

Program EVALPLOT  
(Version 2018-1)

by

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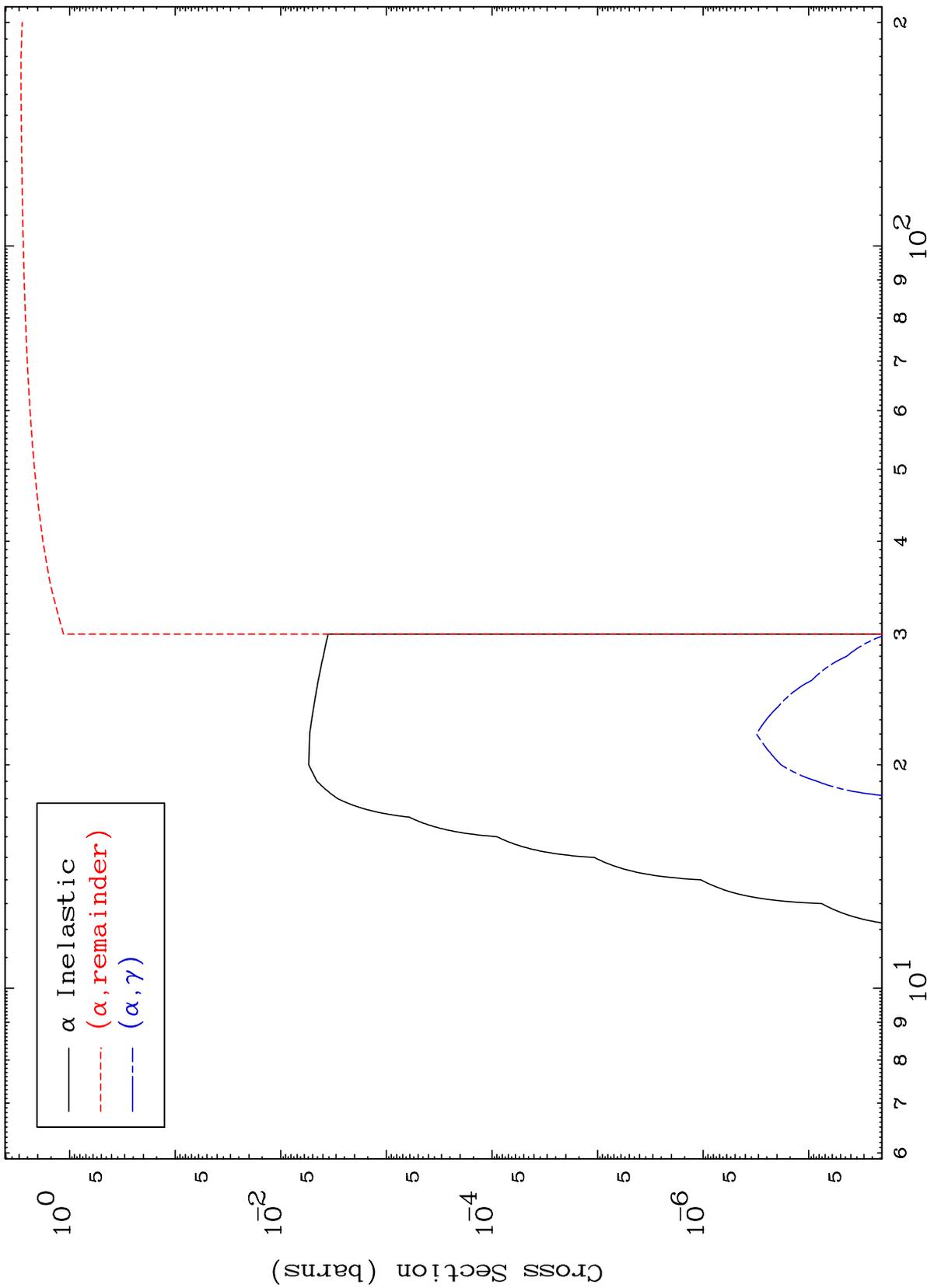
Web: [redcullen1.net/HOMEPAGE.NEW](http://redcullen1.net/HOMEPAGE.NEW)

Press Mouse Button to Start

MAT 8346

0 Kelvin  $\alpha$  Major Cross Sections

83-Bi-216



Incident Energy (MeV)

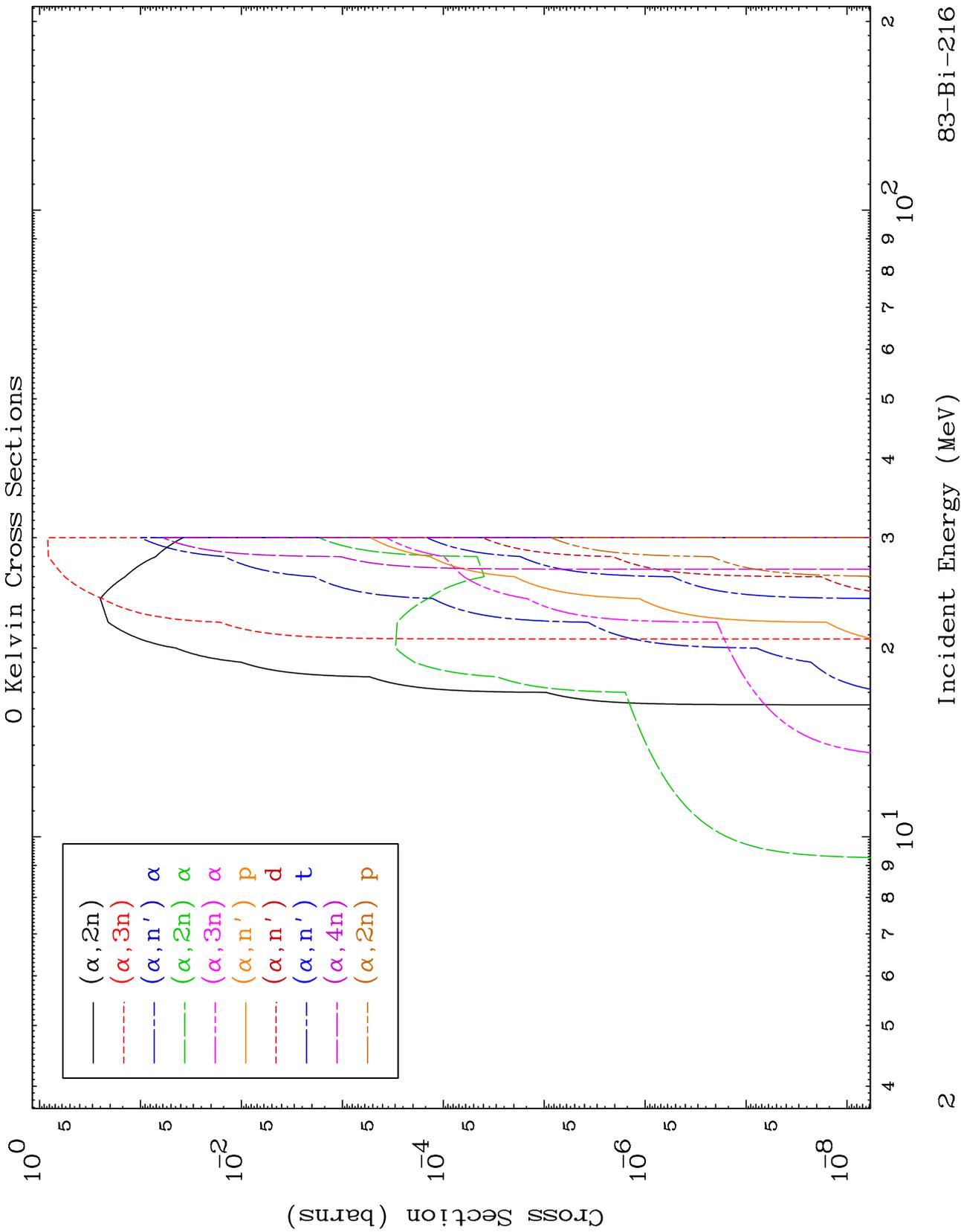
83-Bi-216

1

MAT 8346

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

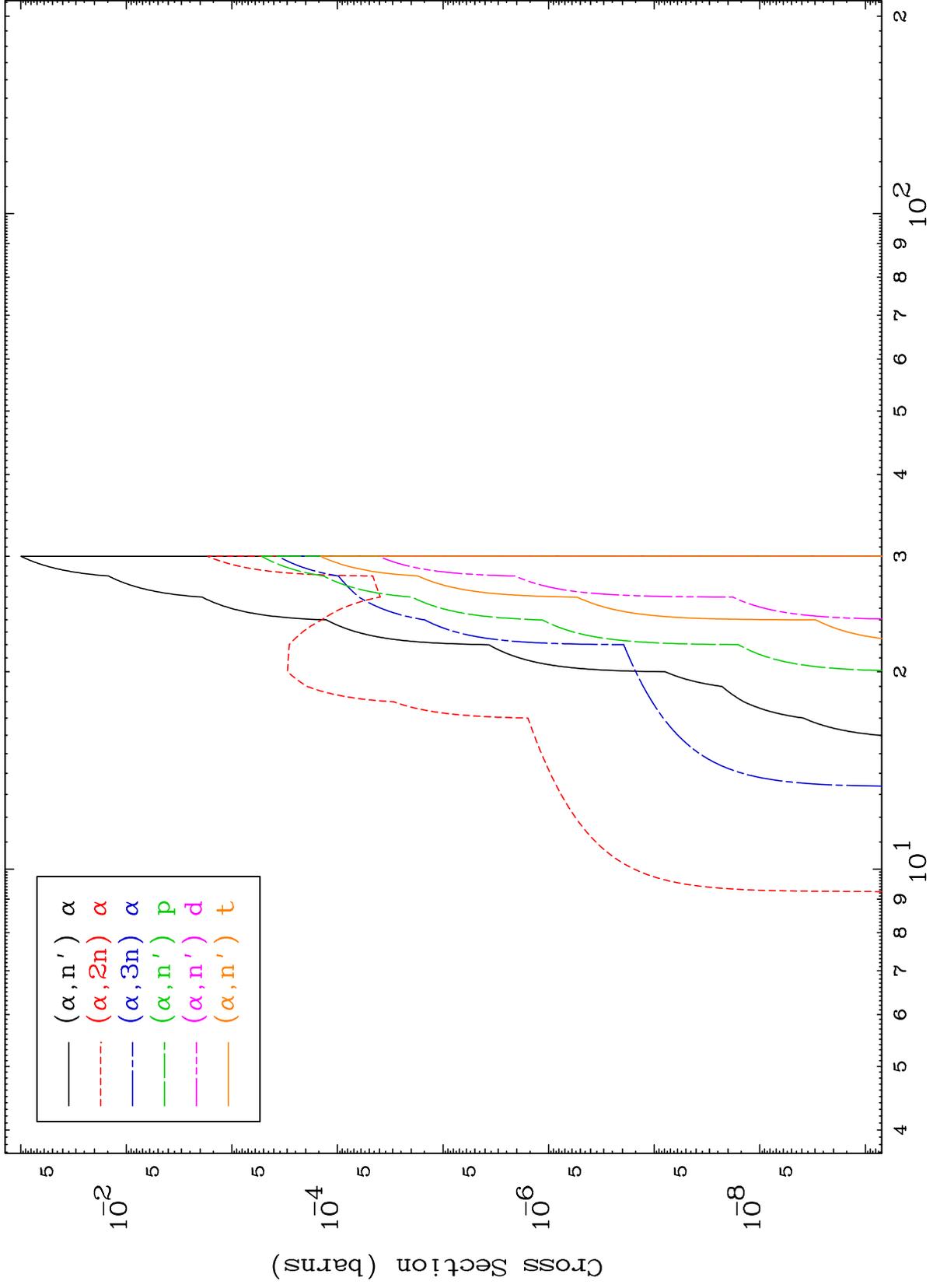
83-Bi-216



Incident Energy (MeV)

83-Bi-216

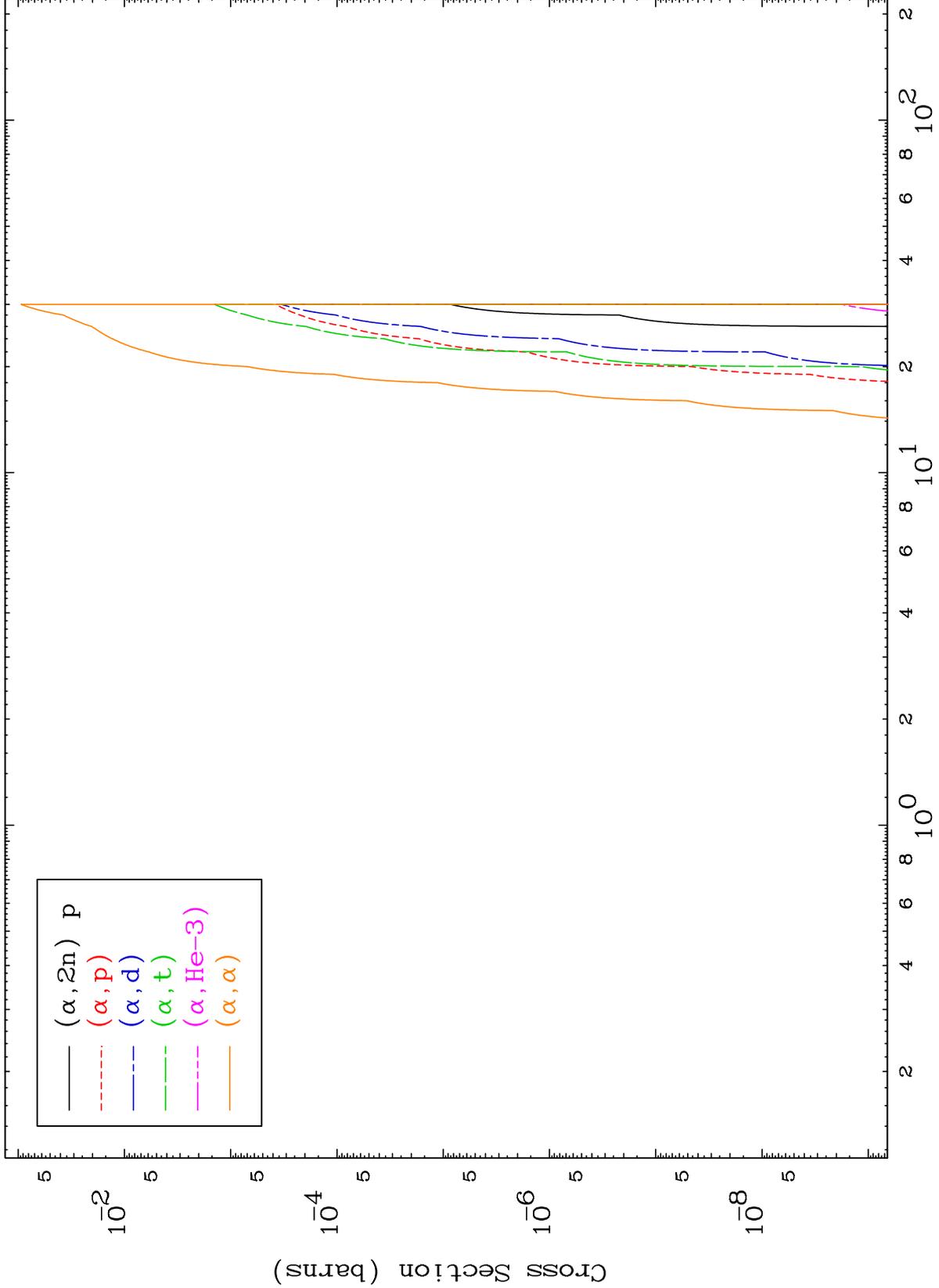
2



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$\alpha$  Charged Particle  
0 Kelvin Cross Sections

83-Bi-216

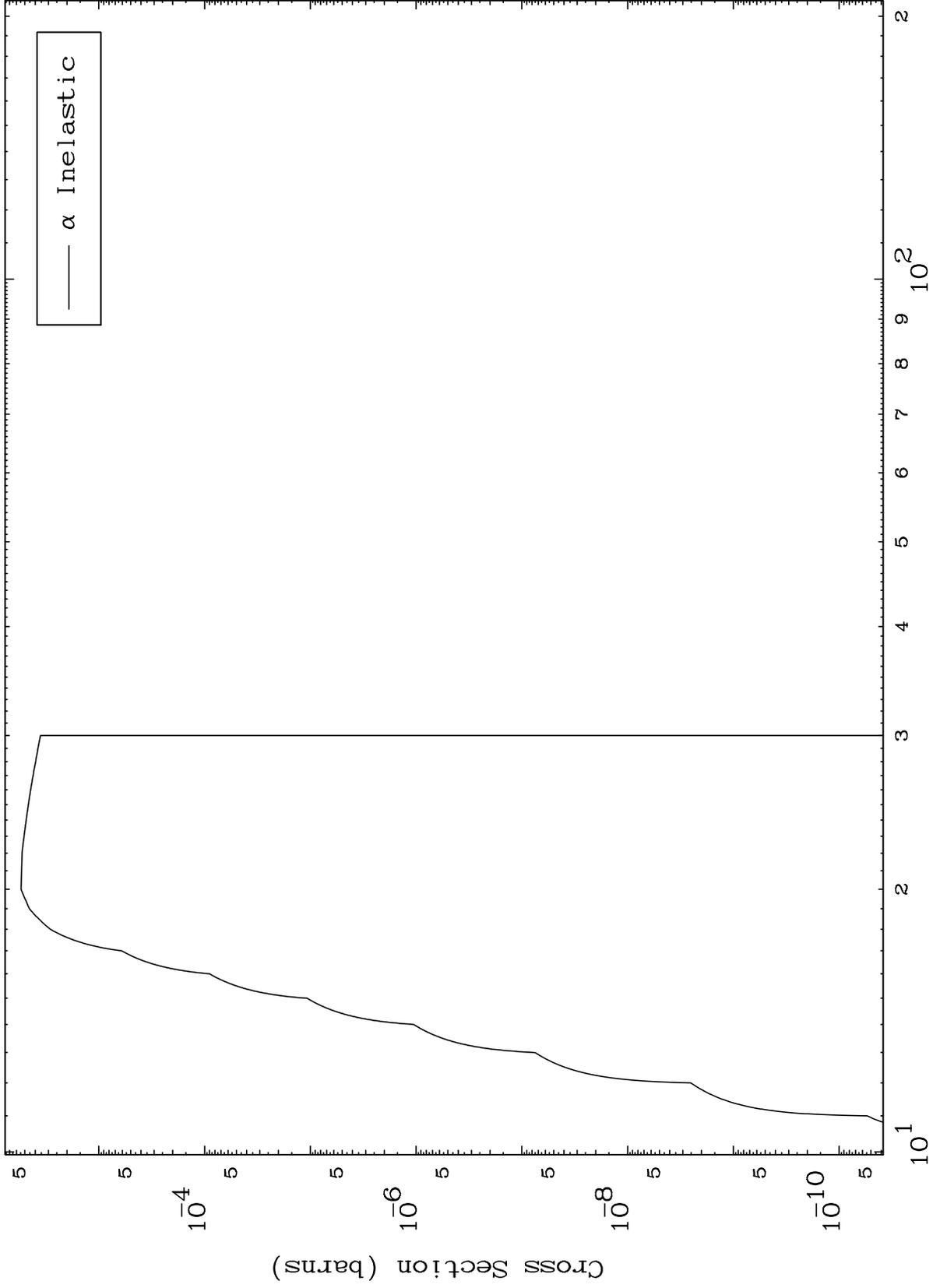


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( $\alpha, n'$ ) Level

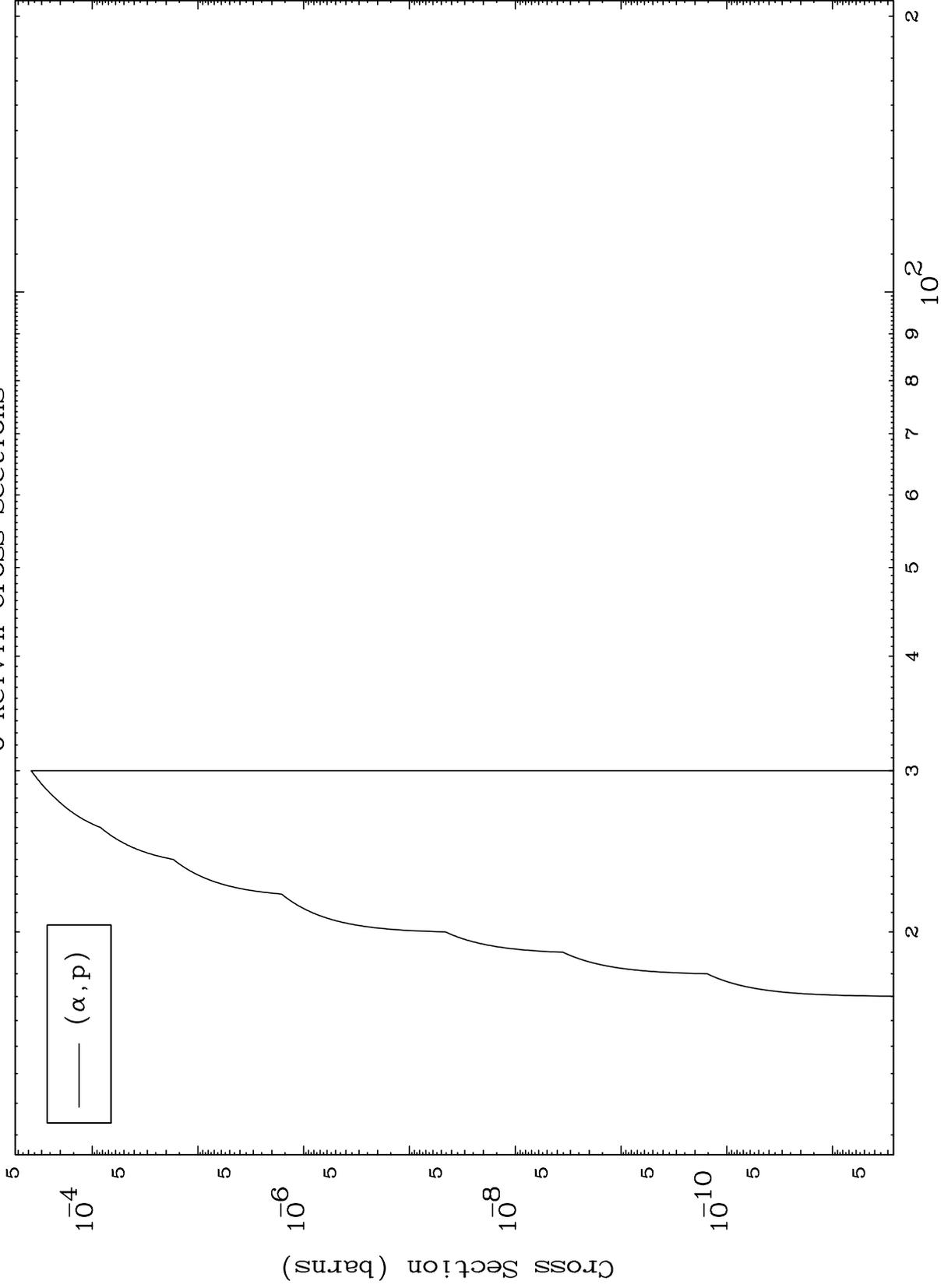
83-Bi-216

0 Kelvin Cross Sections



Incident Energy (MeV)

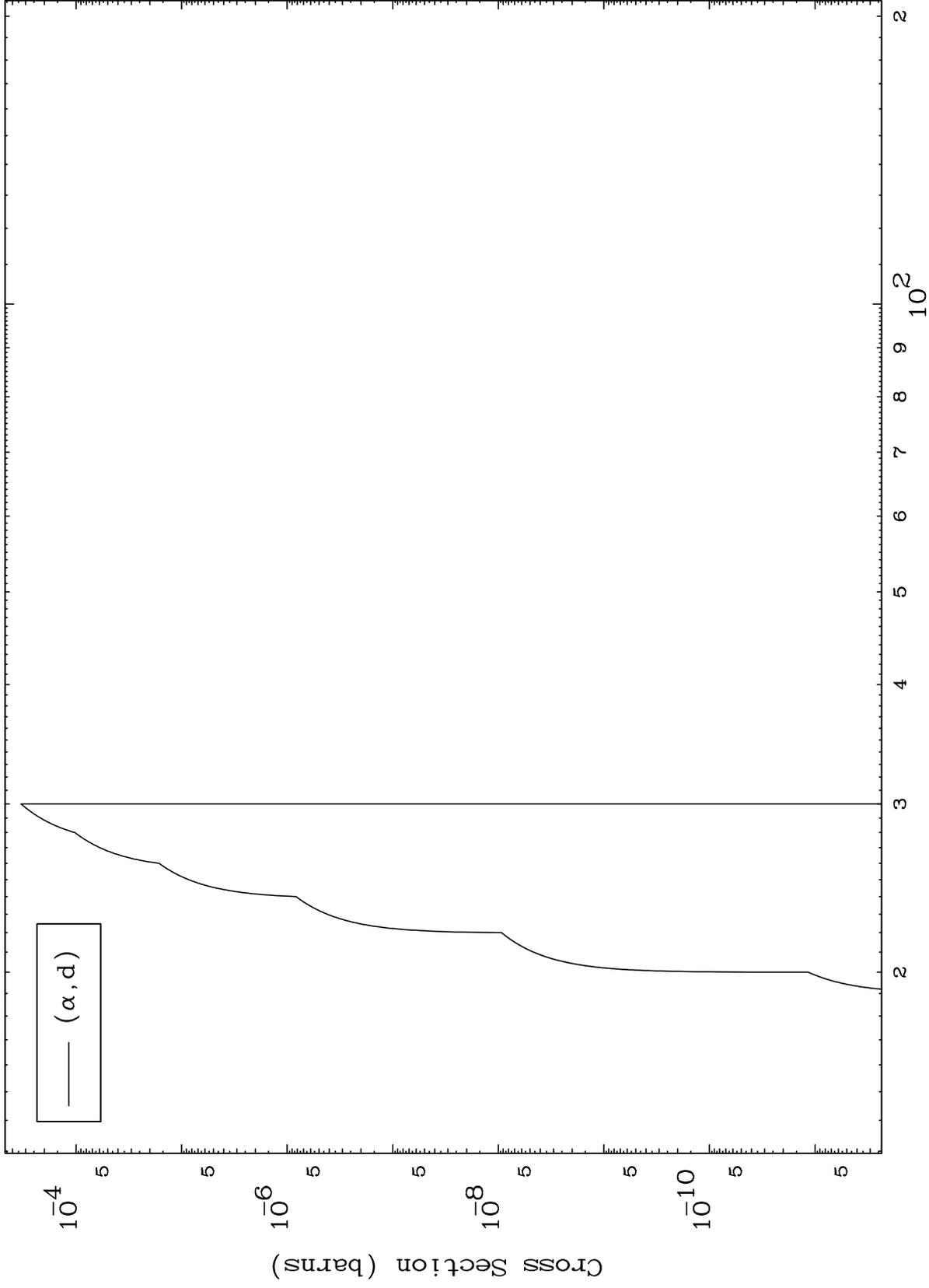
83-Bi-216



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( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

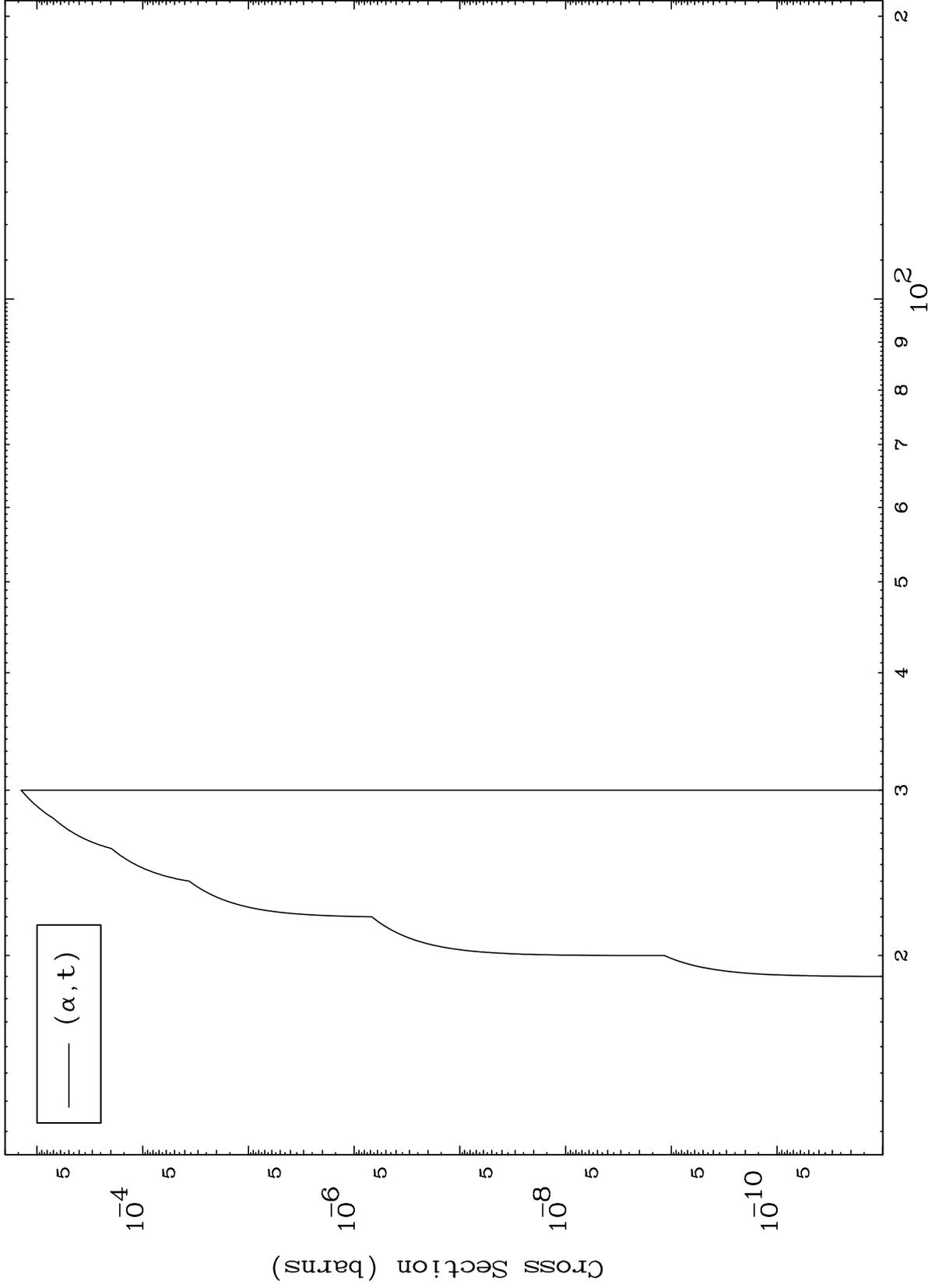
83-Bi-216

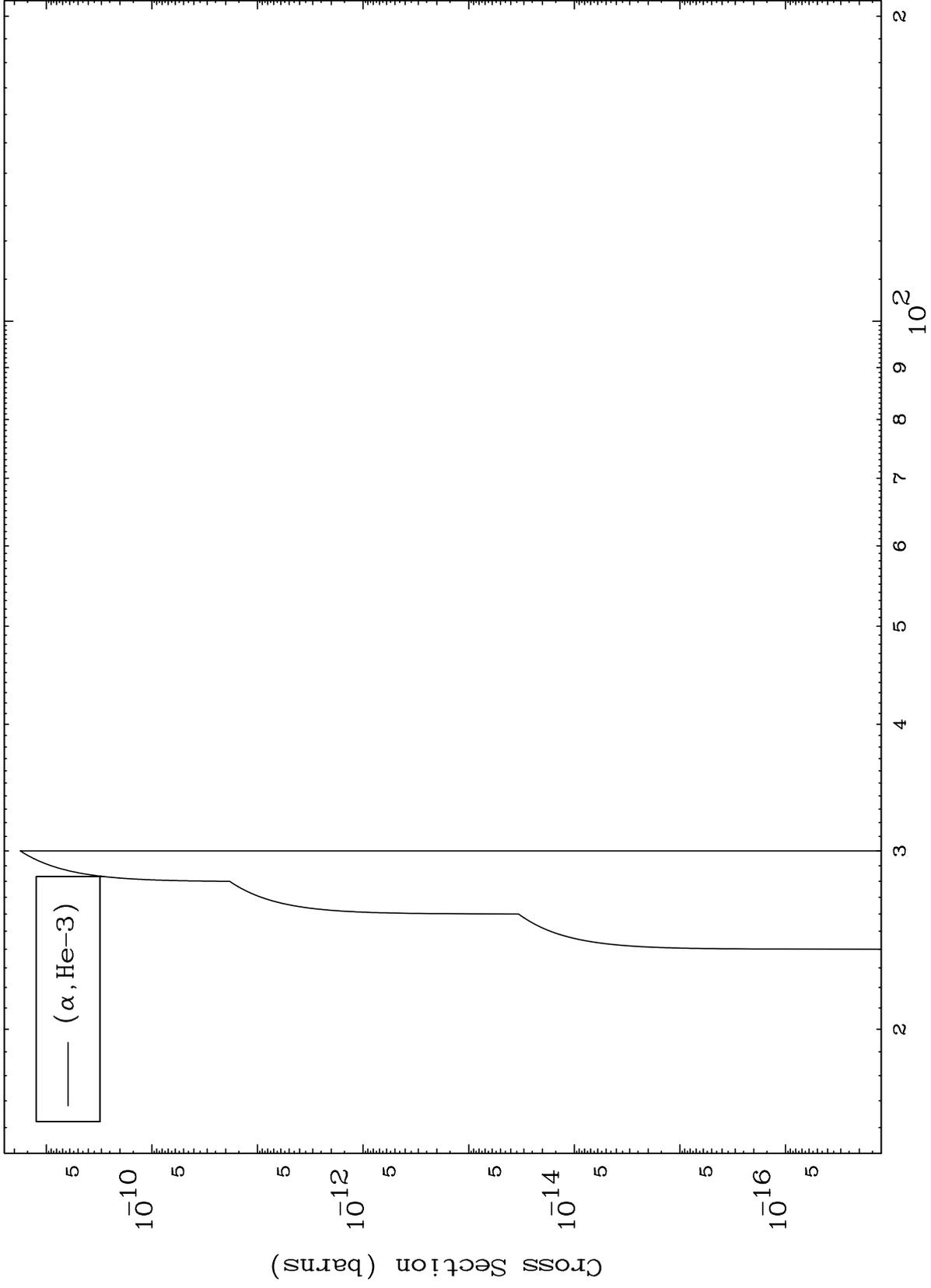


7

Incident Energy (MeV)

83-Bi-216



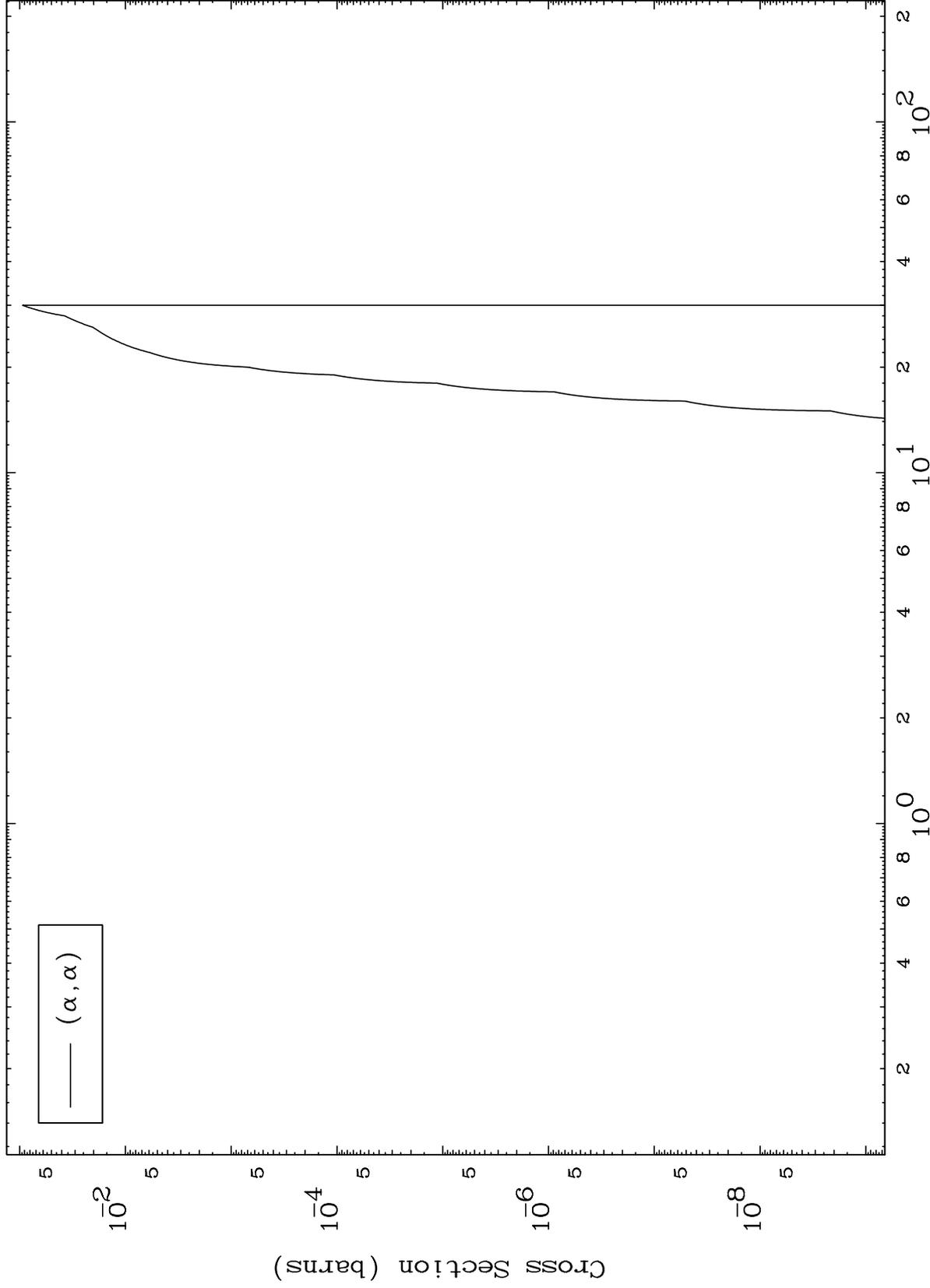


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( $\alpha, \alpha$ ) Levels

83-Bi-216

0 Kelvin Cross Sections



10

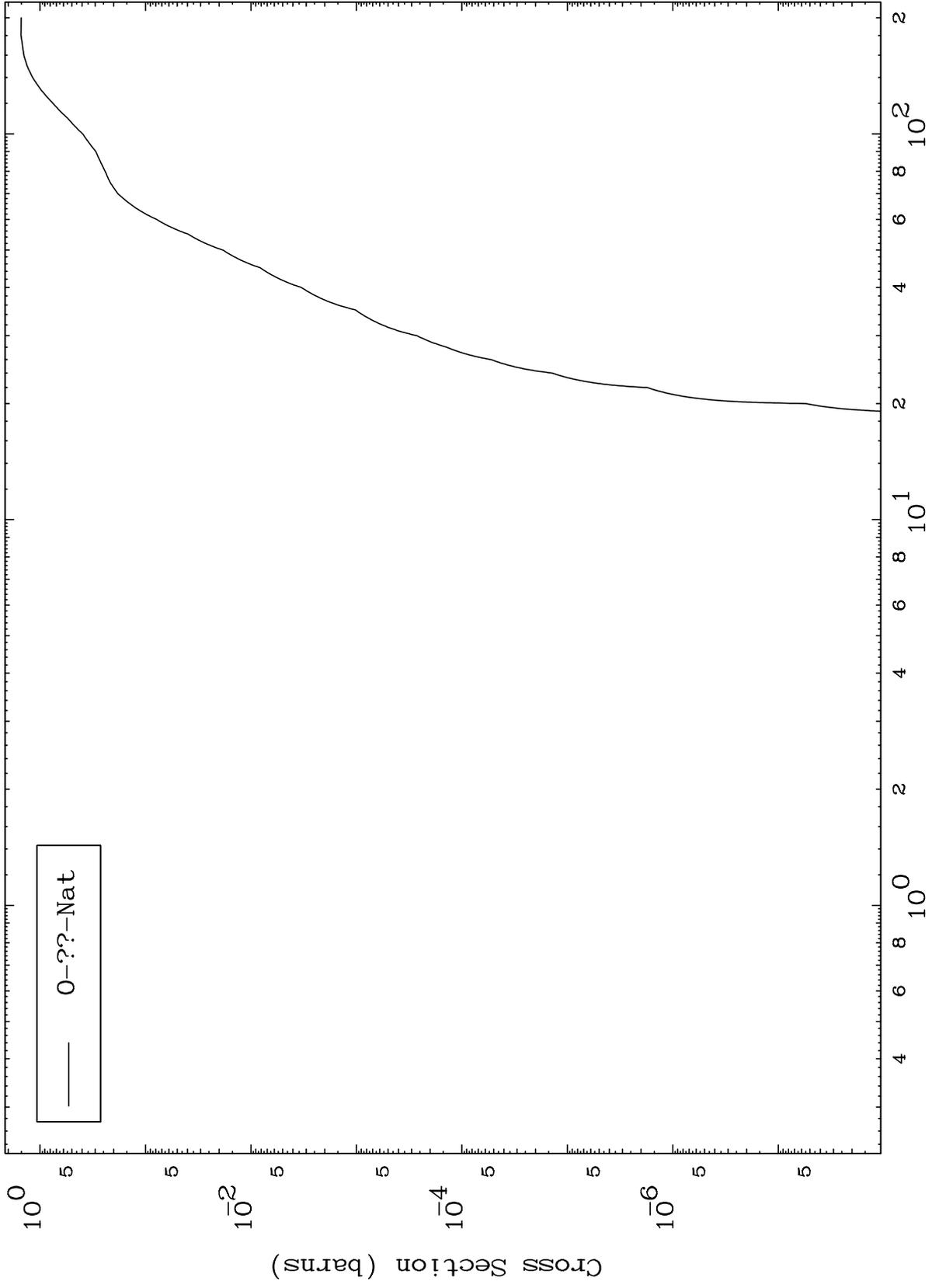
Incident Energy (MeV)

83-Bi-216

MAT 8346

83-Bi-216

$\alpha$  Fission  
Radionuclide Production Cross Section

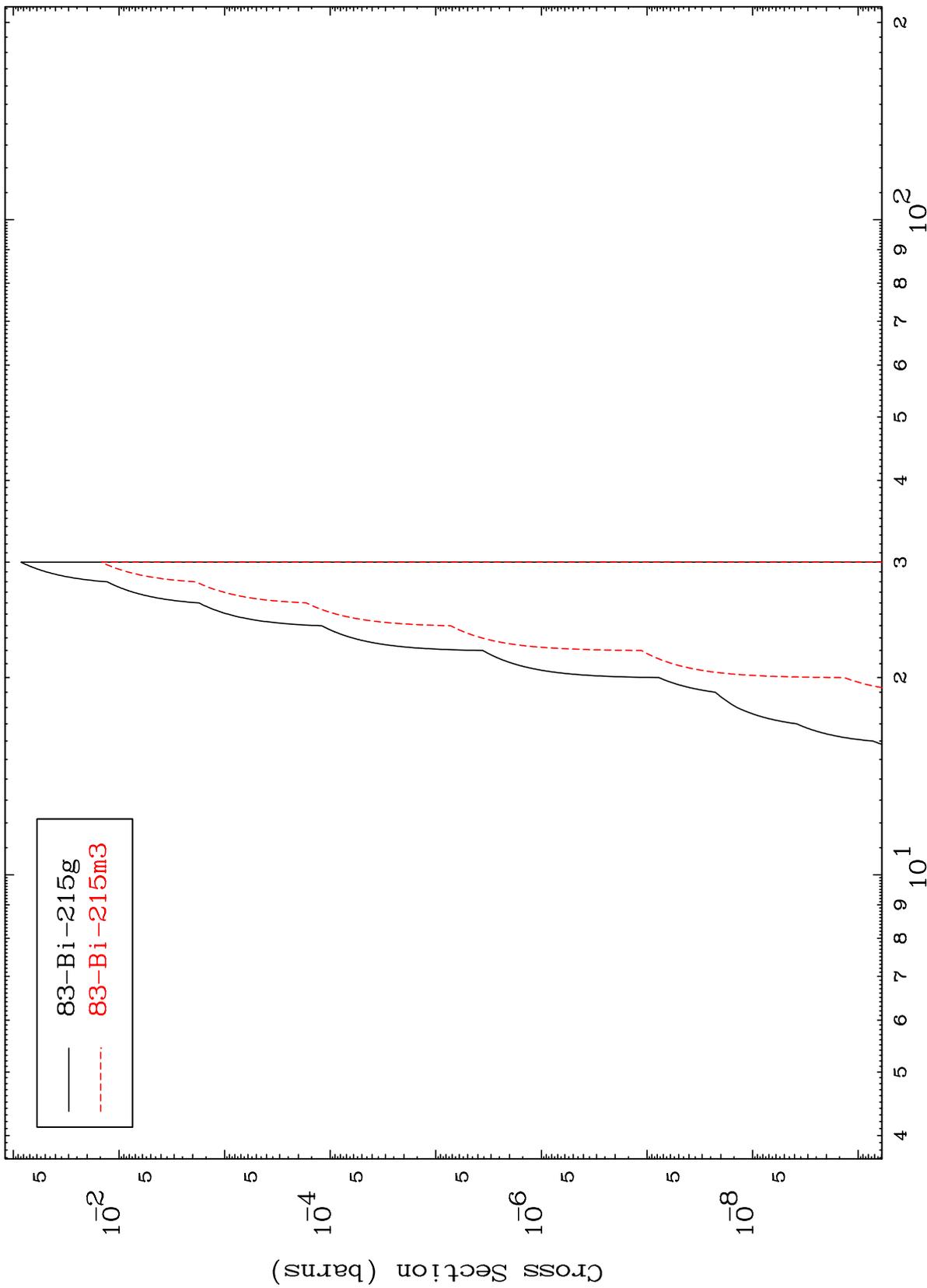


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83-Bi-216

$(\alpha, n')$   $\alpha$

Radionuclide Production Cross Section



12

Incident Energy (MeV)

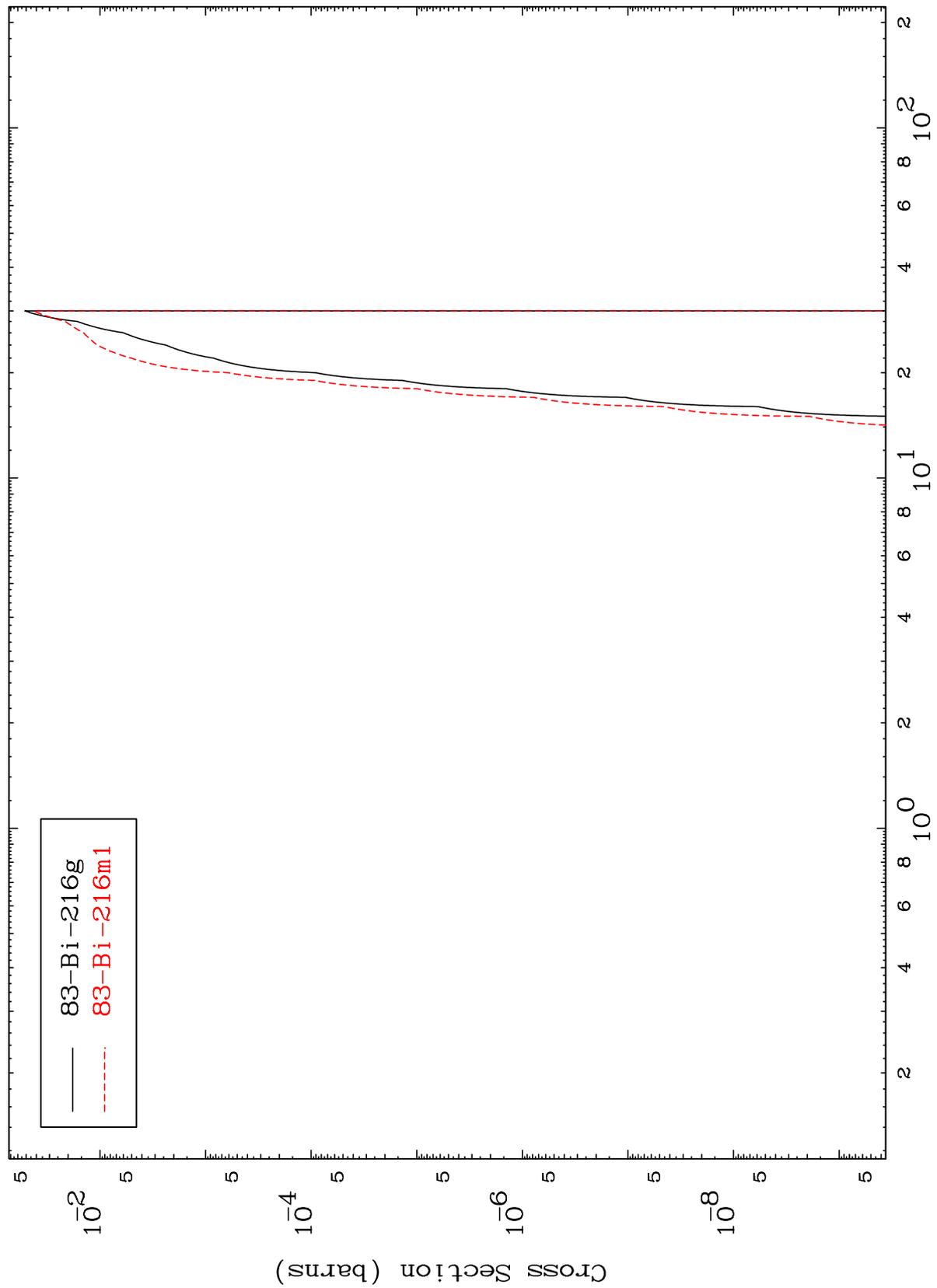
83-Bi-216

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83-Bi-216

( $\alpha, \alpha$ )

Radionuclide Production Cross Section



83-Bi-216g  
83-Bi-216m1

83-Bi-216

Incident Energy (MeV)

13