

Program EVALPLOT
(Version 2018-1)

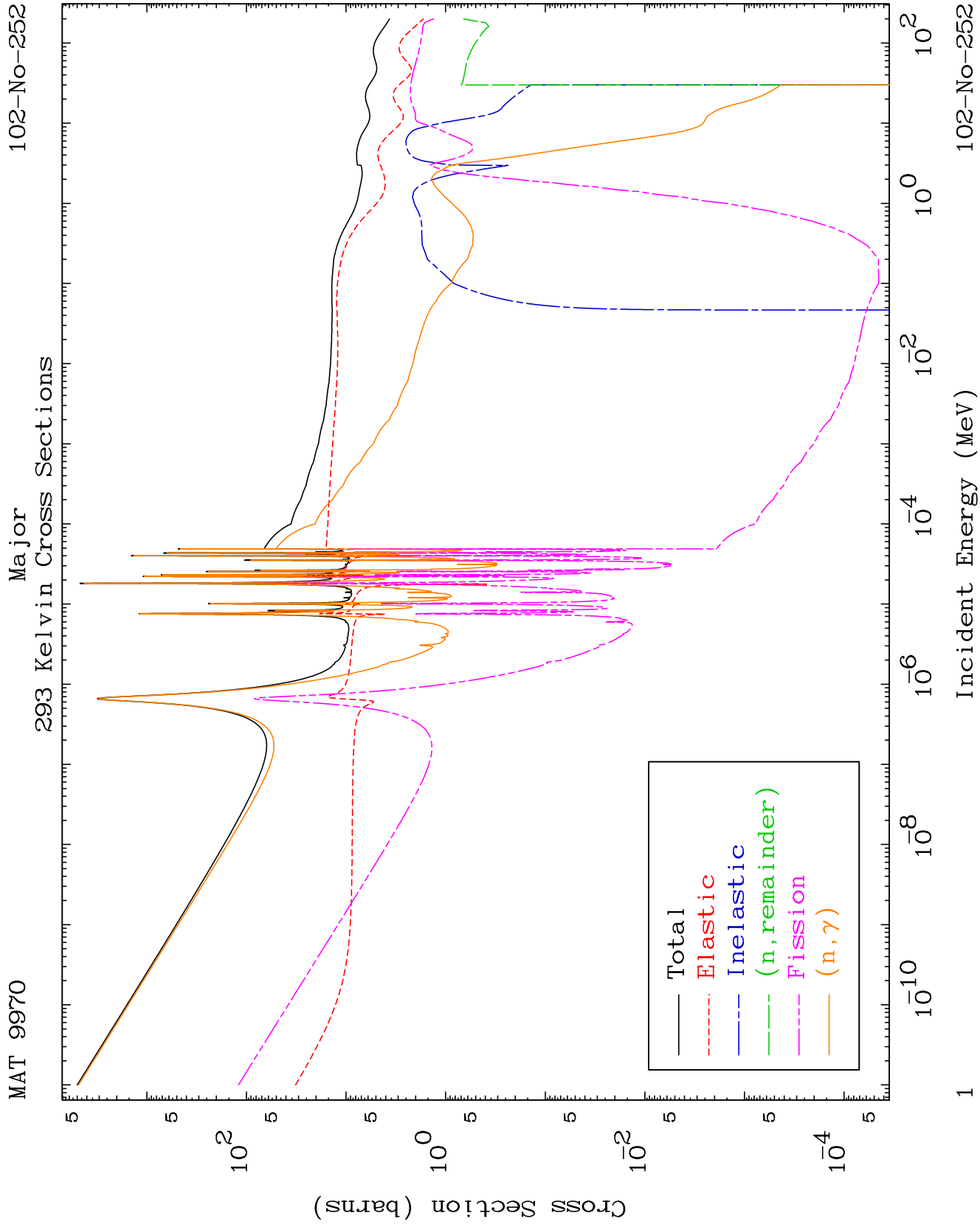
by

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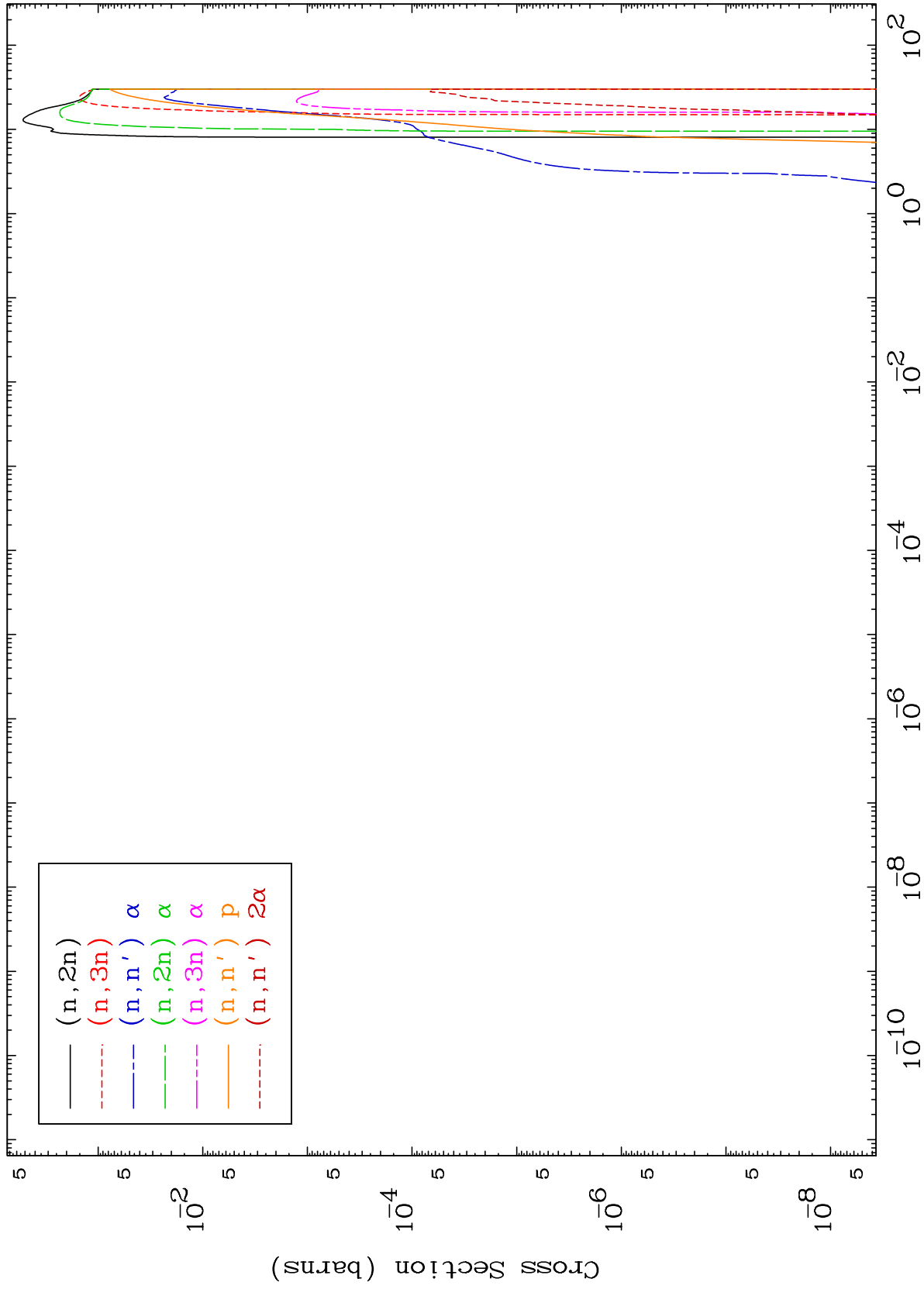
Press Mouse Button to Start



MAT 9970

Neutron Production
293 Kelvin Cross Sections

102-No-252



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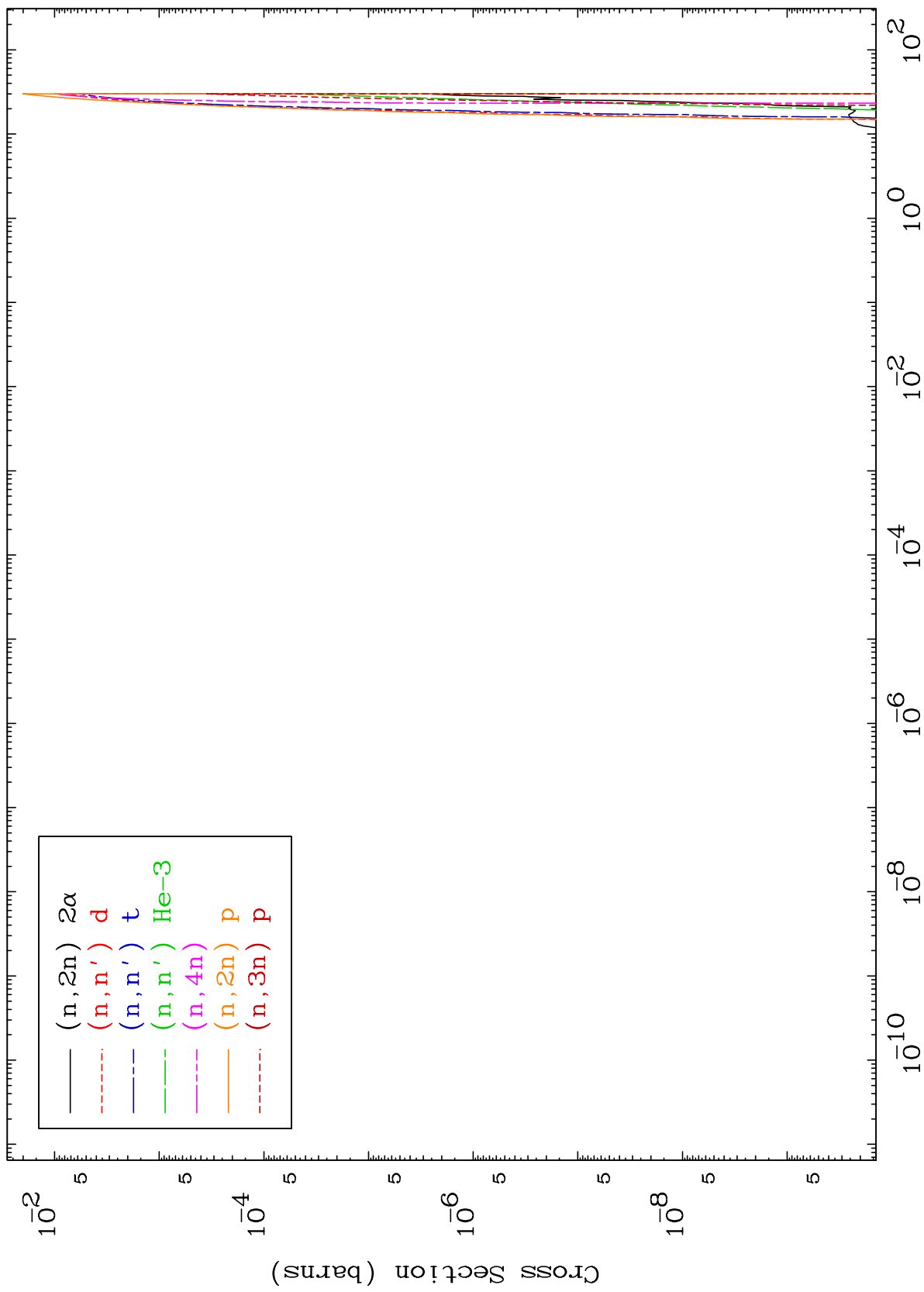
Incident Energy (MeV)

102-No-252

MAT 9970

Neutron Production
293 Kelvin Cross Sections

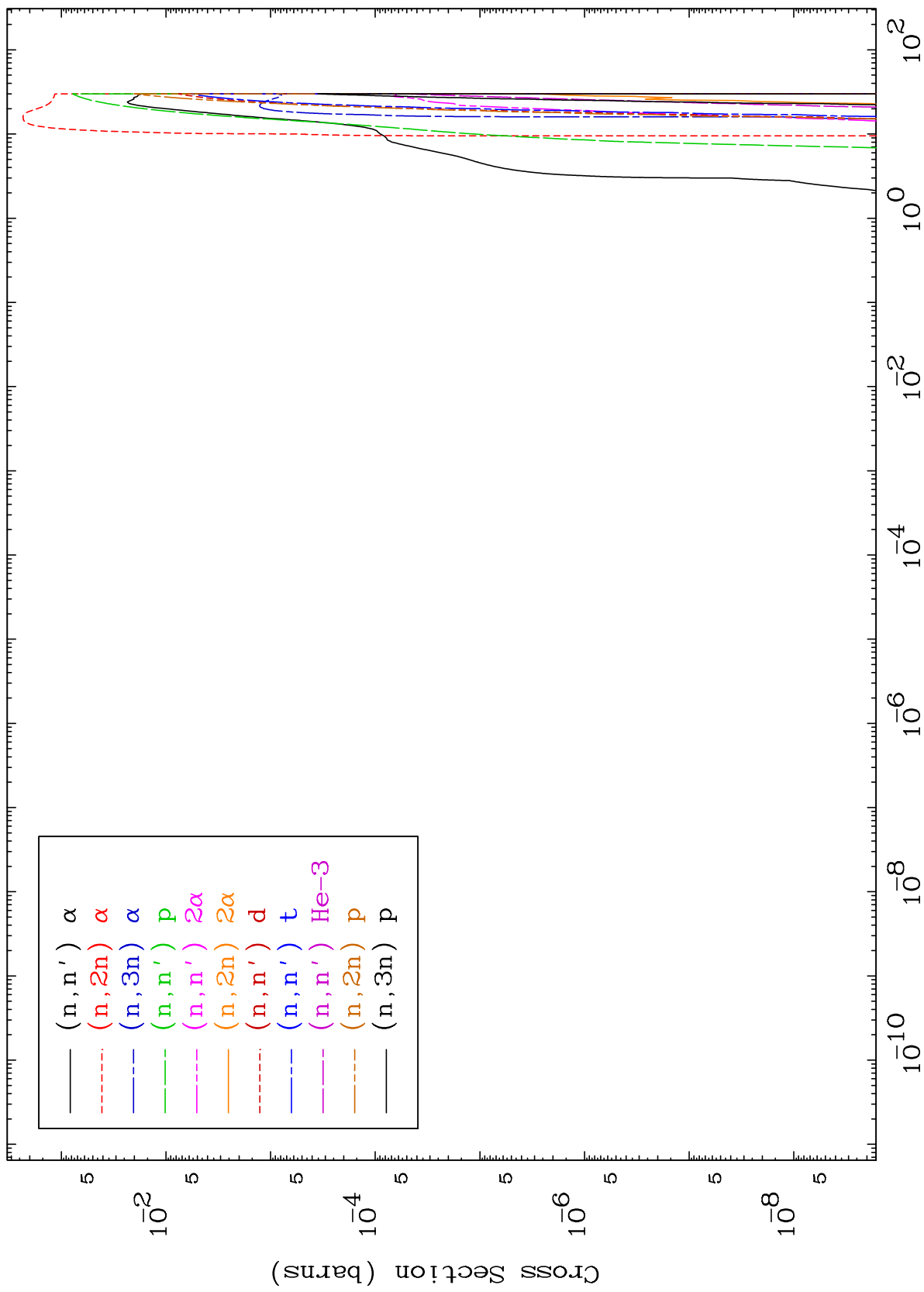
102-No-252



MAT 9970

Charged Particle
293 Kelvin Cross Sections

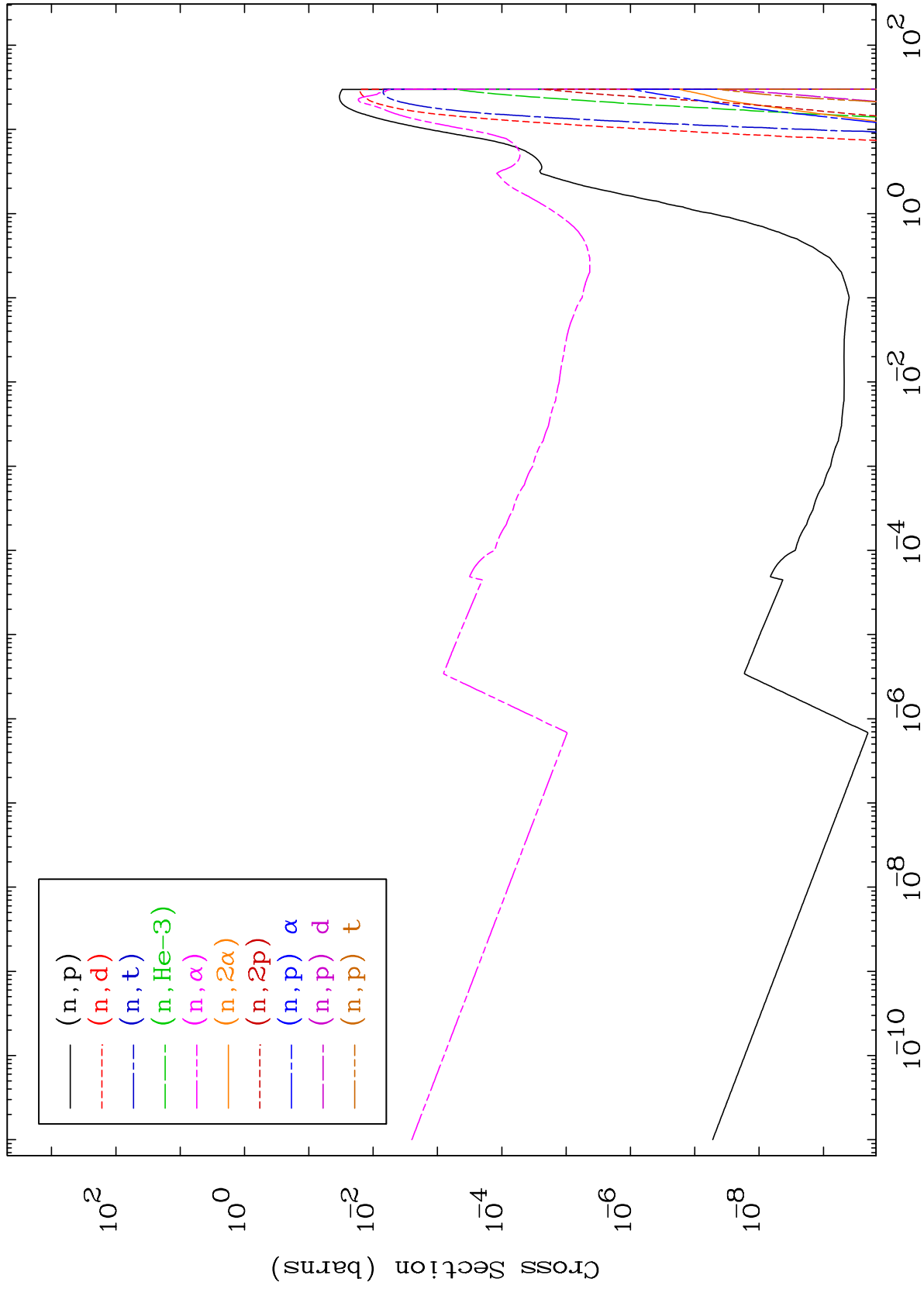
102-No-252

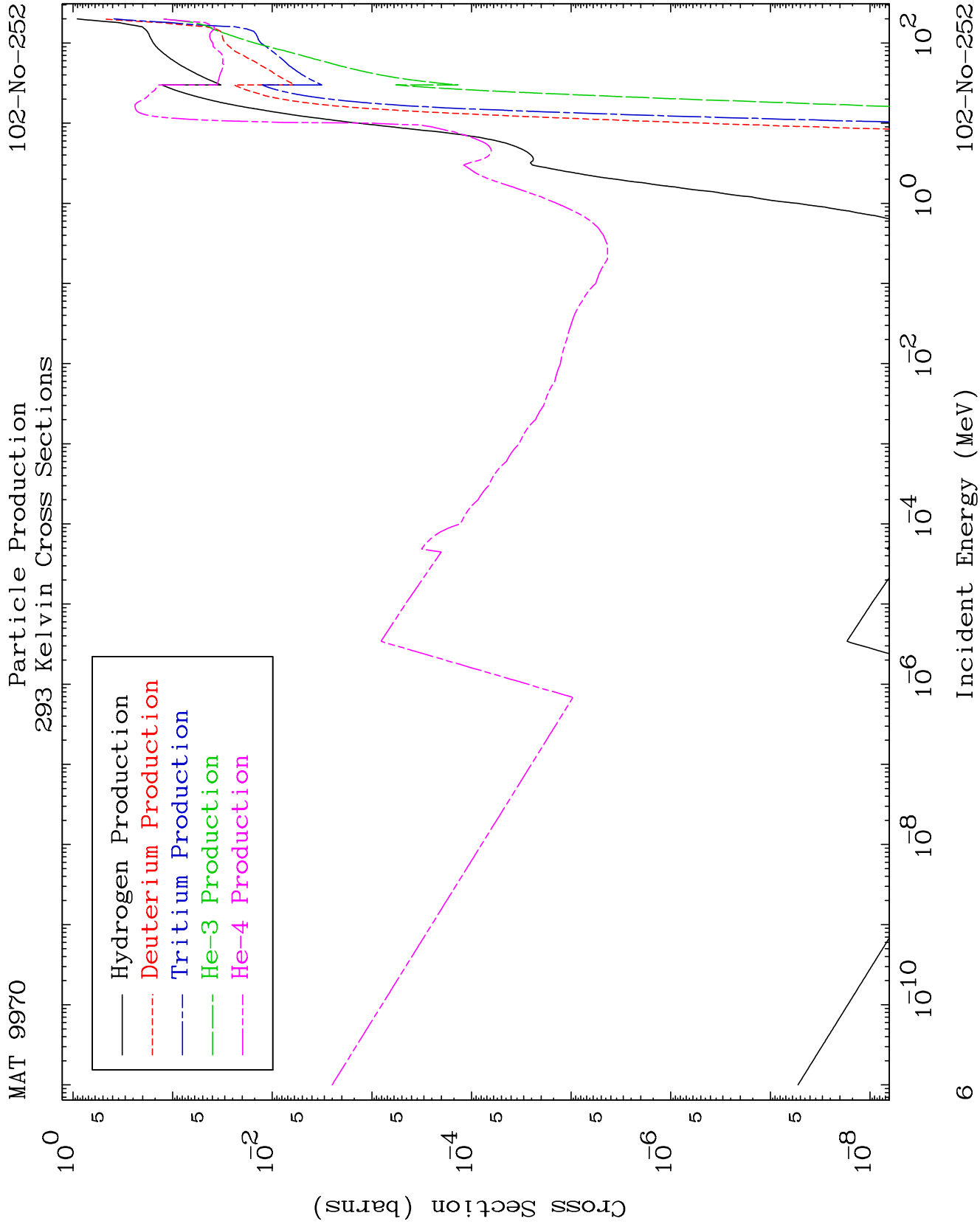


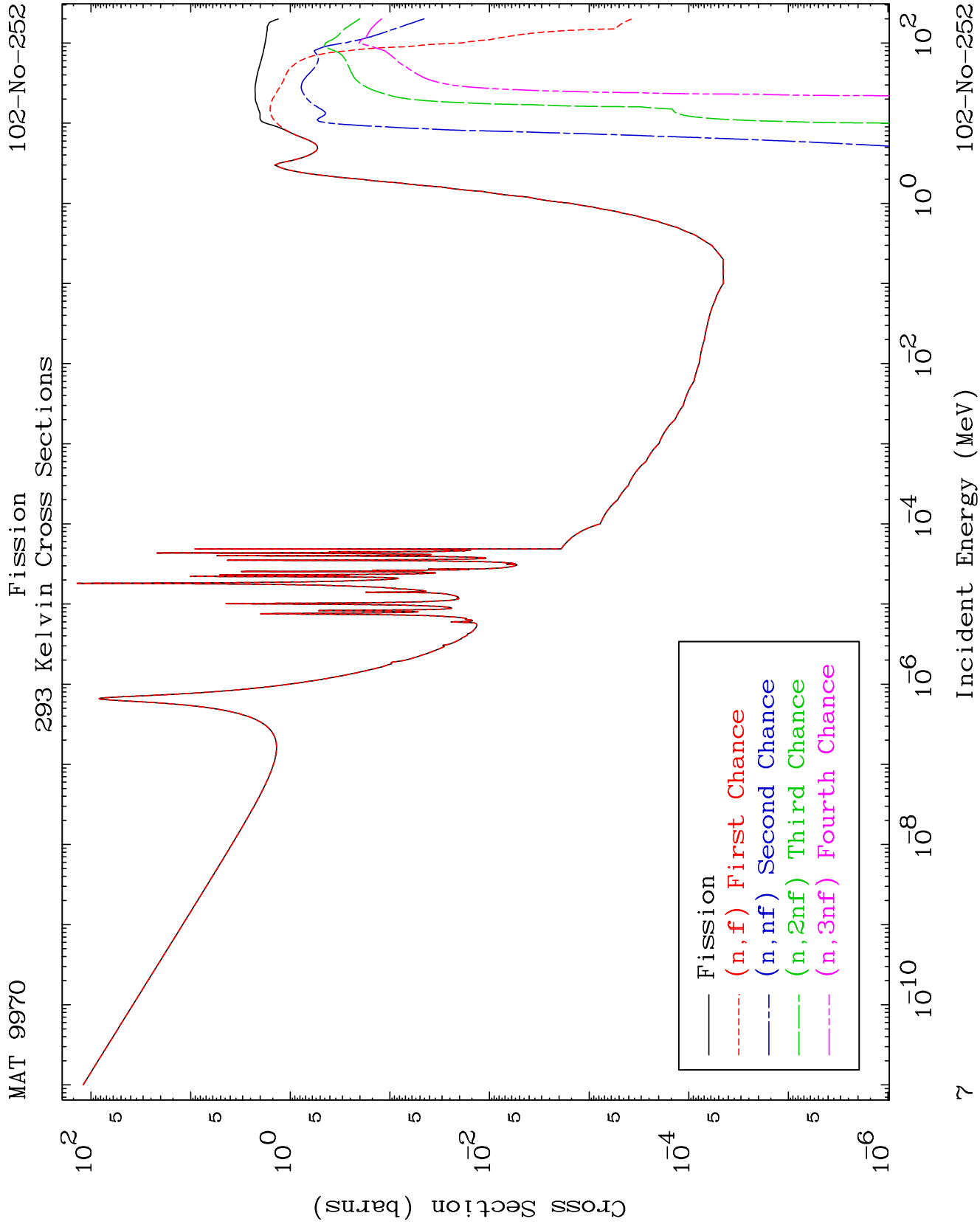
MAT 9970

Charged Particle
293 Kelvin Cross Sections

102-No-252



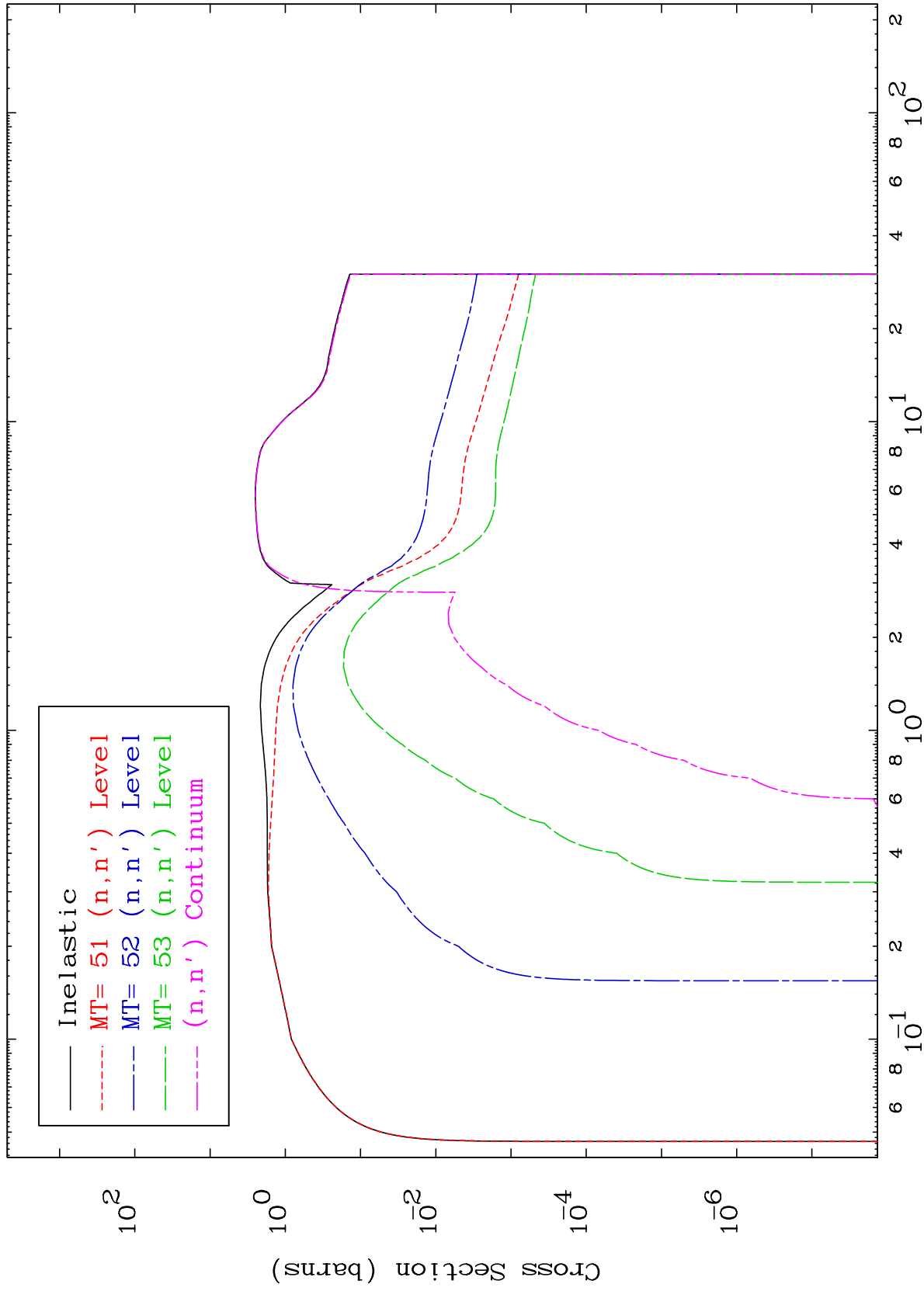




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(n,n') Level
293 Kelvin Cross Sections

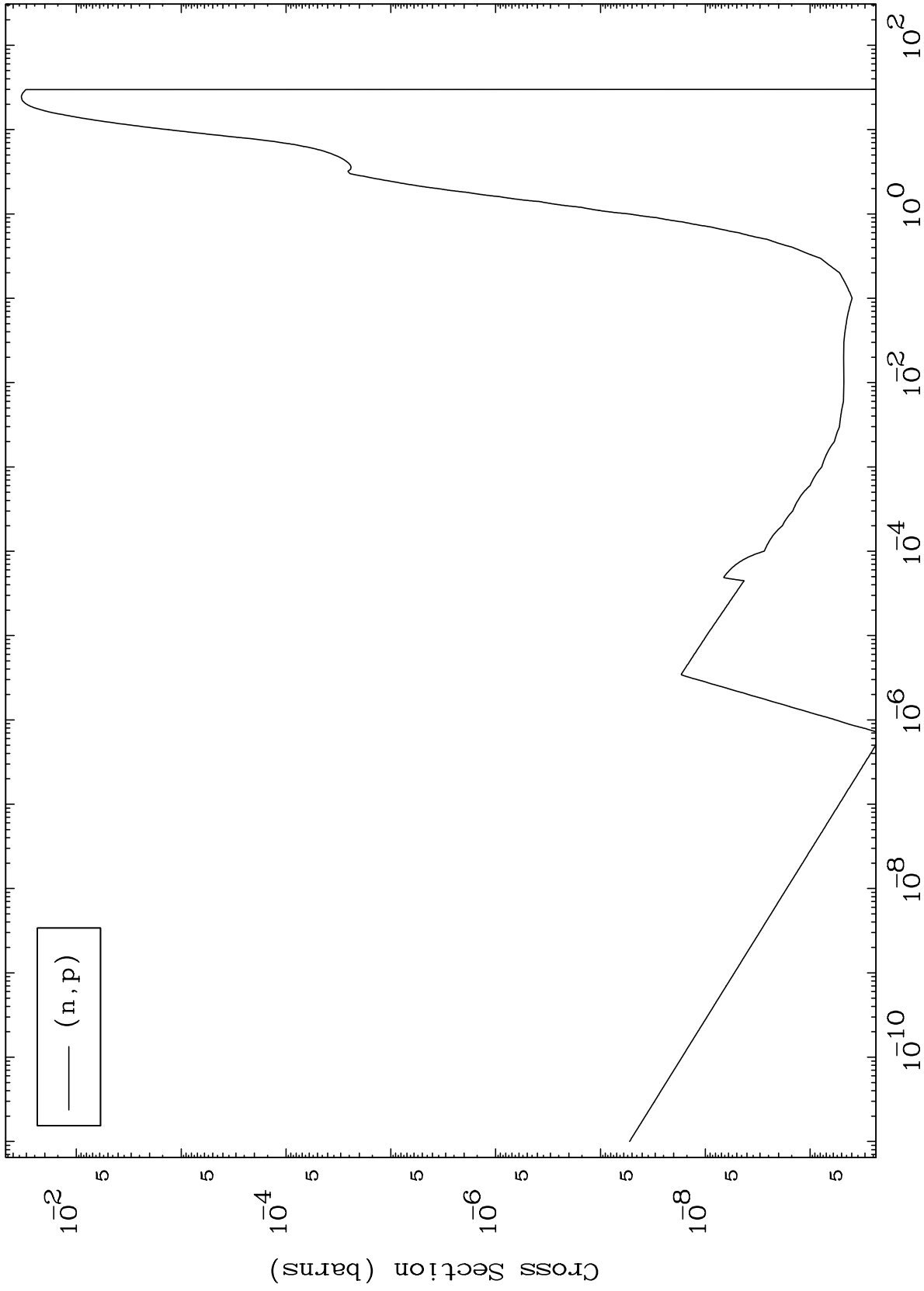
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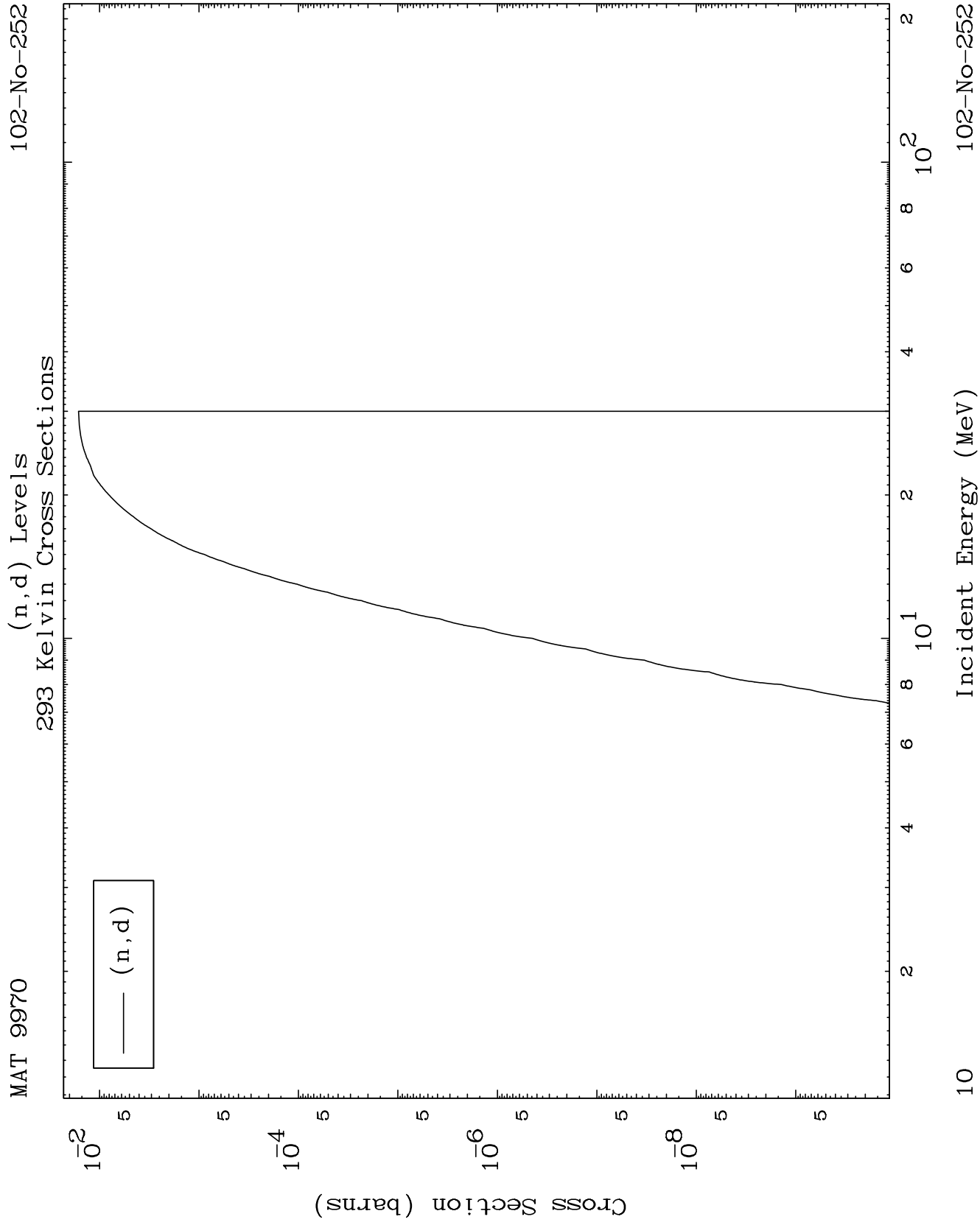


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(n,p) Levels
293 Kelvin Cross Sections

102-No-252

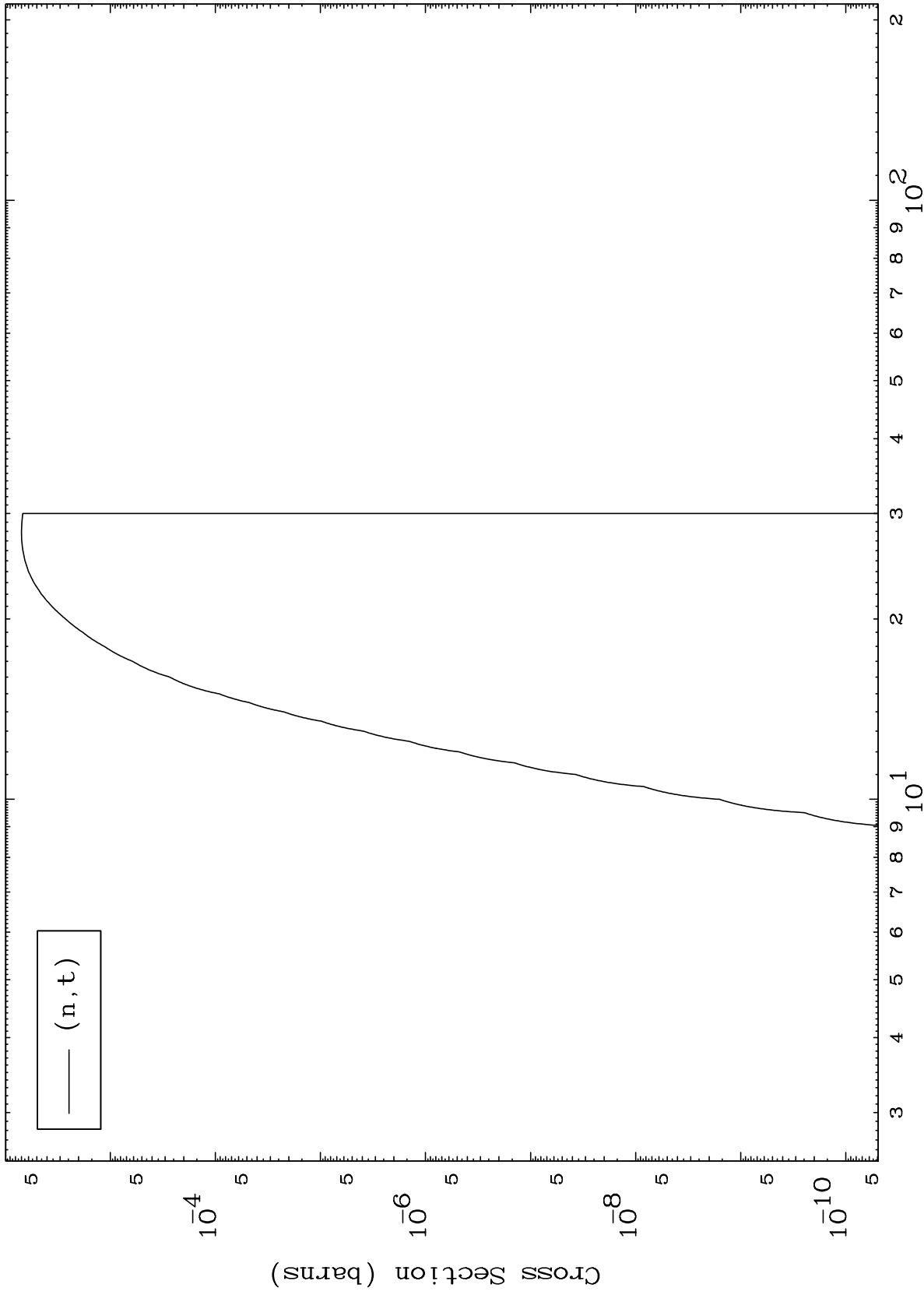




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(n,t) Levels
293 Kelvin Cross Sections

102-No-252



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Incident Energy (MeV)

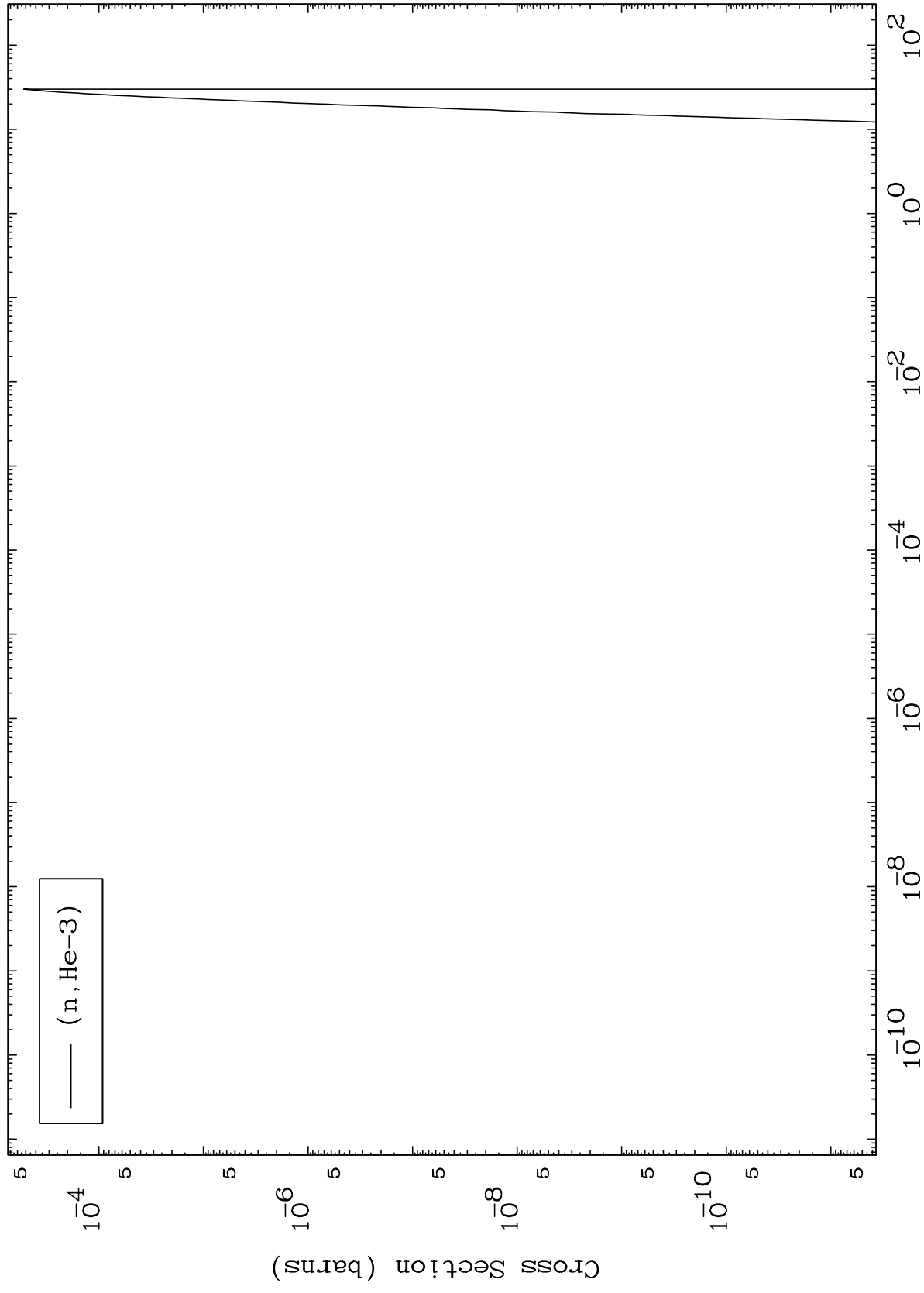
102-No-252

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(n, He3) Levels

102-No-252

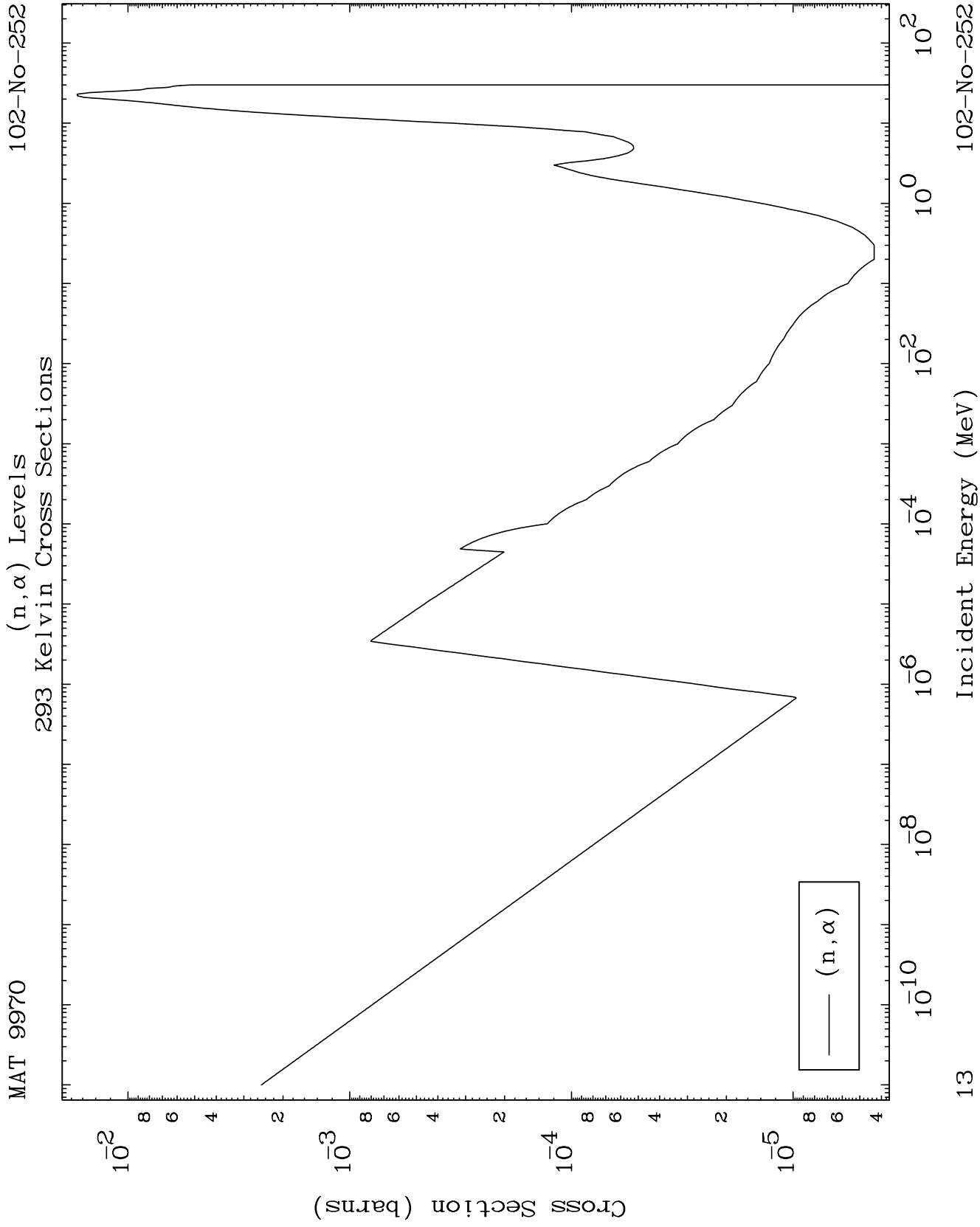
293 Kelvin Cross Sections

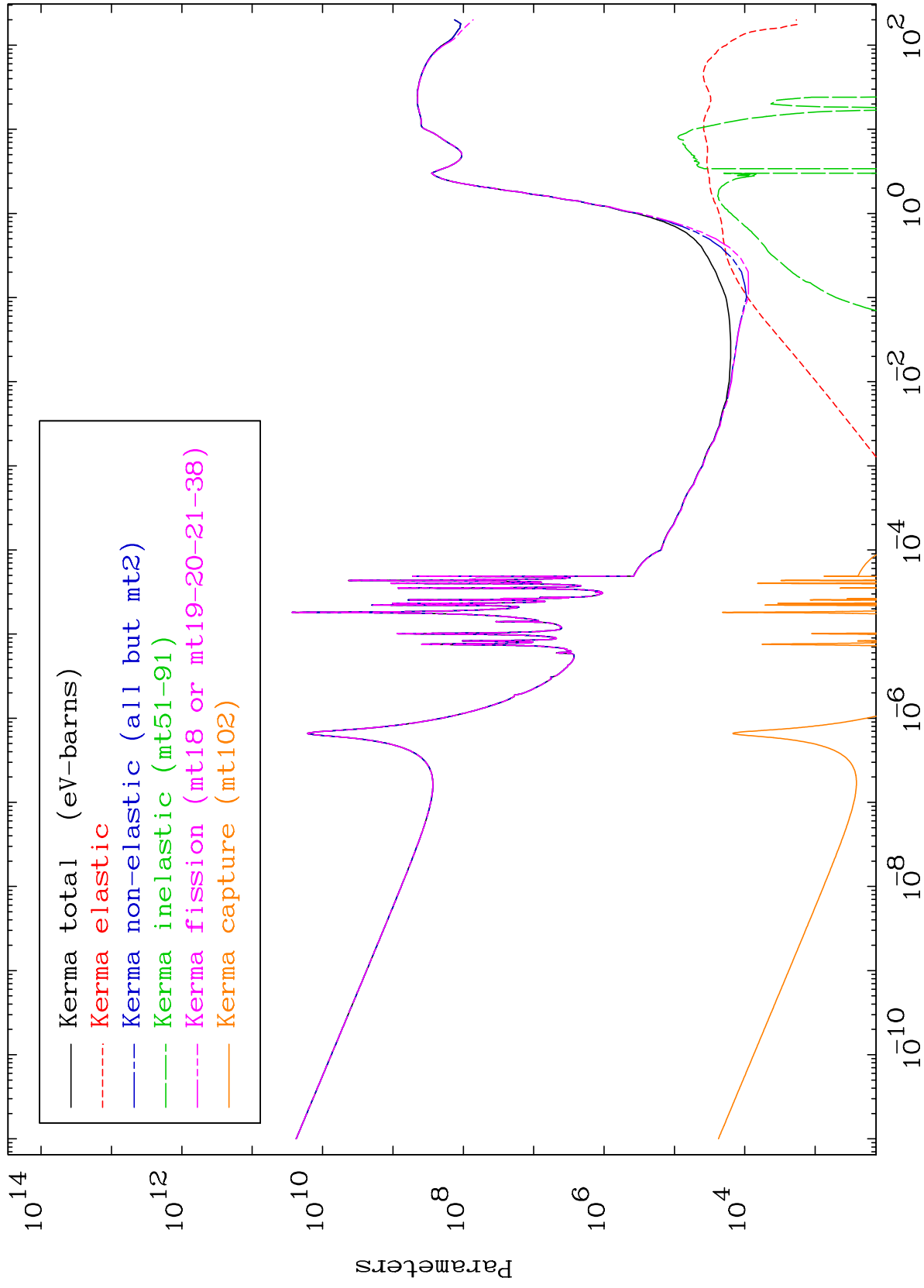


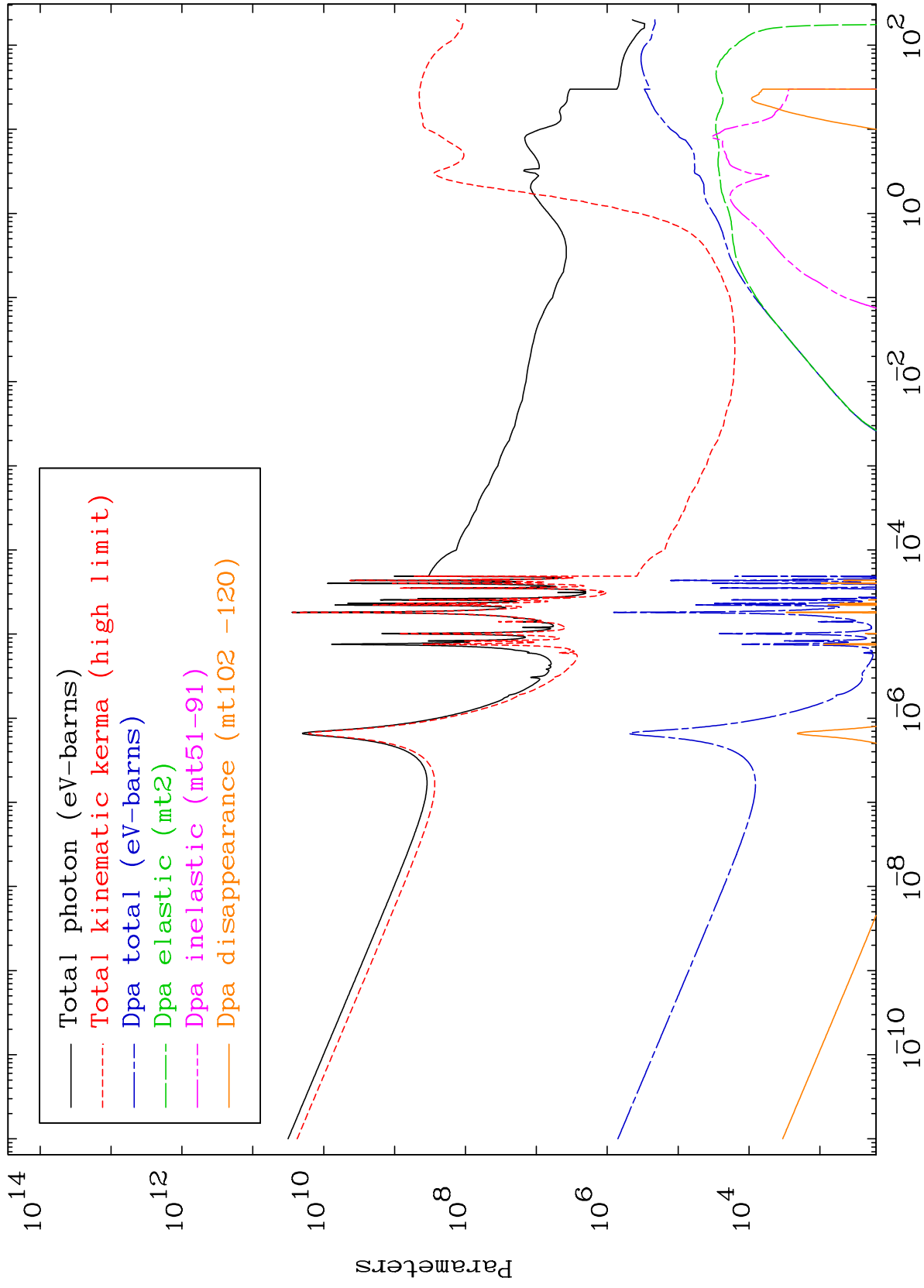
12

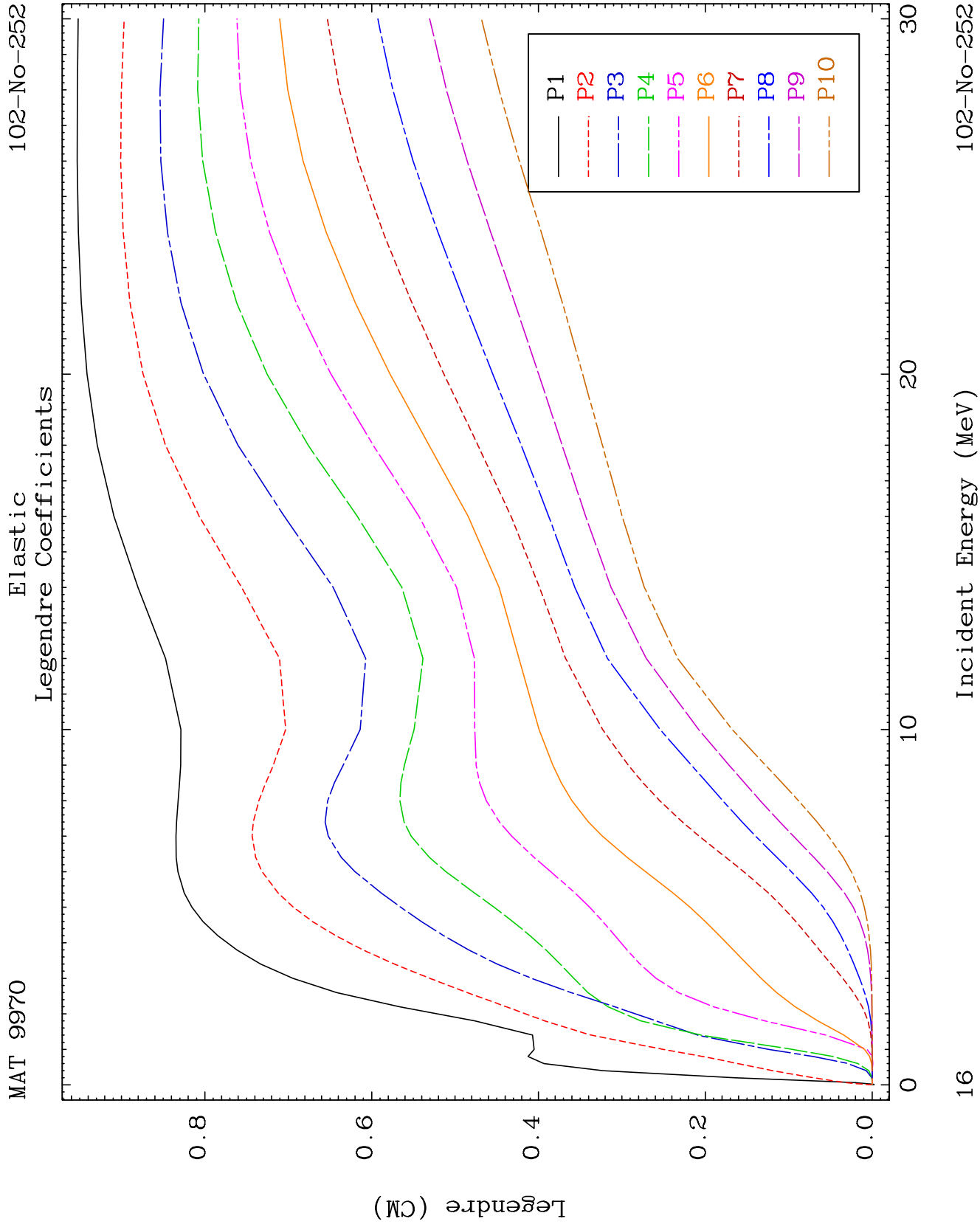
Incident Energy (MeV)

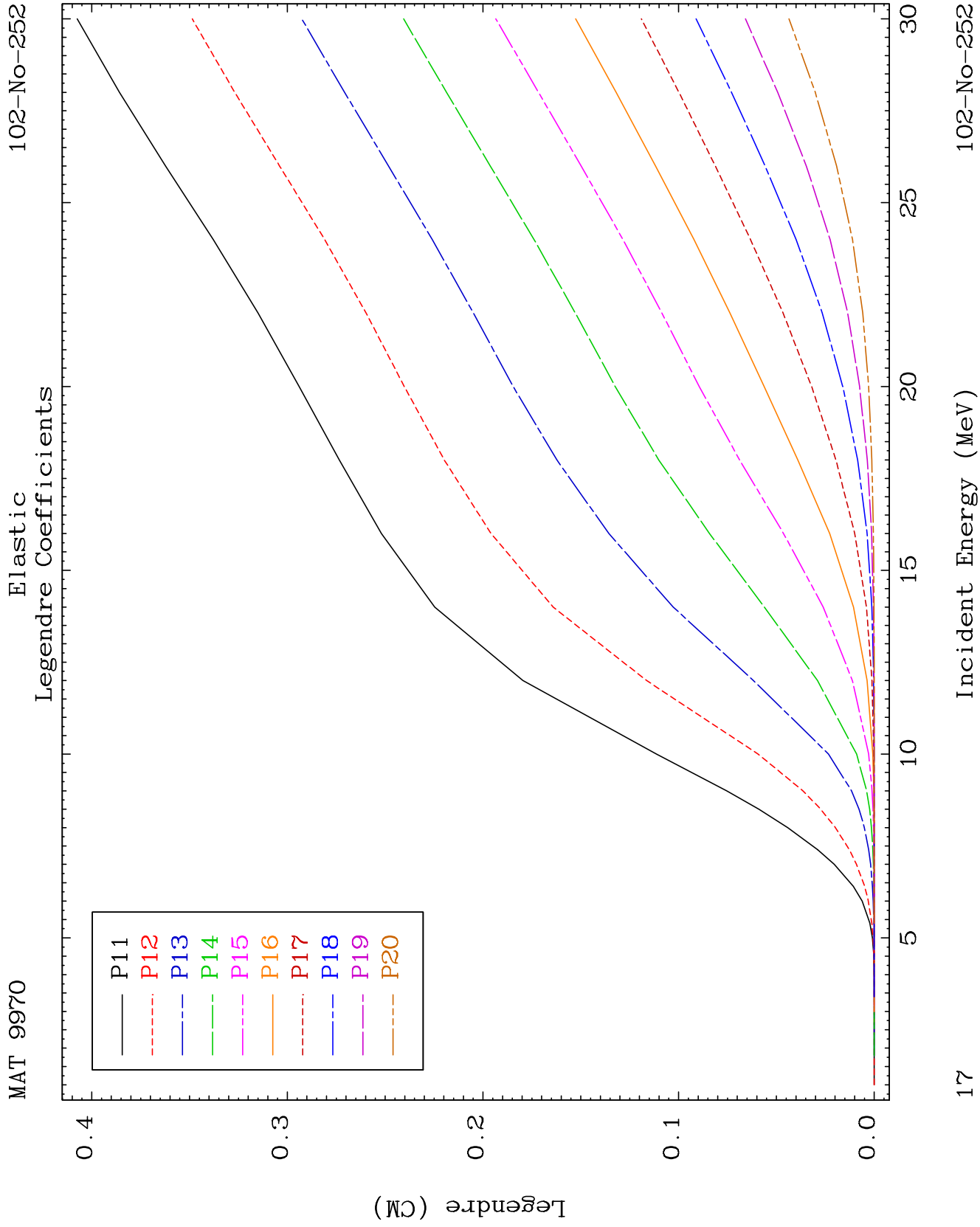
102-No-252

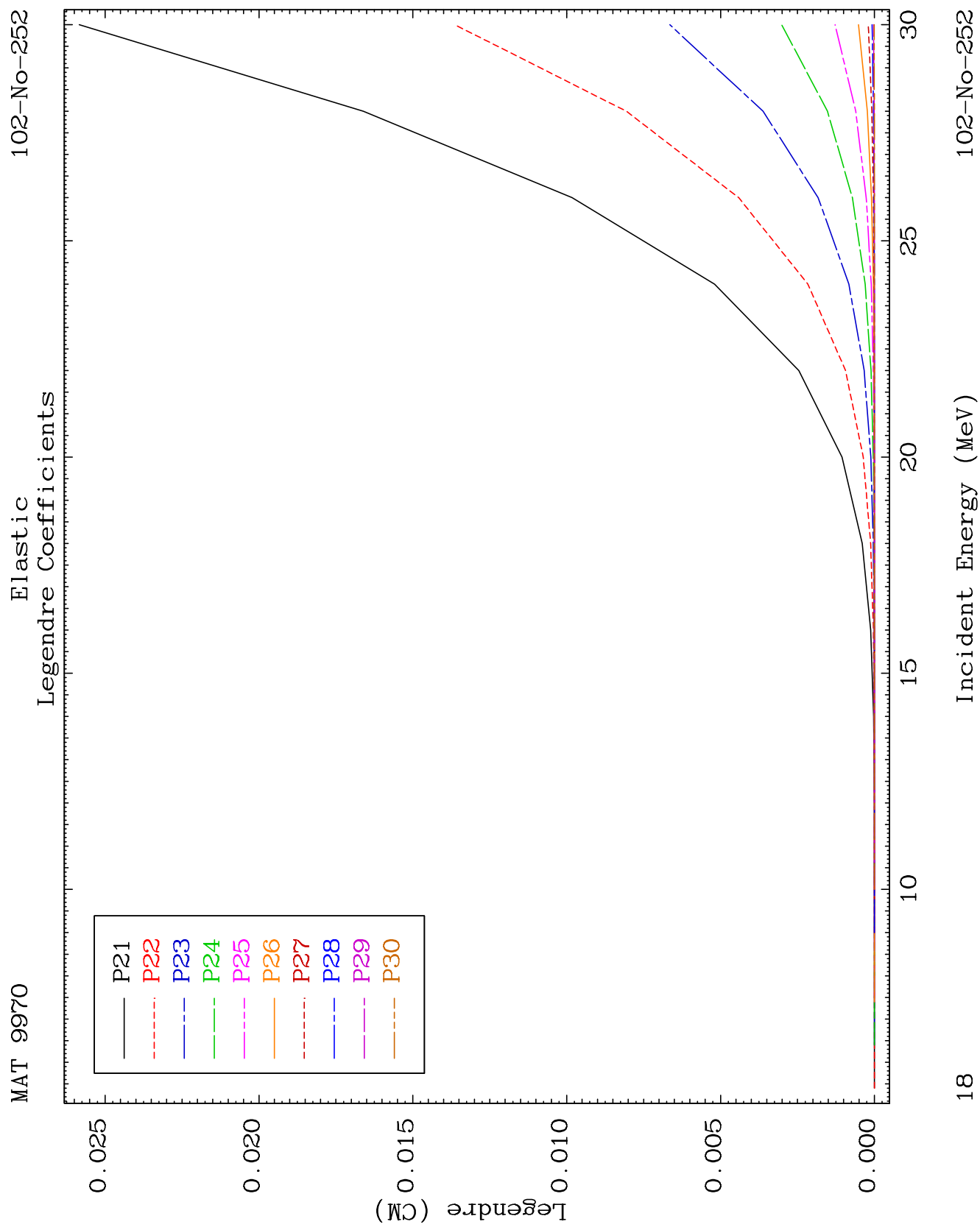


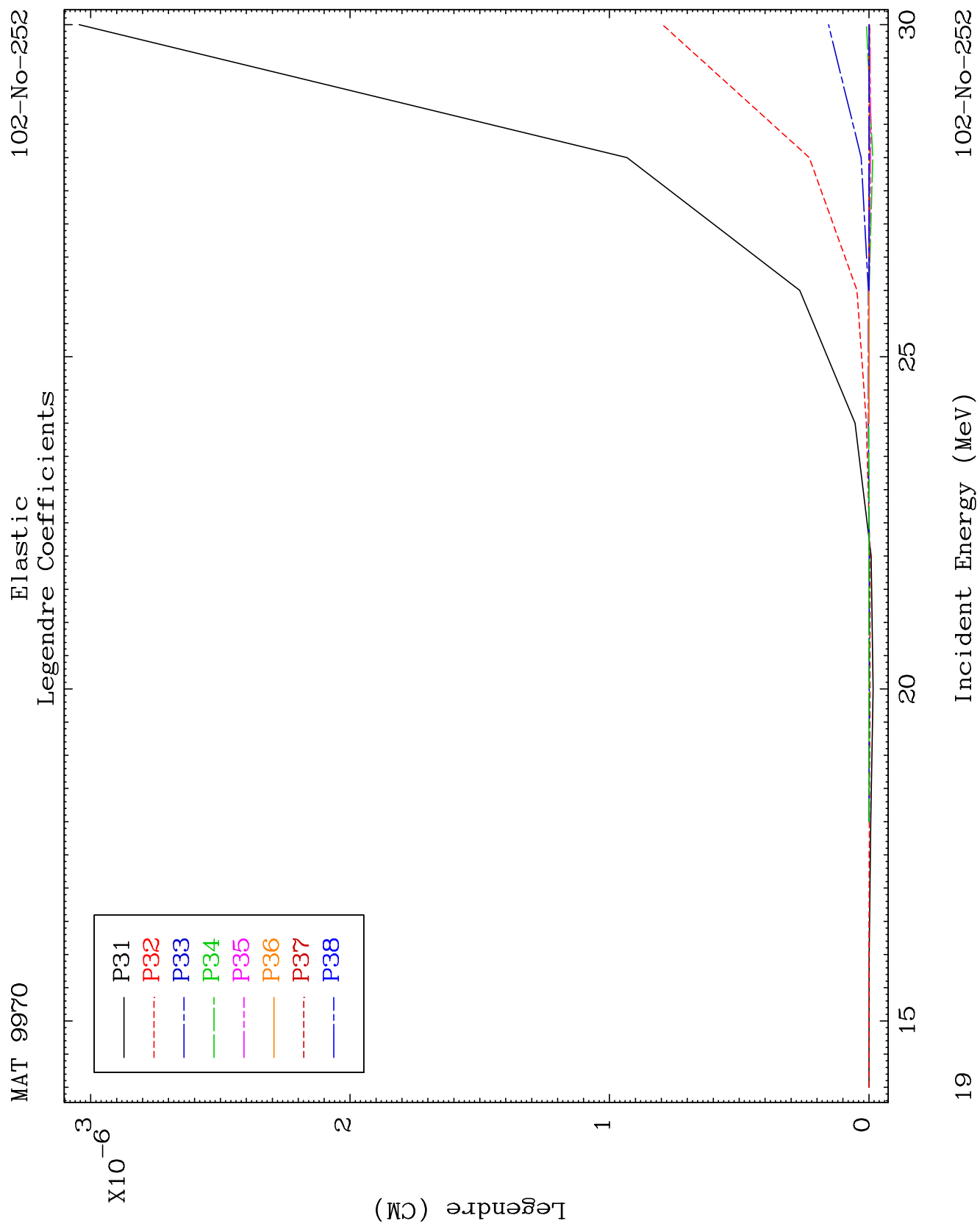


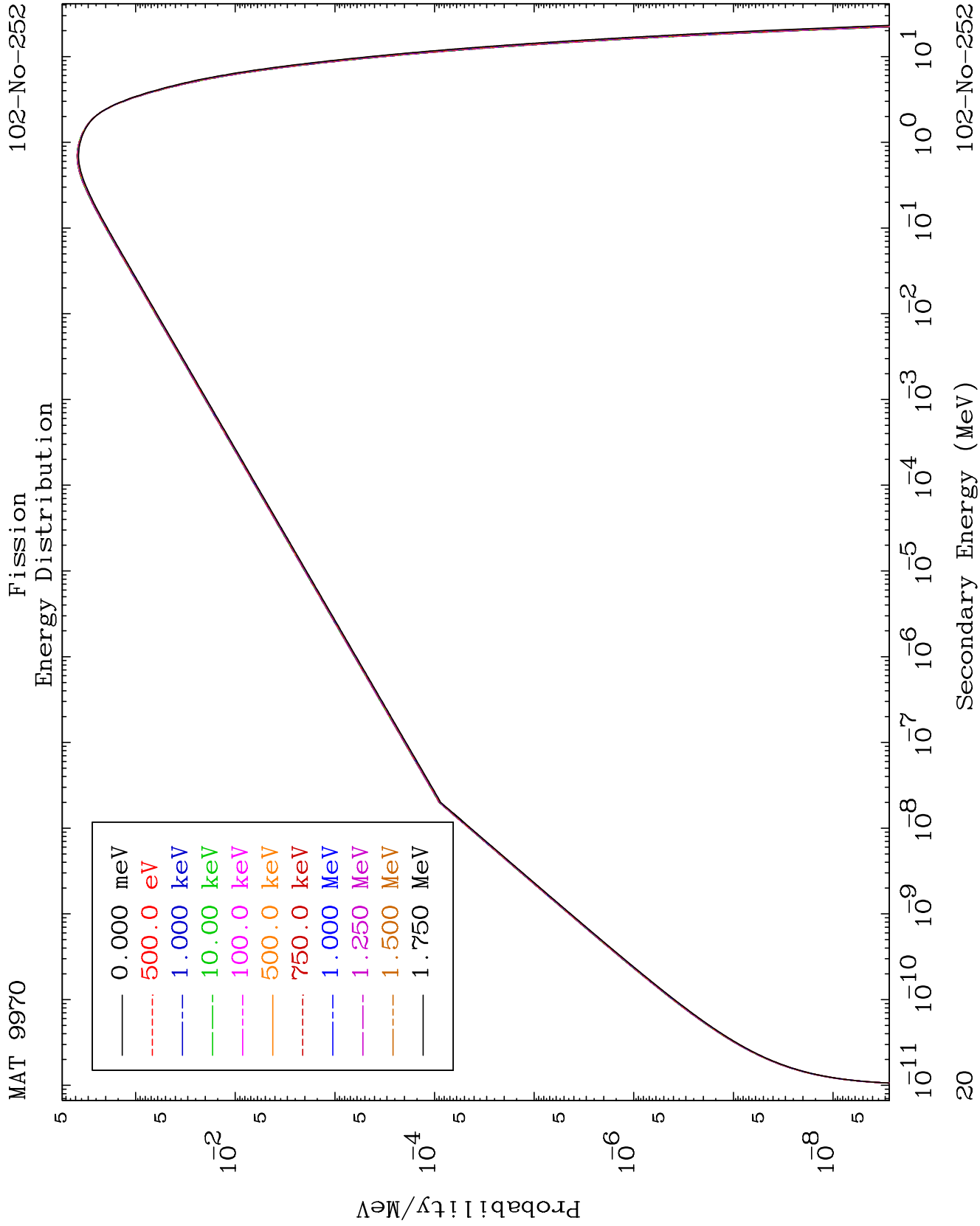


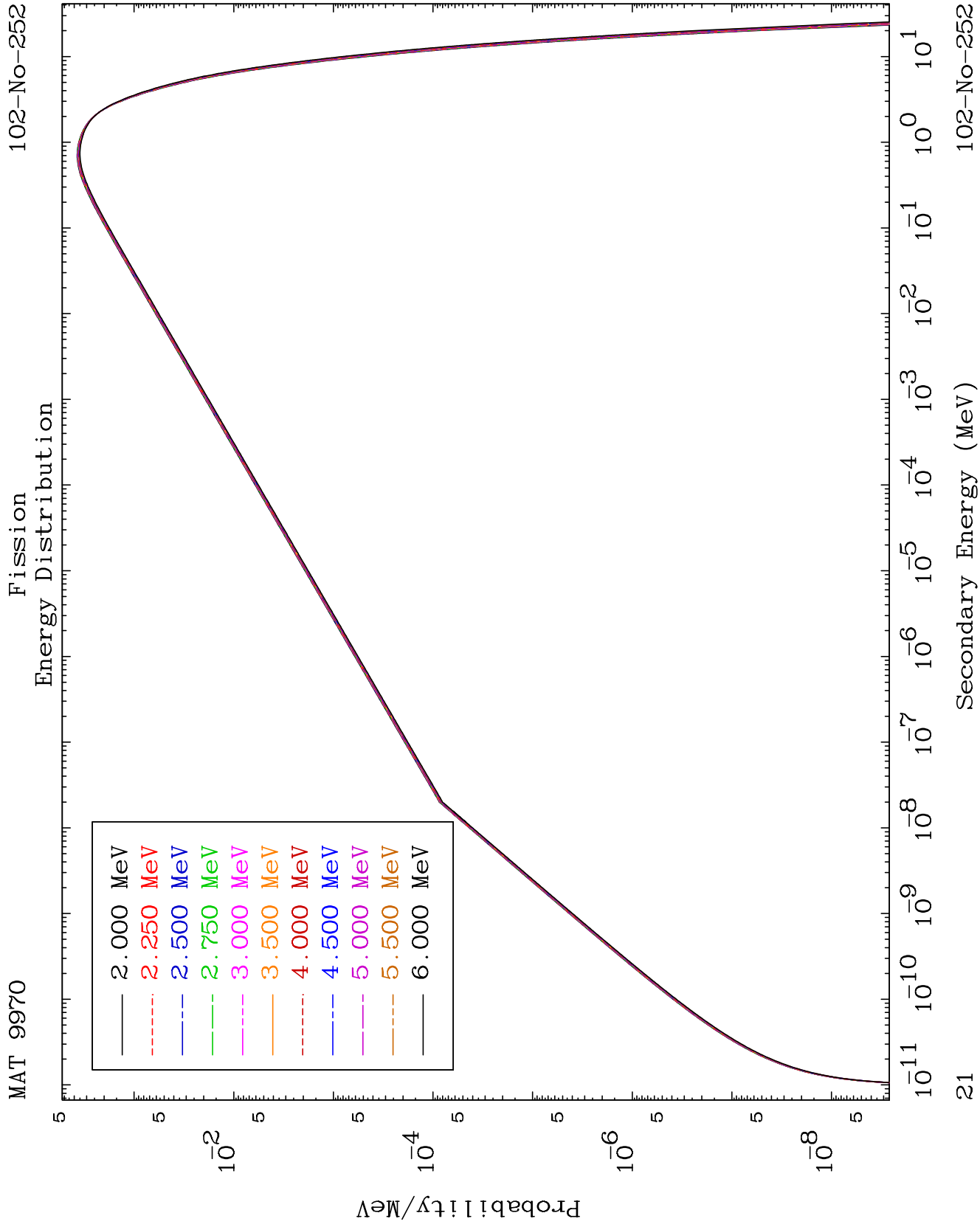








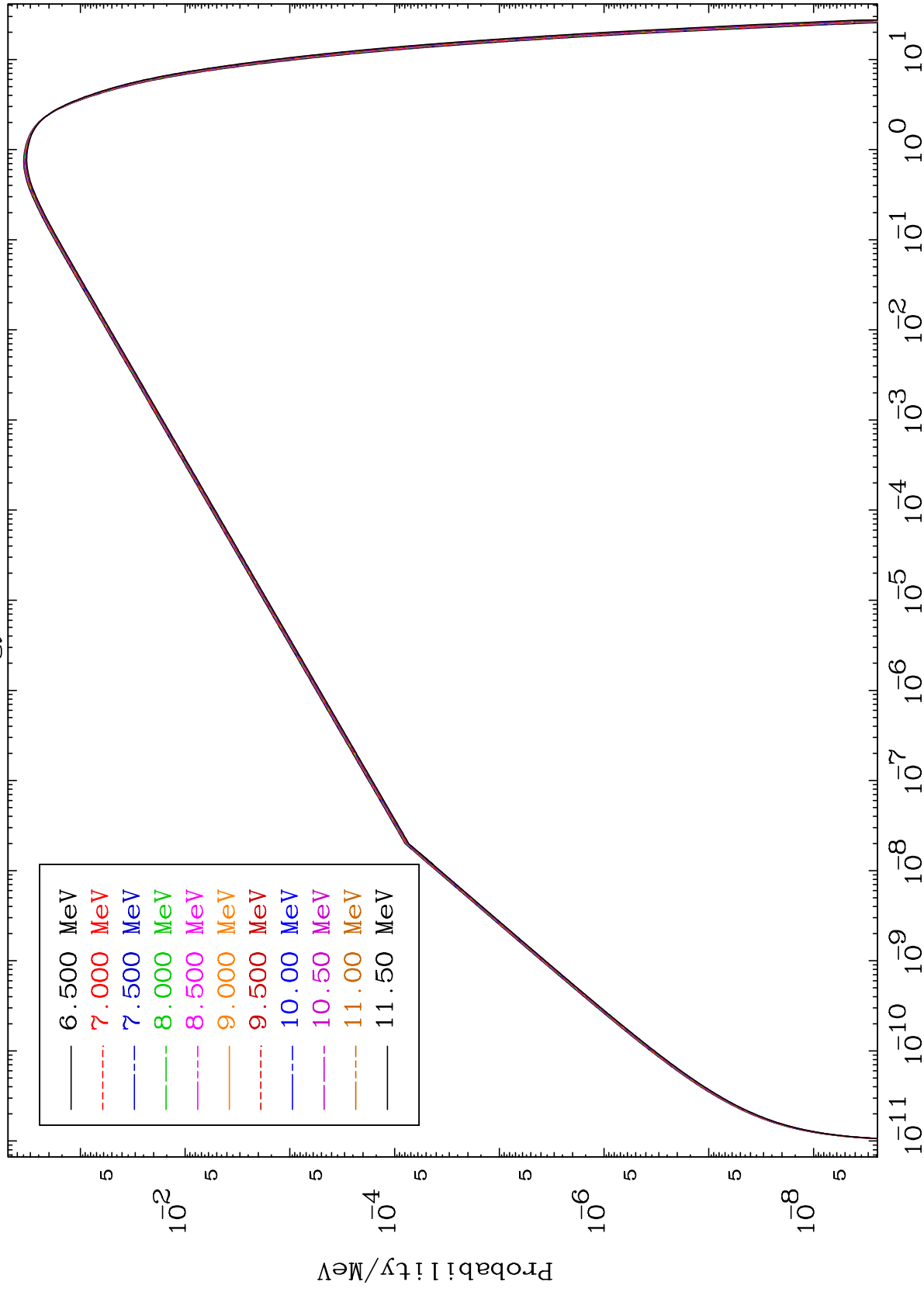




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Fission
Energy Distribution

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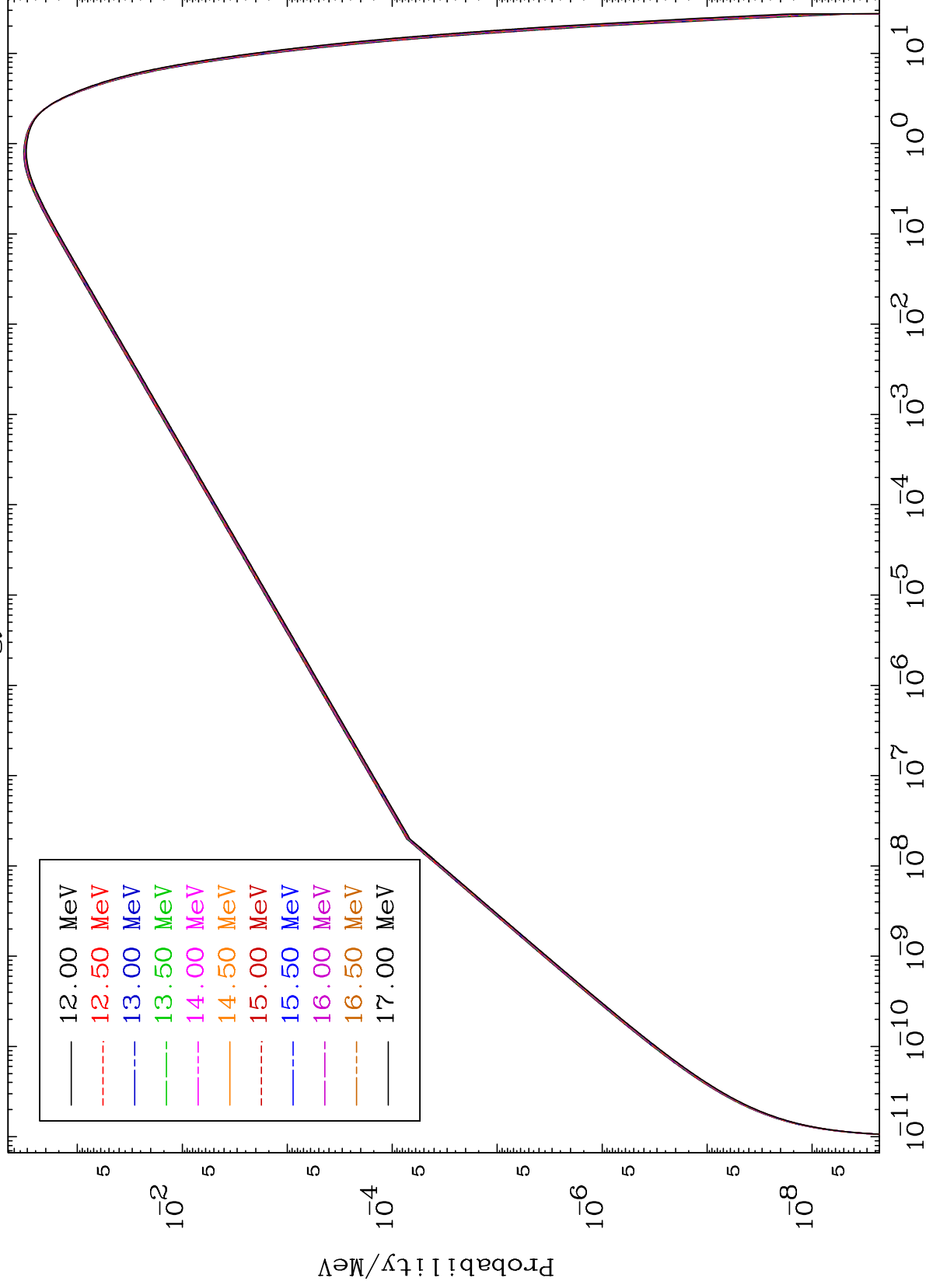
Secondary Energy (MeV)

102-No-252

MAT 9970

Fission
Energy Distribution

102-No-252



23

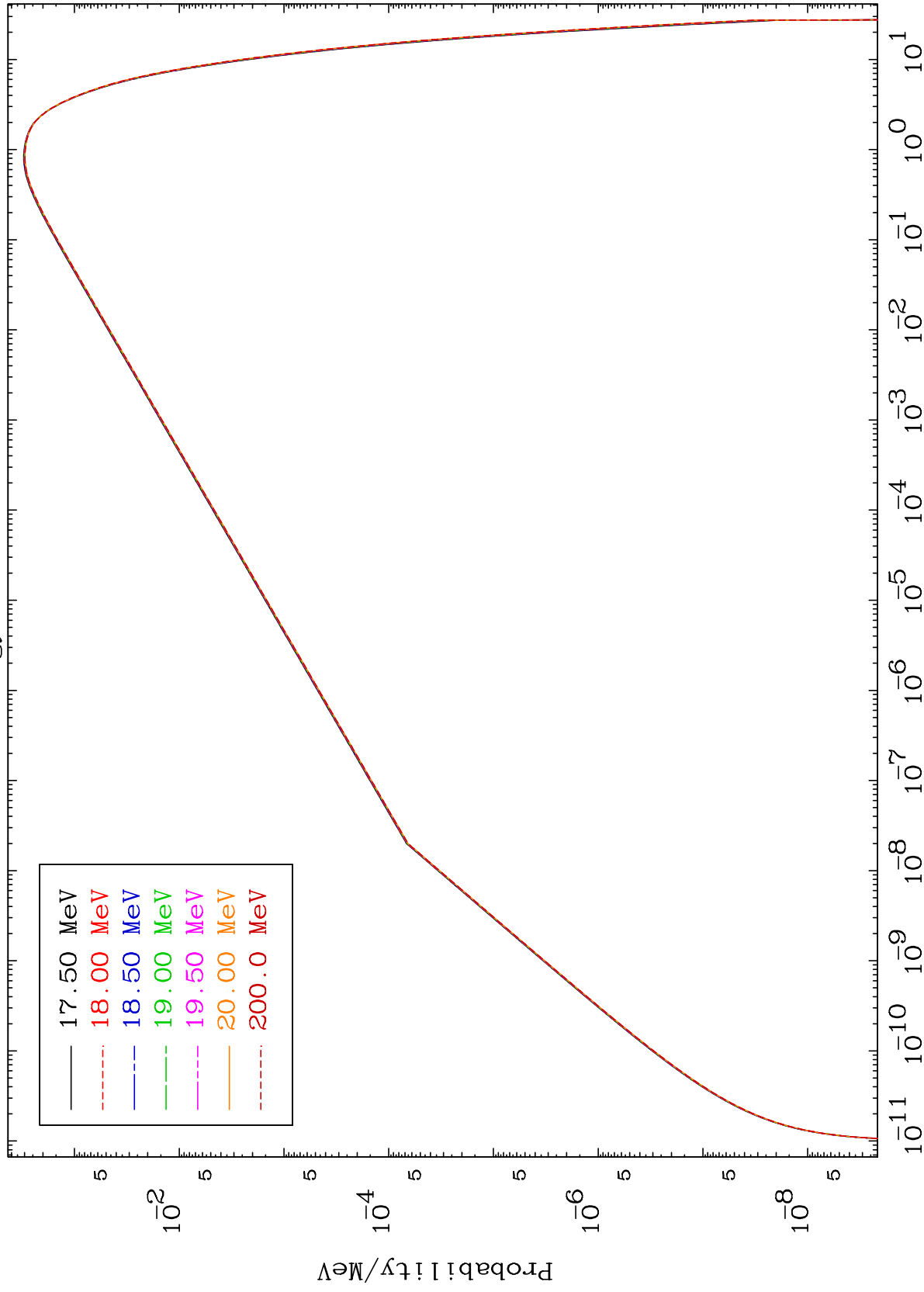
Secondary Energy (MeV)

102-No-252

MAT 9970

Fission
Energy Distribution

102-No-252



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Secondary Energy (MeV)

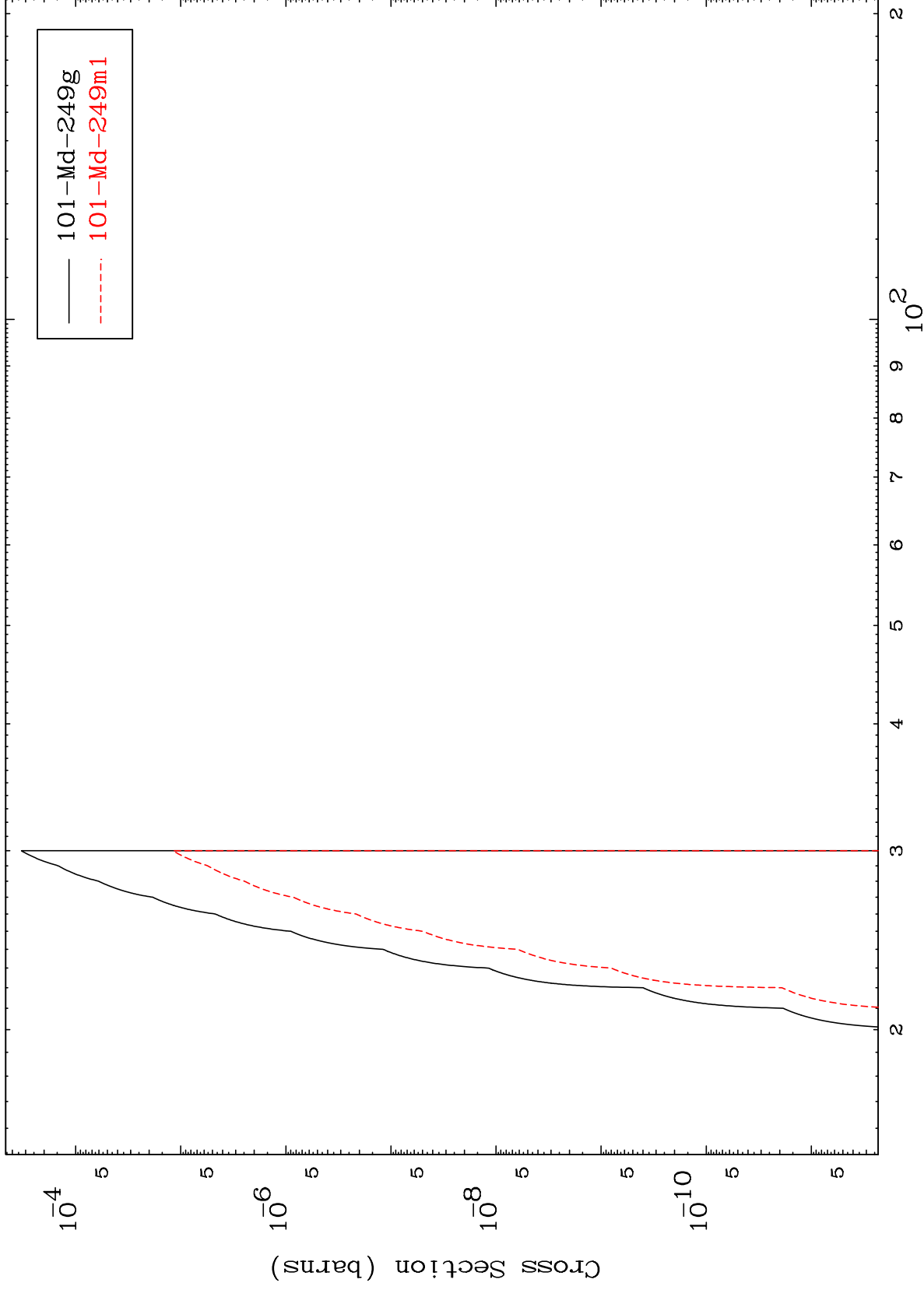
102-No-252

MAT 9970

(n,2n) d

Radionuclide Production Cross Section

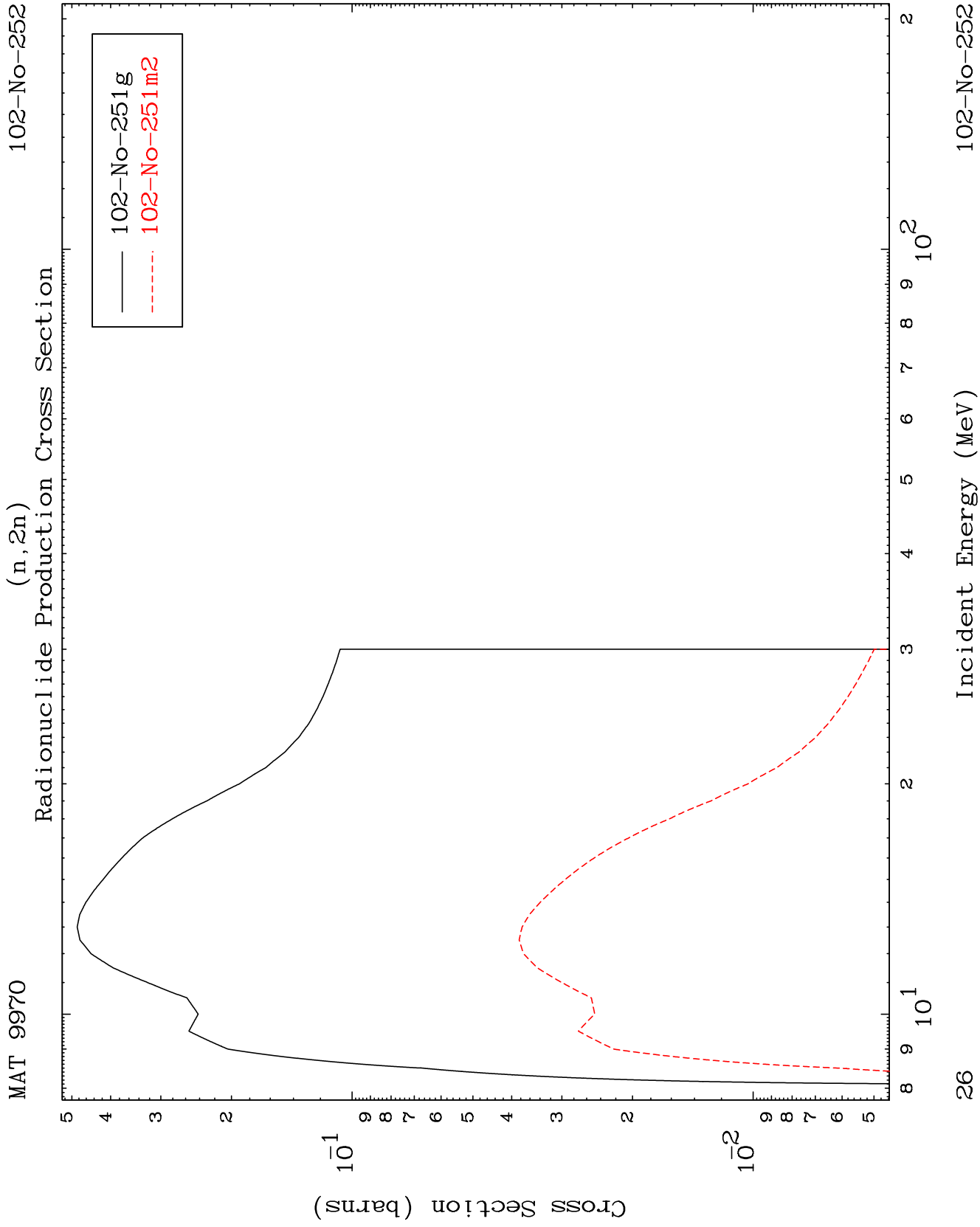
102-No-252



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Incident Energy (MeV)

102-No-252

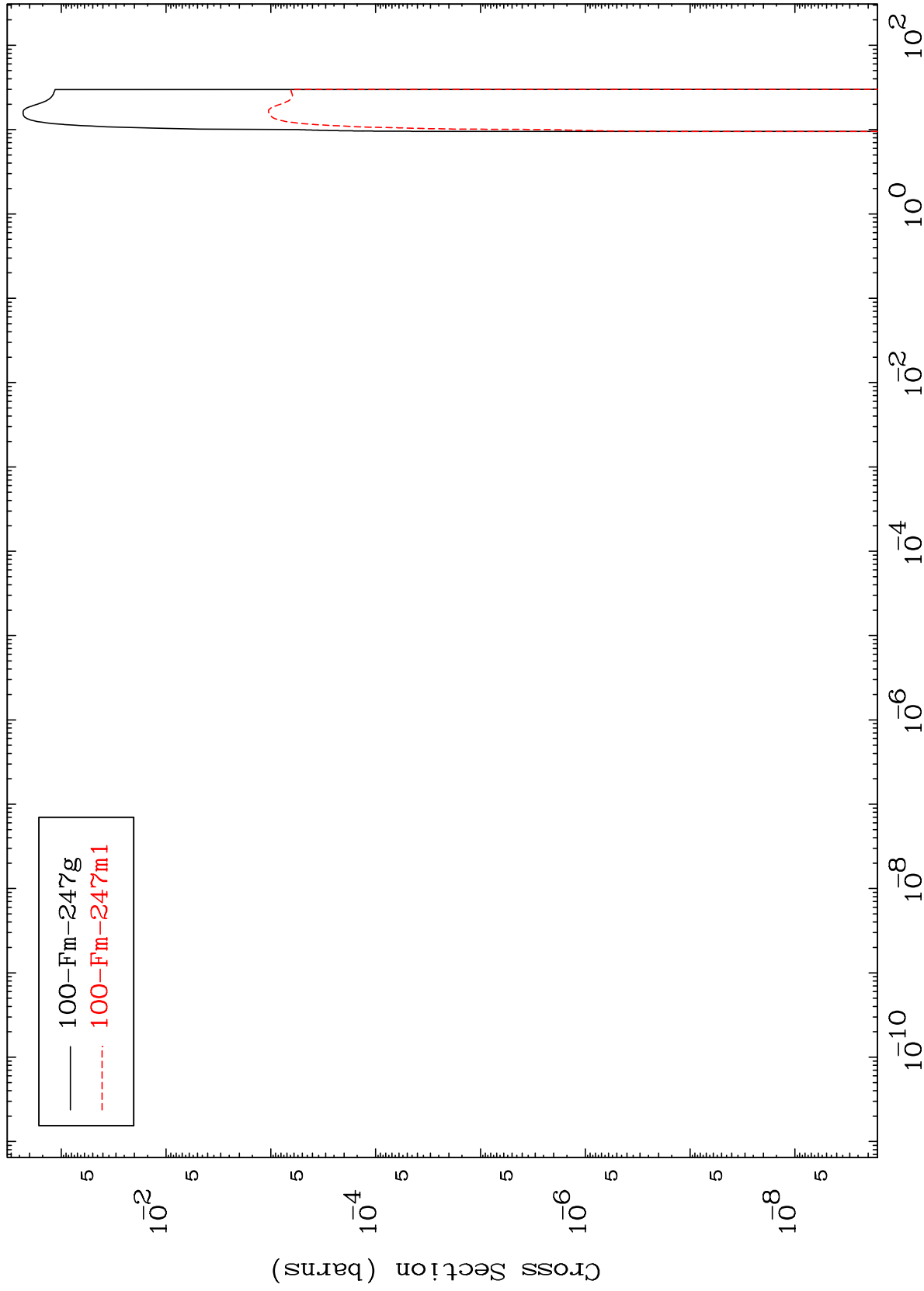


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(n,2n) α

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Radionuclide Production Cross Section



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Incident Energy (MeV)

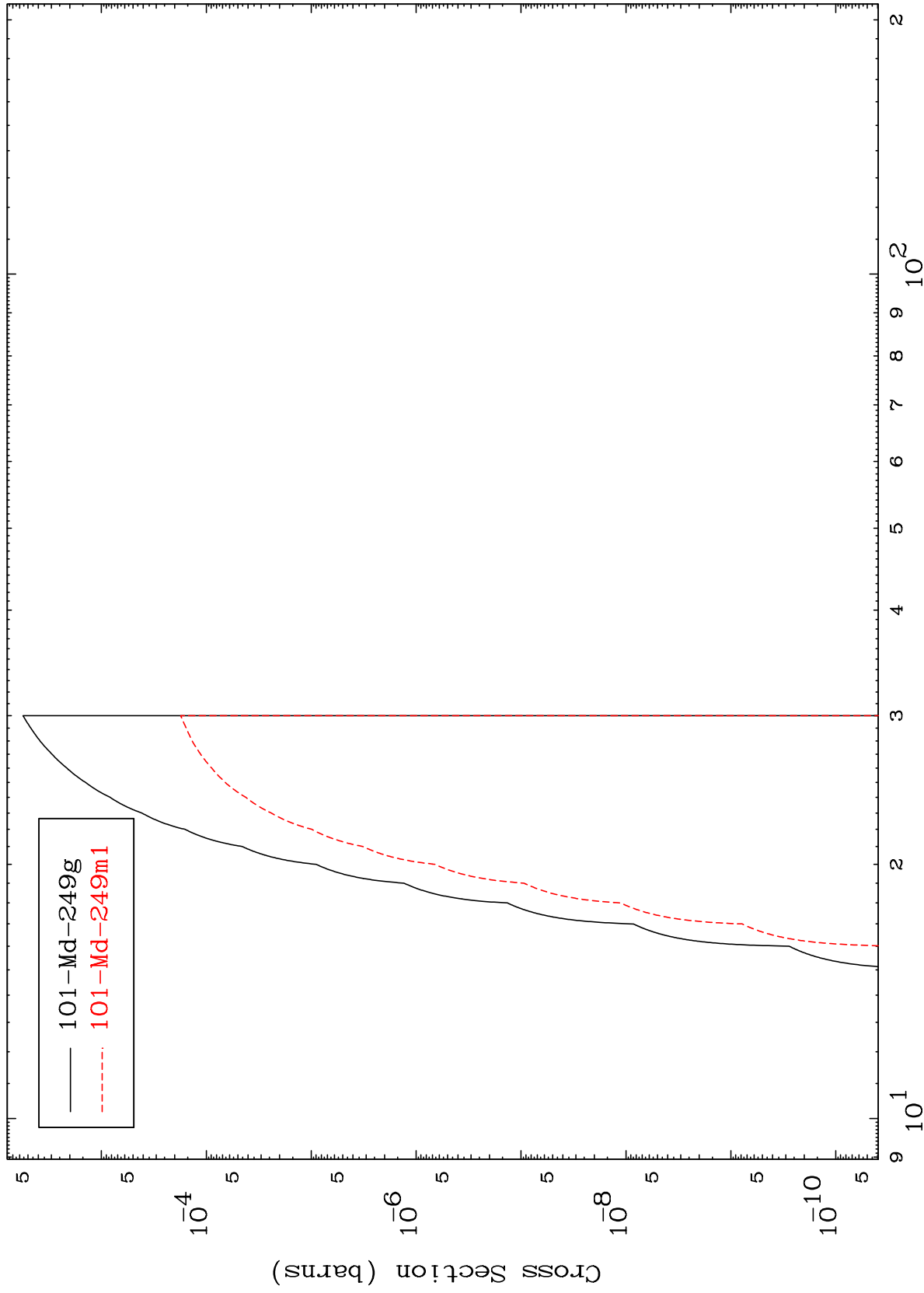
102-No-252

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(n,n') t

102-No-252

Radionuclide Production Cross Section



28

Incident Energy (MeV)

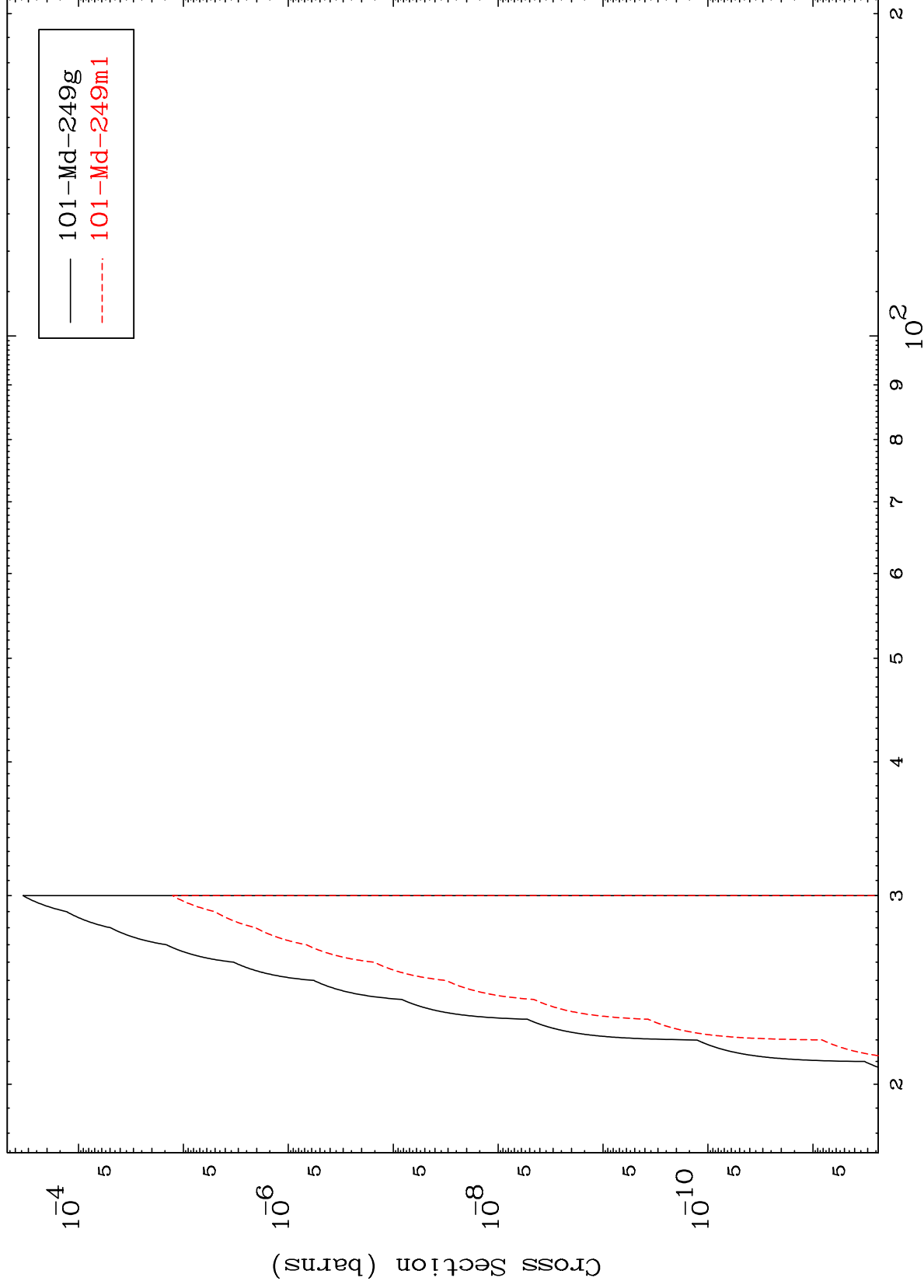
102-No-252

MAT 9970

(n,3n) p

102-No-252

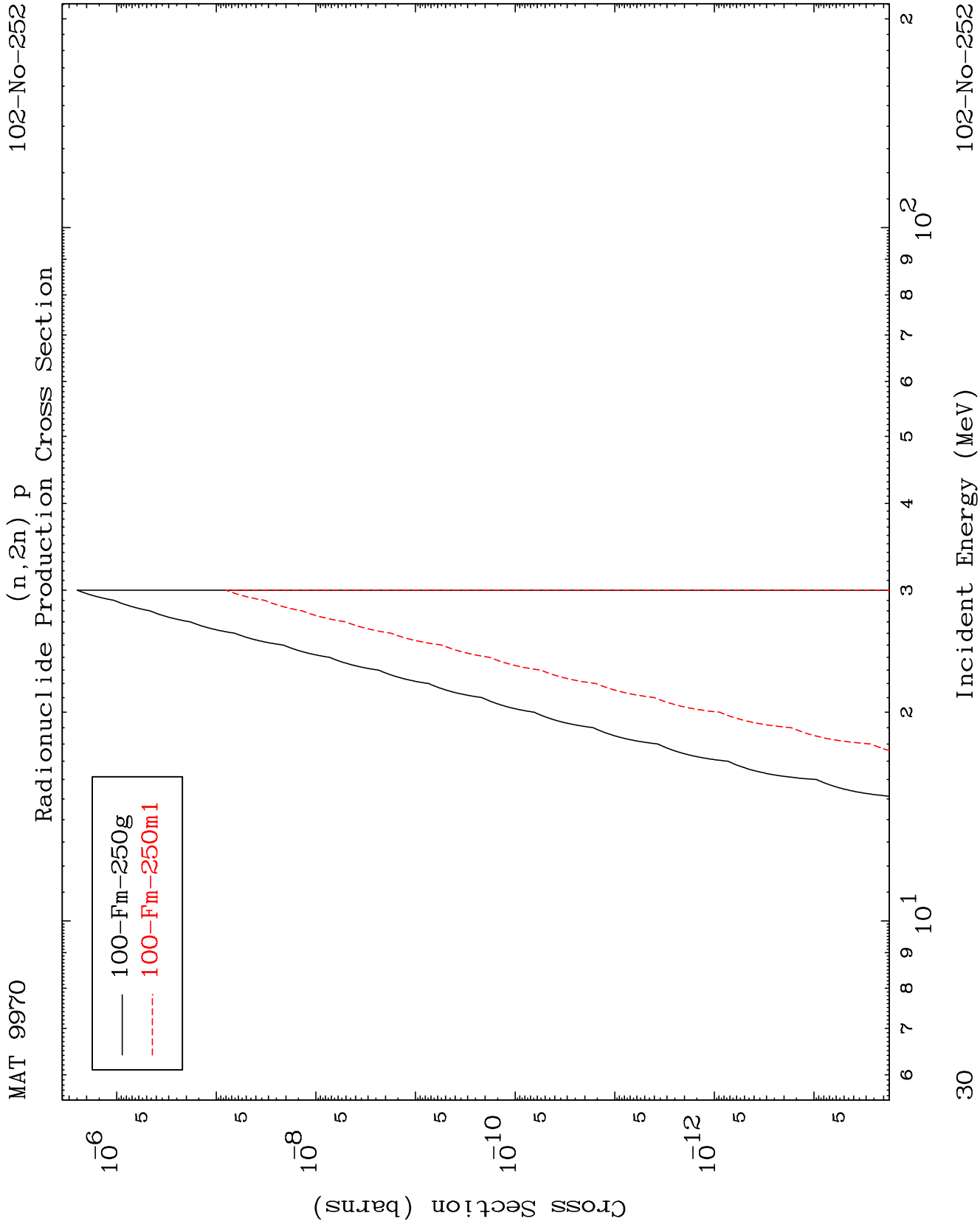
Radionuclide Production Cross Section

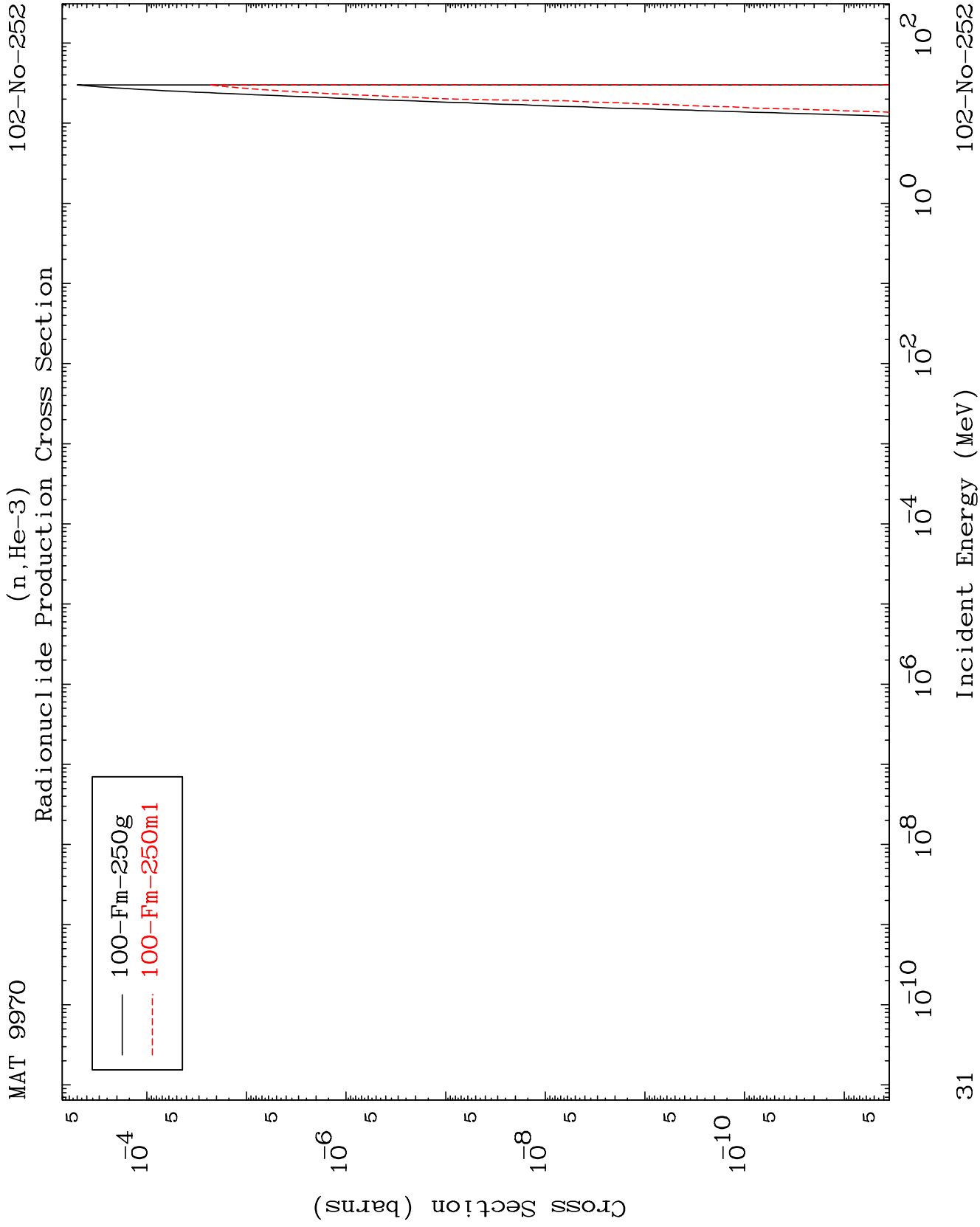


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Incident Energy (MeV)

102-No-252



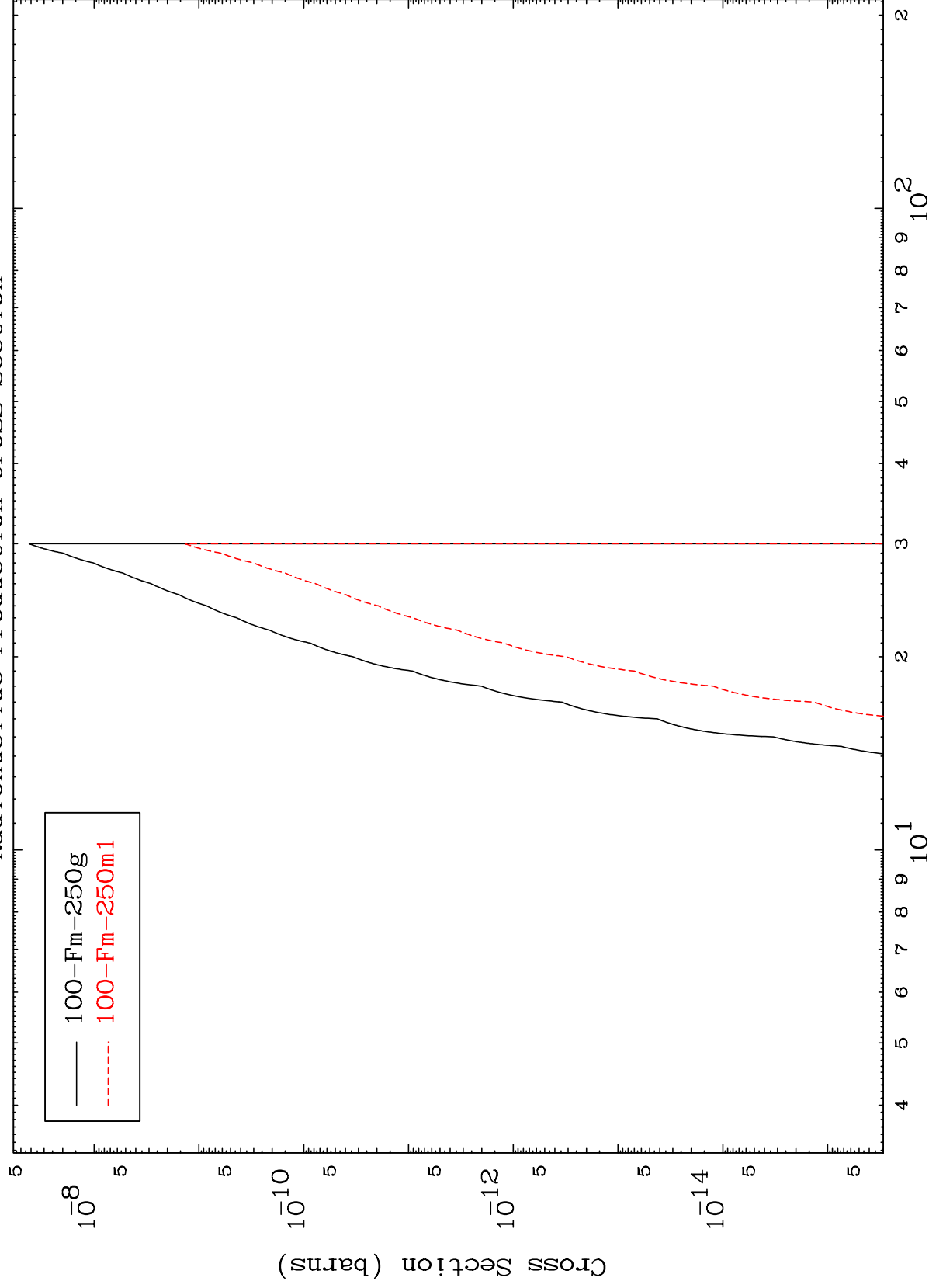


MAT 9970

(n,p) d

102-No-252

Radionuclide Production Cross Section



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Incident Energy (MeV)

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