

Program EVALPLOT  
(Version 2018-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail: [redcullen1@comcast.net](mailto:redcullen1@comcast.net)

Web: [redcullen1.net/HOMEPAGE.NEW](http://redcullen1.net/HOMEPAGE.NEW)

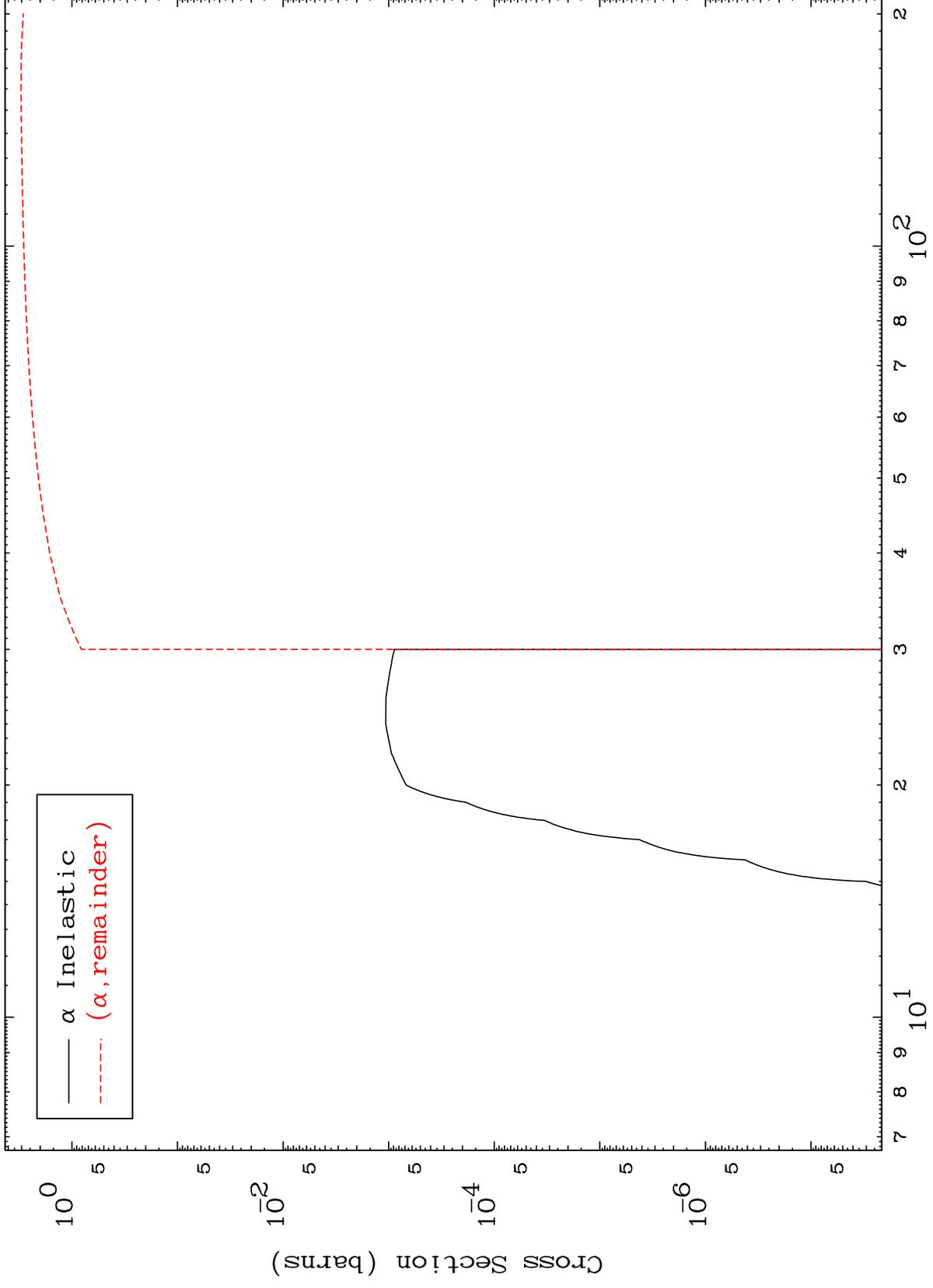
Press Mouse Button to Start

MAT 9740

$\alpha$  Major

97-Bk-245

0 Kelvin Cross Sections

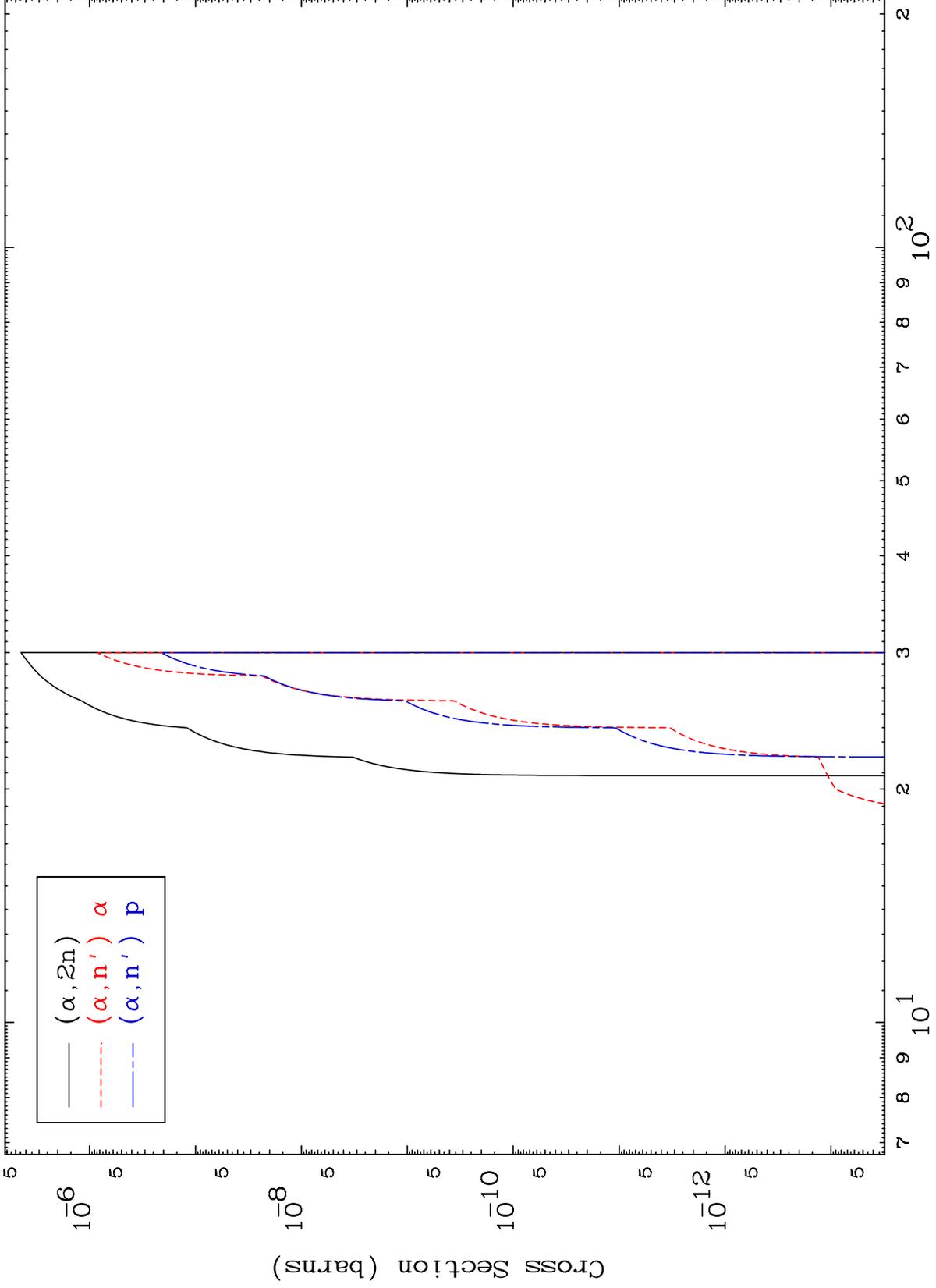


—  $\alpha$  Inelastic  
- - - ( $\alpha$ , remainder)

MAT 9740

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

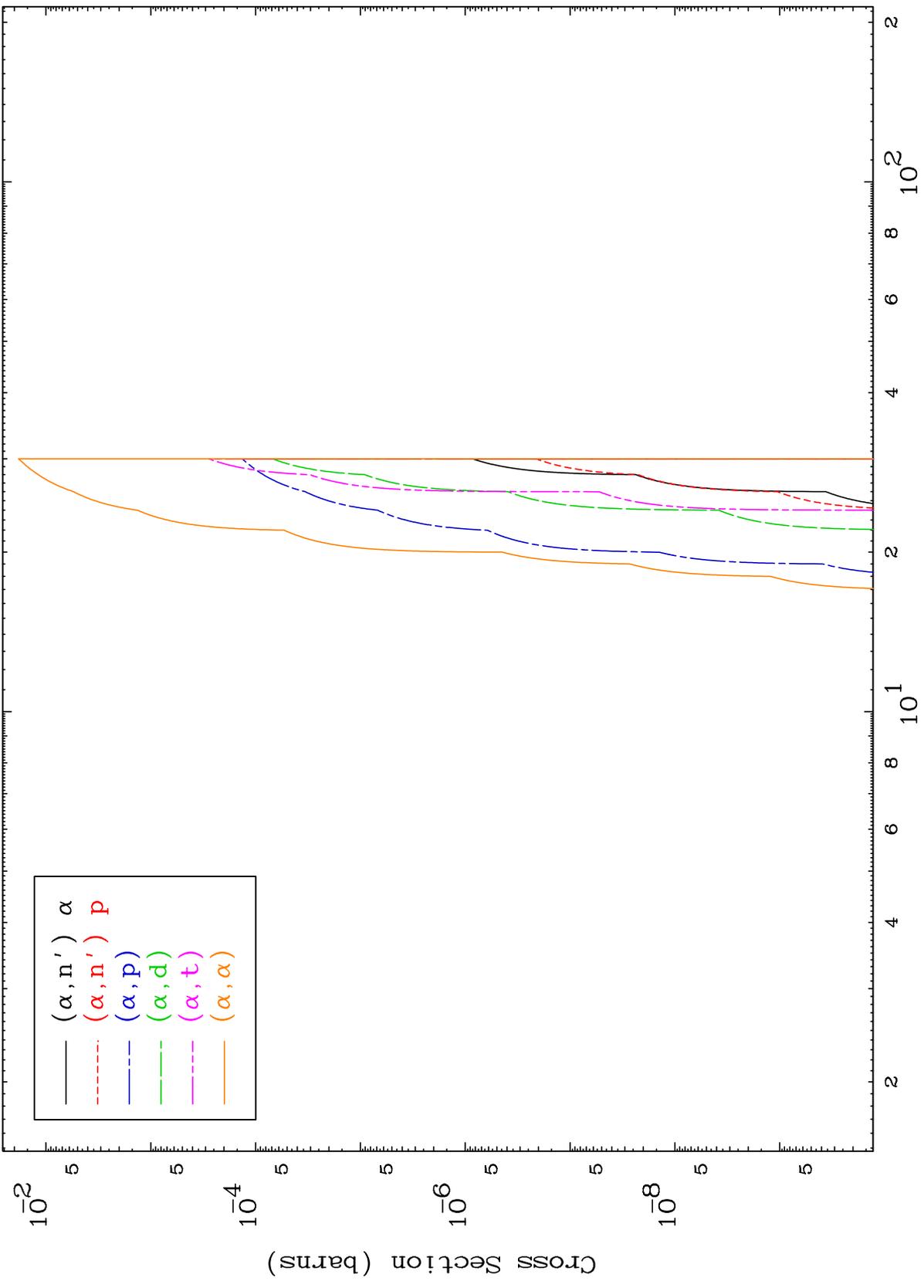
97-Bk-245



2

Incident Energy (MeV)

97-Bk-245

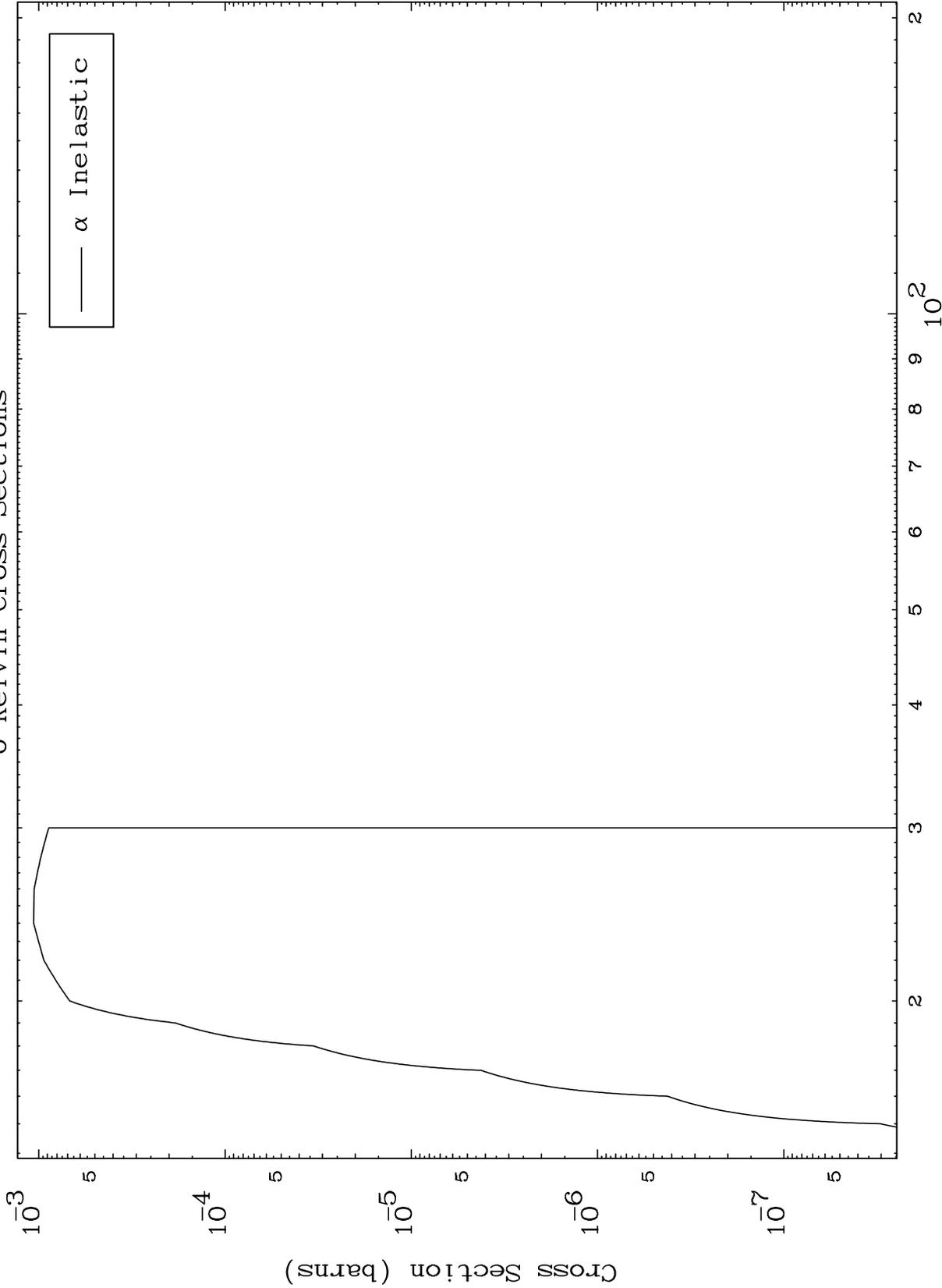


MAT 9740

( $\alpha, n'$ ) Level

97-Bk-245

0 Kelvin Cross Sections



$\alpha$  Inelastic

Incident Energy (MeV)

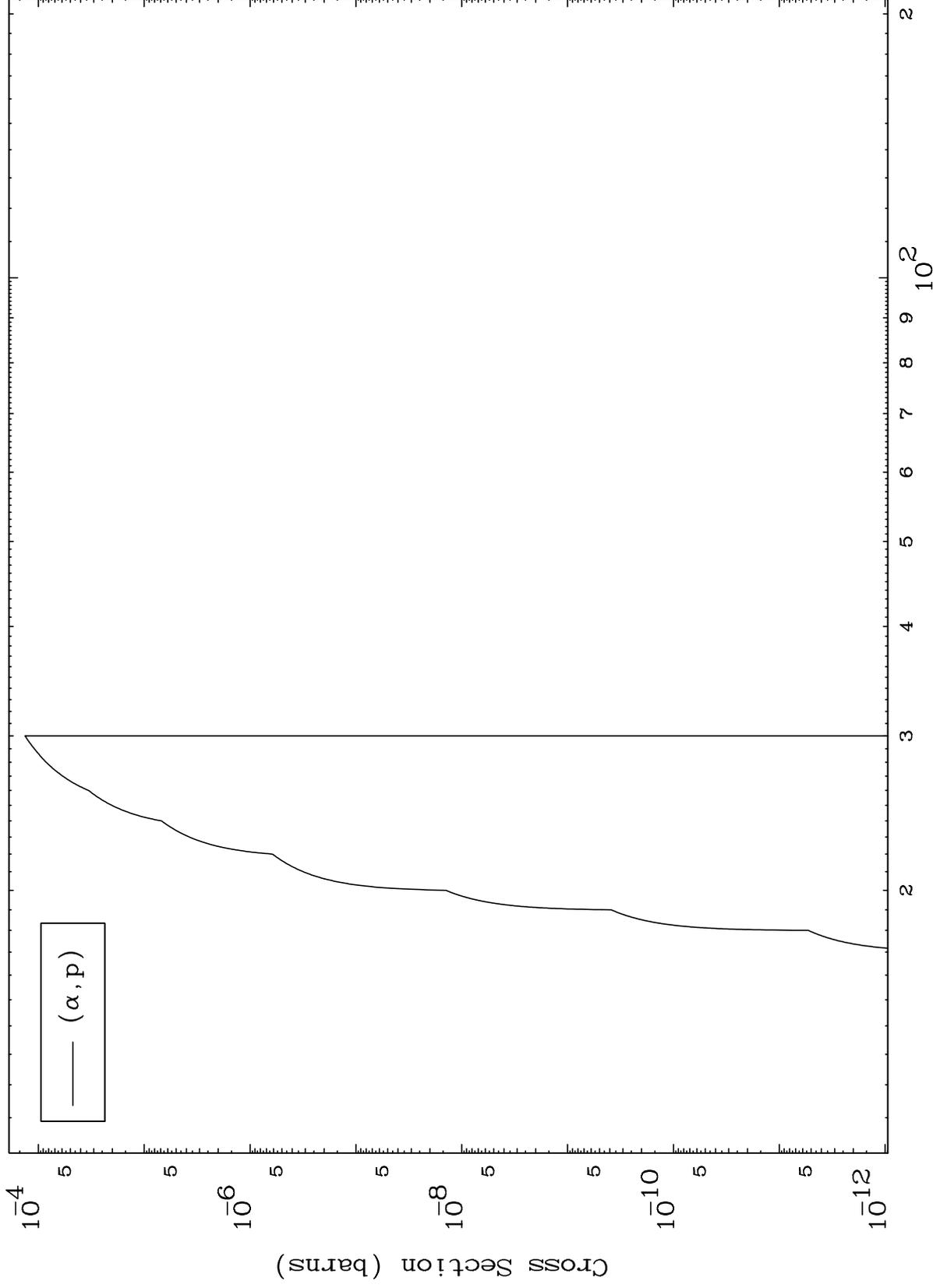
97-Bk-245

4

MAT 9740

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

97-Bk-245



5

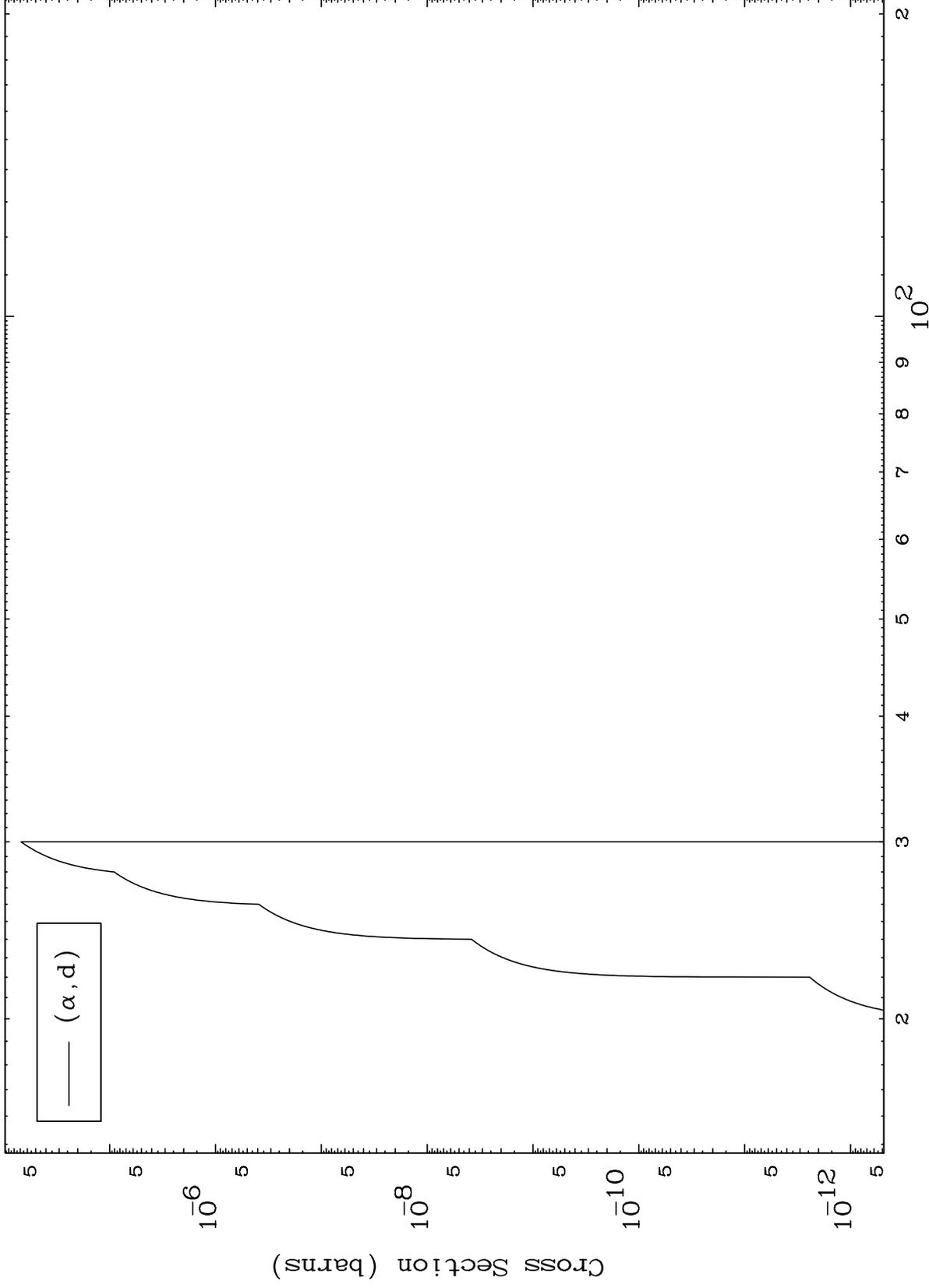
Incident Energy (MeV)

97-Bk-245

MAT 9740

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

97-Bk-245



6

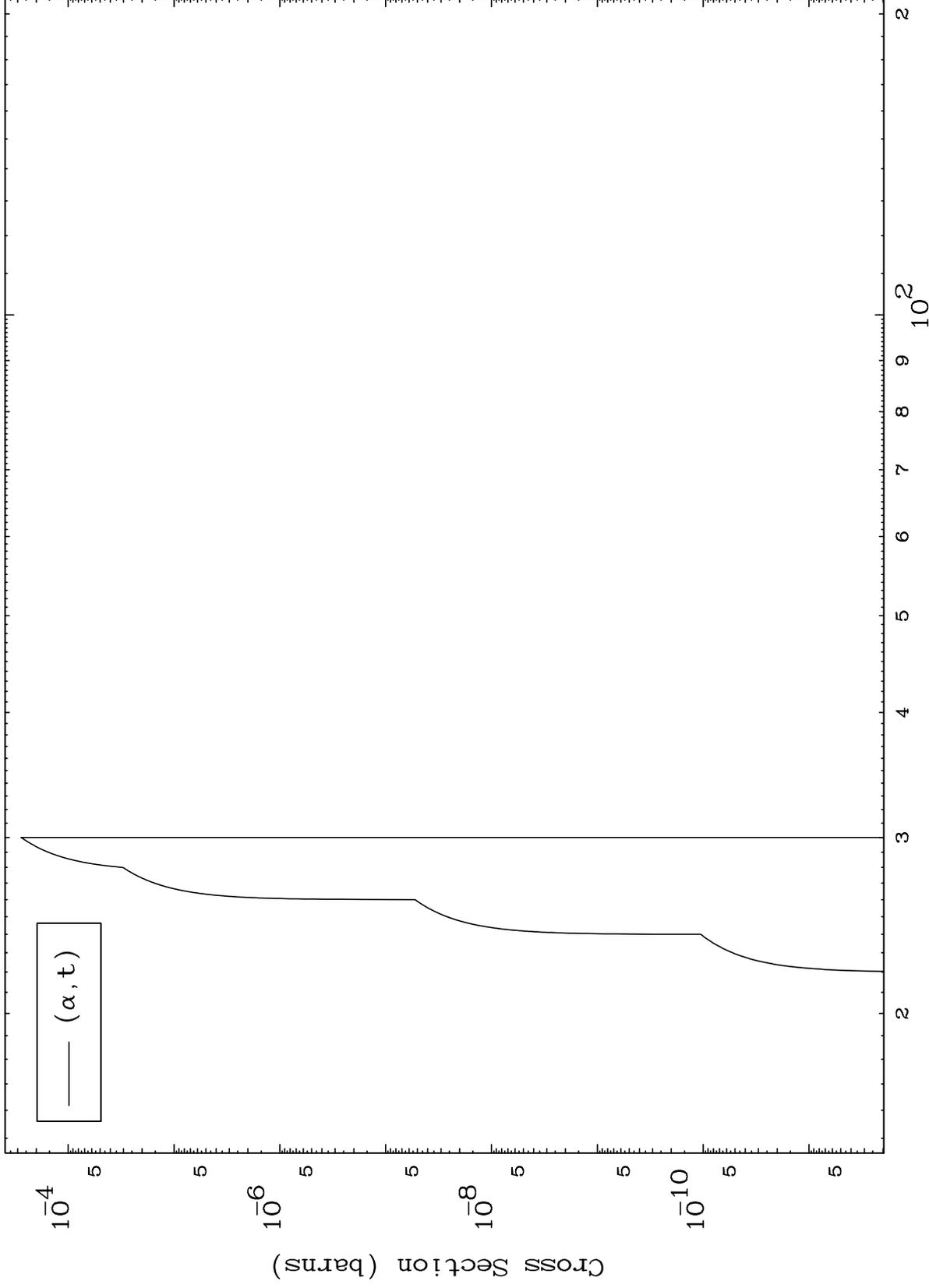
Incident Energy (MeV)

97-Bk-245

MAT 9740

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

97-Bk-245



7

Incident Energy (MeV)

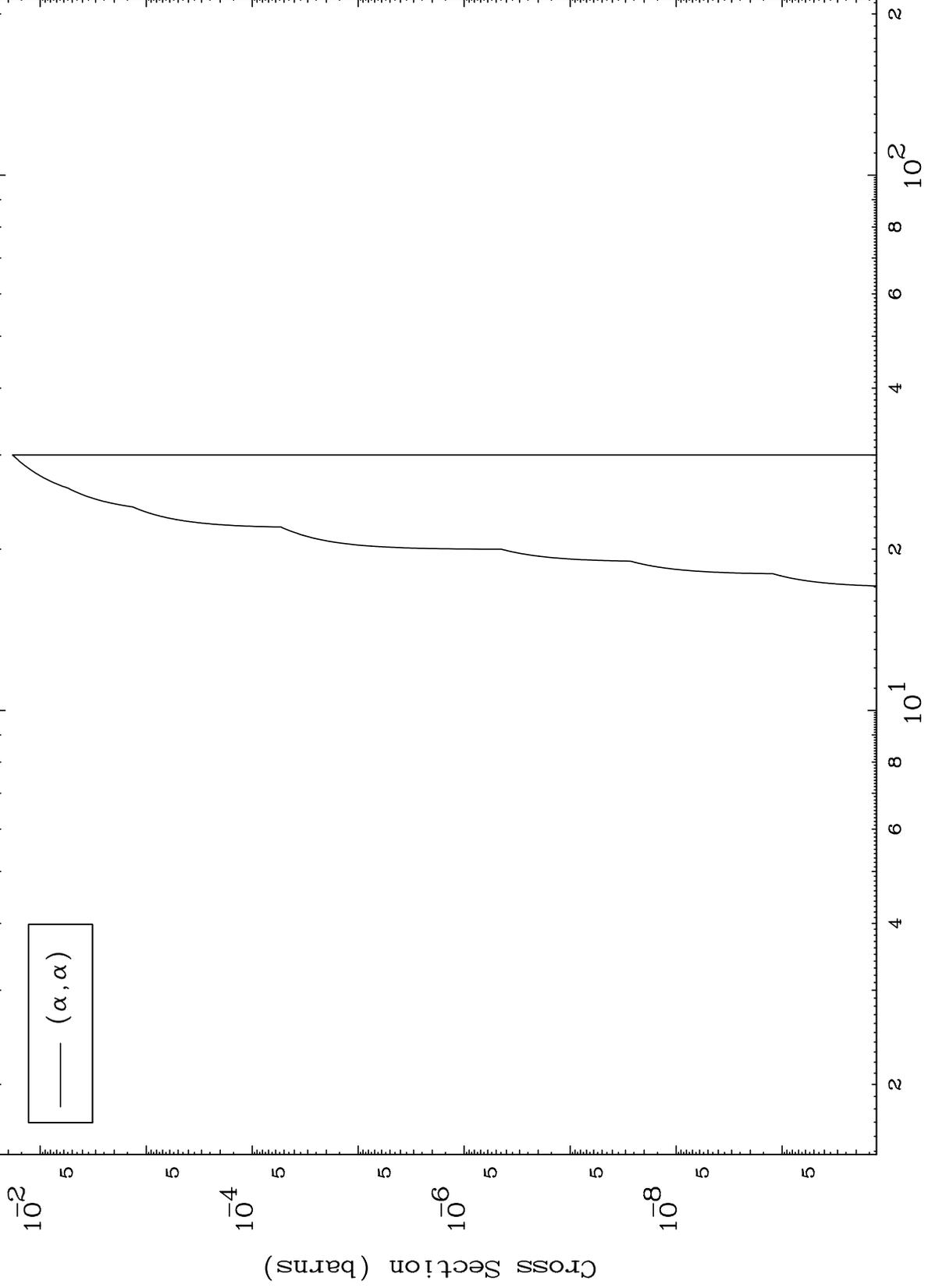
97-Bk-245

MAT 9740

( $\alpha, \alpha$ ) Levels

97-Bk-245

0 Kelvin Cross Sections



8

Incident Energy (MeV)

97-Bk-245

MAT 9740

97-Bk-245

$\alpha$  Fission  
Radionuclide Production Cross Section

