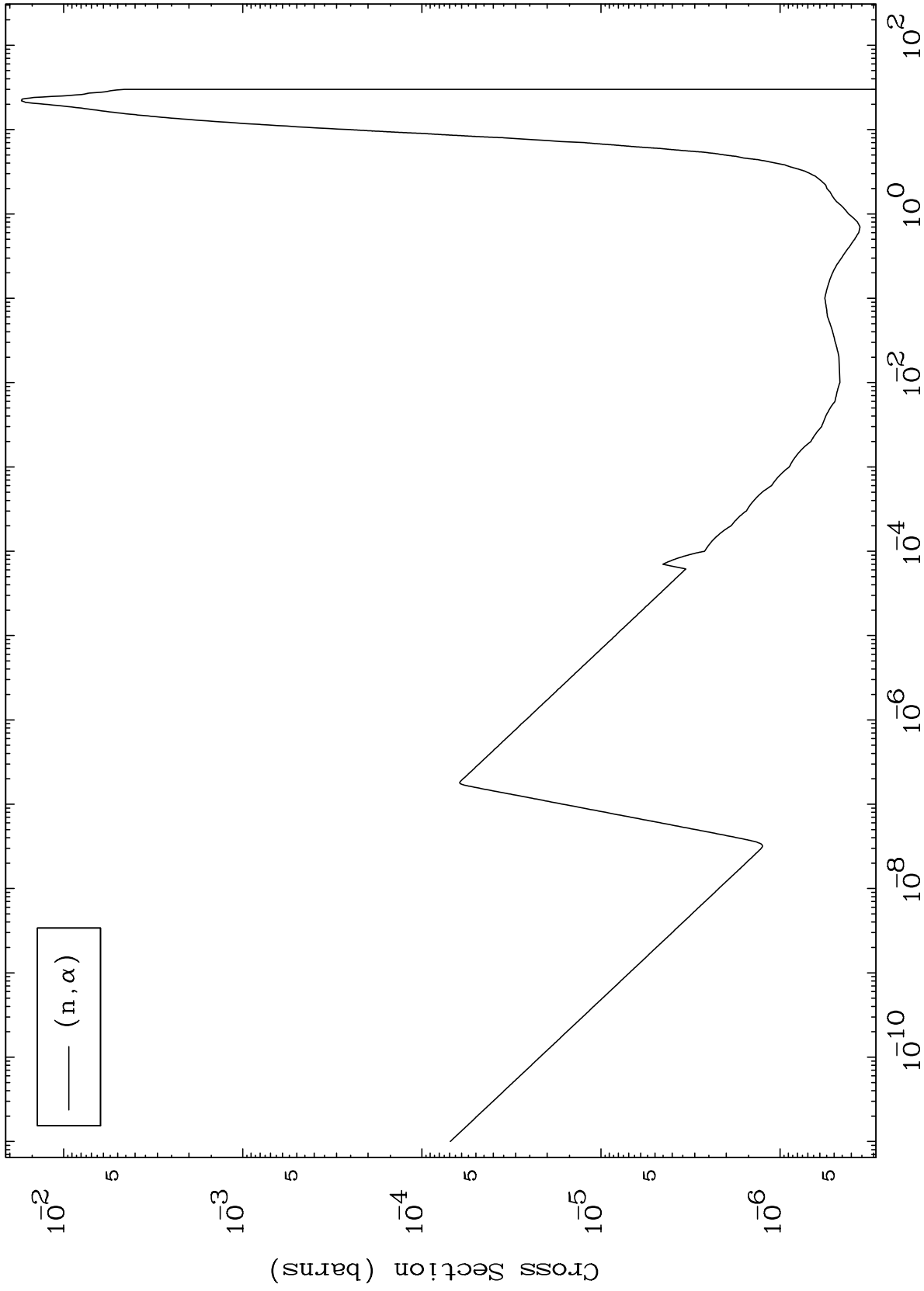
Incident Energy (MeV)

101-Md-254

MAT 9960

(n,α) Levels  
293 Kelvin Cross Sections

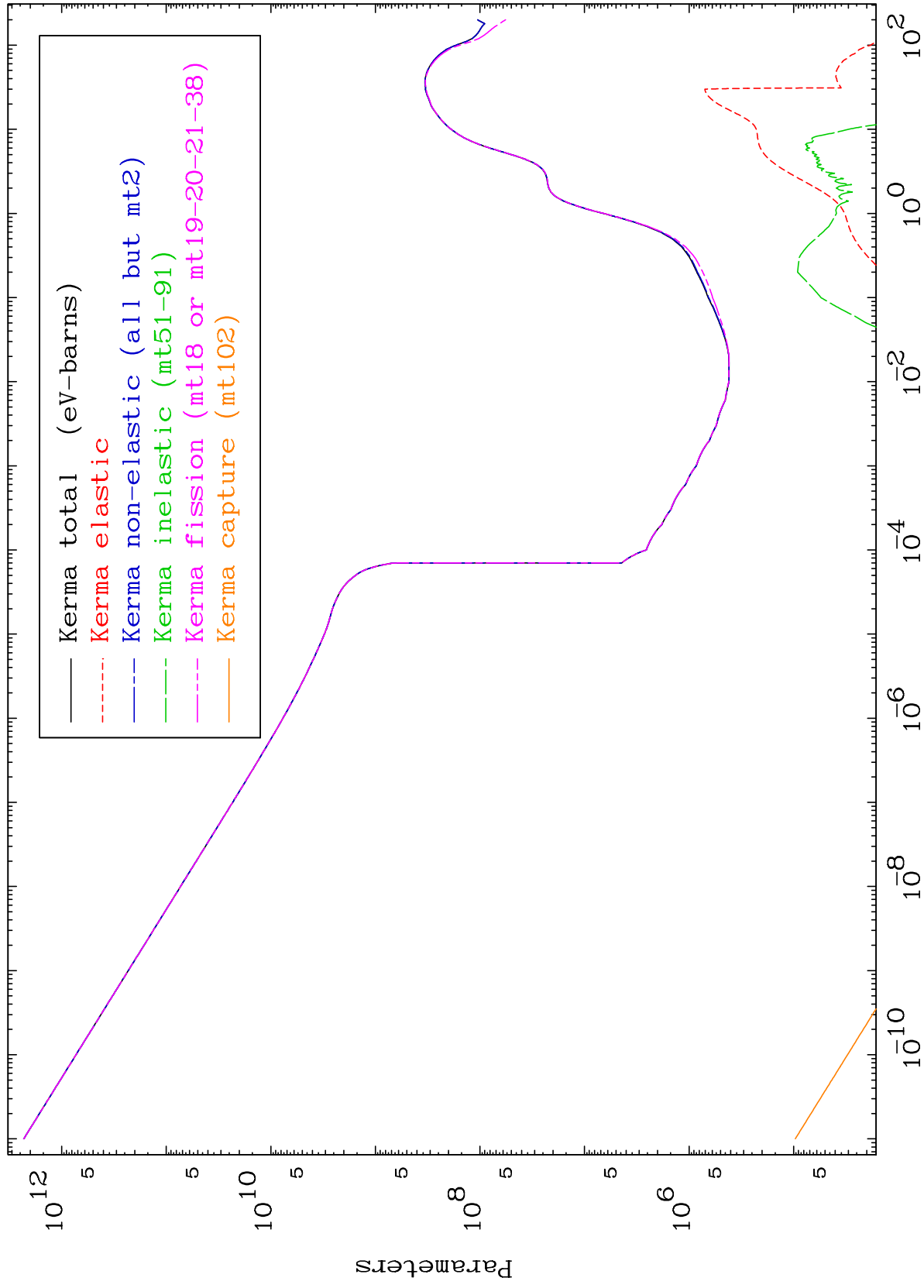
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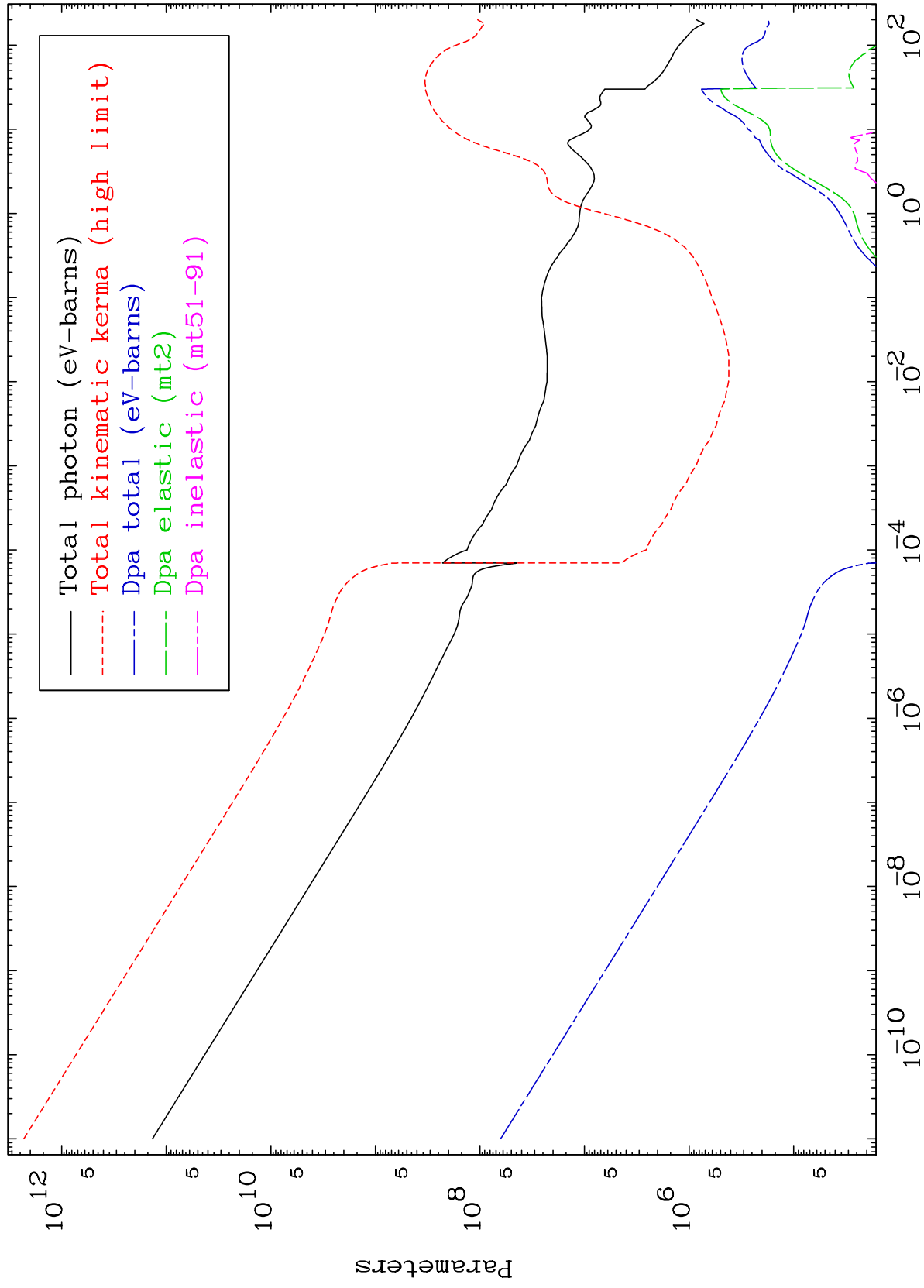


15

Incident Energy (MeV)

101-Md-254





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Elastic Legendre Coefficients

101-Md-254



18

Incident Energy (MeV)

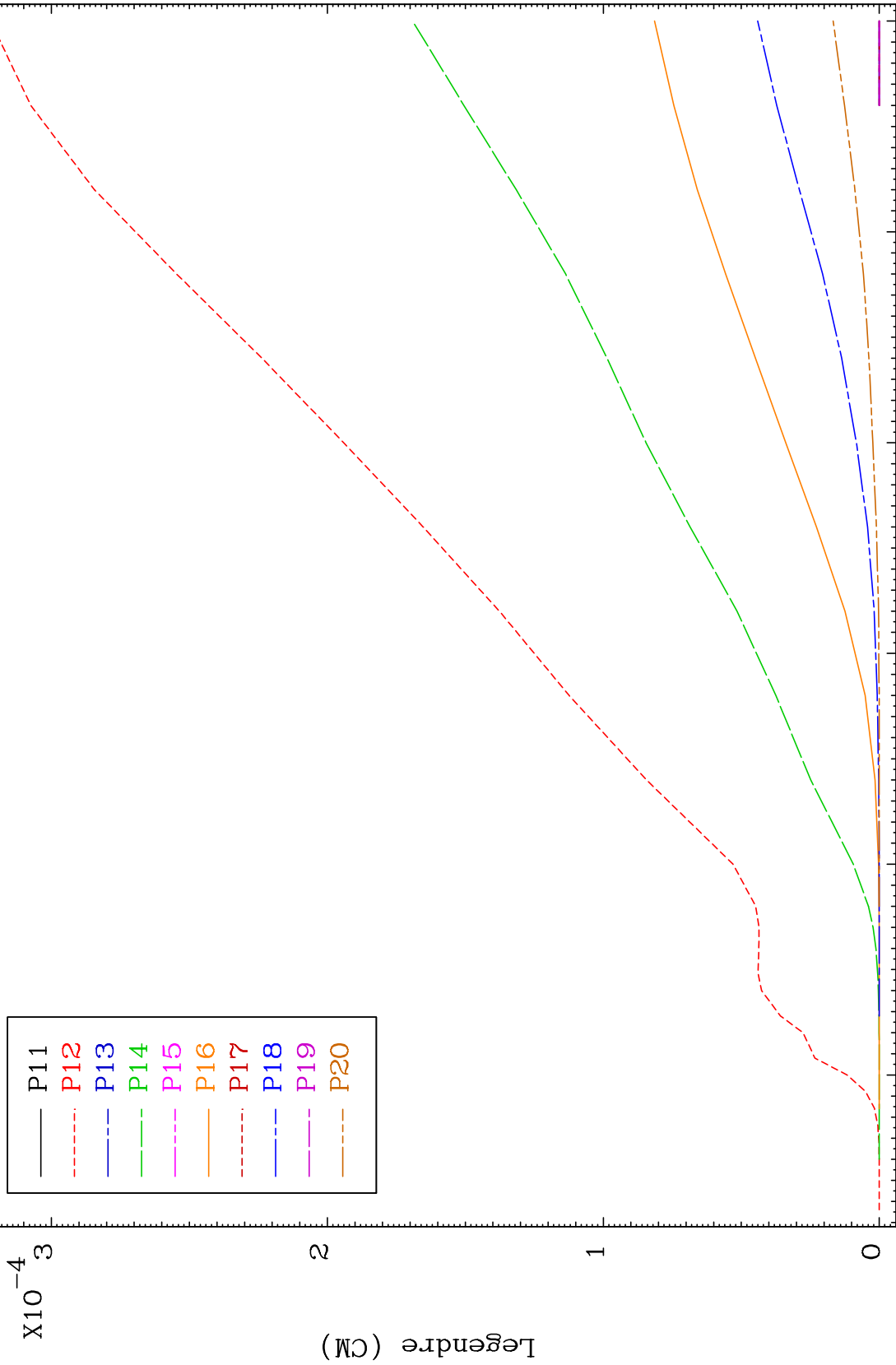
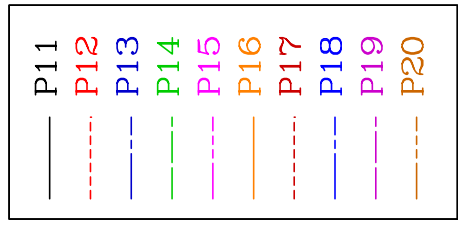
101-Md-254

30

MAT 9960

Elastic Legendre Coefficients

101-Md-254



19

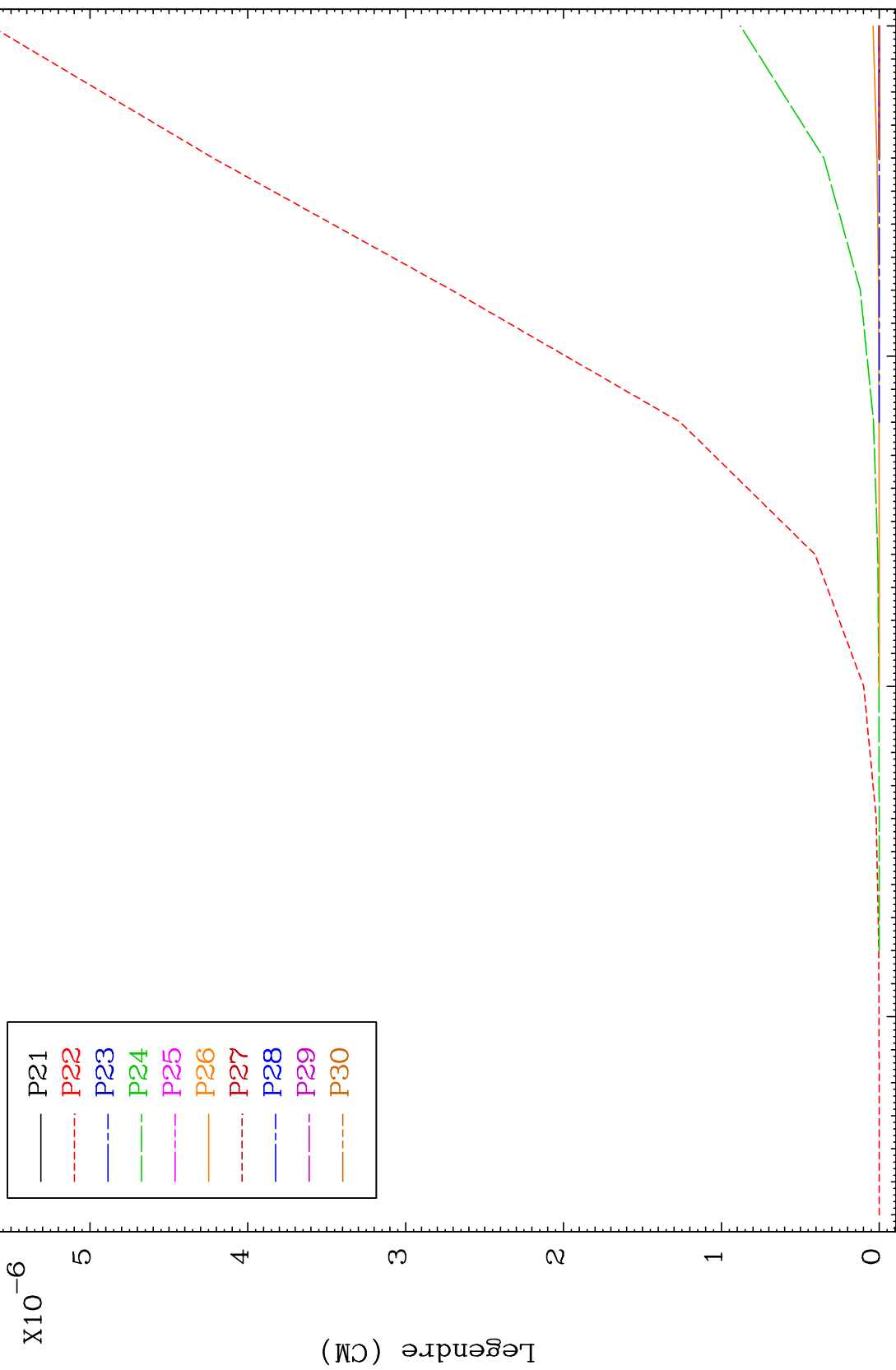
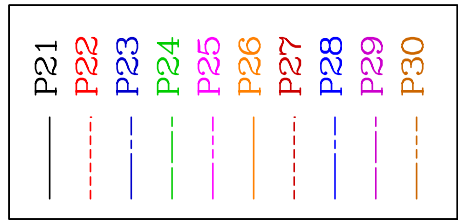
Incident Energy (MeV)

101-Md-254

MAT 9960

Elastic  
Legendre Coefficients

101-Md-254



20

15

20

Incident Energy (MeV)

25

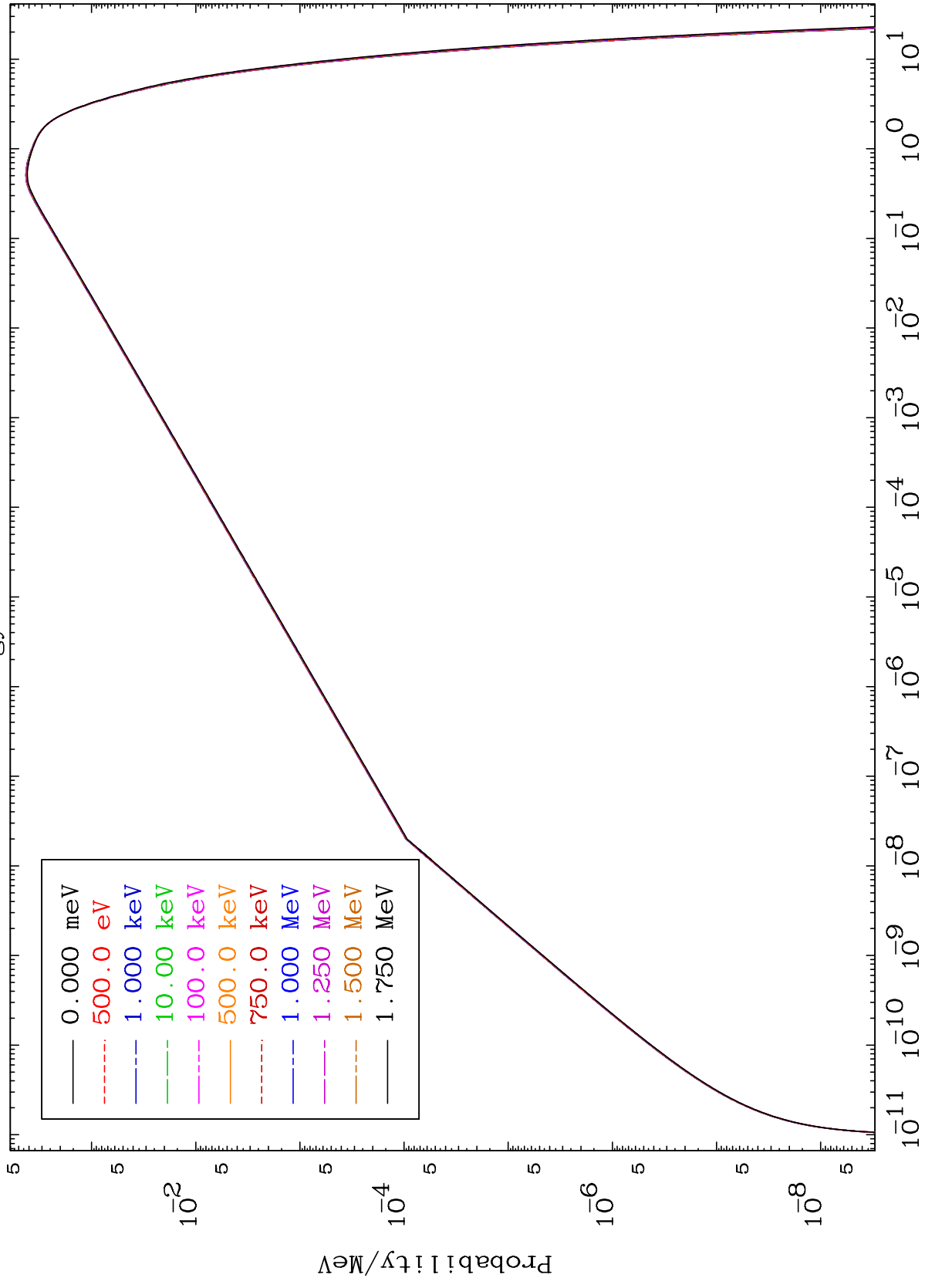
101-Md-254

30

MAT 9960

### Fission Energy Distribution

101-Md-254

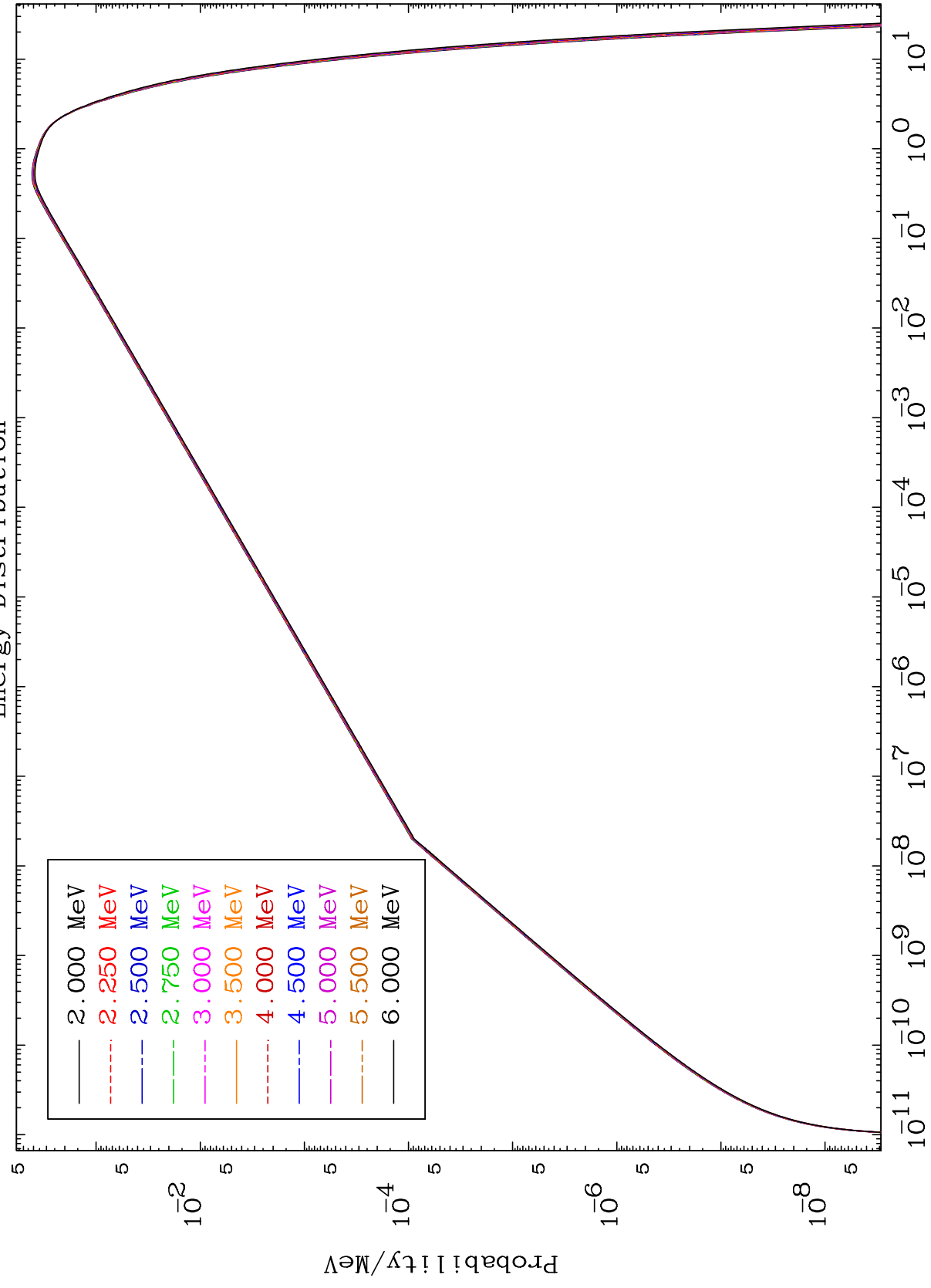


101-Md-254

MAT 9960

### Fission Energy Distribution

101-Md-254

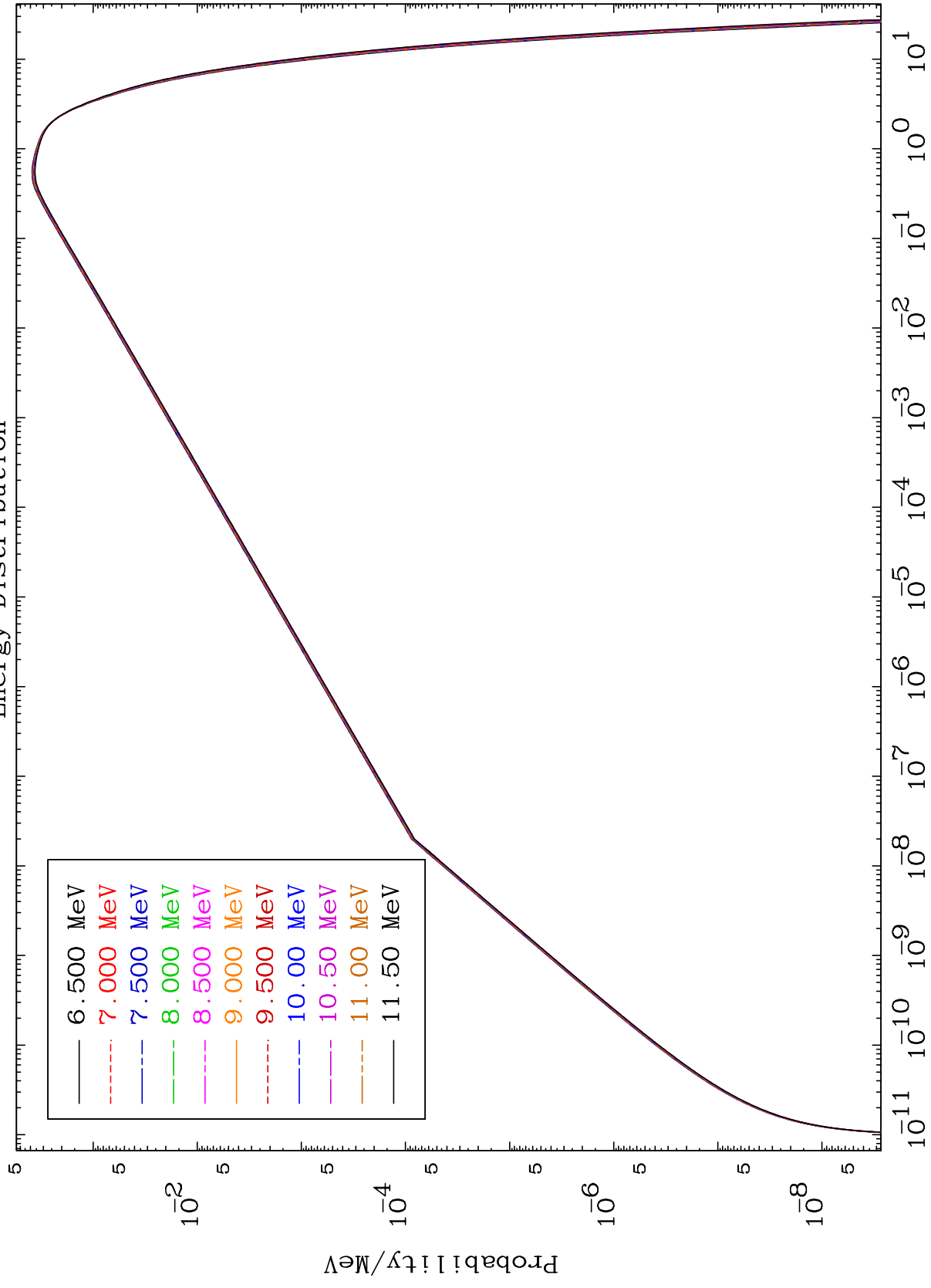


101-Md-254

MAT 9960

### Fission Energy Distribution

101-Md-254

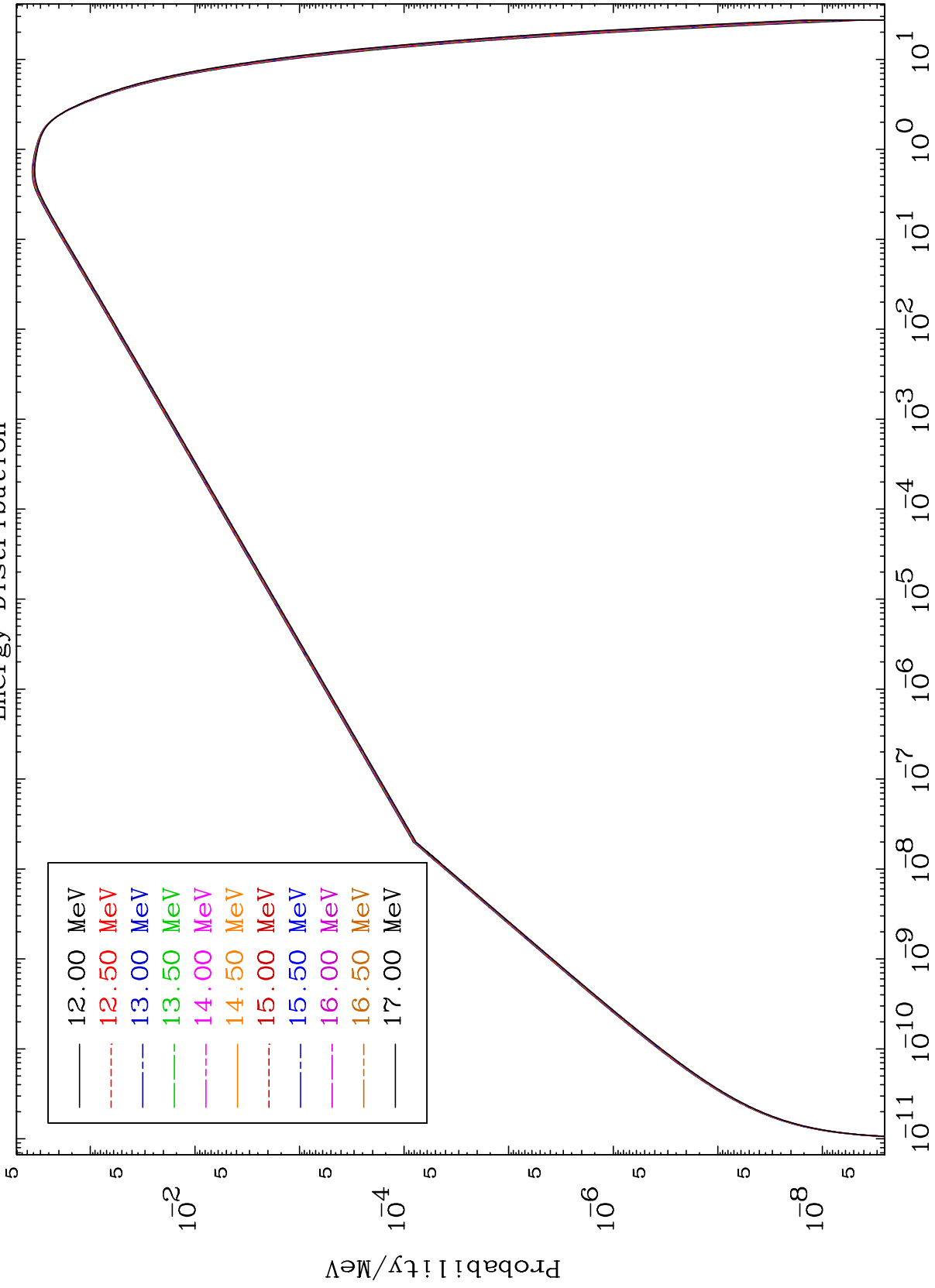


101-Md-254

MAT 9960

Fission  
Energy Distribution

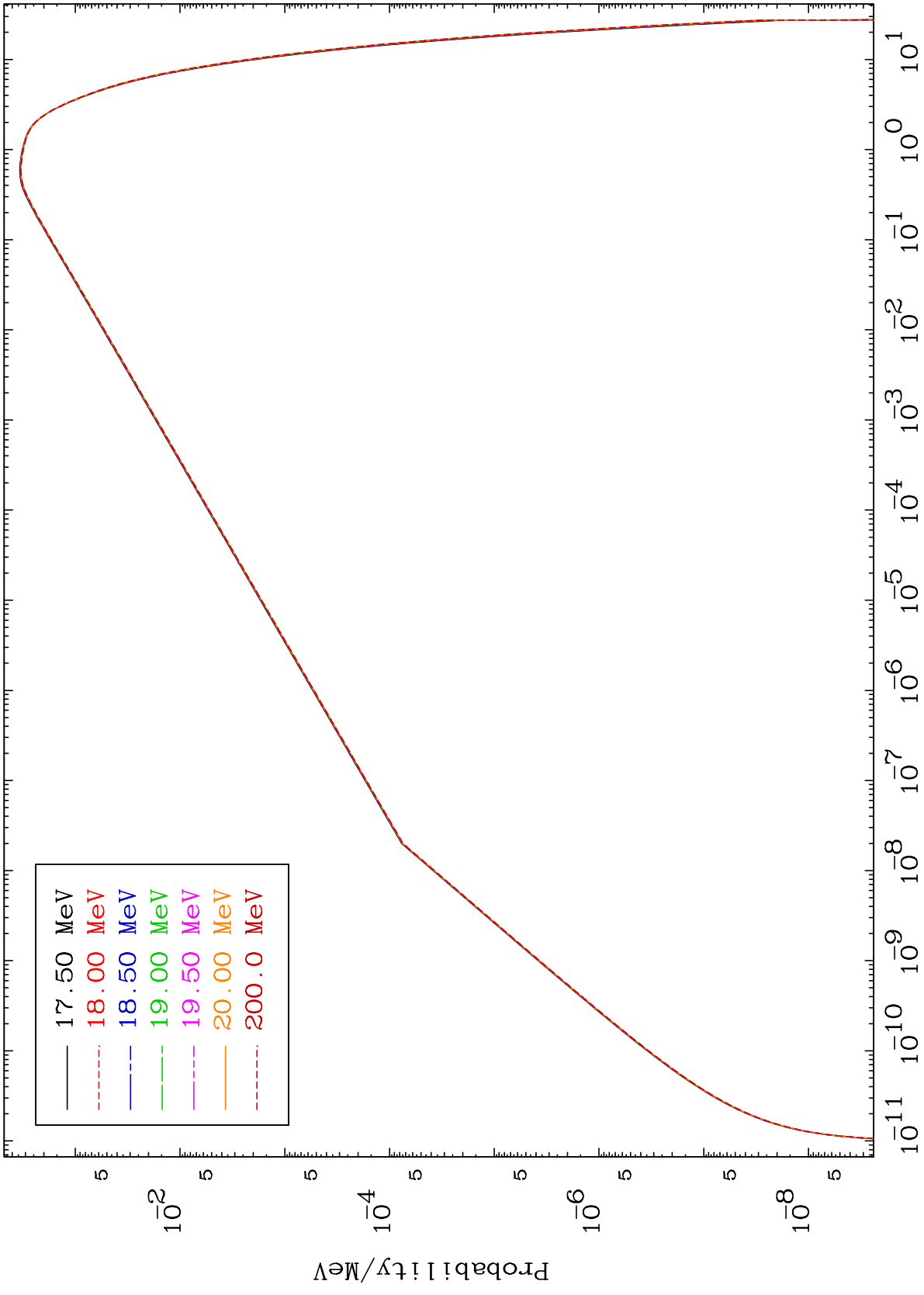
101-Md-254



MAT 9960

### Fission Energy Distribution

101-Md-254

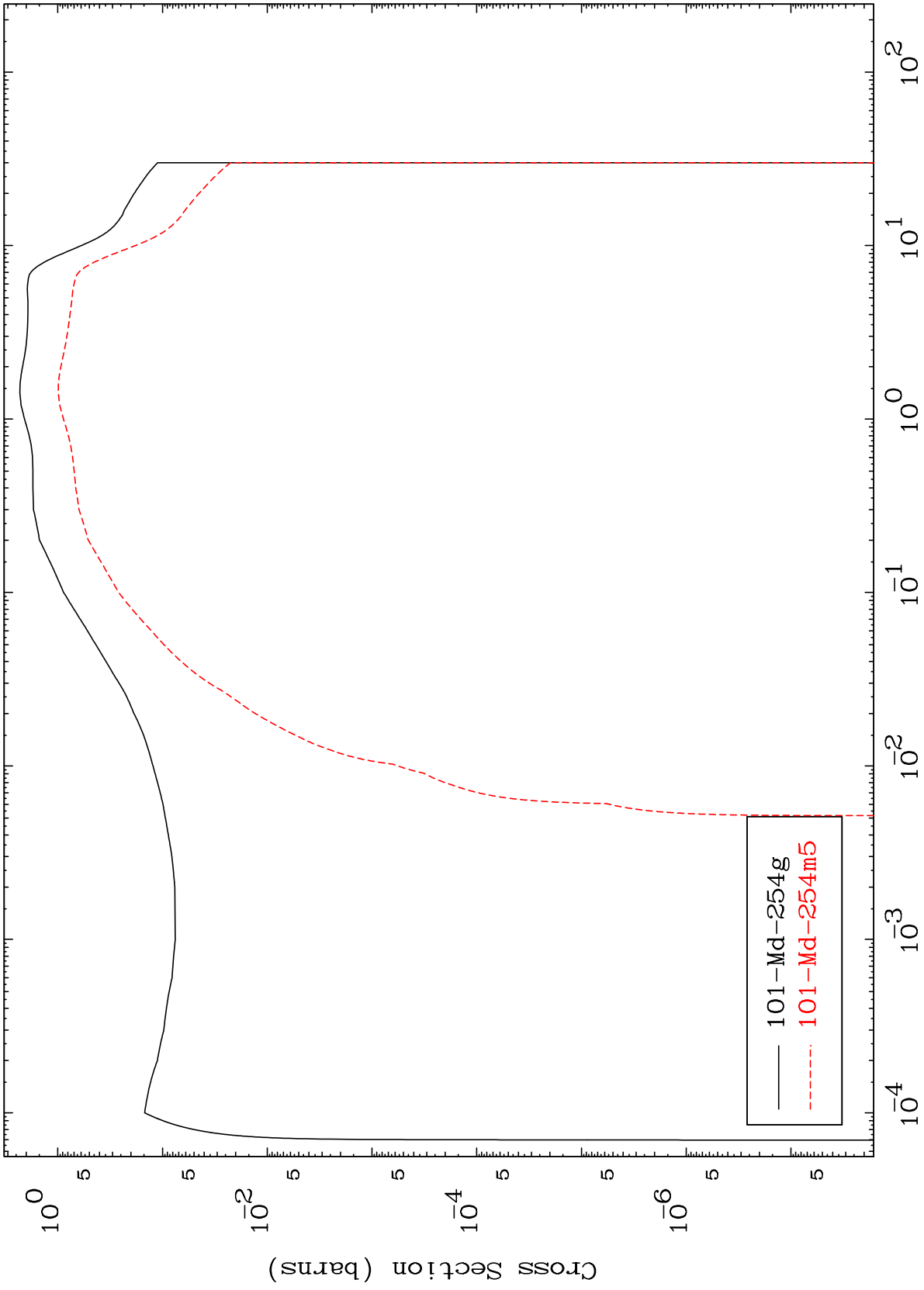


101-Md-254

MAT 9960

Inelastic  
Radionuclide Production Cross Section

101-Md-254



MAT 9960

(n,n')  $\alpha$

101-Md-254

Radionuclide Production Cross Section

