

Program EVALPLOT  
(Version 2018-1)

by

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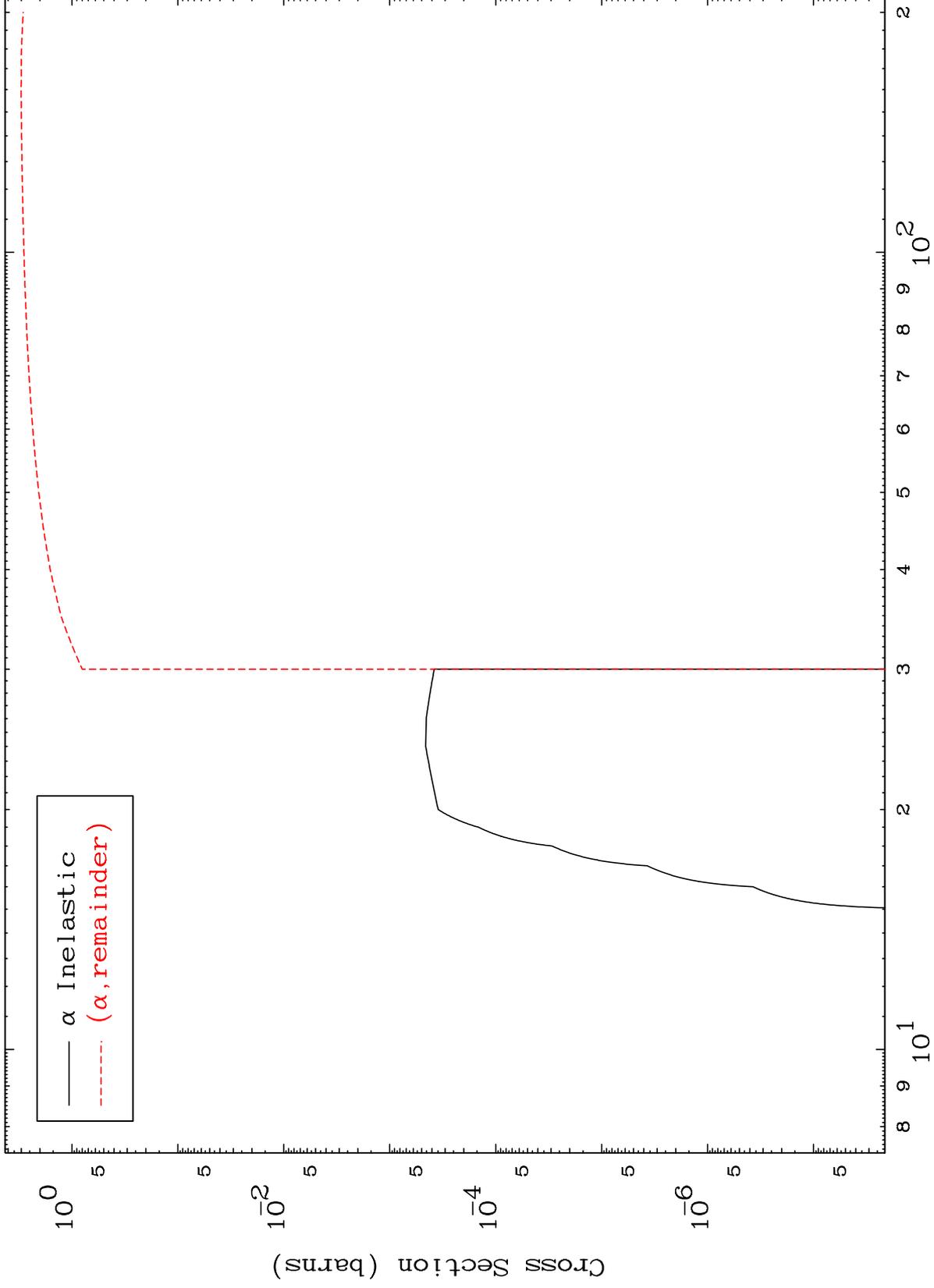
Press Mouse Button to Start

MAT 9731

0 Kelvin

$\alpha$  Major Cross Sections

97-Bk-242



1

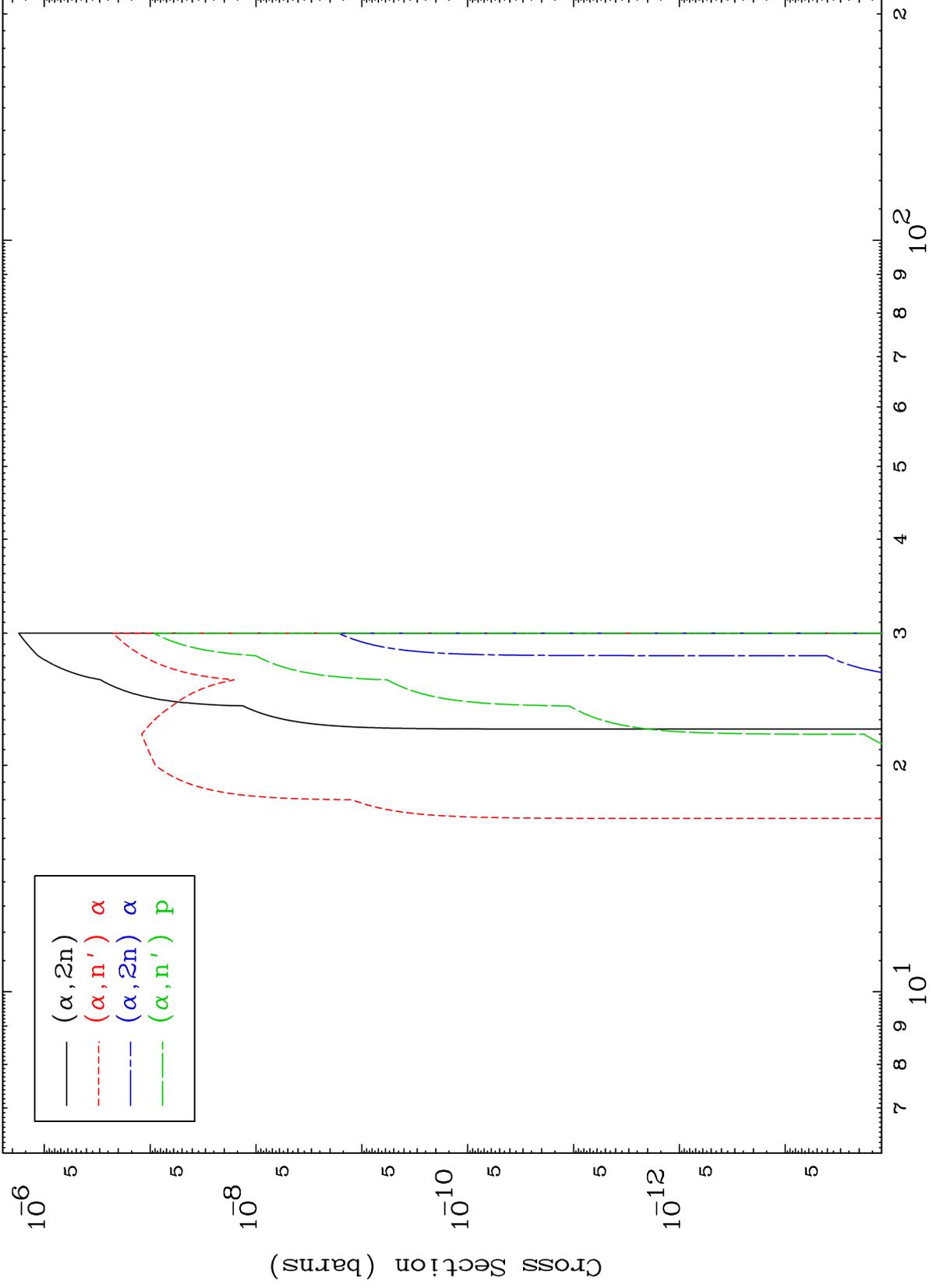
Incident Energy (MeV)

97-Bk-242

MAT 9731

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

97-Bk-242

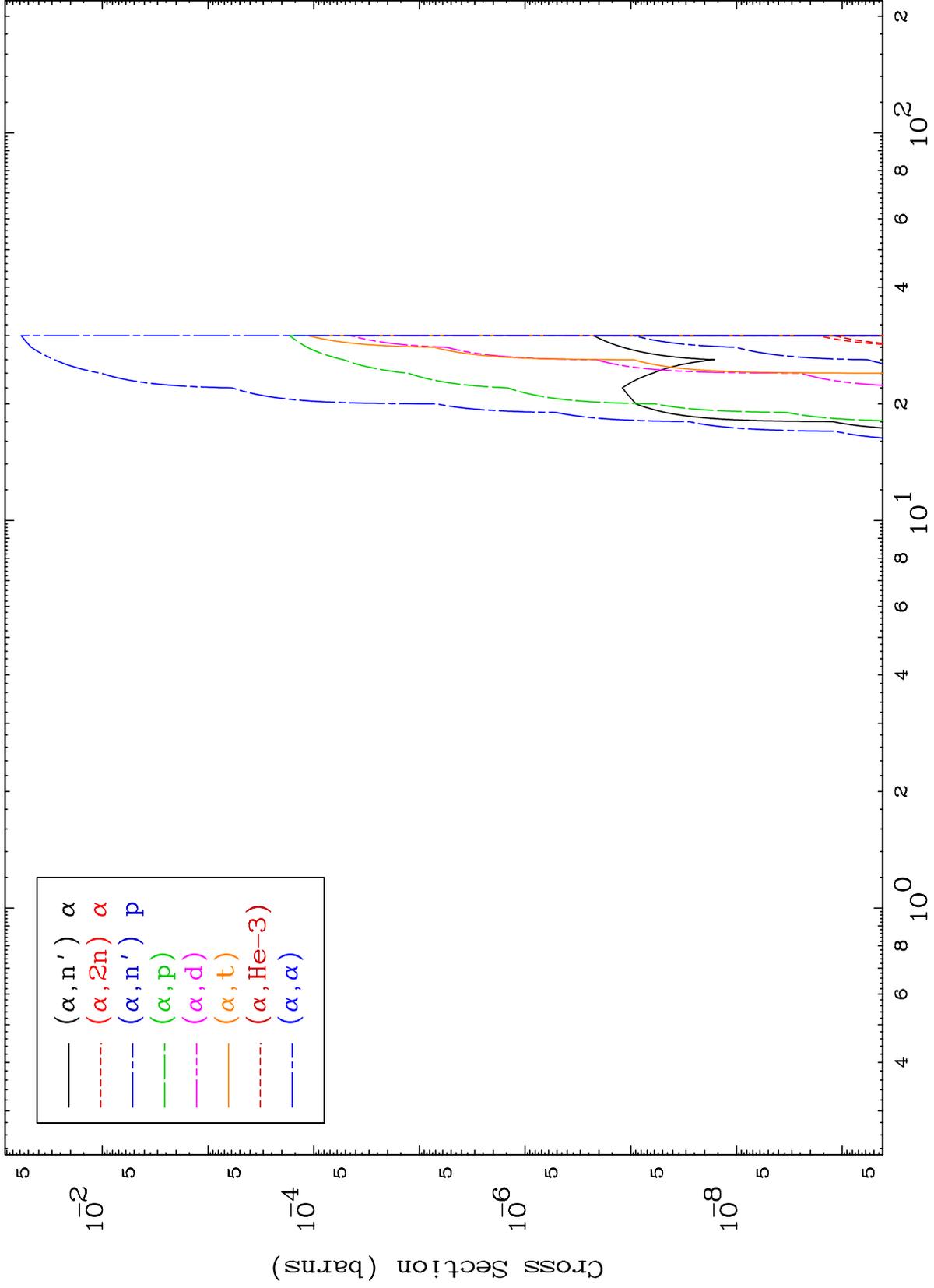


MAT 9731

$\alpha$  Charged Particle

97-Bk-242

0 Kelvin Cross Sections



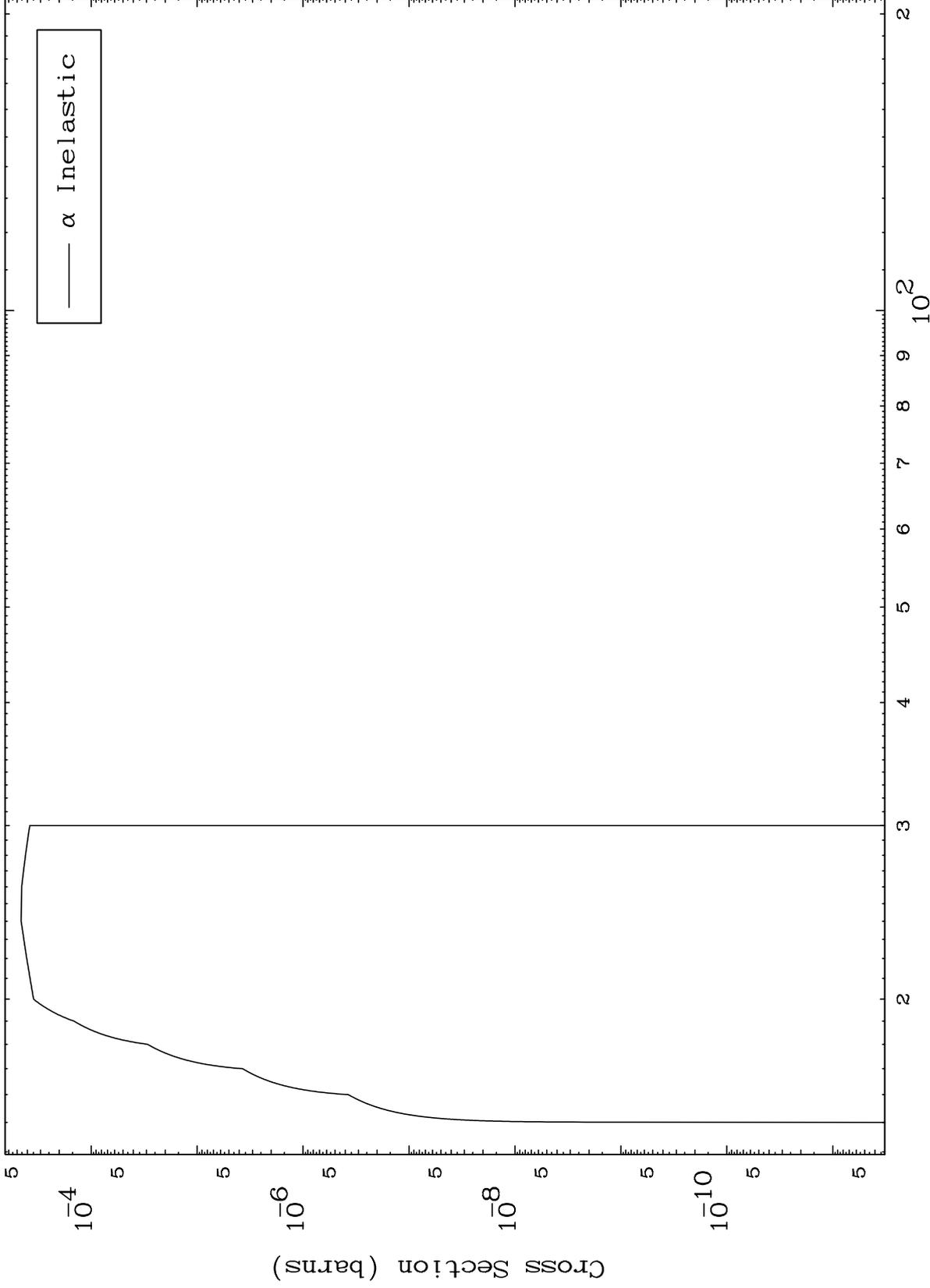
Incident Energy (MeV)

97-Bk-242

MAT 9731

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

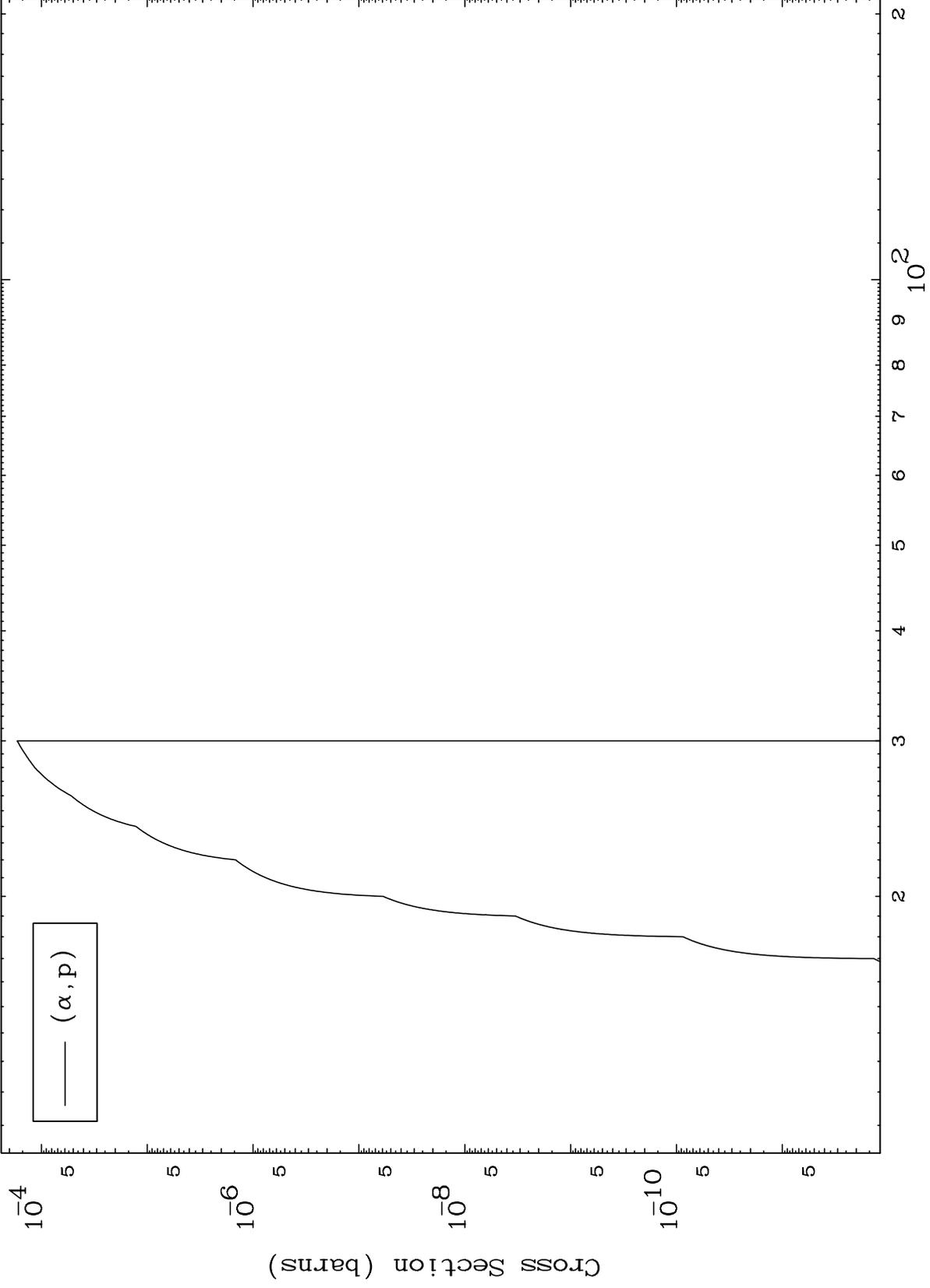
97-Bk-242



4

Incident Energy (MeV)

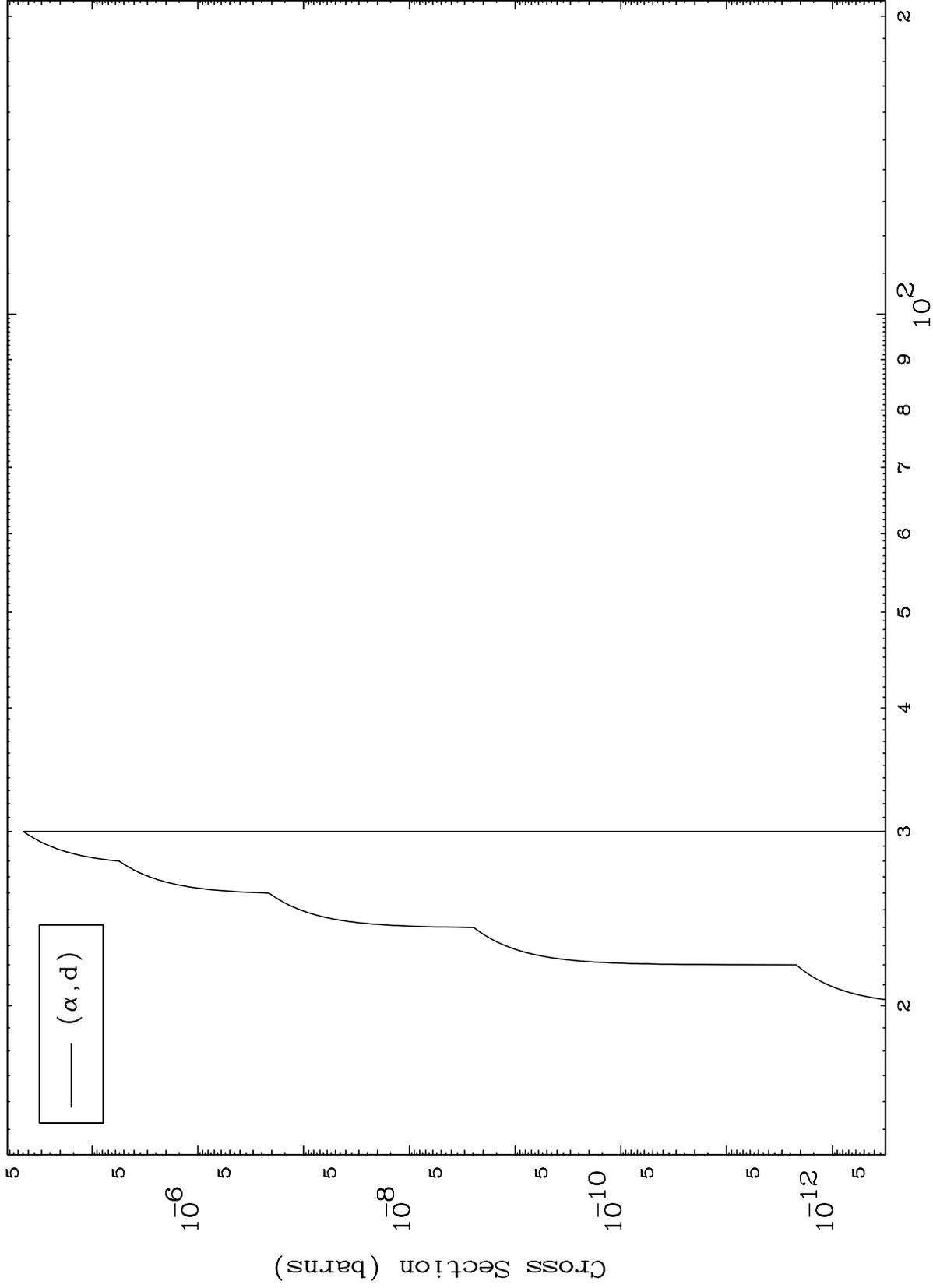
97-Bk-242



MAT 9731

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

97-Bk-242



6

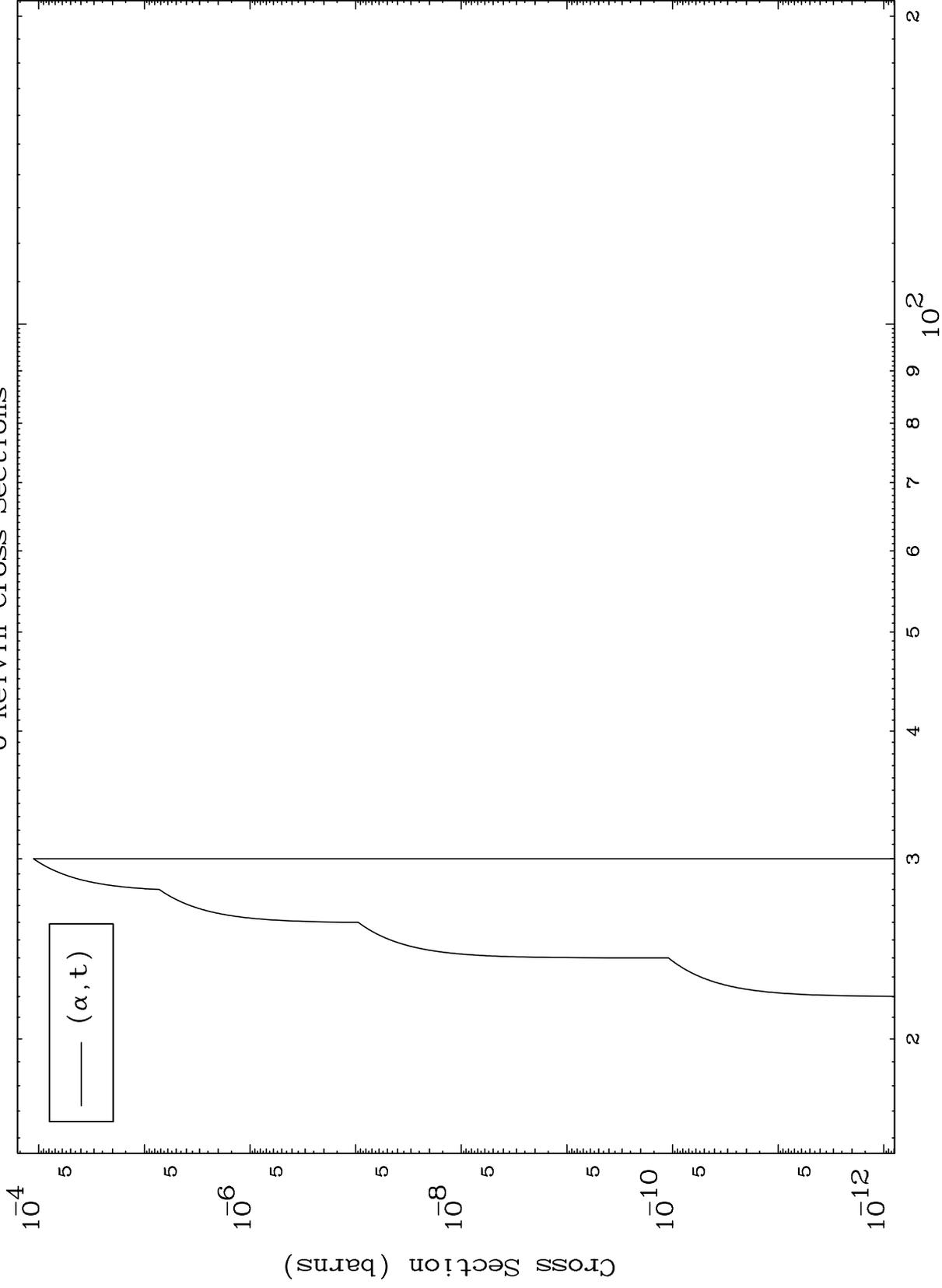
Incident Energy (MeV)

97-Bk-242

MAT 9731

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

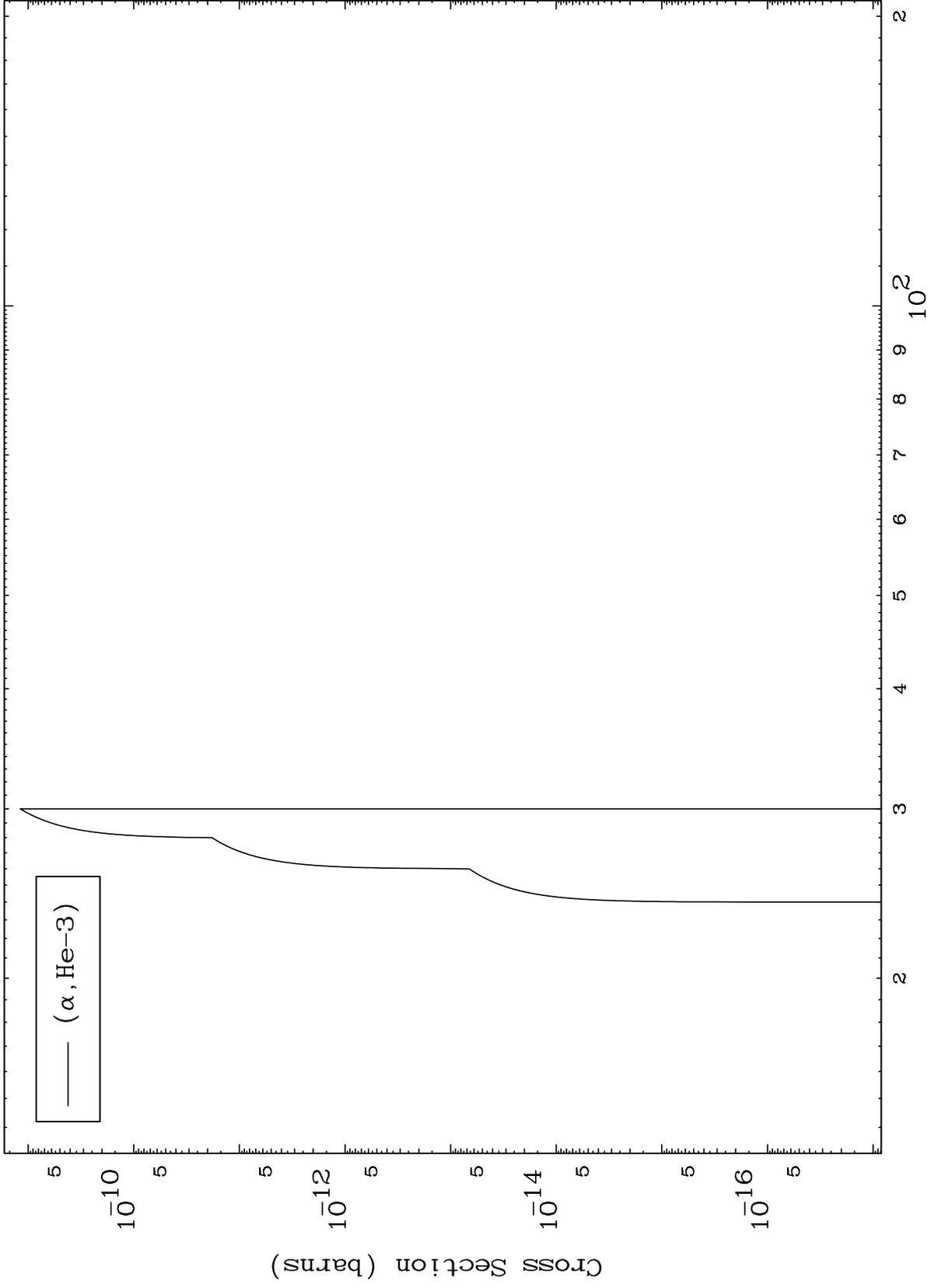
97-Bk-242



7

Incident Energy (MeV)

97-Bk-242

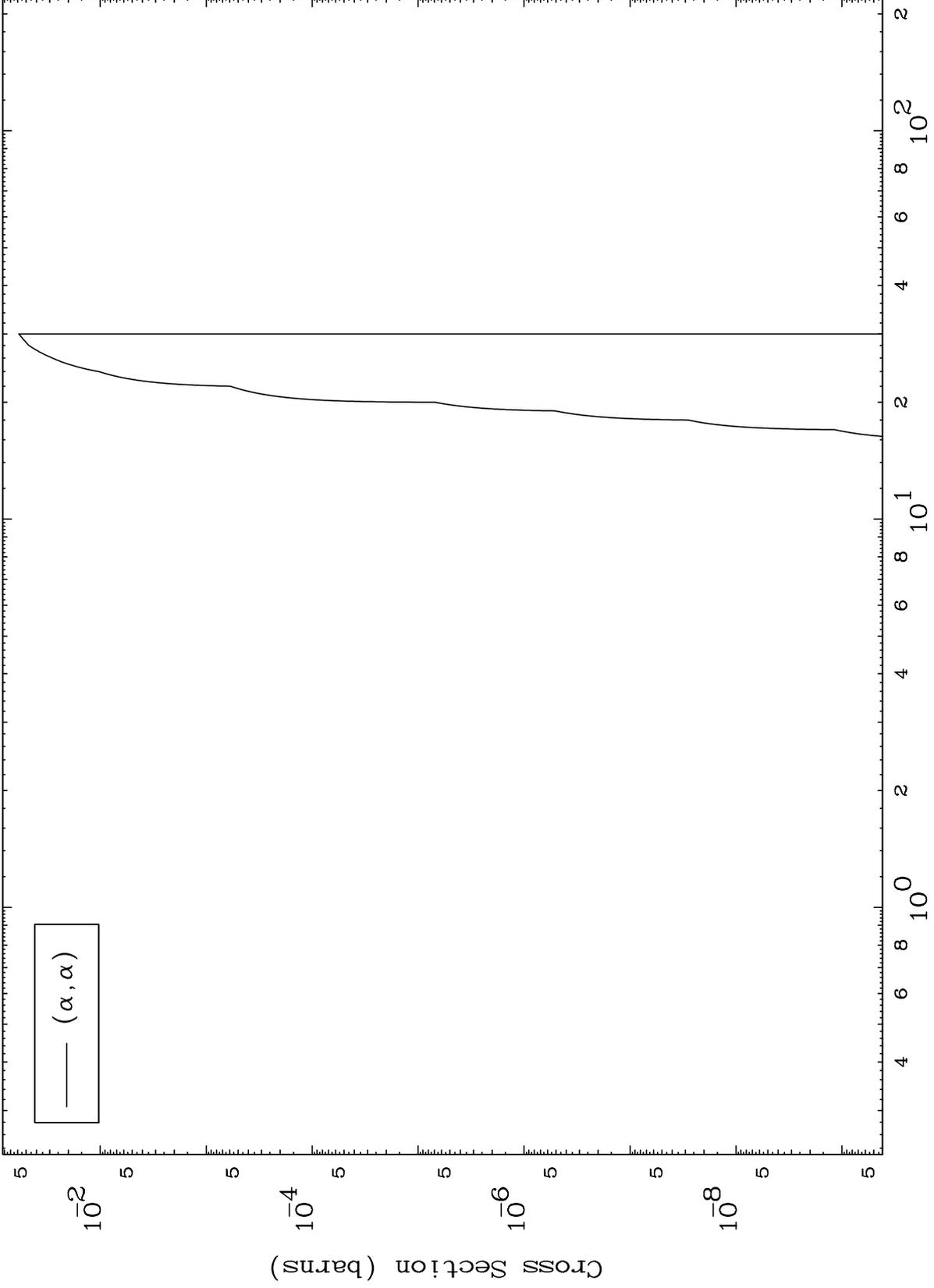


MAT 9731

( $\alpha, \alpha$ ) Levels

97-Bk-242

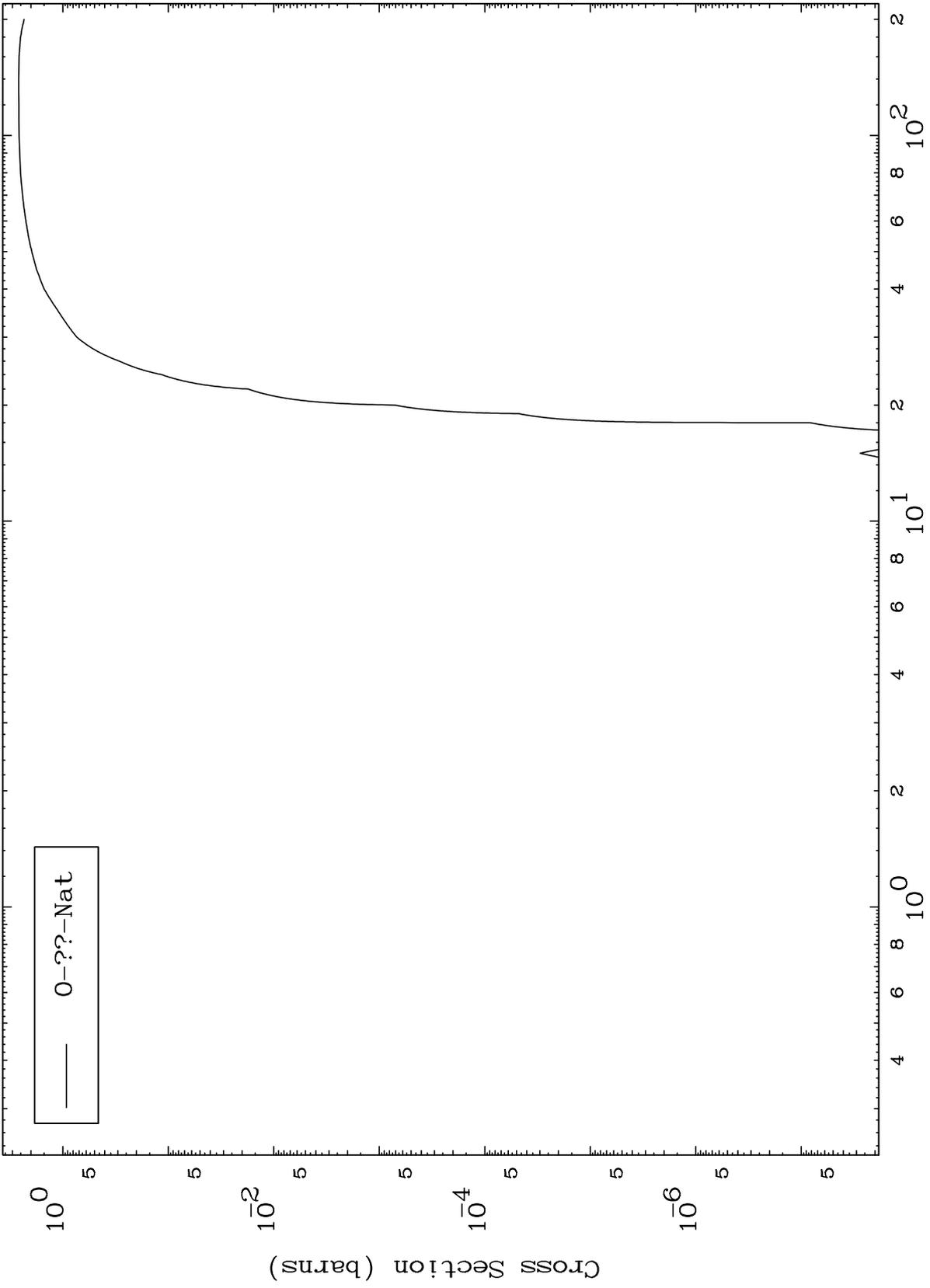
0 Kelvin Cross Sections



MAT 9731

97-Bk-242

$\alpha$  Fission  
Radionuclide Production Cross Section



— 0-??-Nat

10

Incident Energy (MeV)

97-Bk-242