

Program EVALPLOT
(Version 2018-1)

by

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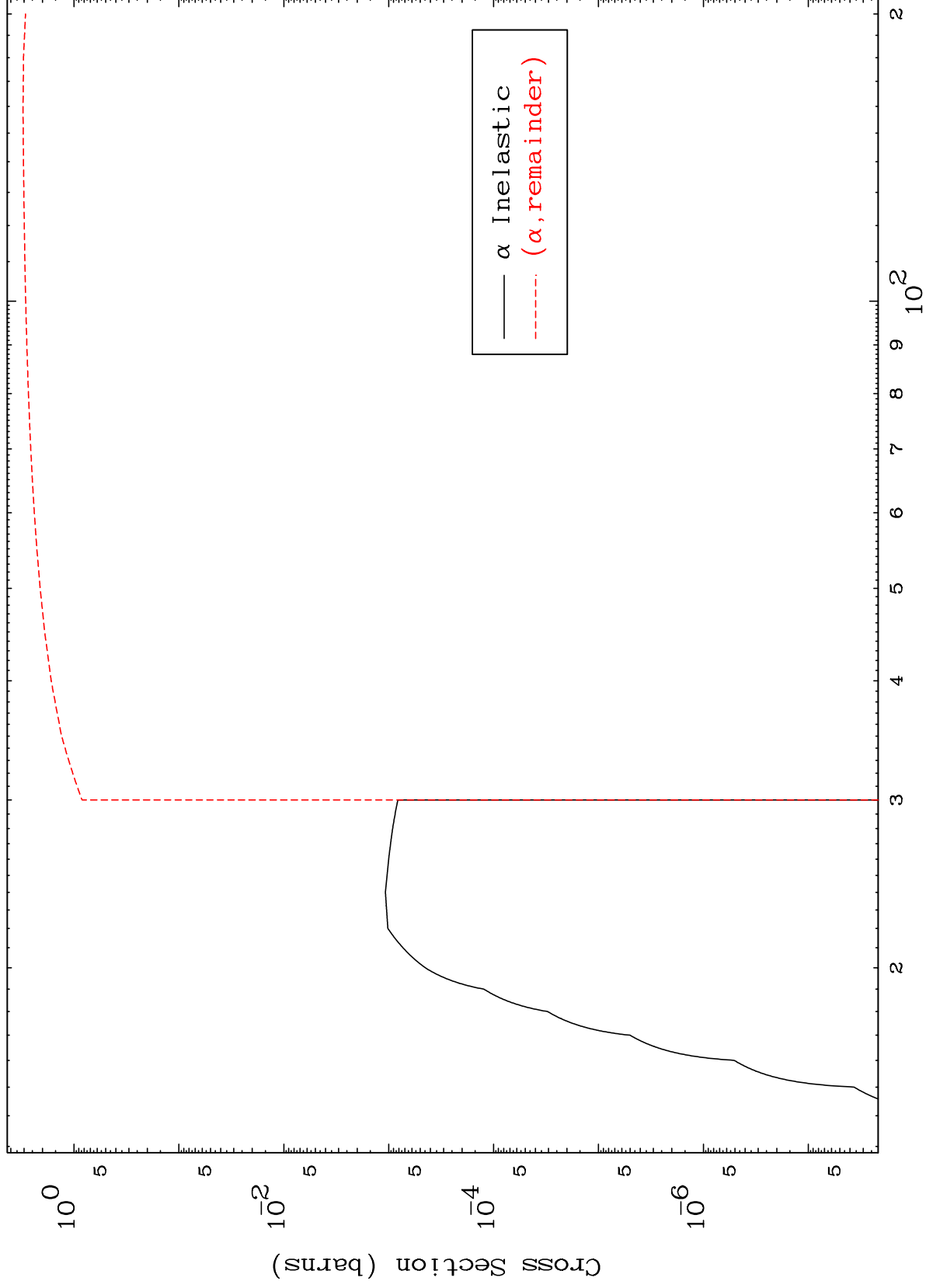
Press Mouse Button to Start

MAT 9752

α Major

97-Bk-249

0 Kelvin Cross Sections



1

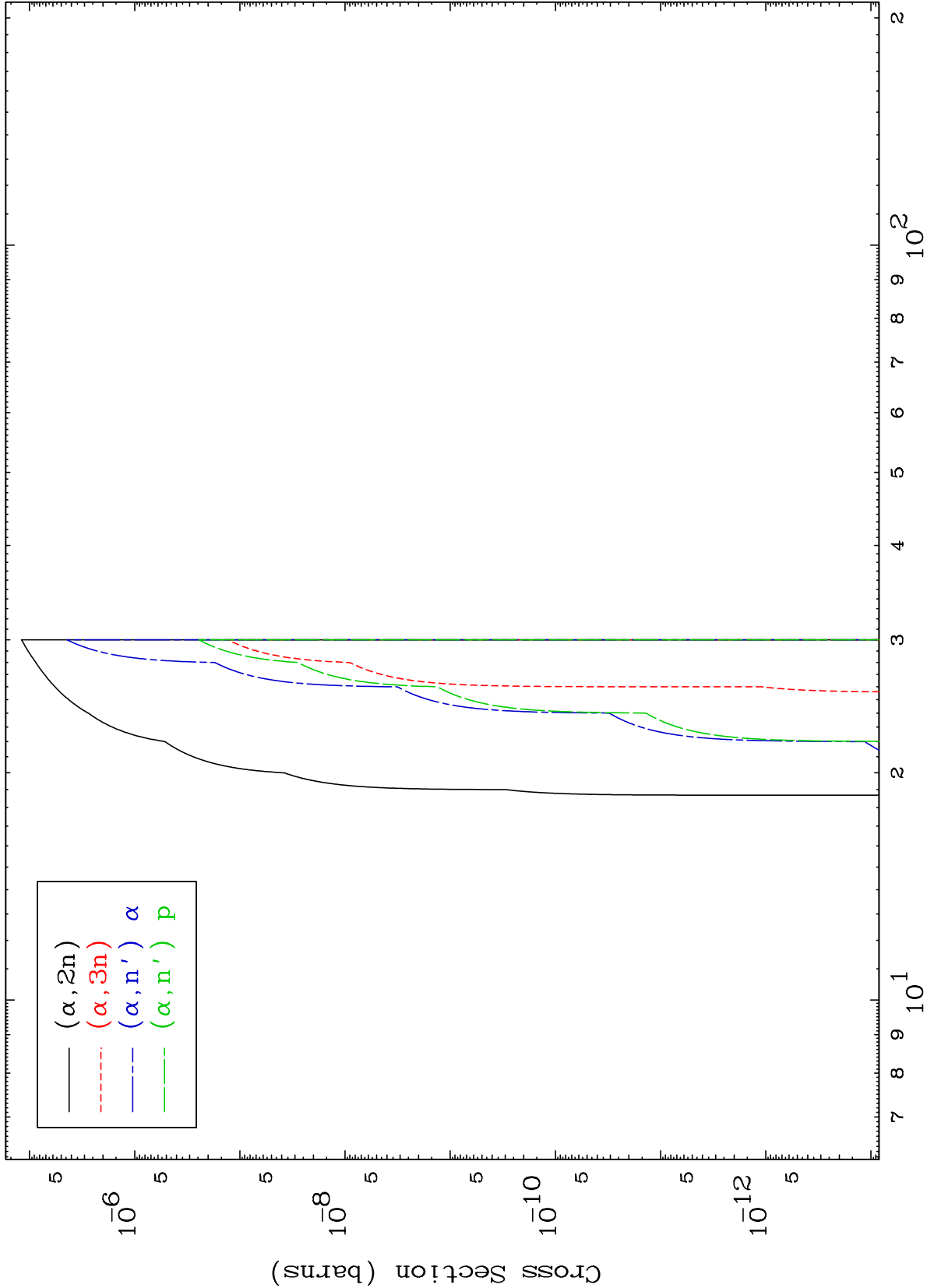
Incident Energy (MeV)

97-Bk-249

MAT 9752

α Neutron Production
0 Kelvin Cross Sections

97-Bk-249

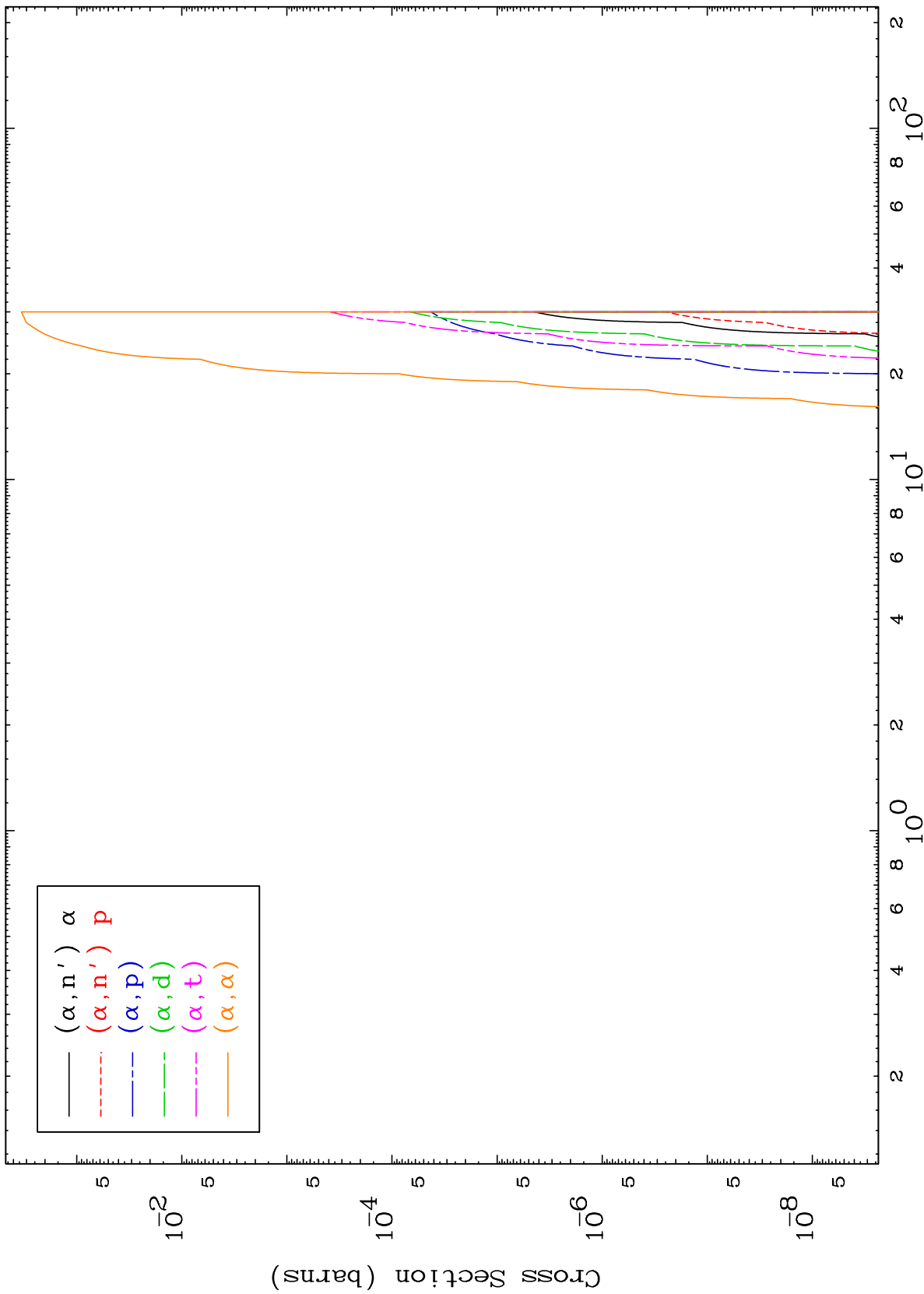


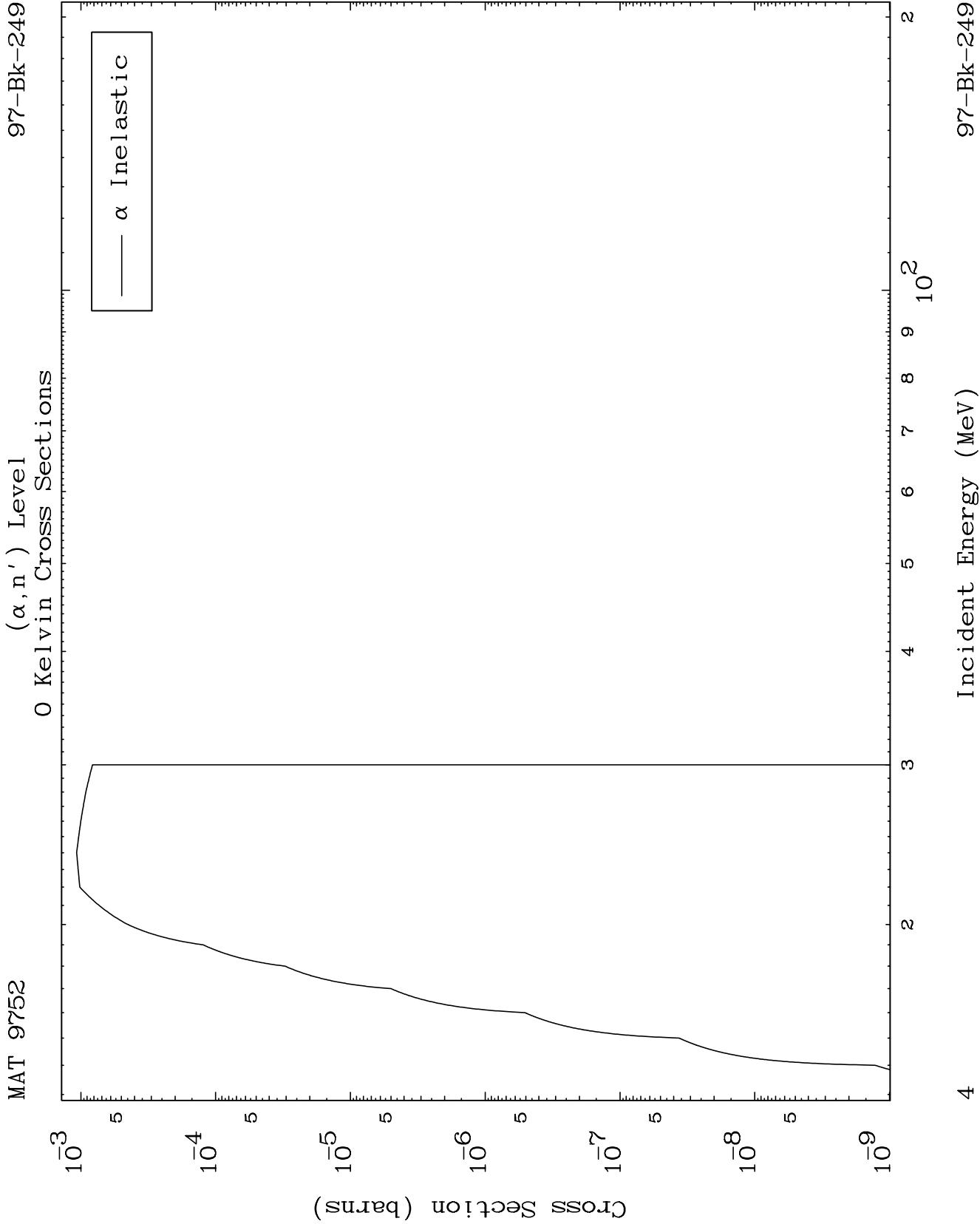
2

Incident Energy (MeV)

97-Bk-249

0 Kelvin Cross Sections



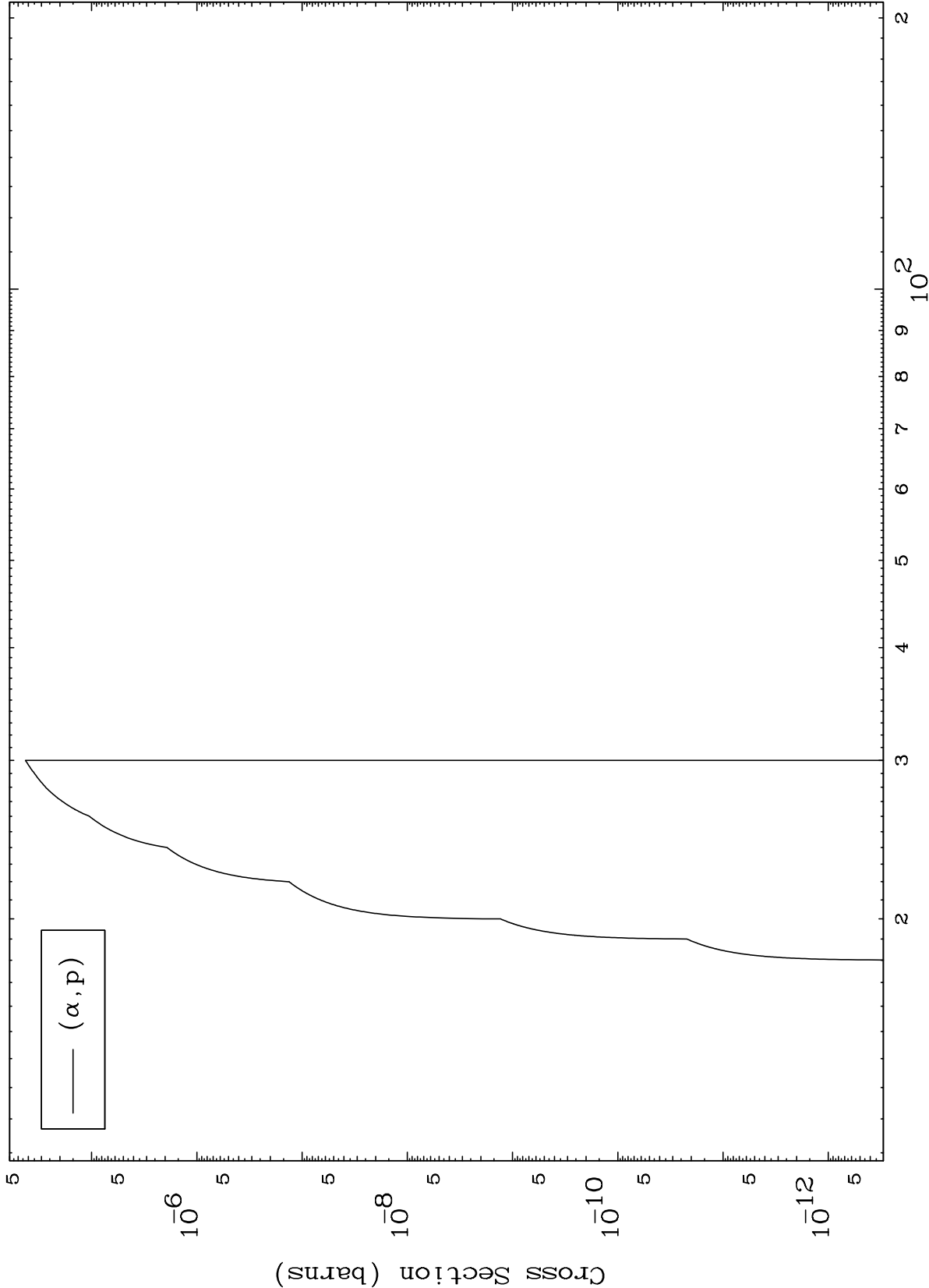


MAT 9752

(α ,p) Levels

97-Bk-249

0 Kelvin Cross Sections

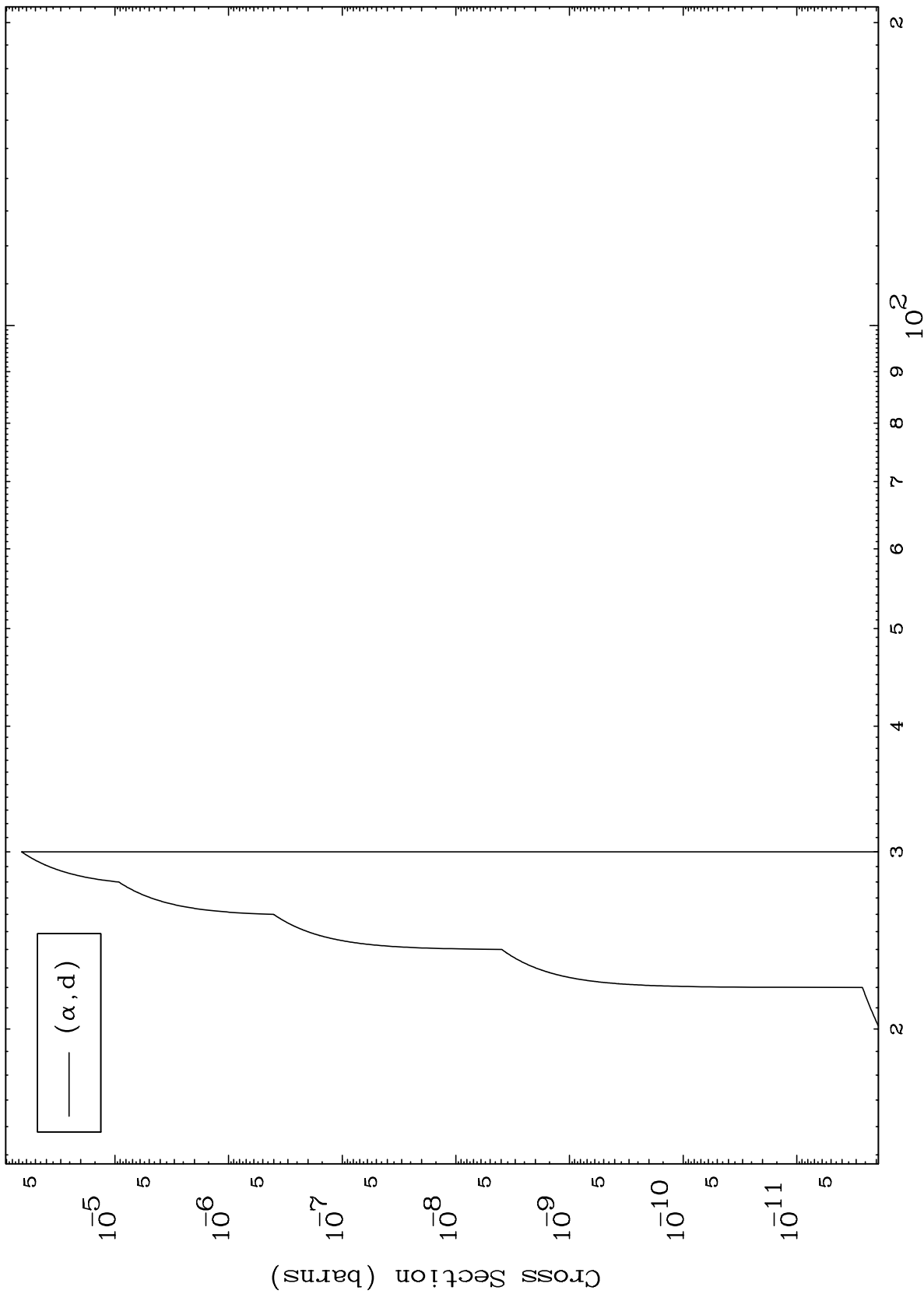


5

Incident Energy (MeV)

97-Bk-249

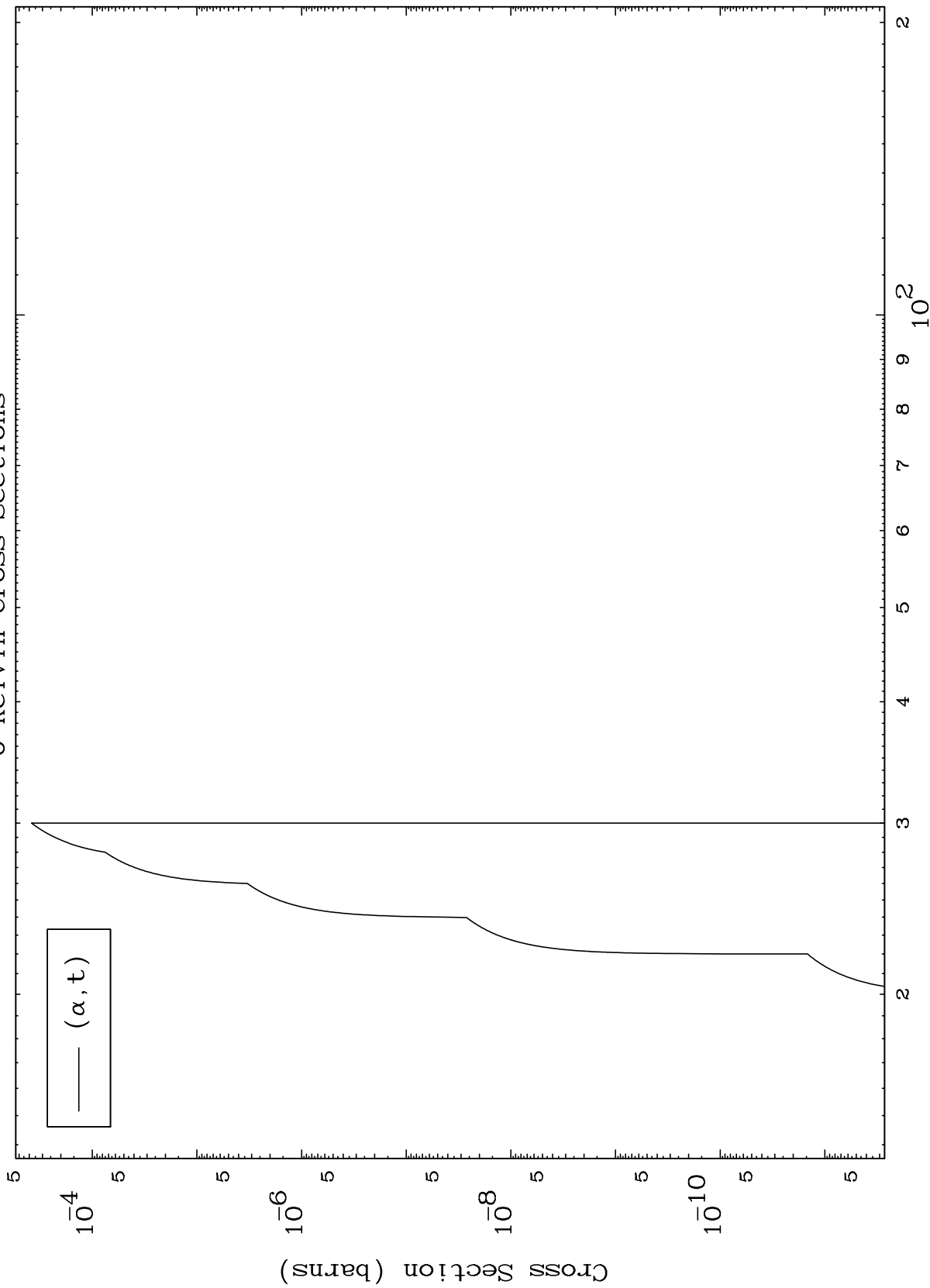
(α, d) Levels
0 Kelvin Cross Sections



MAT 9752

97-Bk-249

(α, t) Levels
0 Kelvin Cross Sections



7

97-Bk-249

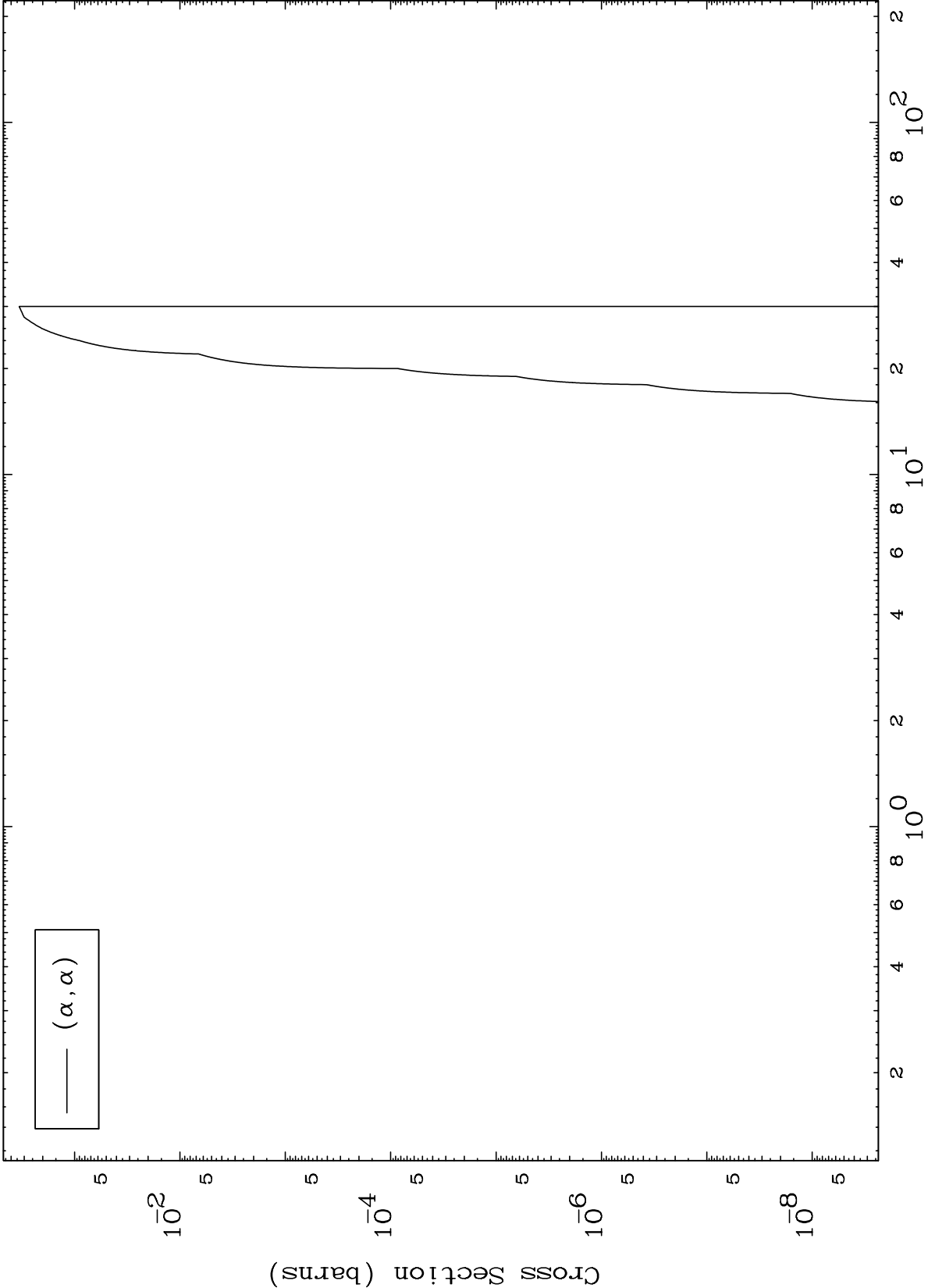
Incident Energy (MeV)

MAT 9752

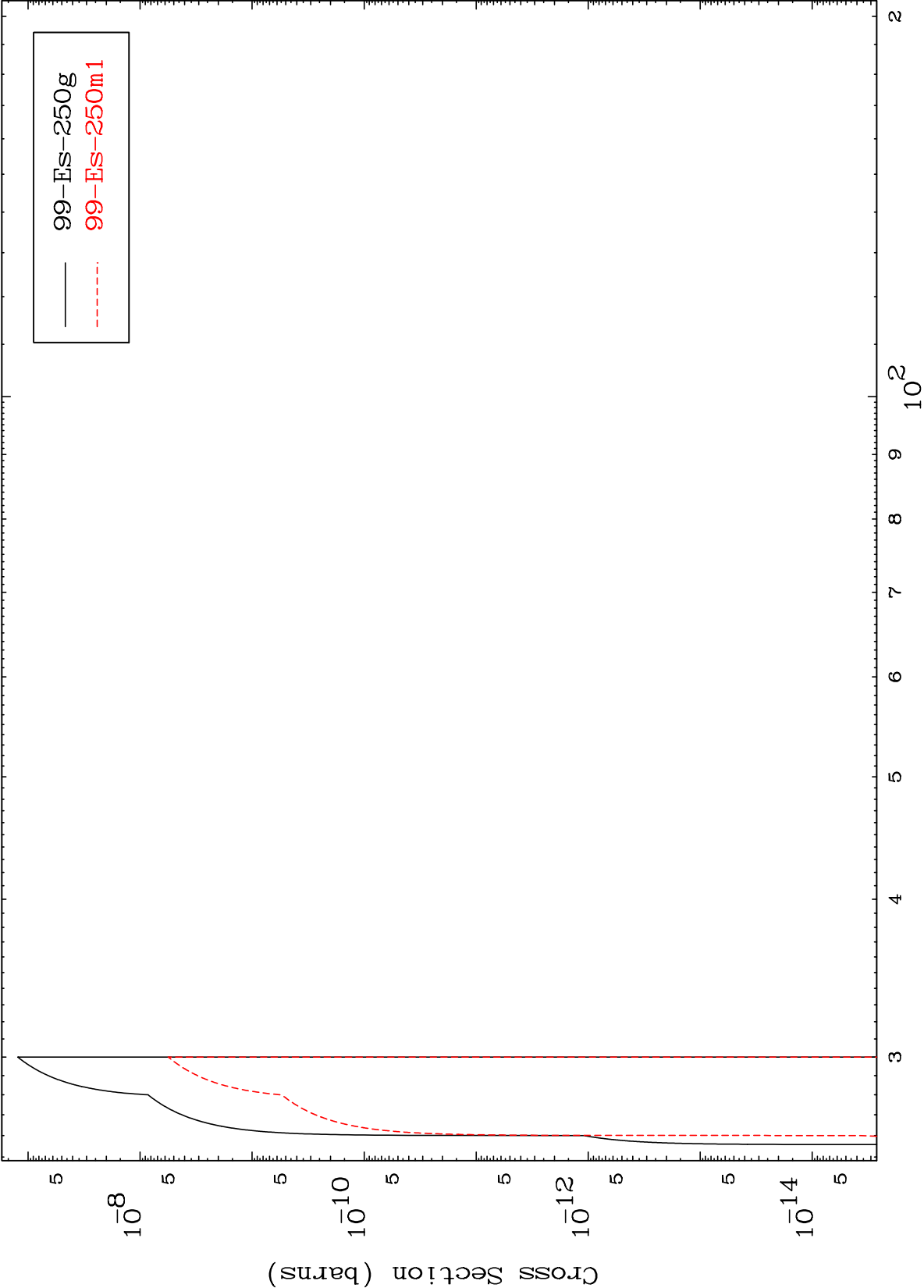
(α, α) Levels

97-Bk-249

0 Kelvin Cross Sections



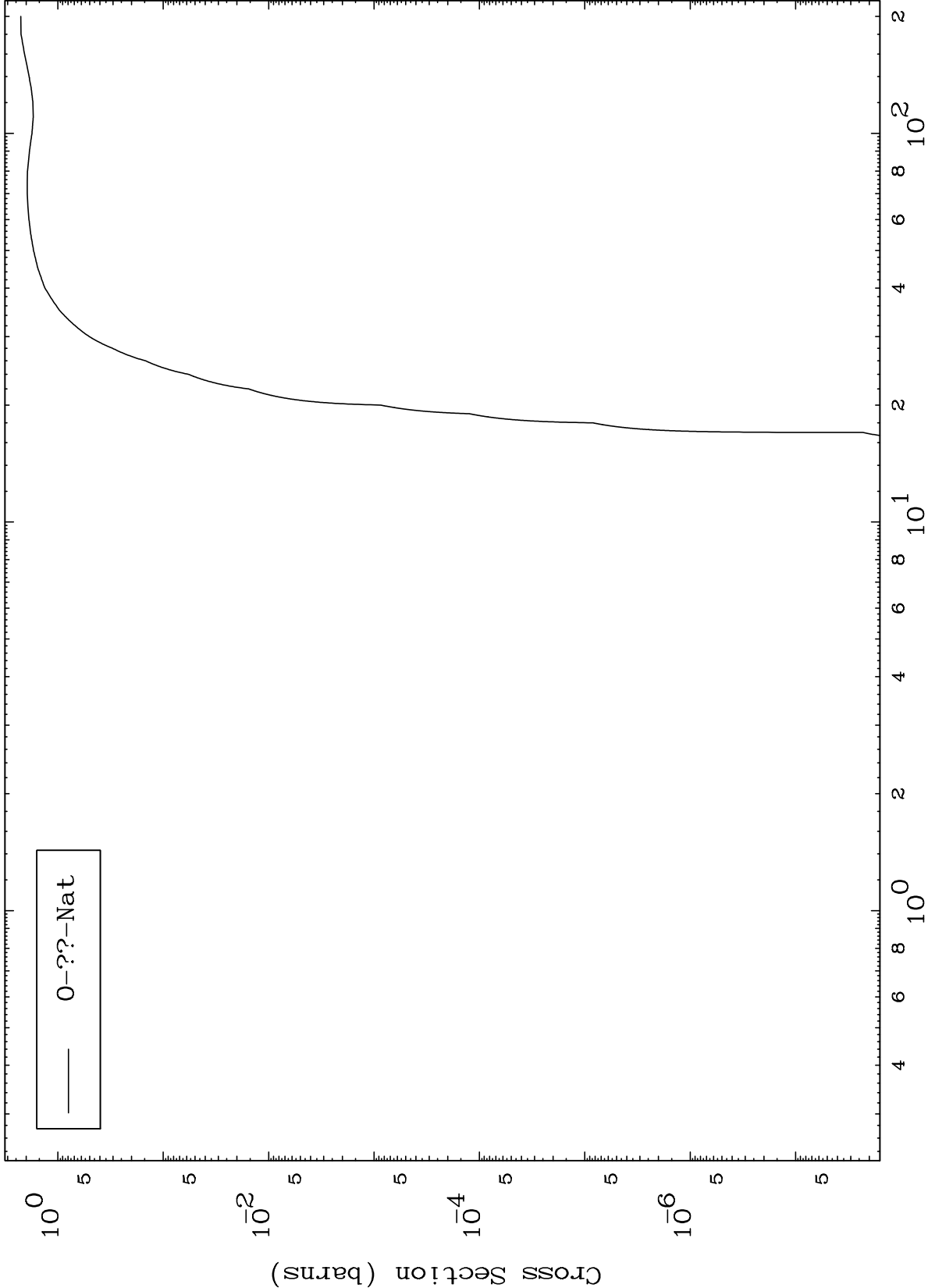
Radionuclide Production Cross Section
($\alpha, 3n$)



MAT 9752

97-Bk-249

α Fission
Radionuclide Production Cross Section



10

Incident Energy (MeV)

97-Bk-249

