

Program EVALPLOT
(Version 2018-1)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail: redcullen1@comcast.net
Web: redcullen1.net/HOMEPAGE.NEW

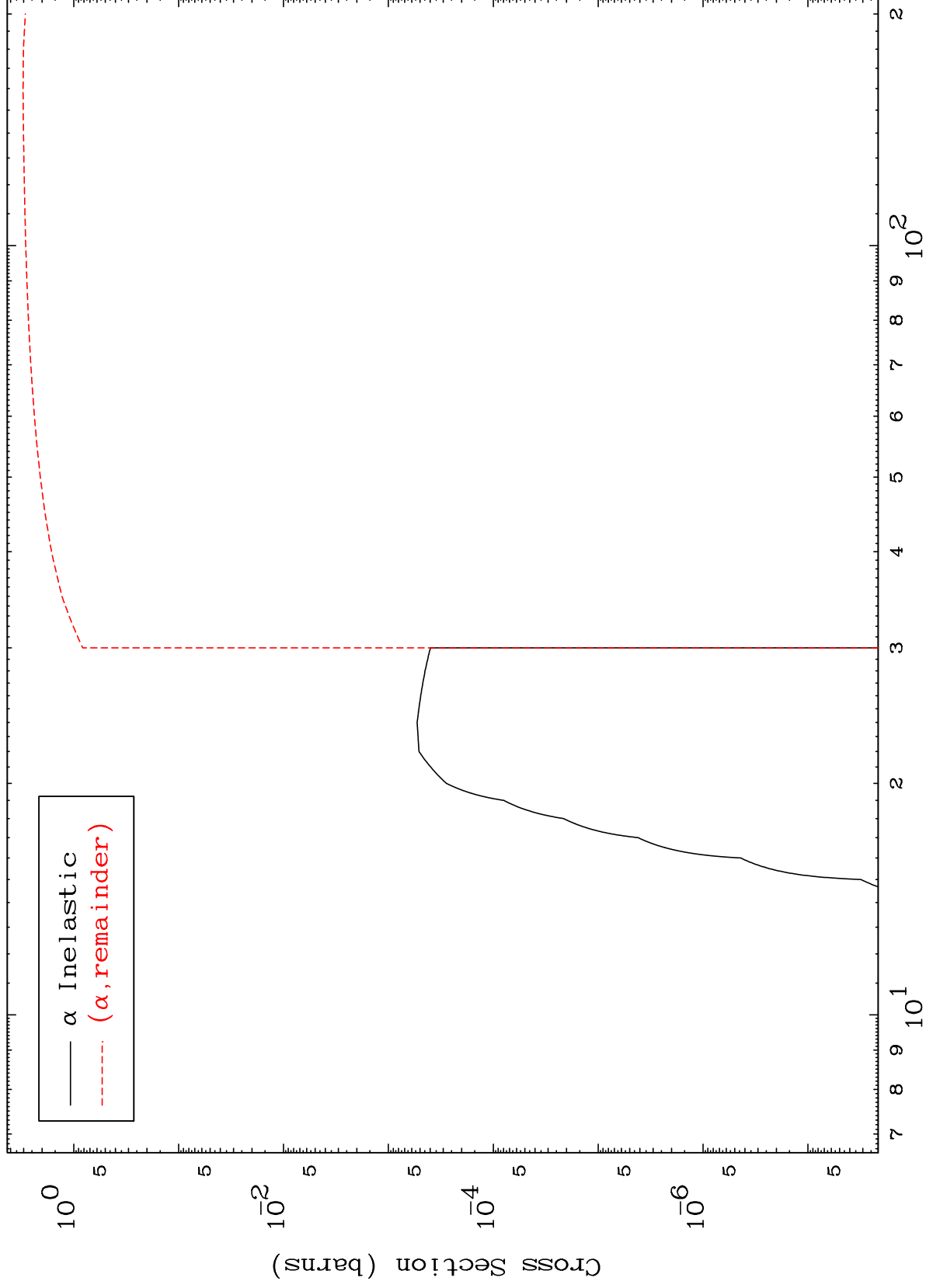
Press Mouse Button to Start

MAT 9743

α Major

97-Bk-246

0 Kelvin Cross Sections



1

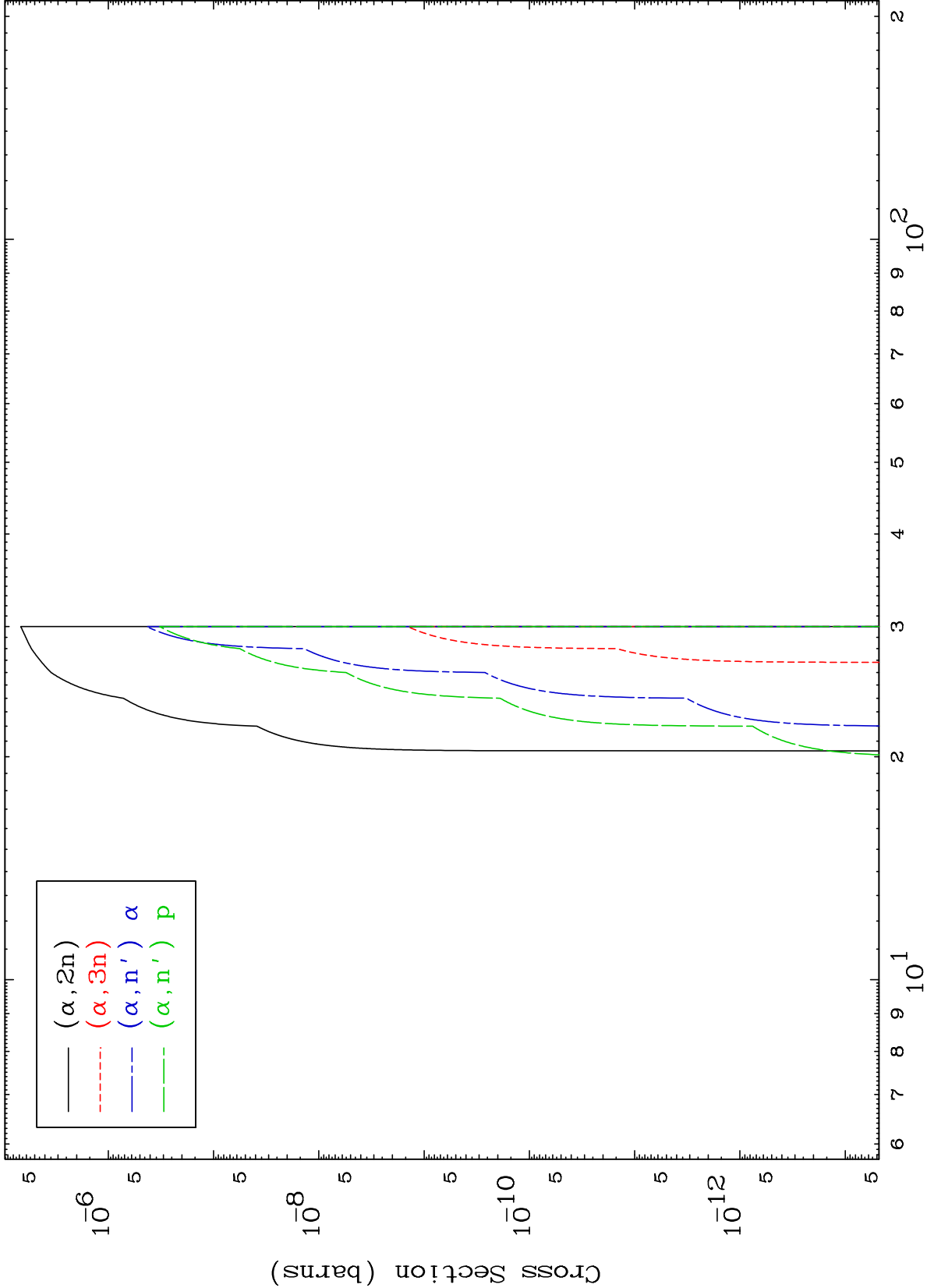
Incident Energy (MeV)

97-Bk-246

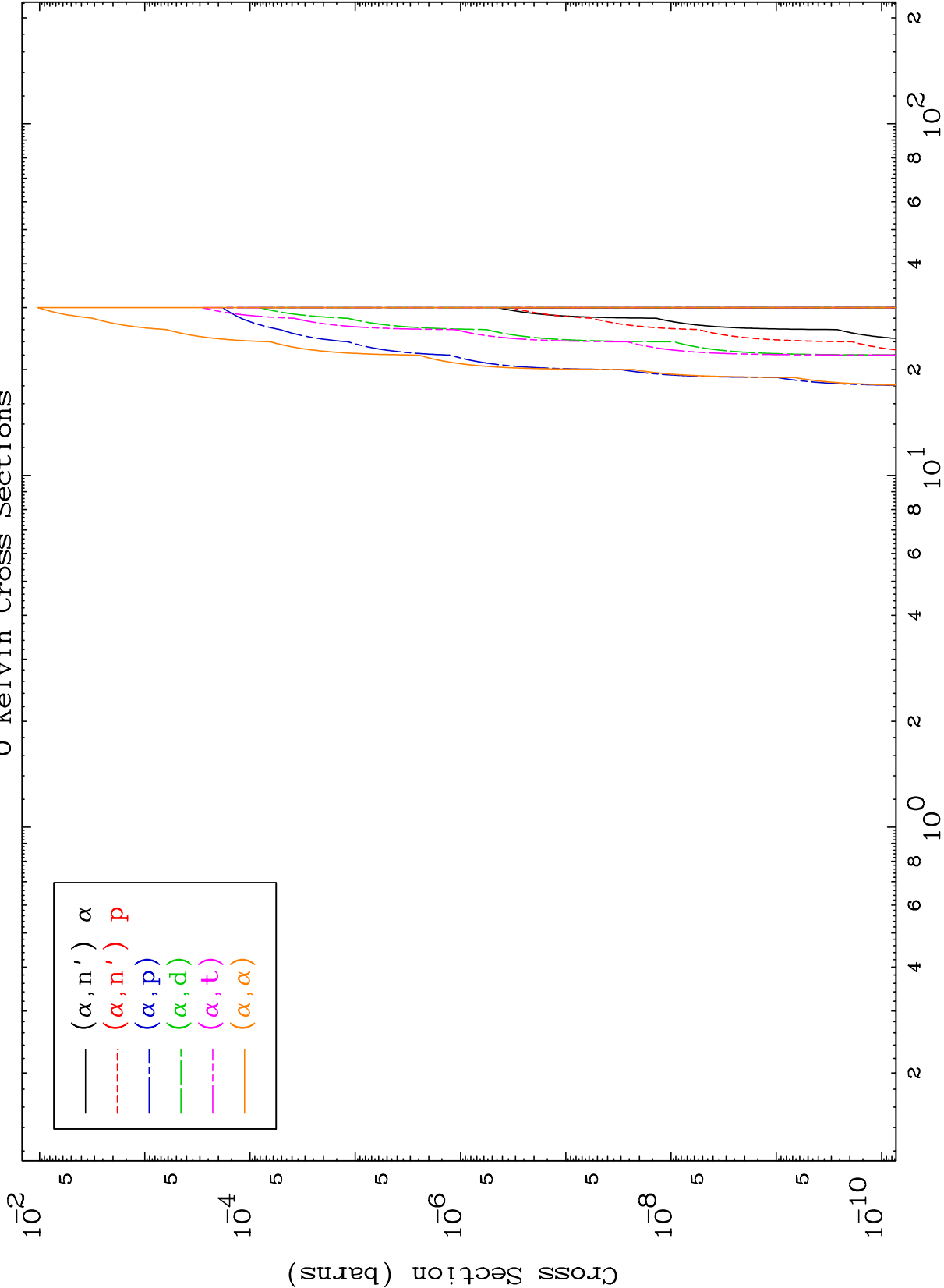
MAT 9743

α Neutron Production
0 Kelvin Cross Sections

97-Bk-246



0 Kelvin Cross Sections

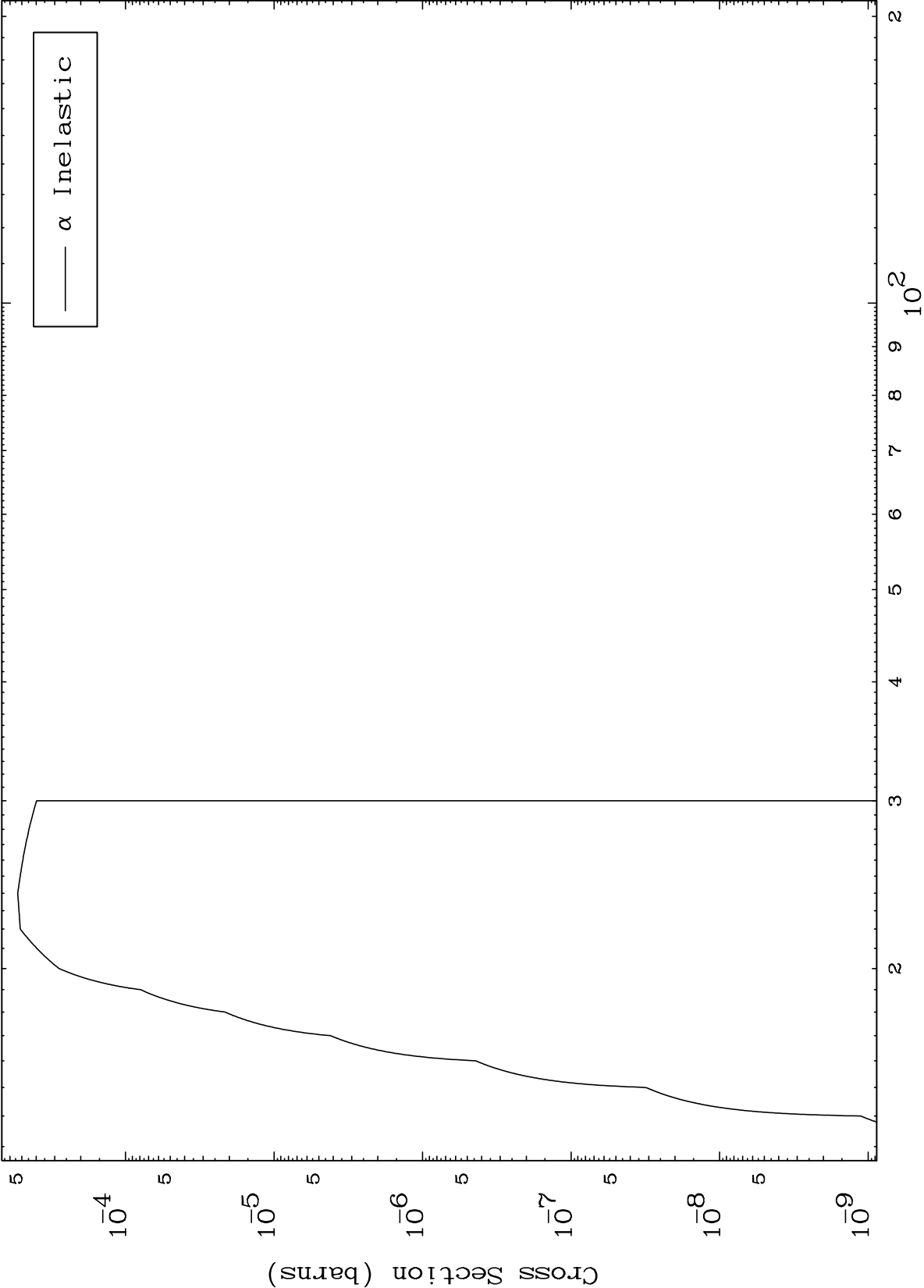


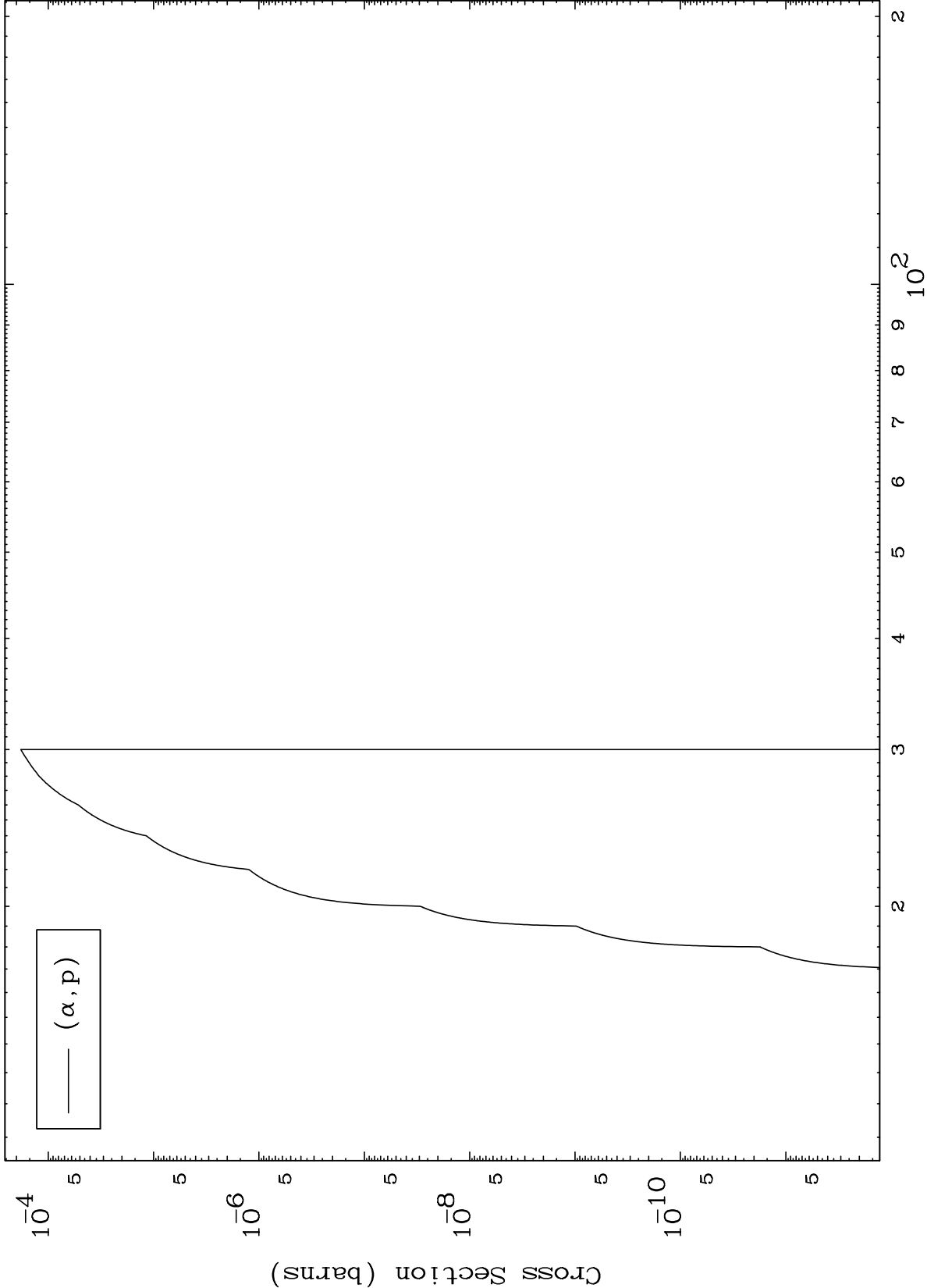
MAT 9743

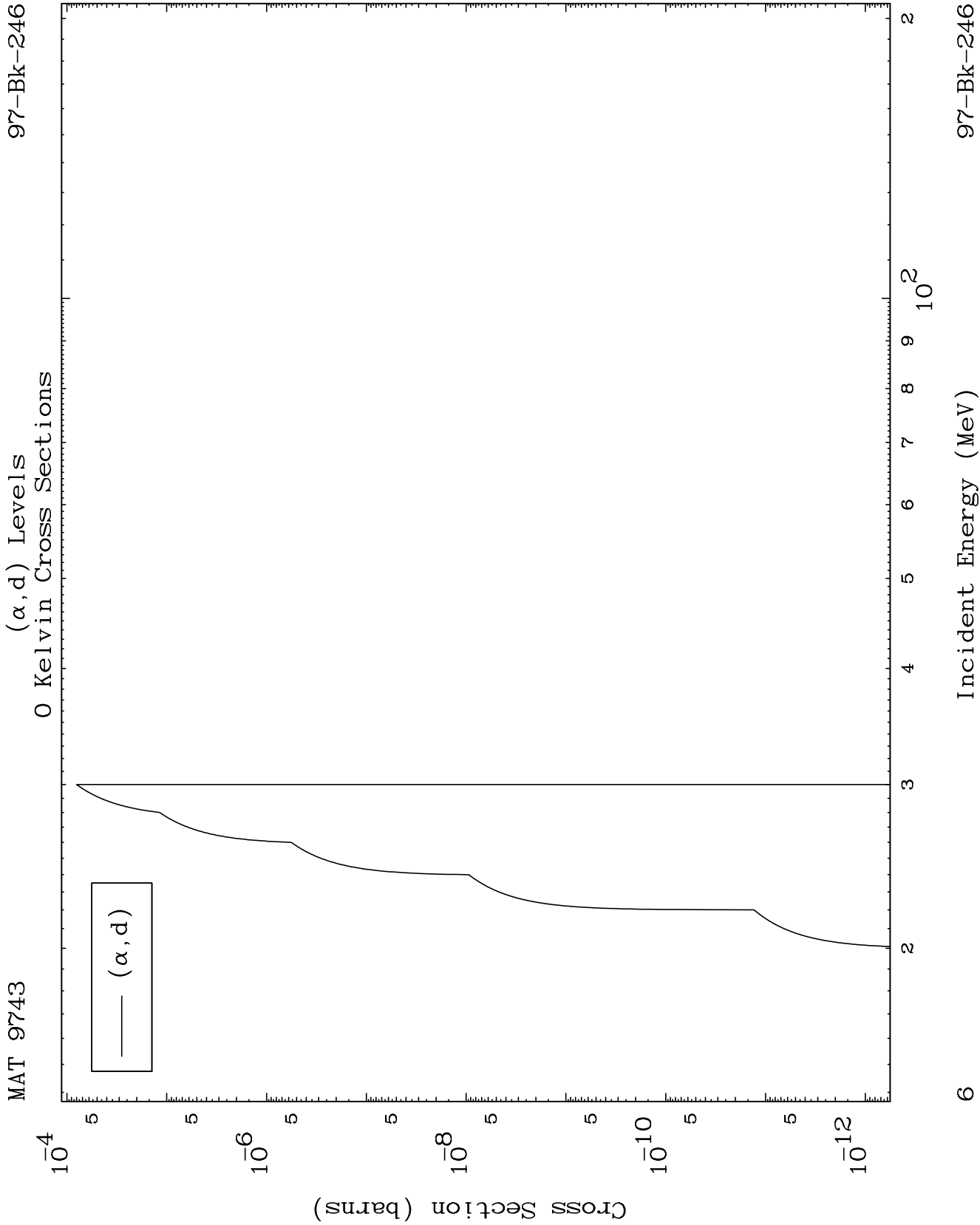
(α, n') Level

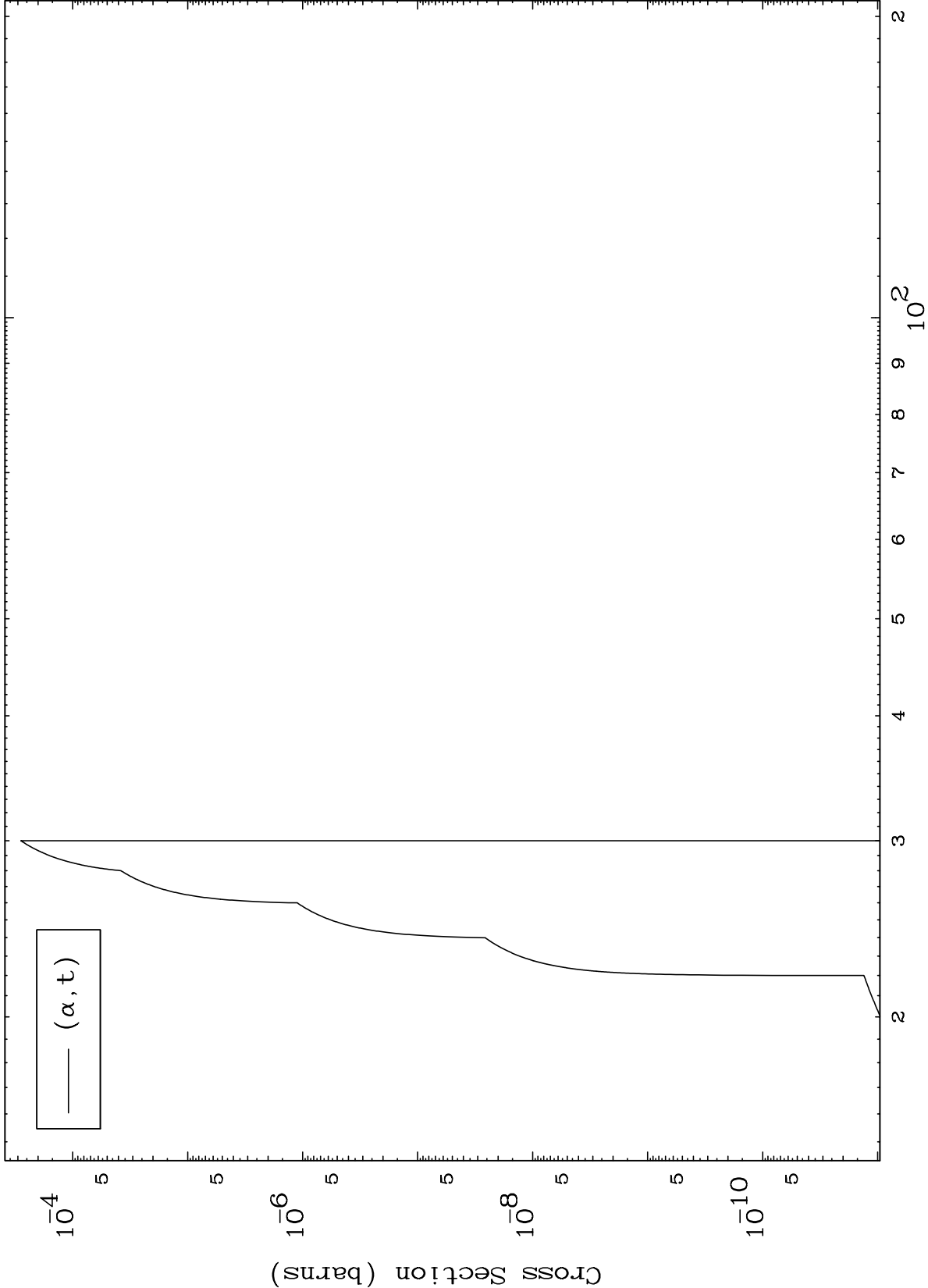
97-Bk-246

0 Kelvin Cross Sections







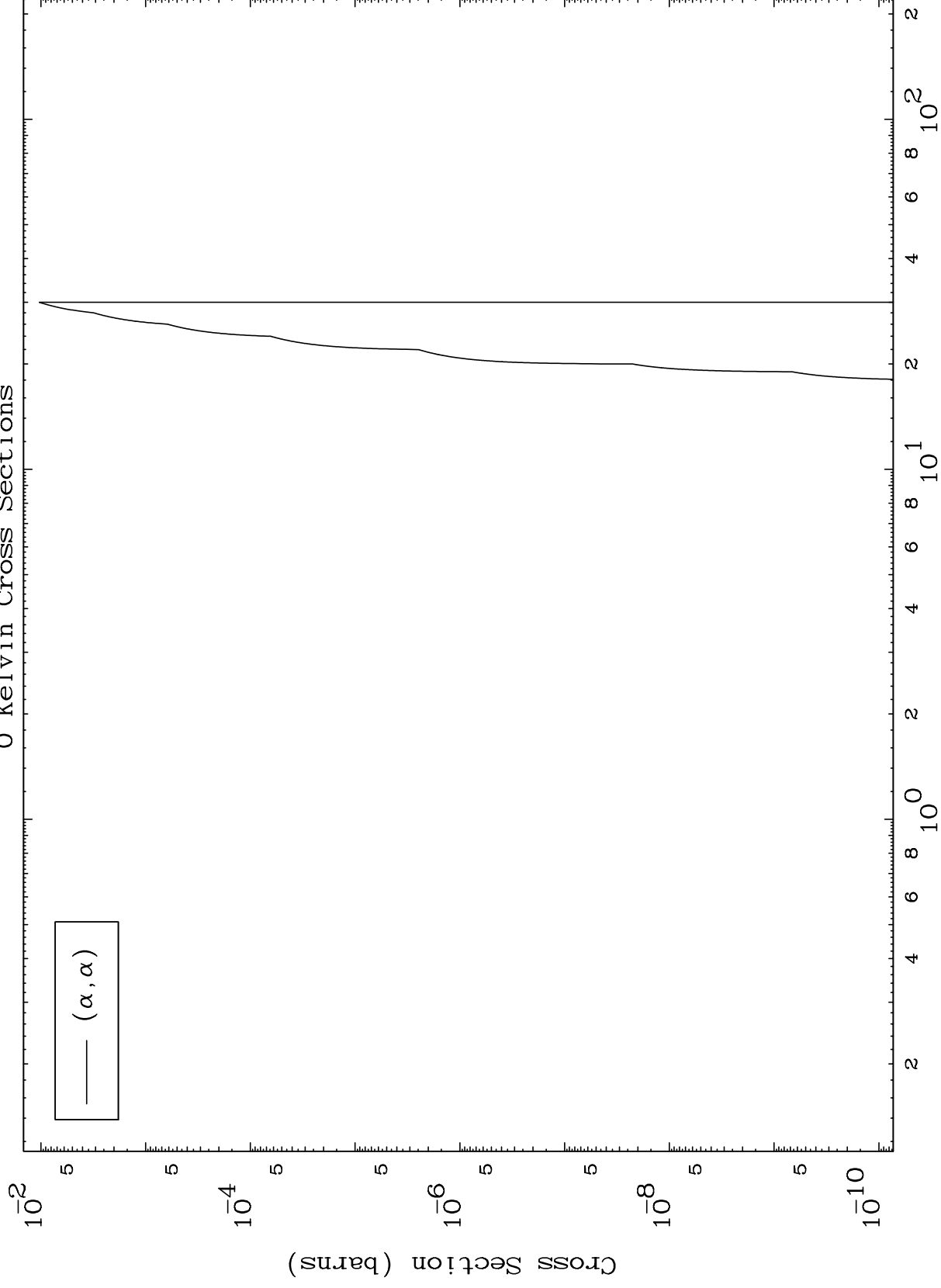


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(α, α) Levels

97-Bk-246

0 Kelvin Cross Sections



8

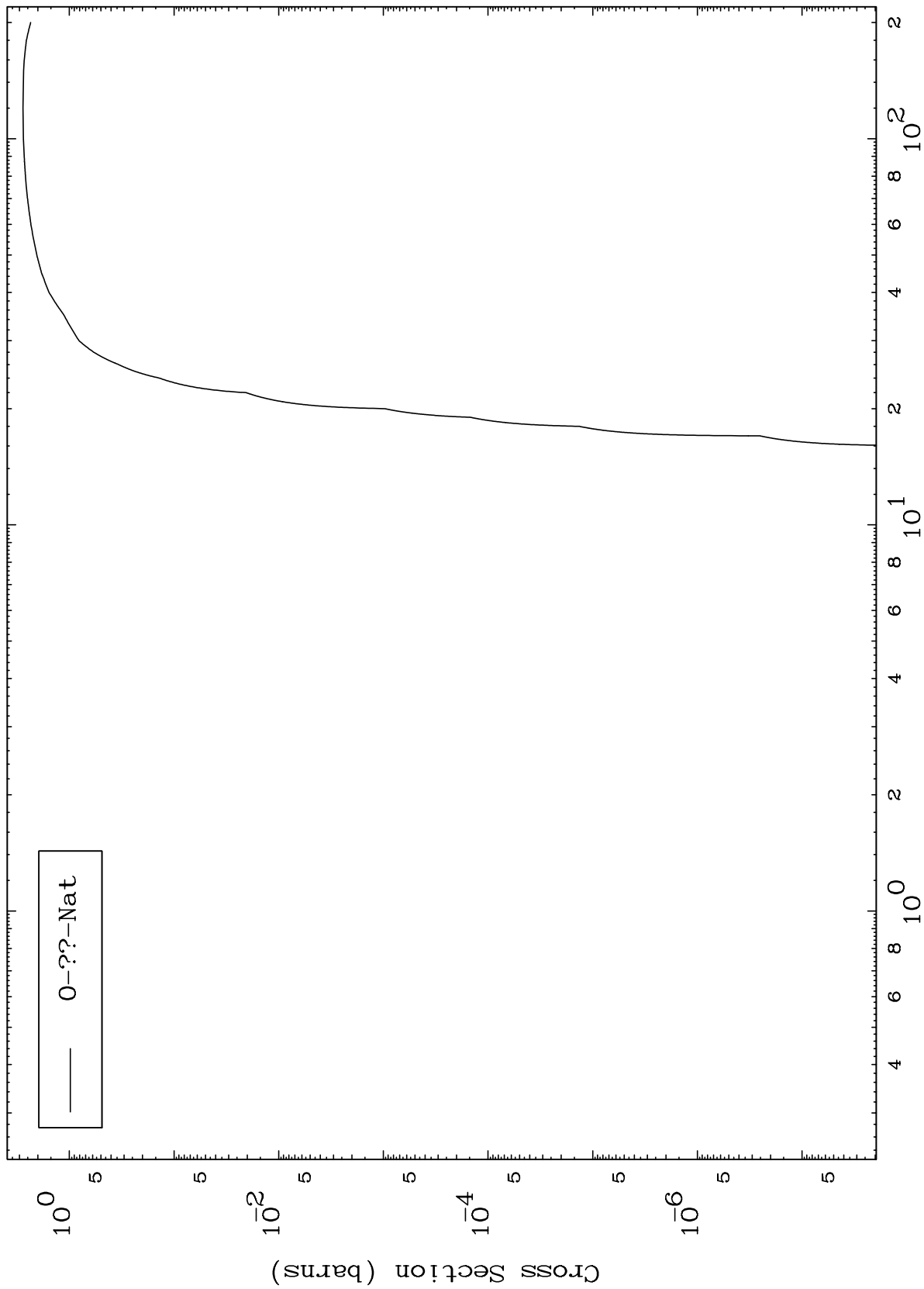
Incident Energy (MeV)

97-Bk-246

MAT 9743

97-Bk-246

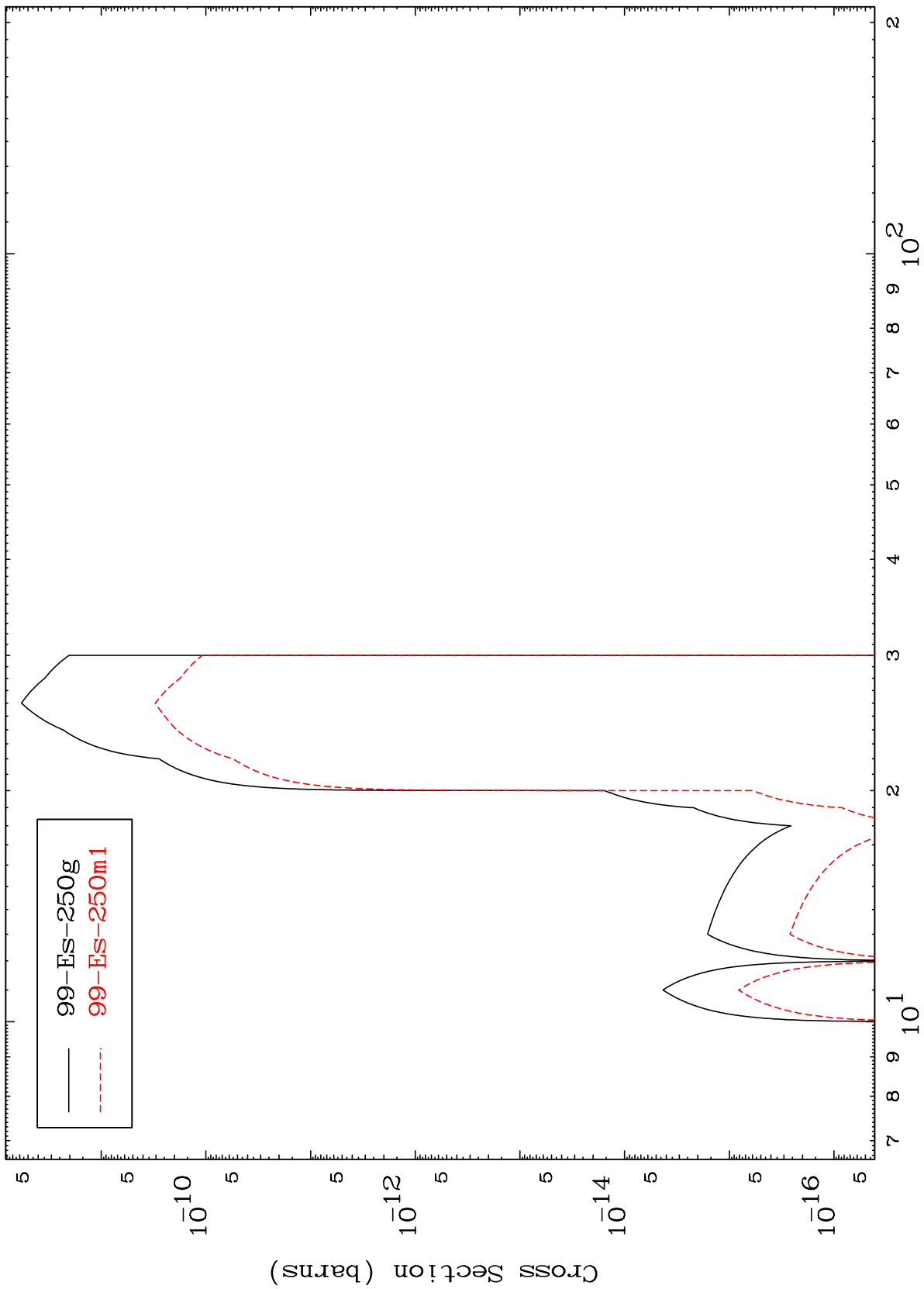
α Fission
Radionuclide Production Cross Section



MAT 9743

97-Bk-246

Radionuclide Production Cross Section
(α, γ)



97-Bk-246

Incident Energy (MeV)

10