

Program EVALPLOT
(Version 2018-1)

by

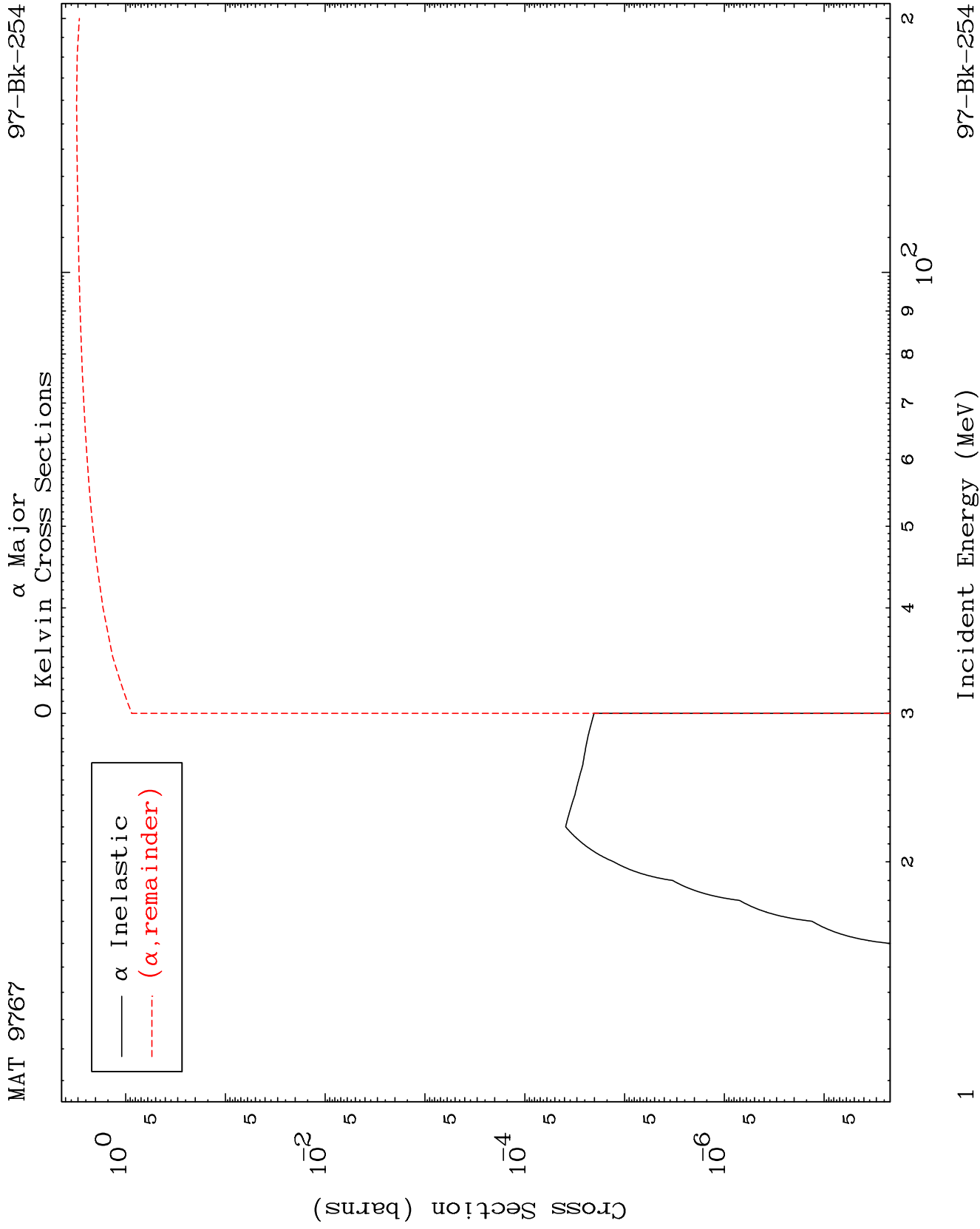
Dermott E. Cullen
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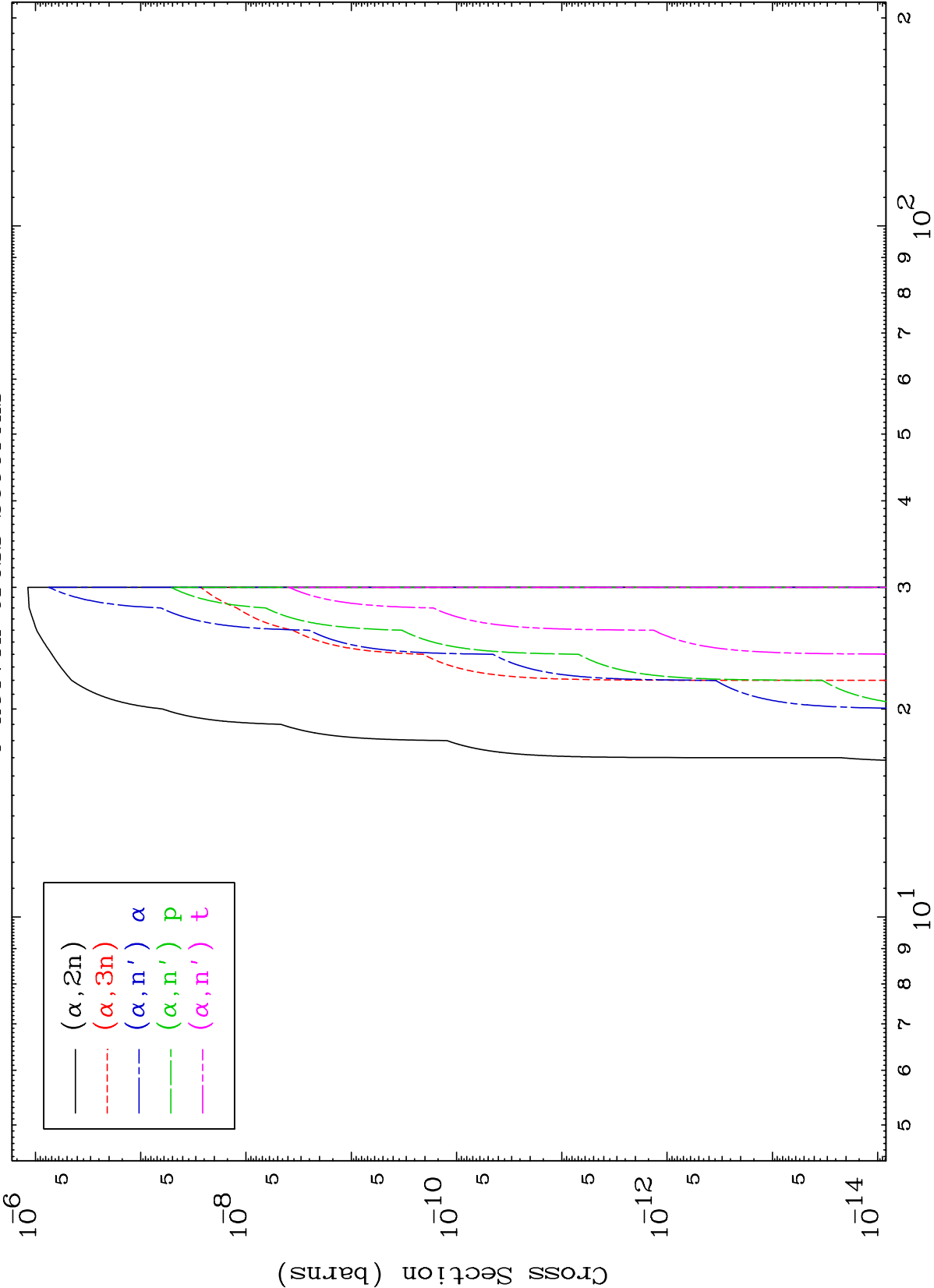
Press Mouse Button to Start



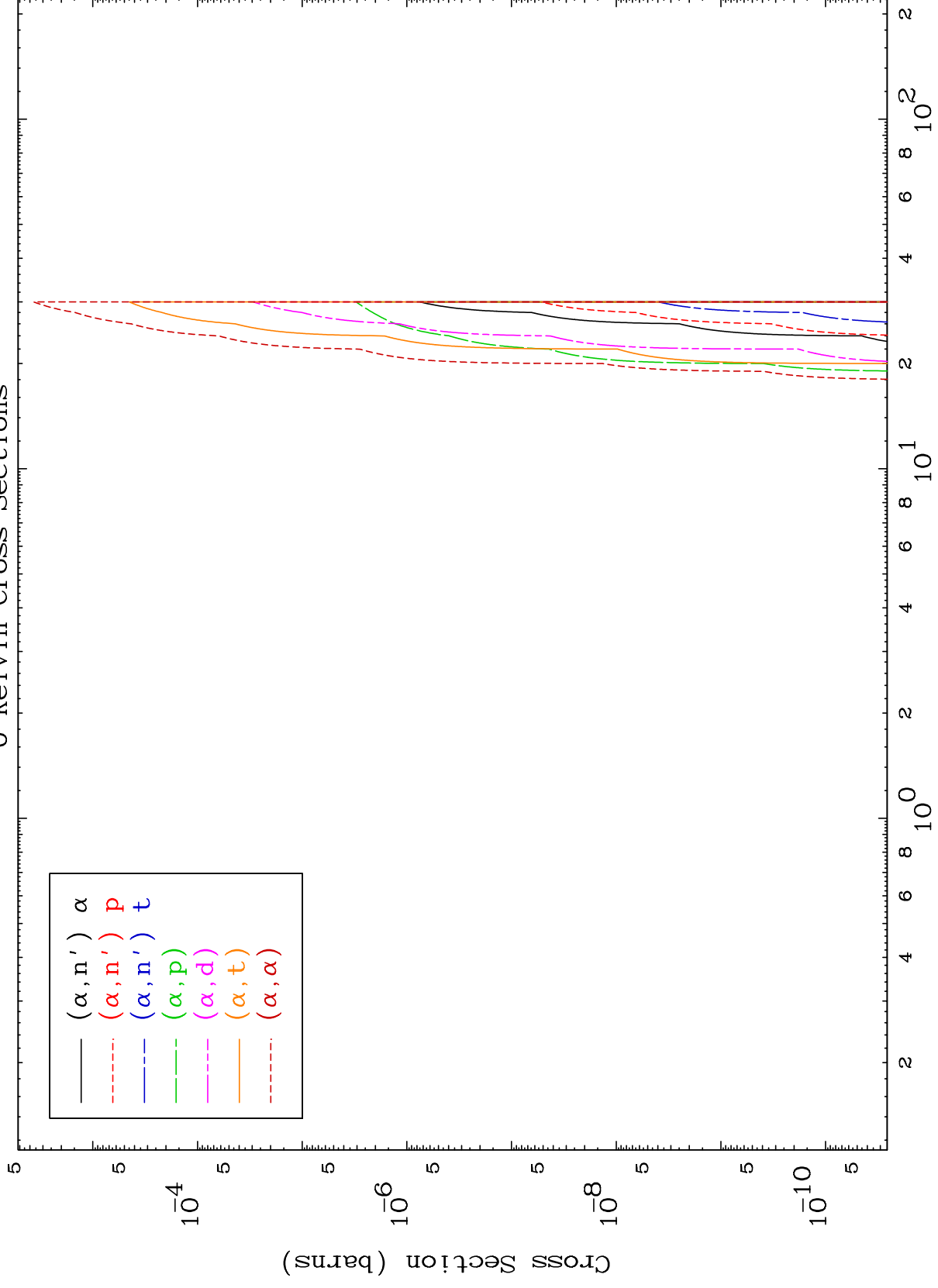
MAT 9767

α Neutron Production
0 Kelvin Cross Sections

97-Bk-254



0 Kelvin Cross Sections



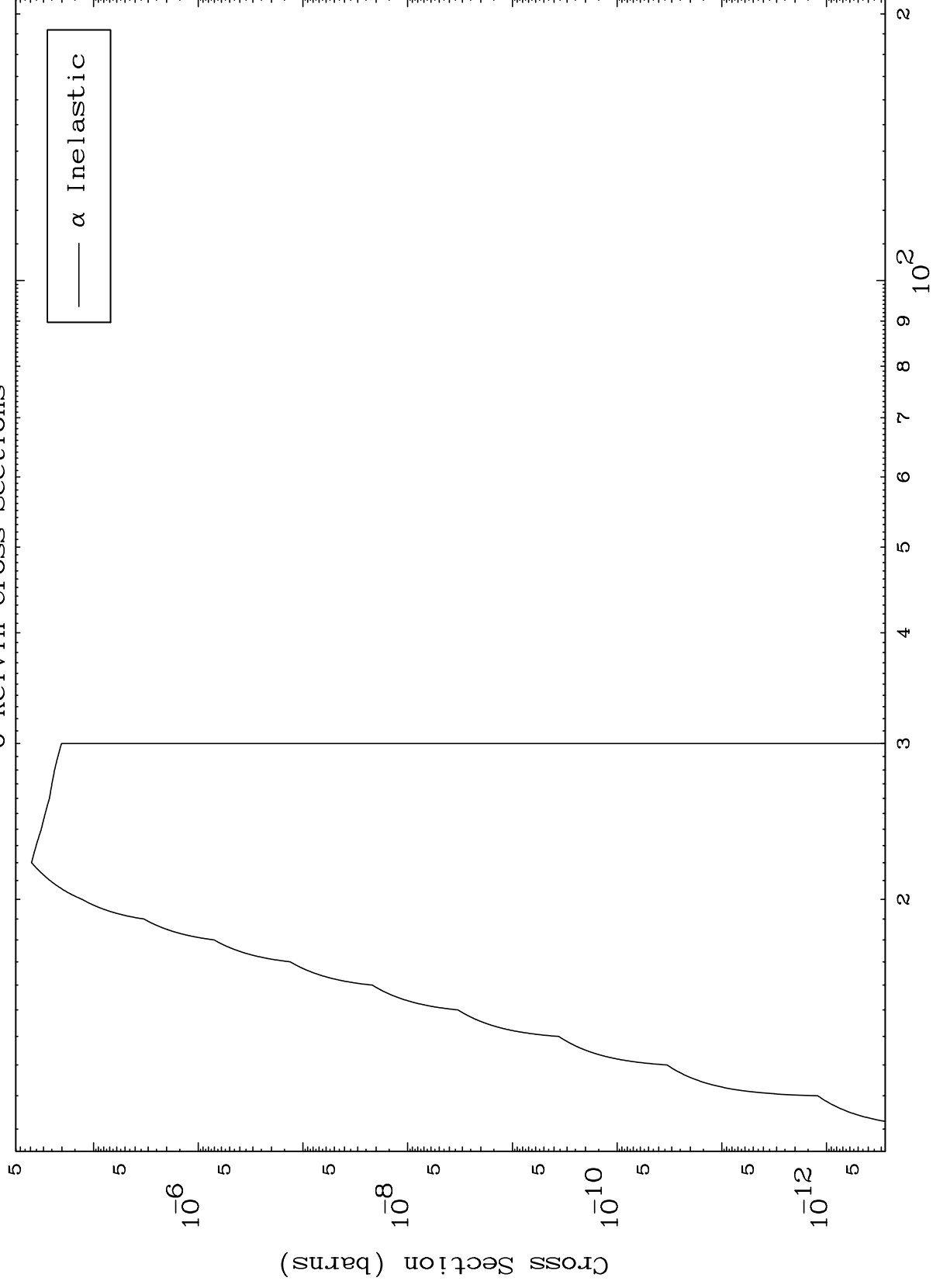
Incident Energy (MeV)

MAT 9767

(α, n') Level

97-Bk-254

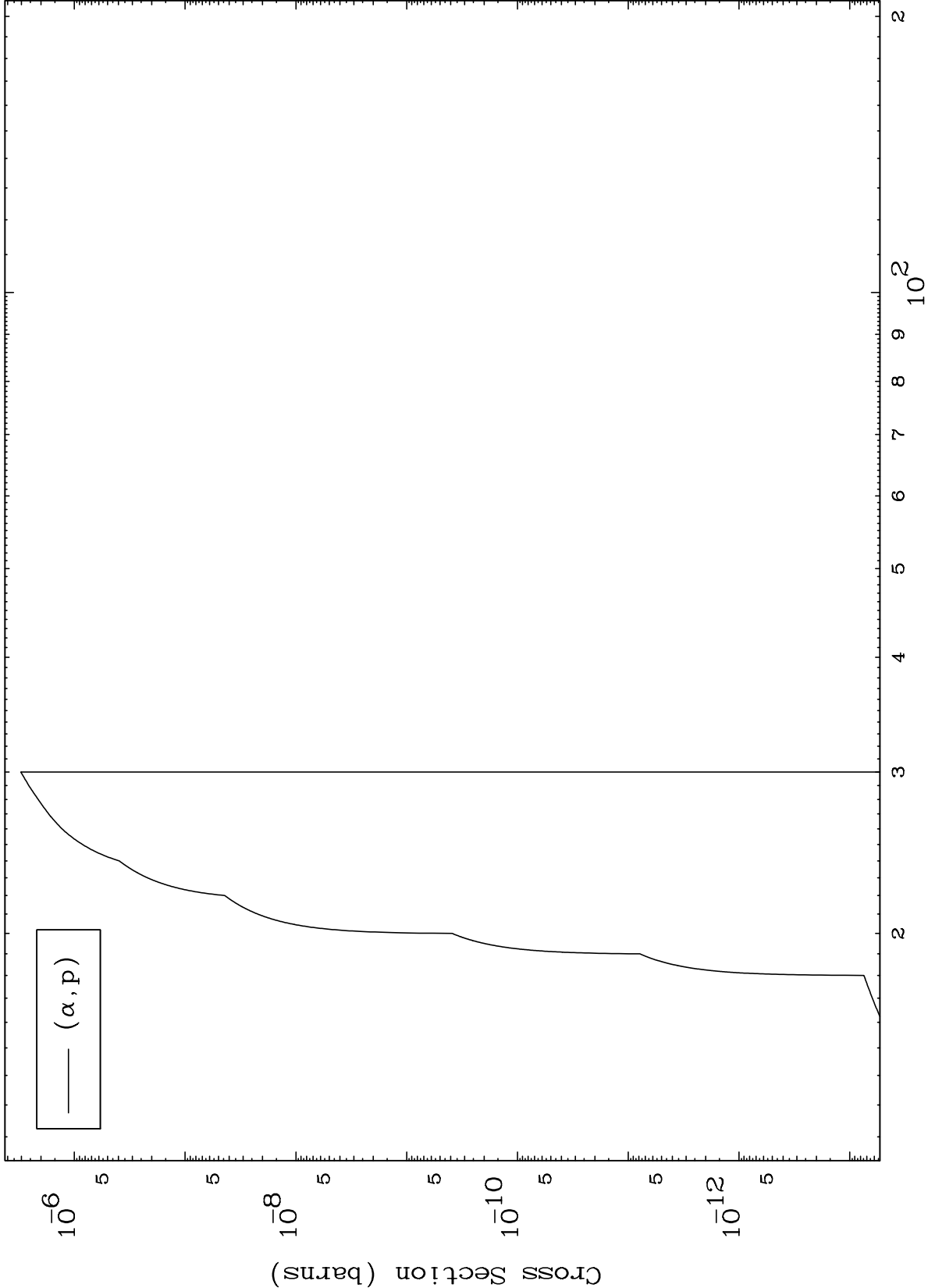
0 Kelvin Cross Sections



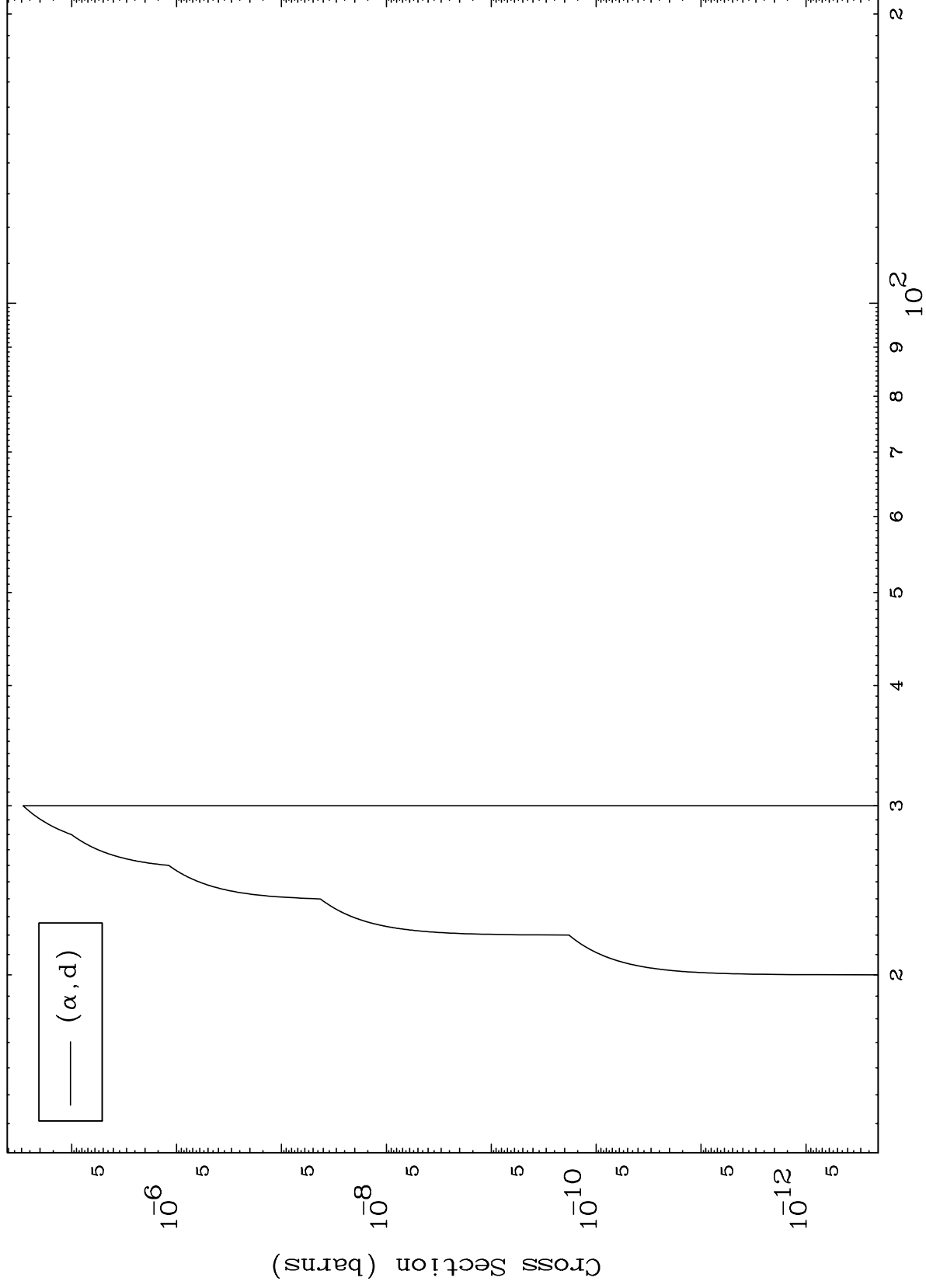
4

Incident Energy (MeV)

97-Bk-254



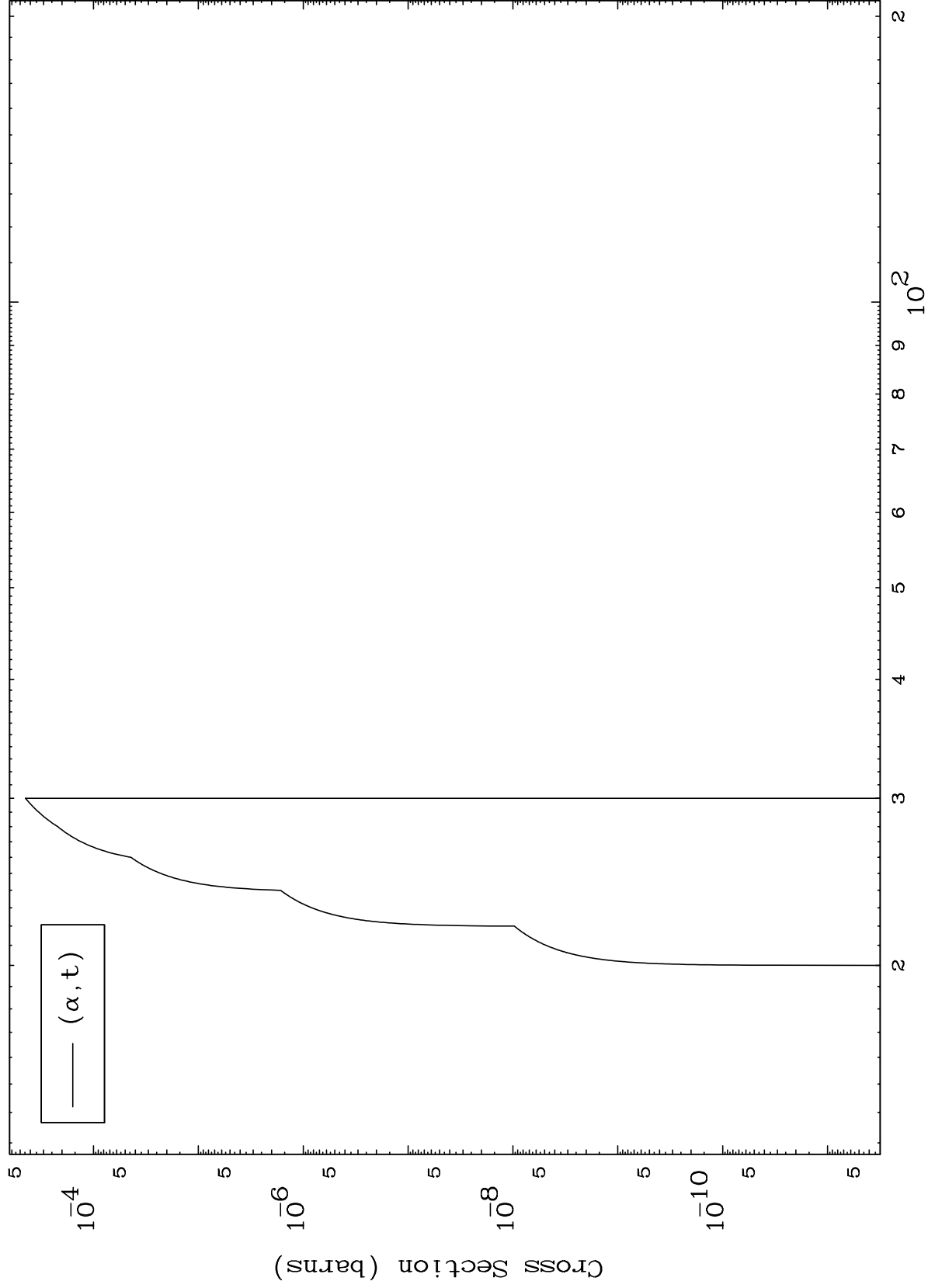
0 Kelvin Cross Sections



MAT 9767

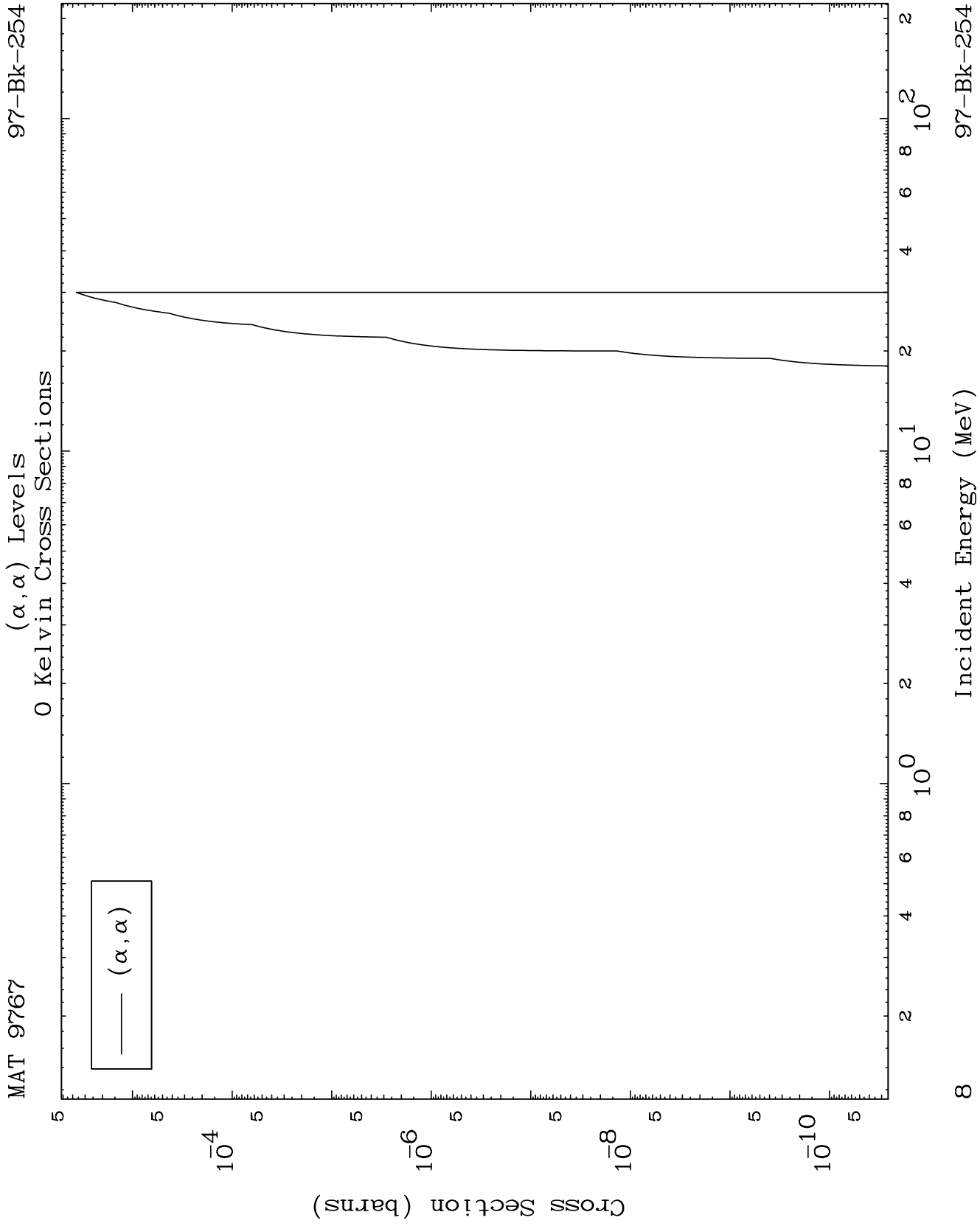
97-Bk-254

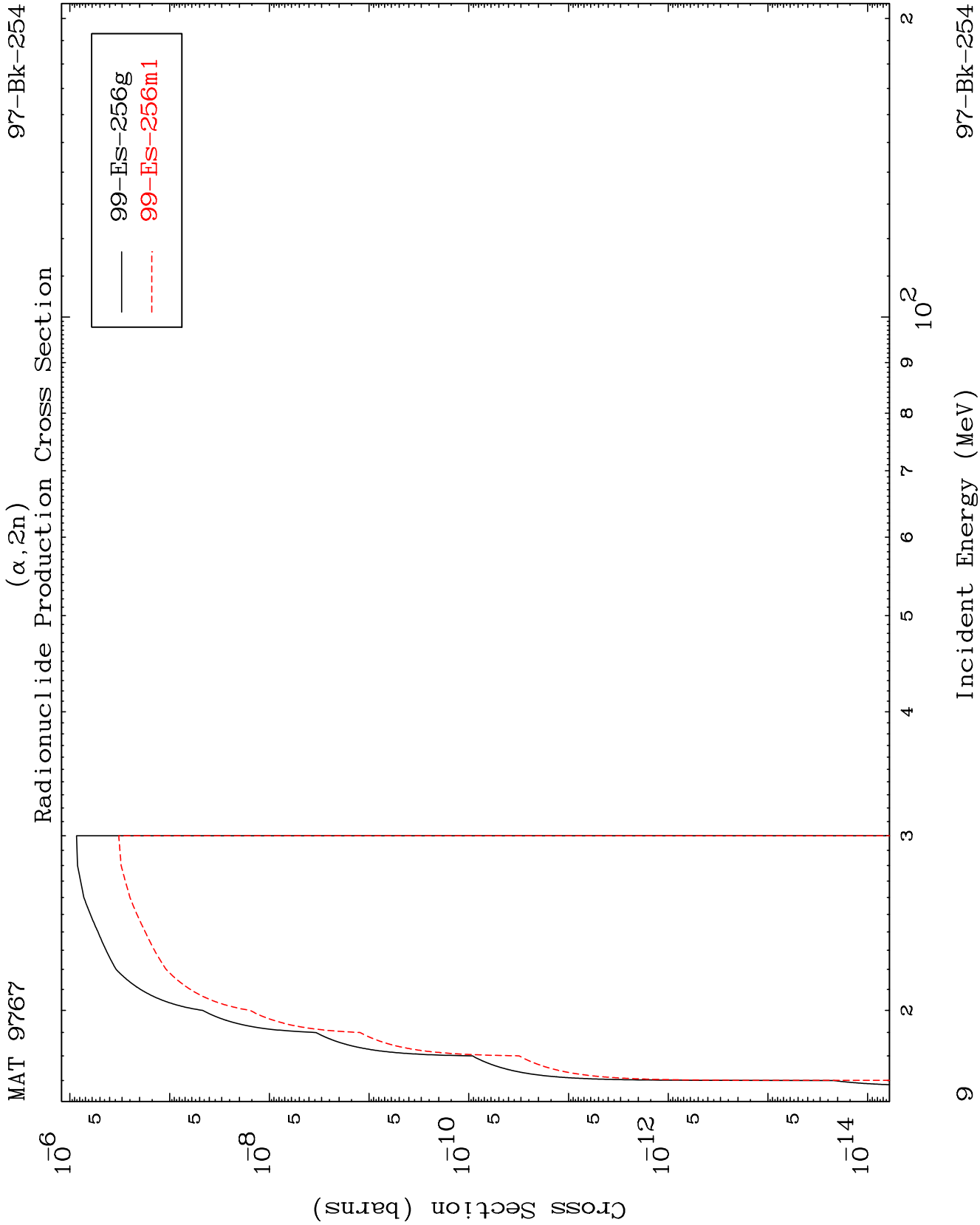
(α, t) Levels
0 Kelvin Cross Sections



7

97-Bk-254

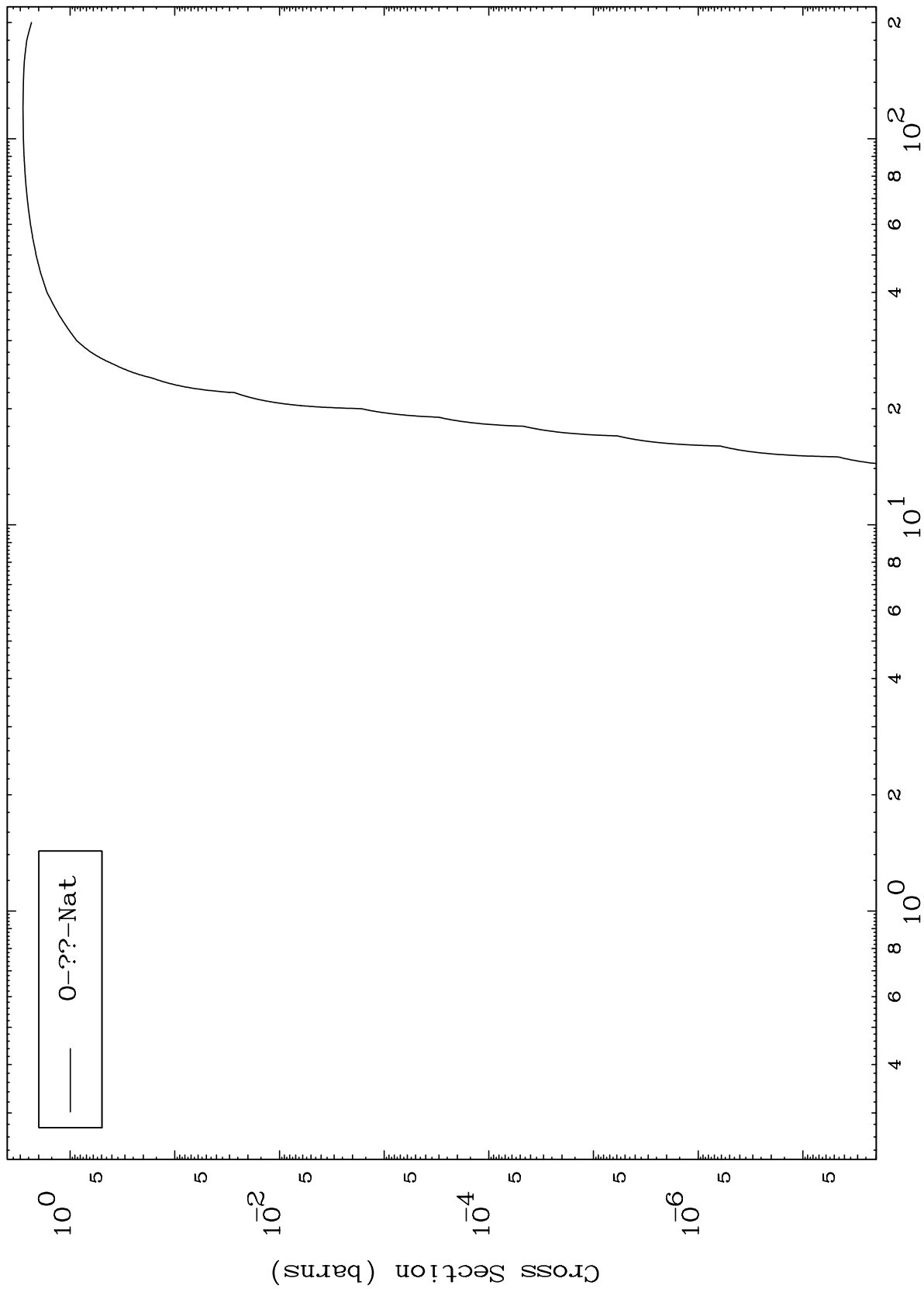




MAT 9767

97-Bk-254

α Fission
Radionuclide Production Cross Section



Incident Energy (MeV)

97-Bk-254