

Program EVALPLOT  
(Version 2018-1)

by

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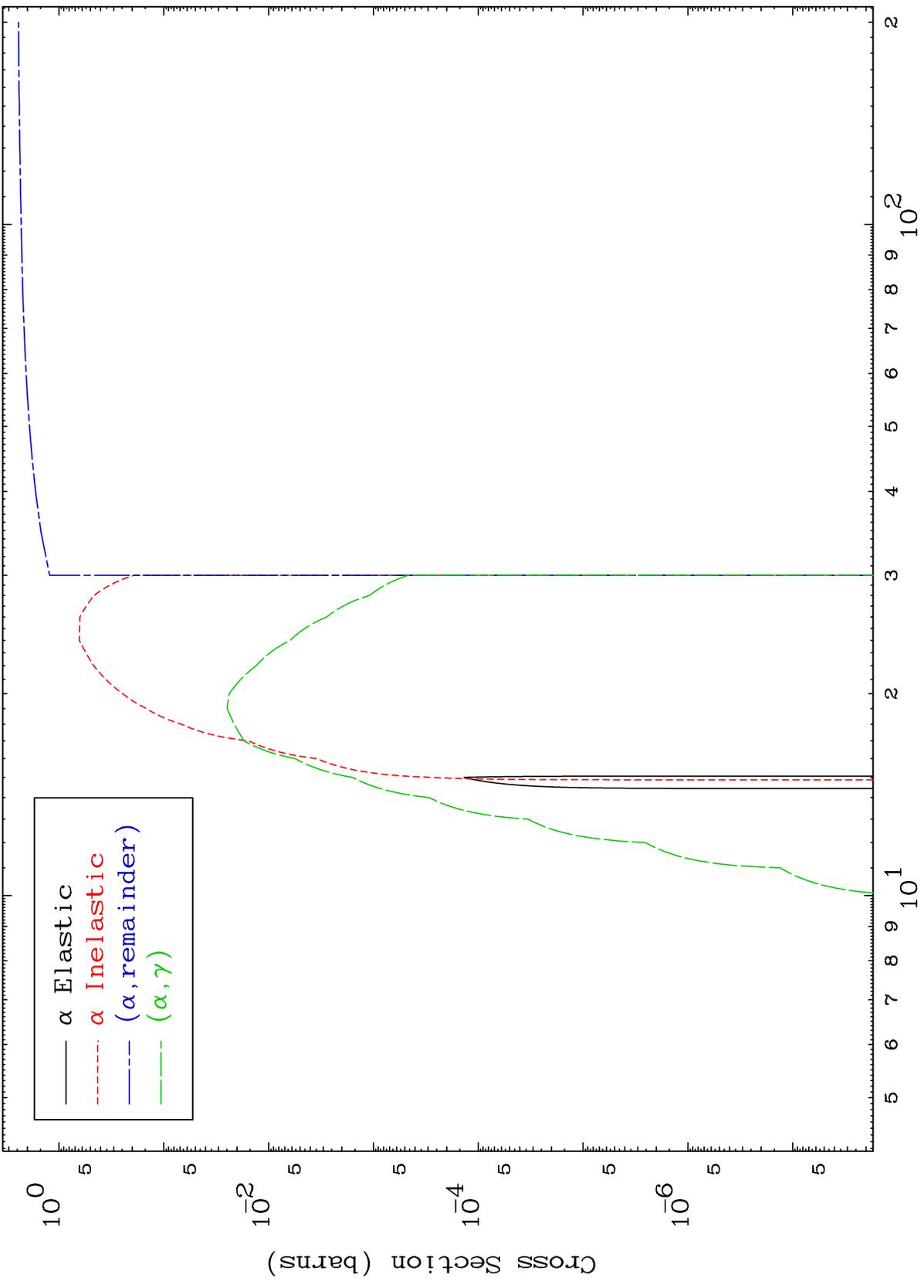
Press Mouse Button to Start

MAT 6607

$\alpha$  Major

66-Dy-150

0 Kelvin Cross Sections



1

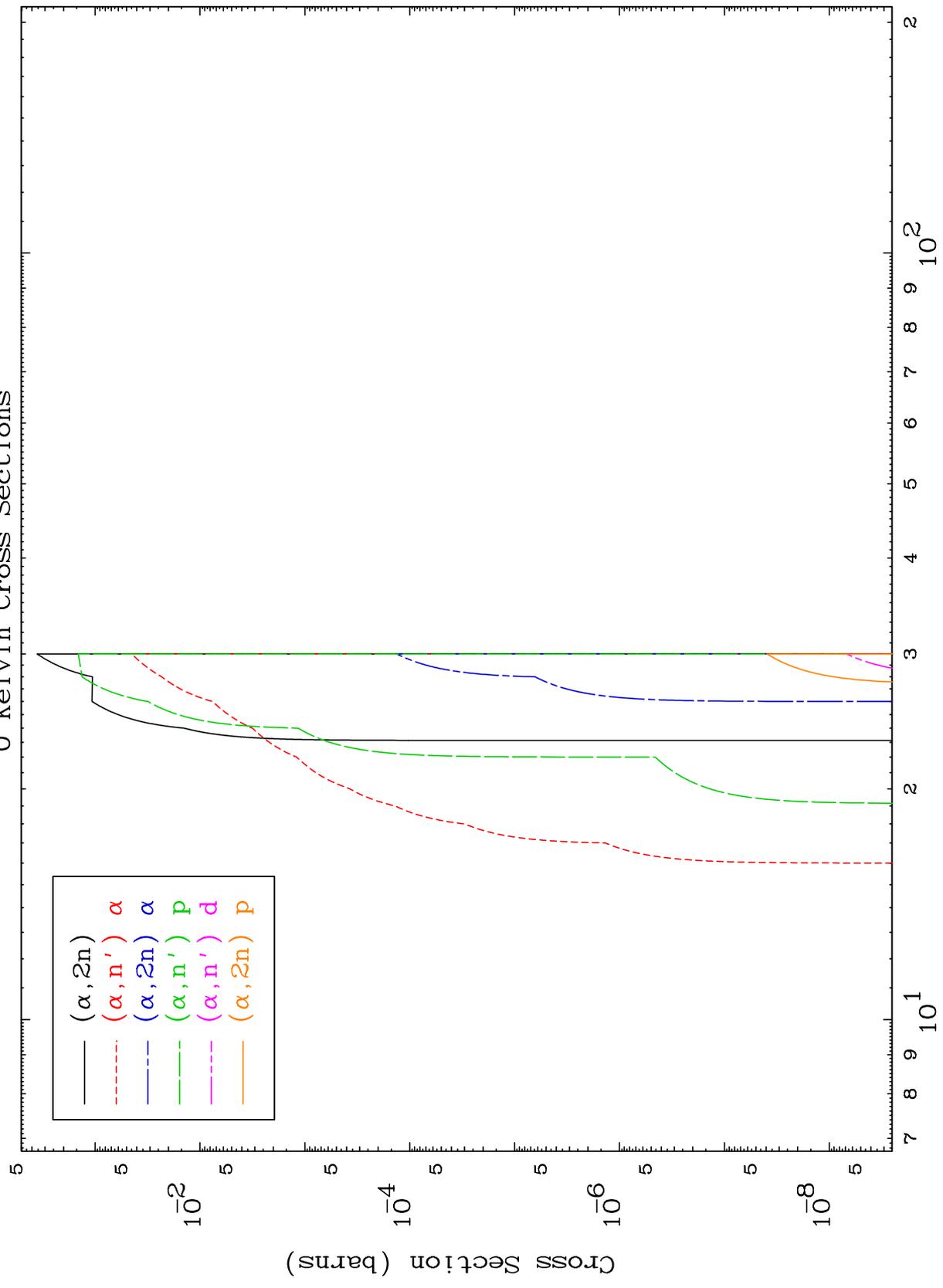
Incident Energy (MeV)

66-Dy-150

MAT 6607

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

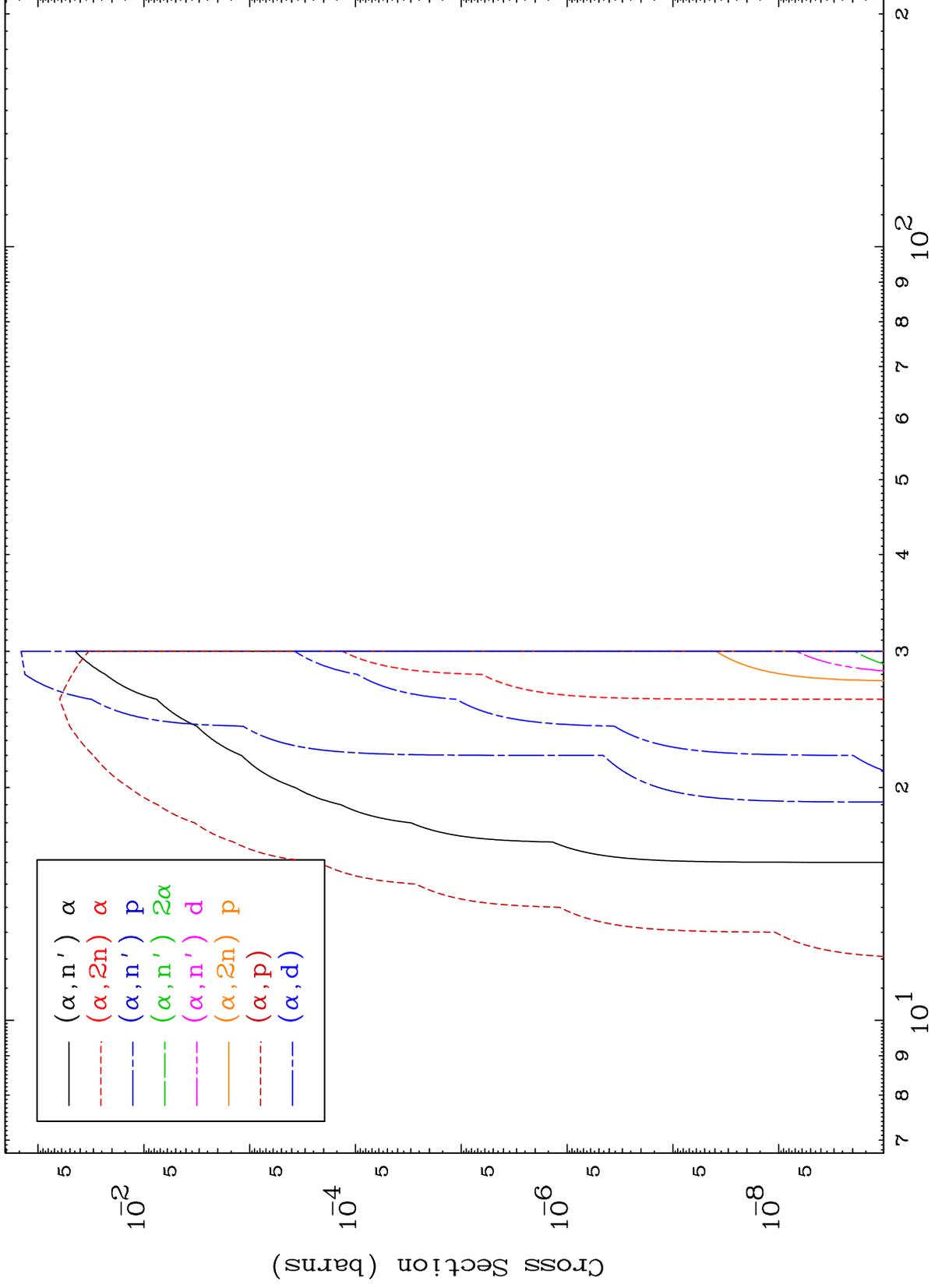
66-Dy-150



2

Incident Energy (MeV)

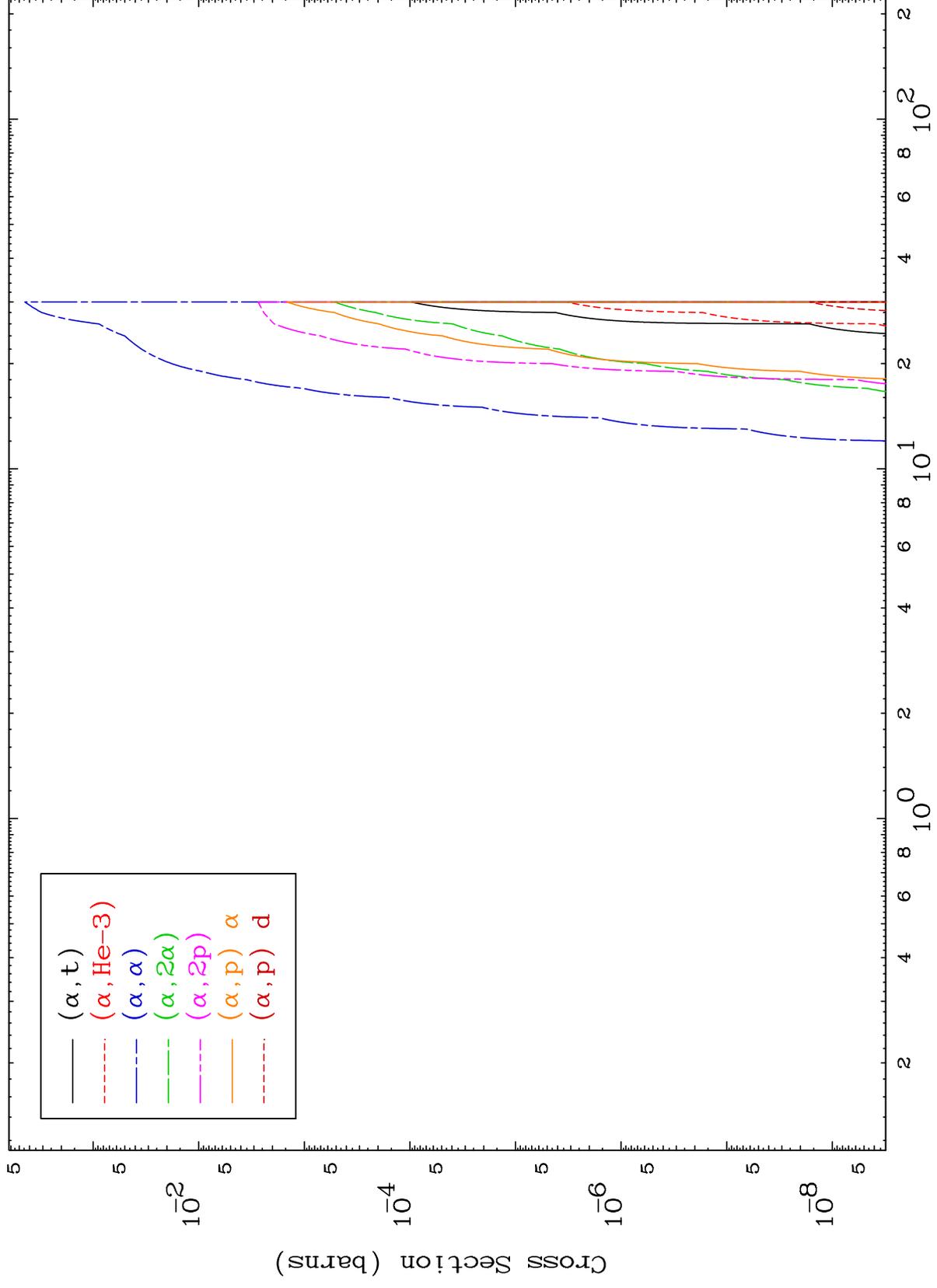
66-Dy-150



MAT 6607

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

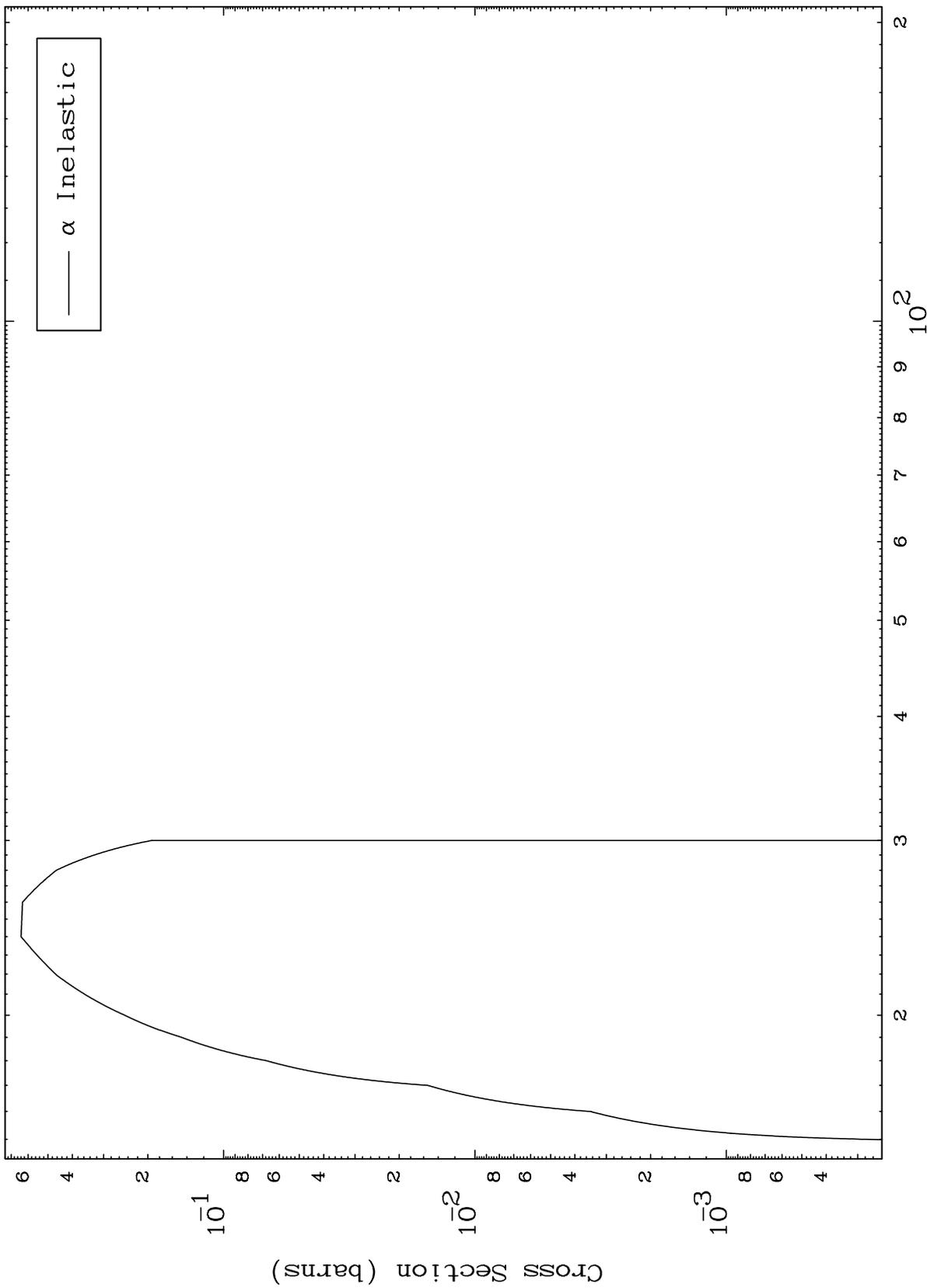
66-Dy-150



MAT 6607

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

66-Dy-150



5

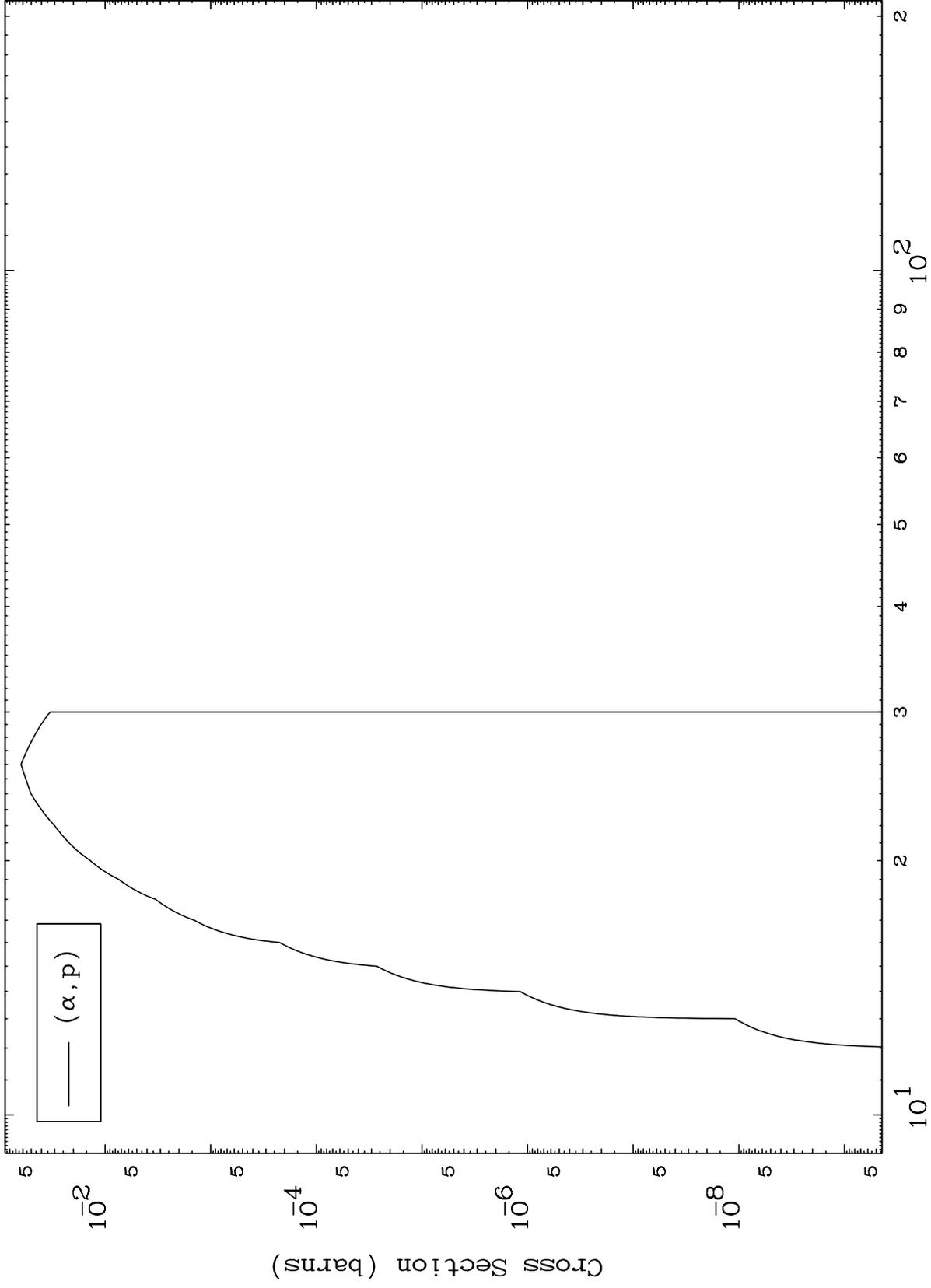
Incident Energy (MeV)

66-Dy-150

MAT 6607

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

66-Dy-150



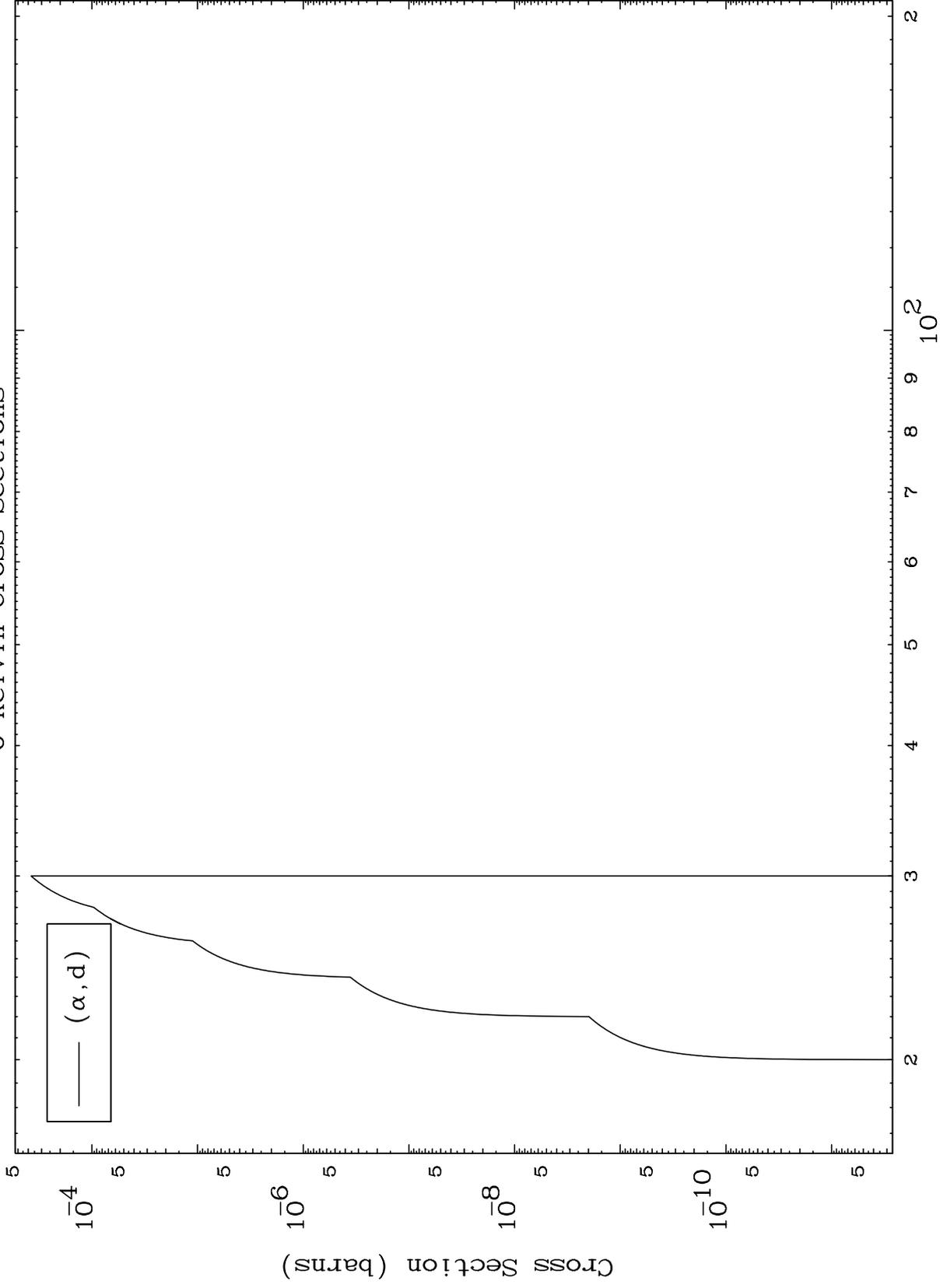
Incident Energy (MeV)

66-Dy-150

MAT 6607

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

66-Dy-150



7

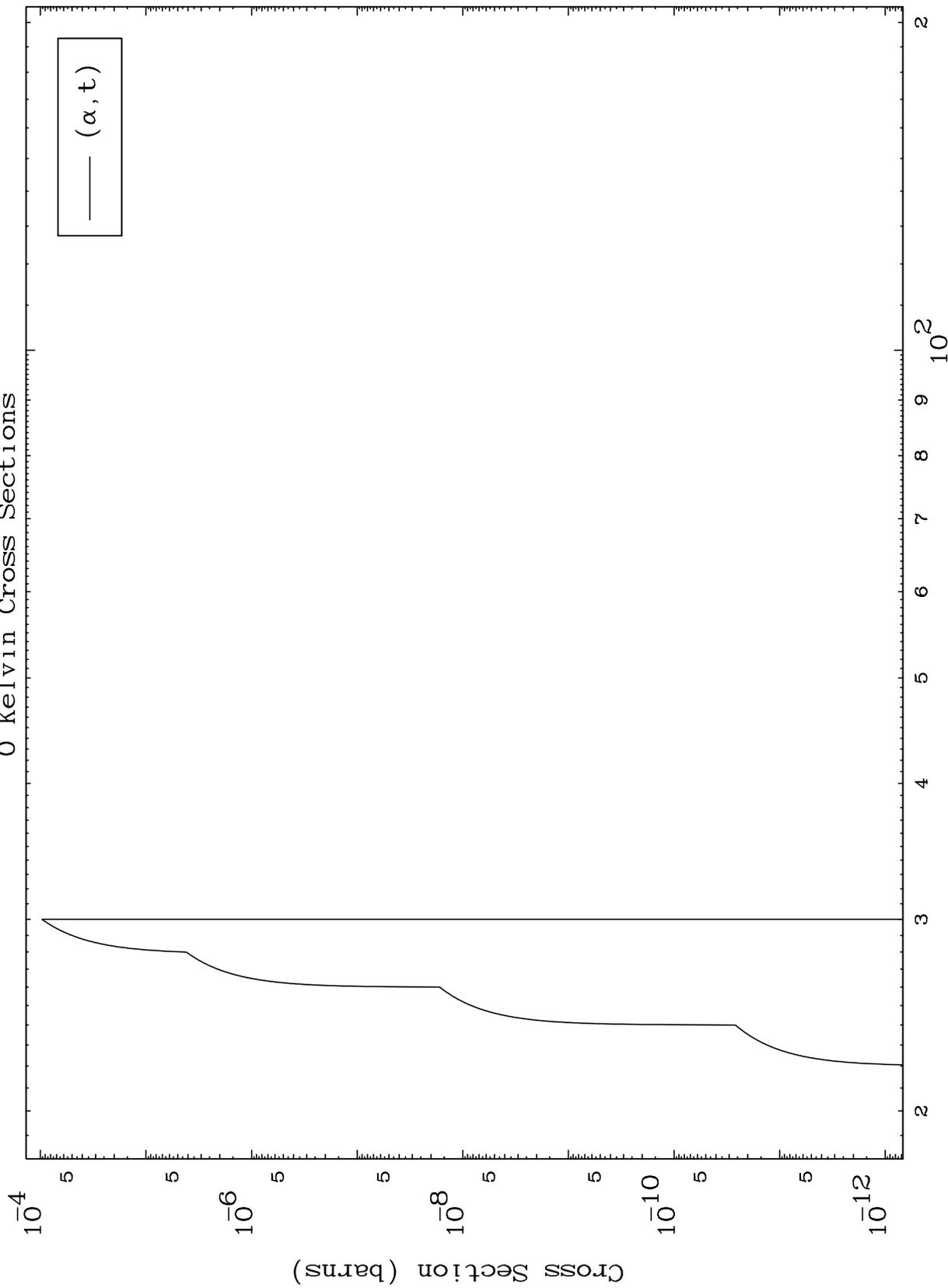
Incident Energy (MeV)

66-Dy-150

MAT 6607

66-Dy-150

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

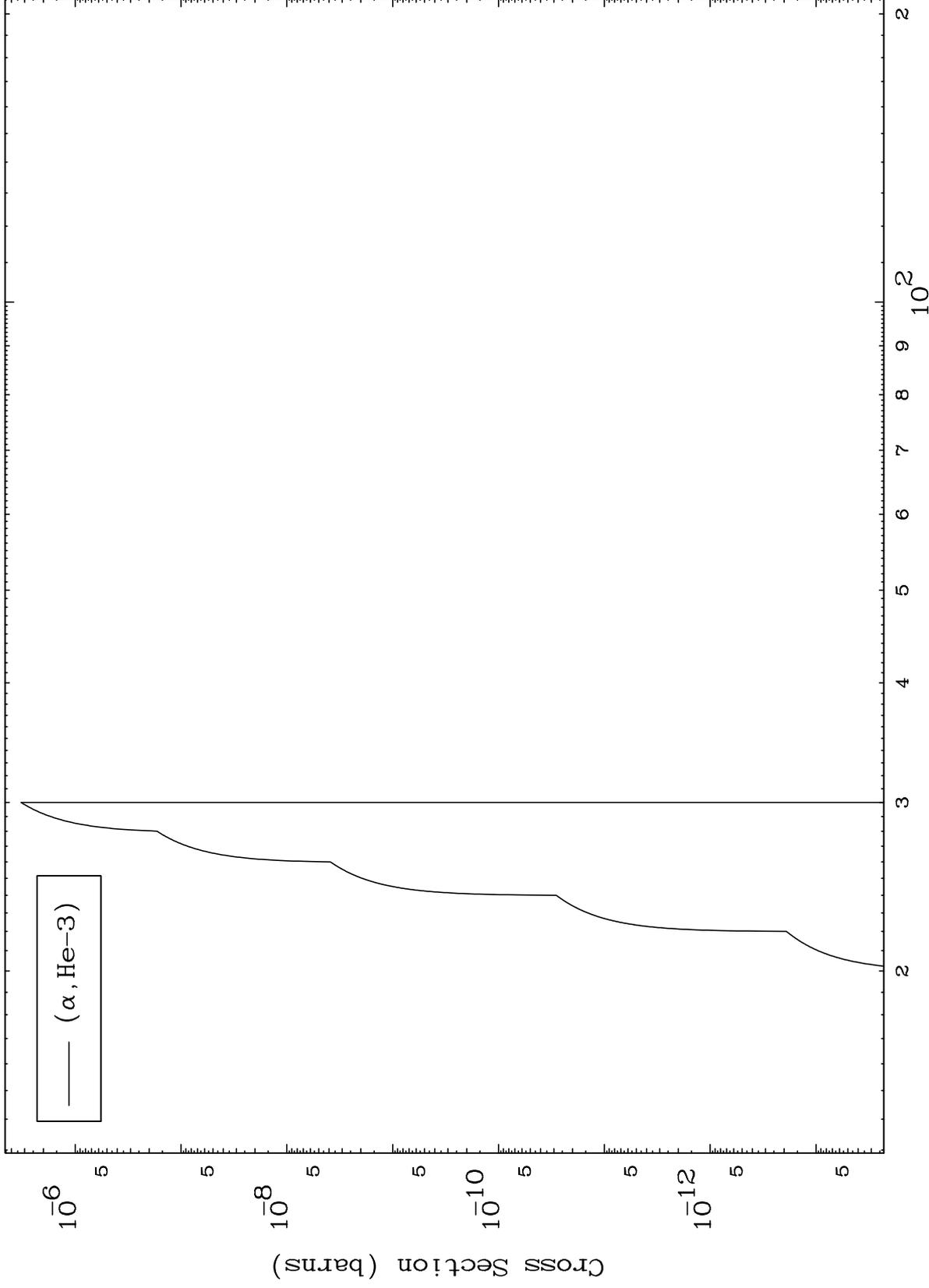


66-Dy-150

Incident Energy (MeV)

8

0 Kelvin Cross Sections

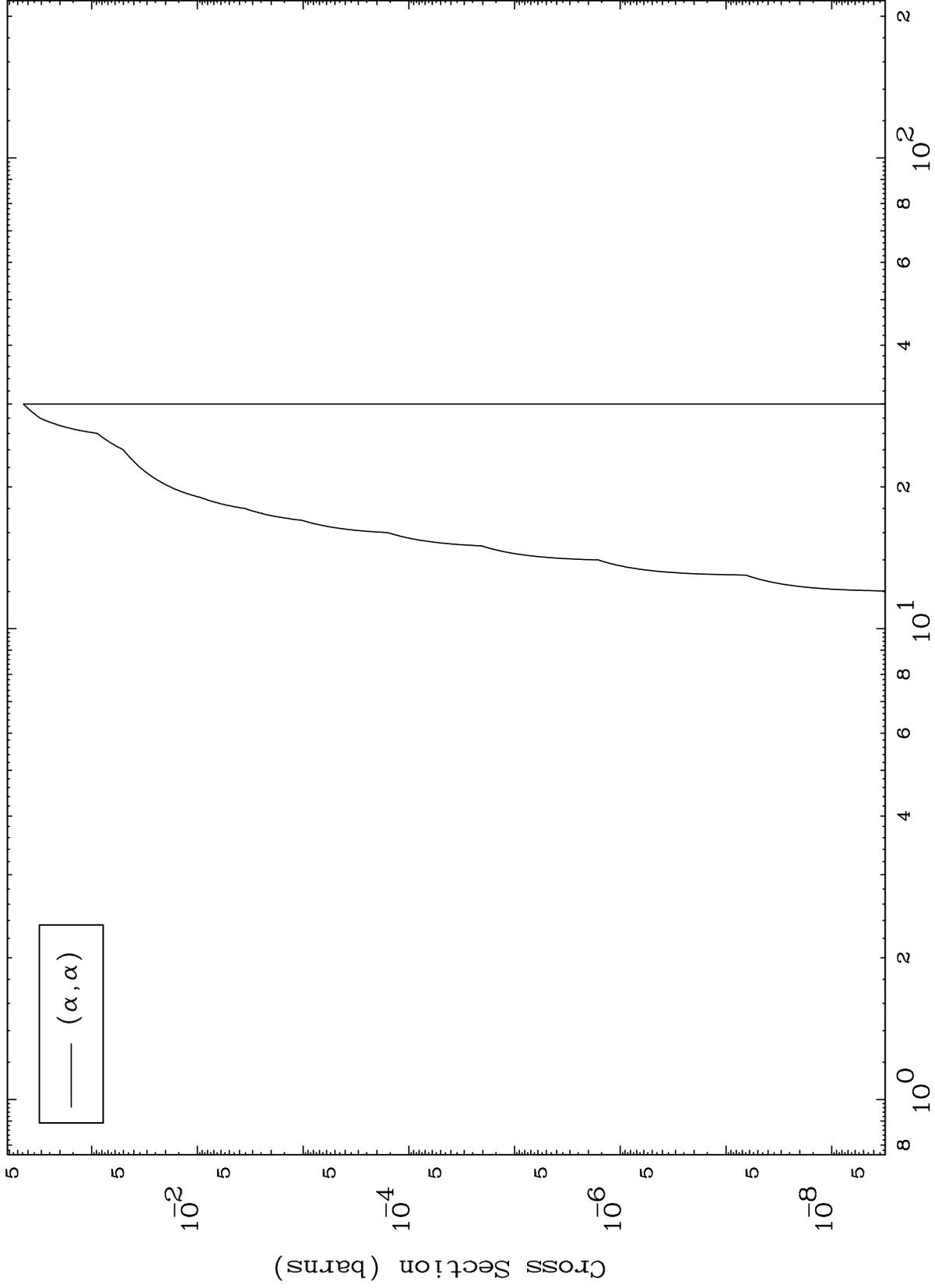


MAT 6607

( $\alpha, \alpha$ ) Levels

66-Dy-150

0 Kelvin Cross Sections



Incident Energy (MeV)

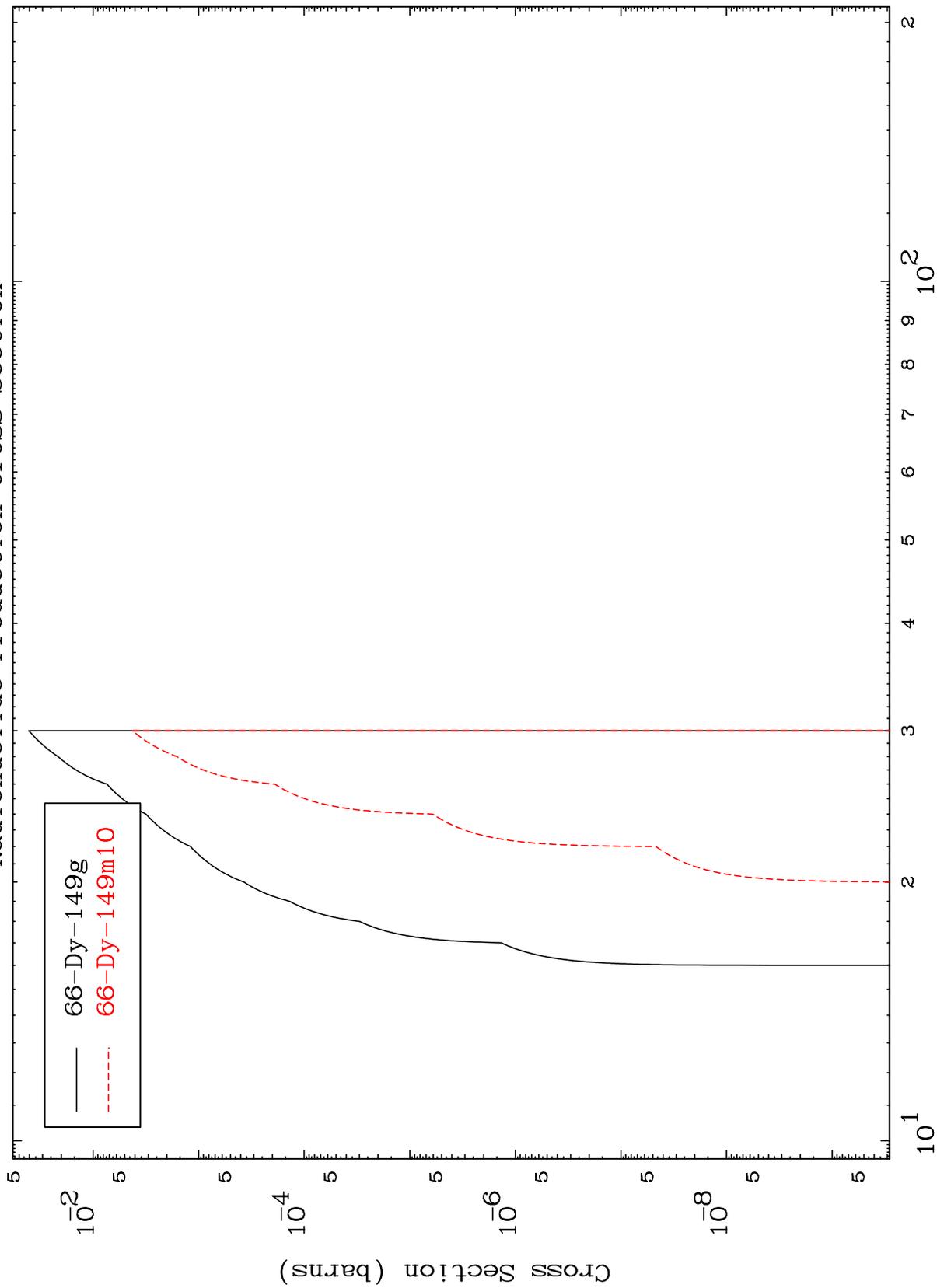
66-Dy-150

MAT 6607

$(\alpha, n')$   $\alpha$

66-Dy-150

Radionuclide Production Cross Section



Incident Energy (MeV)

66-Dy-150

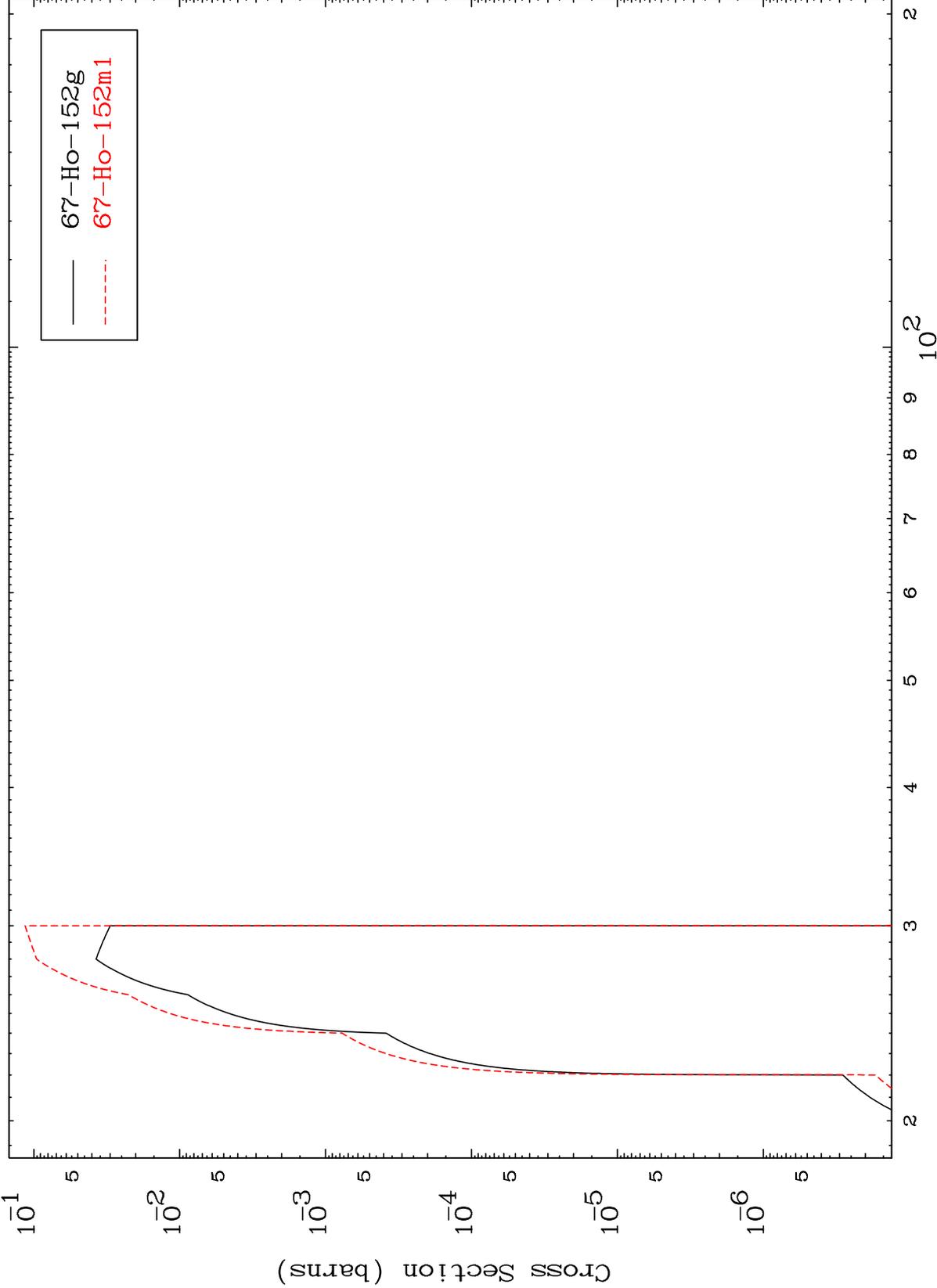
11

MAT 6607

$(\alpha, n')$  p

66-Dy-150

Radionuclide Production Cross Section



12

Incident Energy (MeV)

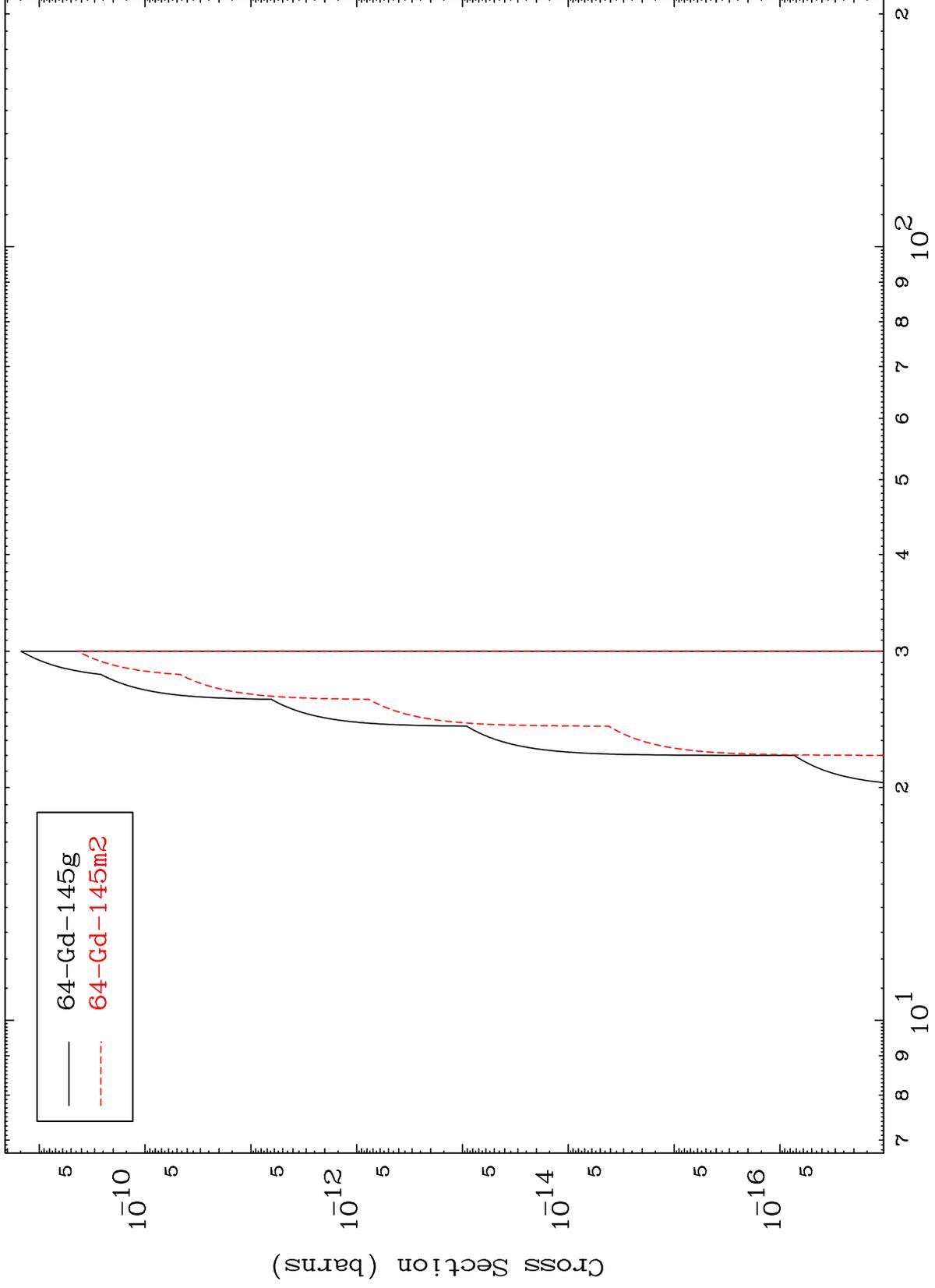
66-Dy-150

MAT 6607

$(\alpha, n')$   $2\alpha$

66-Dy-150

Radionuclide Production Cross Section



64-Gd-145g  
64-Gd-145m2

13

Incident Energy (MeV)

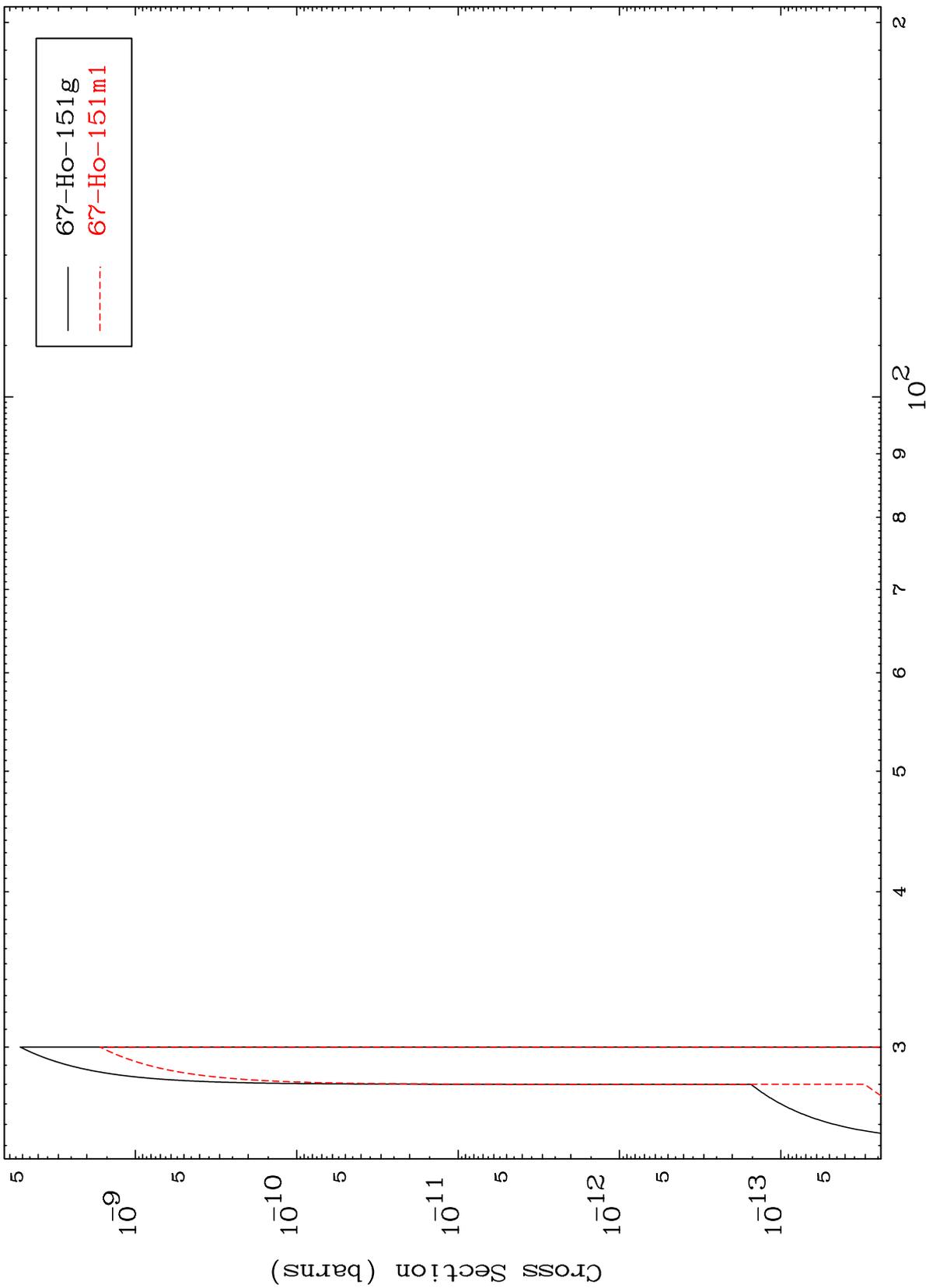
66-Dy-150

MAT 6607

$(\alpha, n')$  d

66-Dy-150

Radionuclide Production Cross Section



14

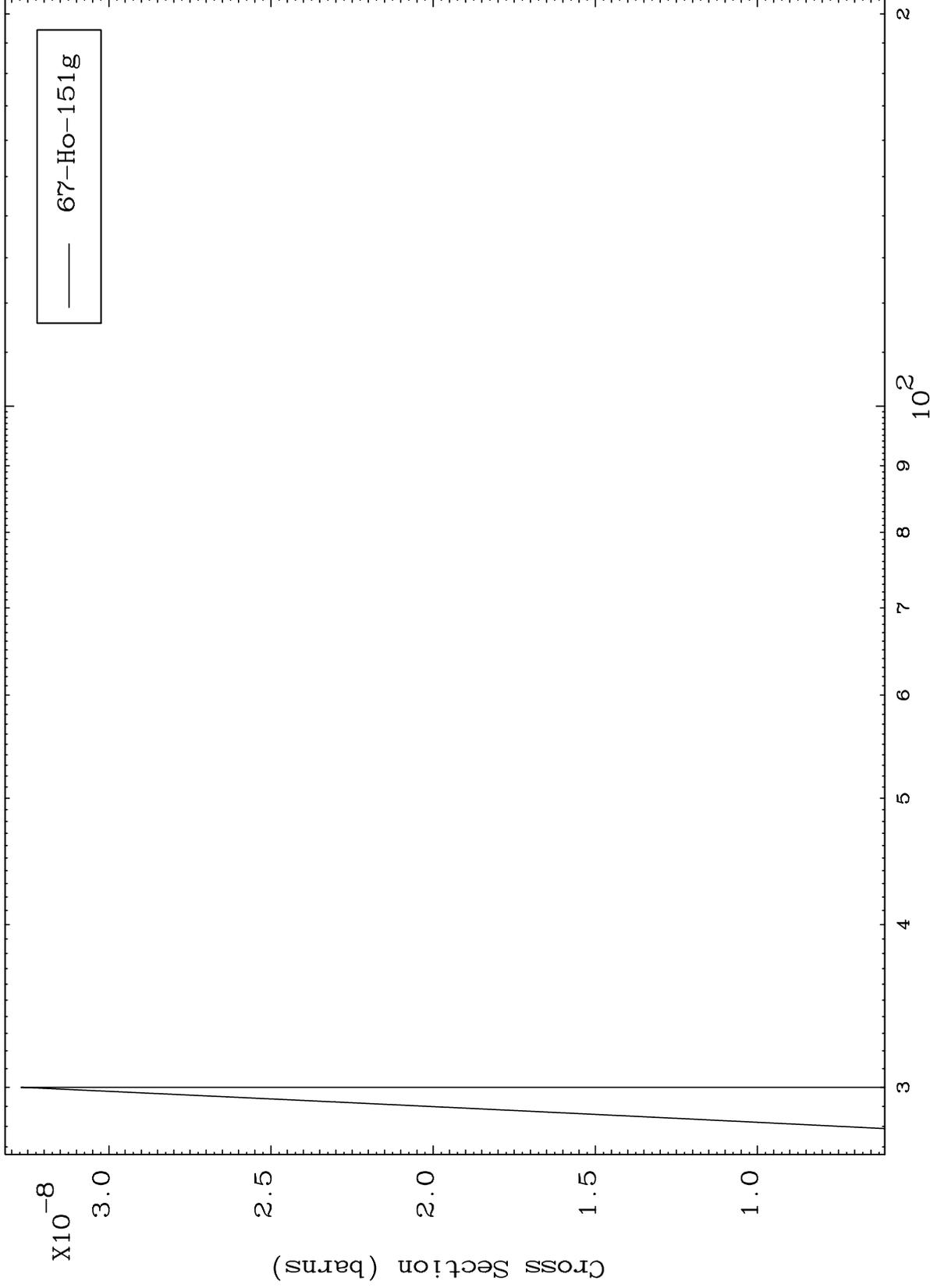
Incident Energy (MeV)

66-Dy-150

MAT 6607

66-Dy-150

$(\alpha, 2n)$  p  
Radionuclide Production Cross Section

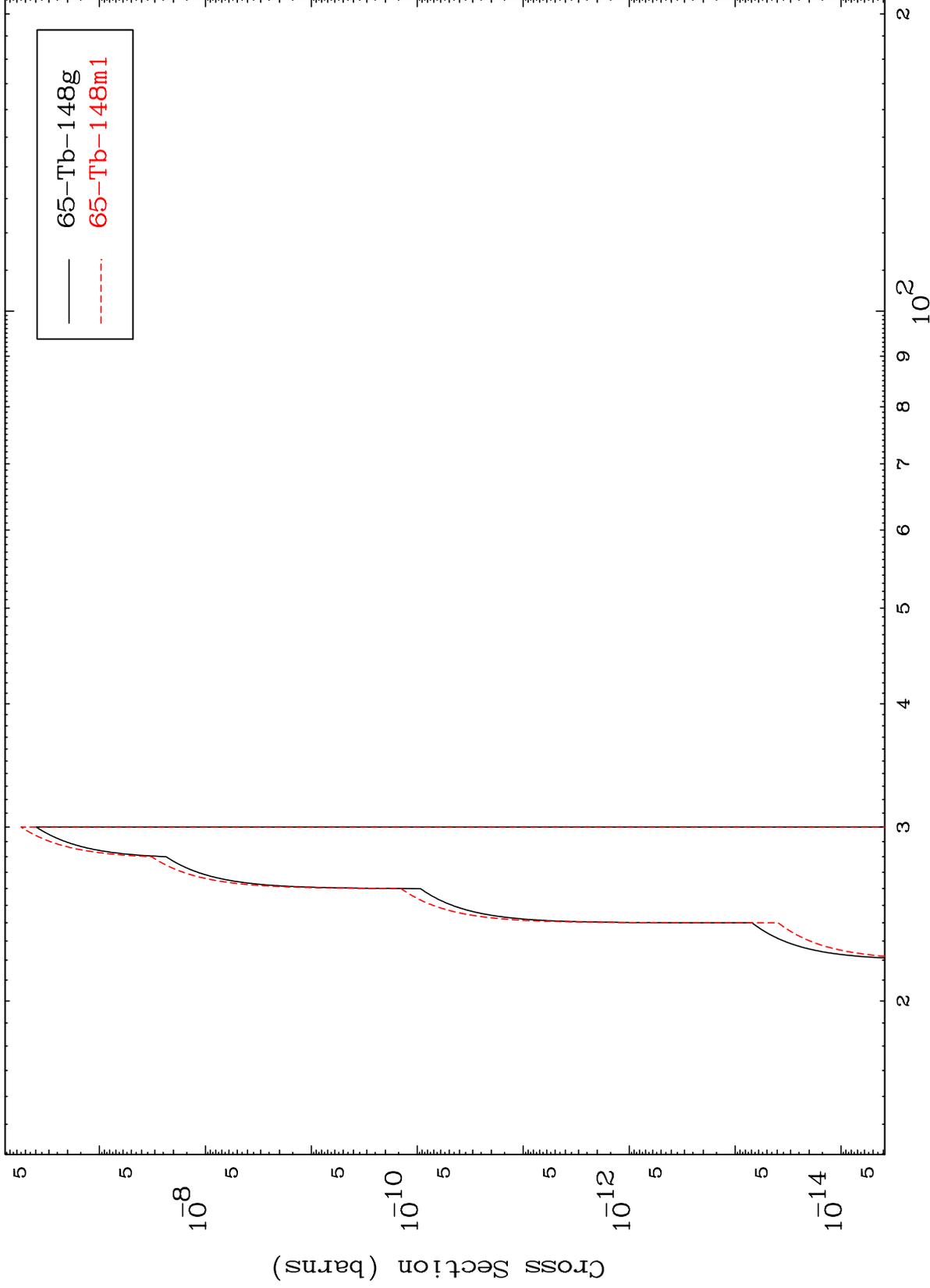


15

Incident Energy (MeV)

66-Dy-150

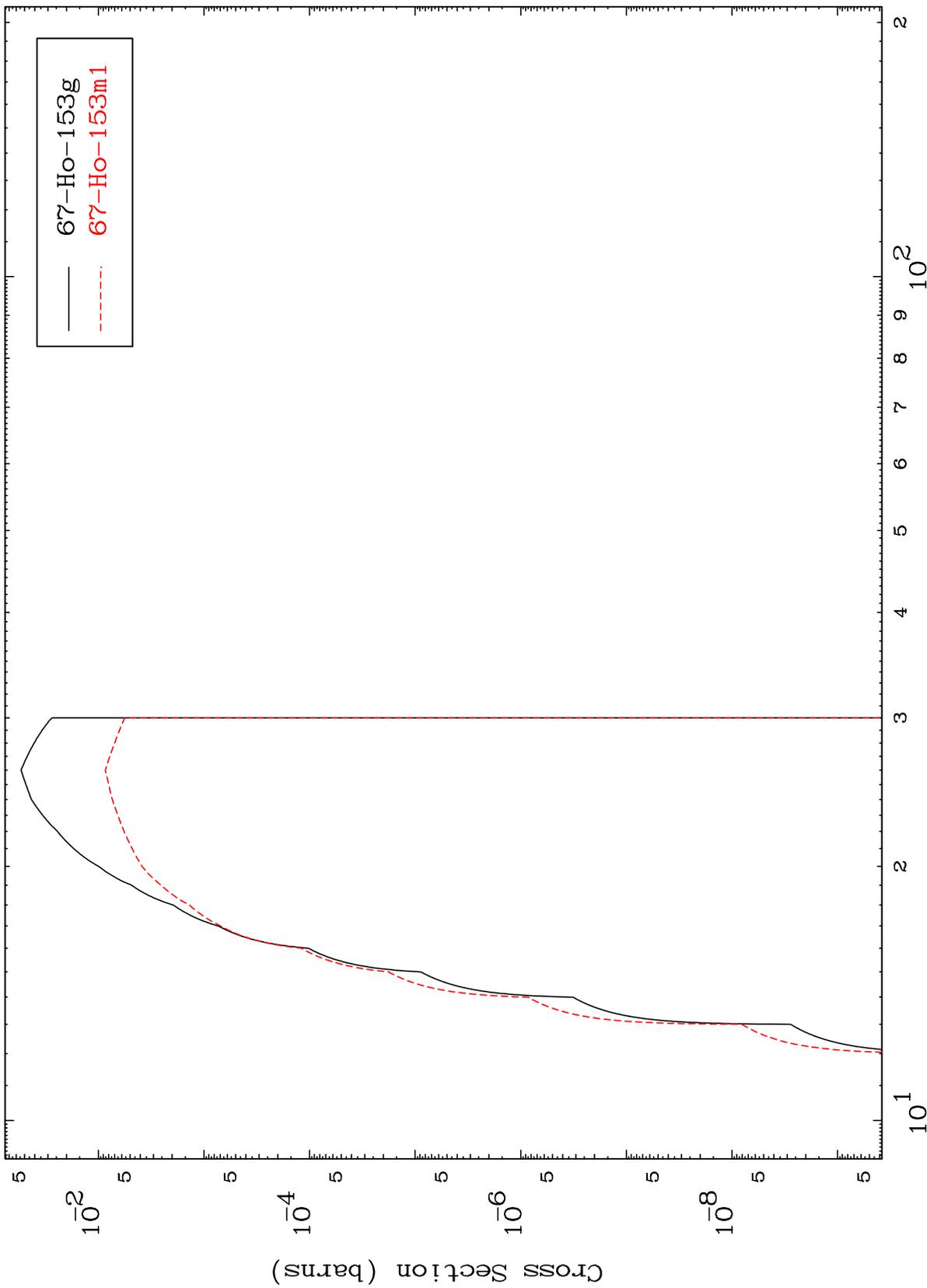
Radionuclide Production Cross Section



MAT 6607

66-Dy-150

( $\alpha, p$ )  
Radionuclide Production Cross Section

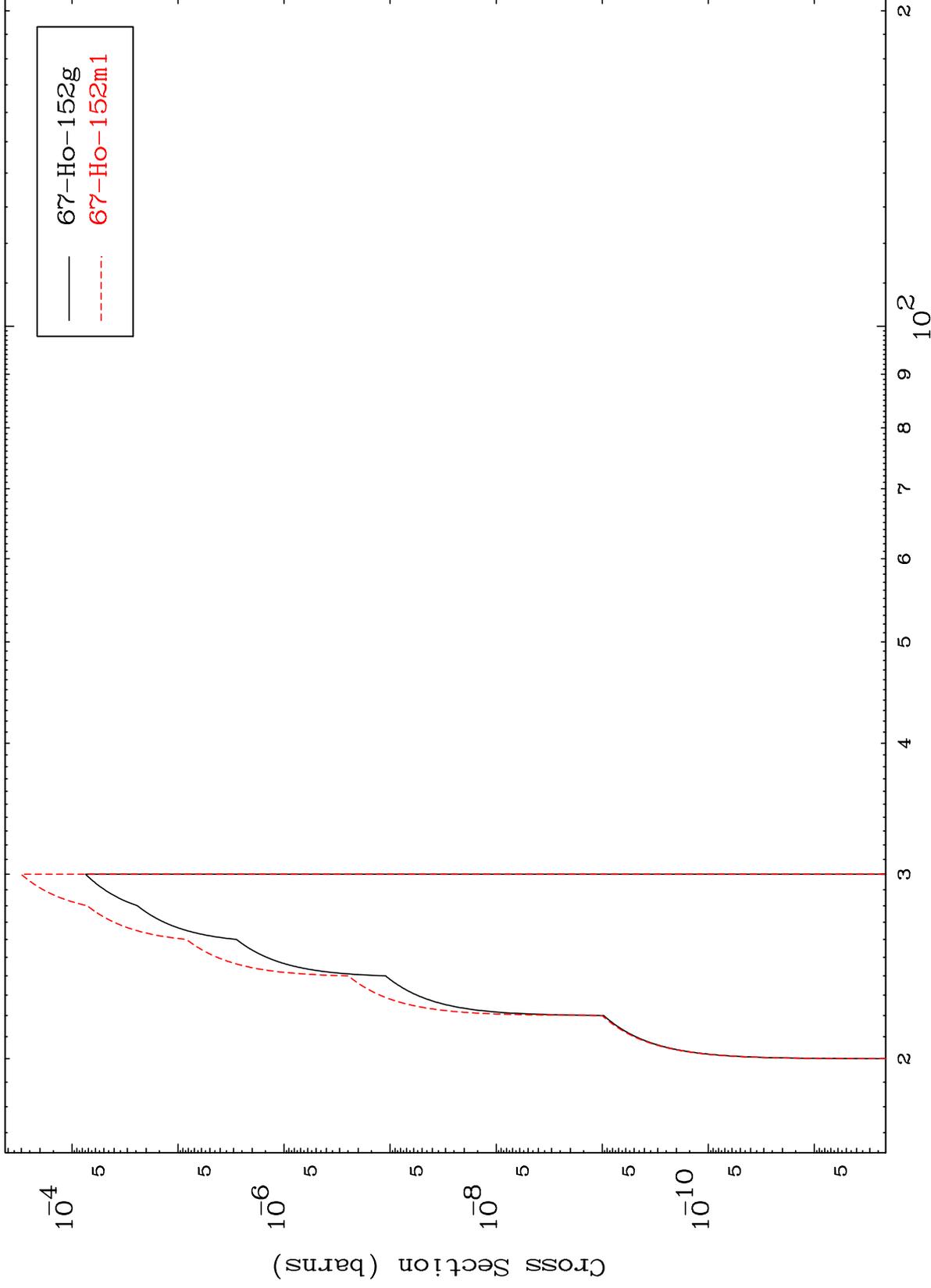


66-Dy-150

Incident Energy (MeV)

17

Radionuclide Production Cross Section

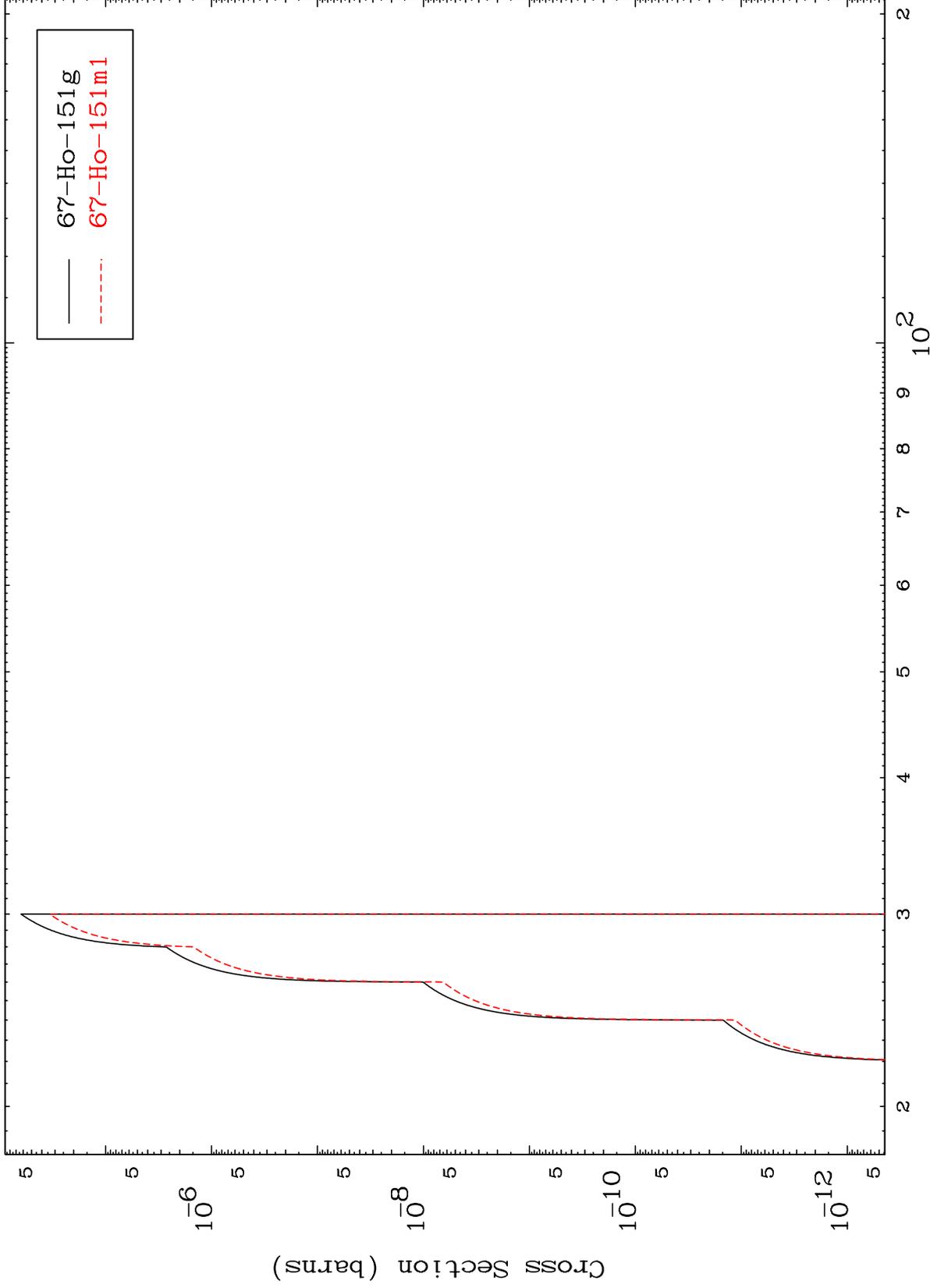


MAT 6607

( $\alpha, t$ )

66-Dy-150

Radionuclide Production Cross Section



19

Incident Energy (MeV)

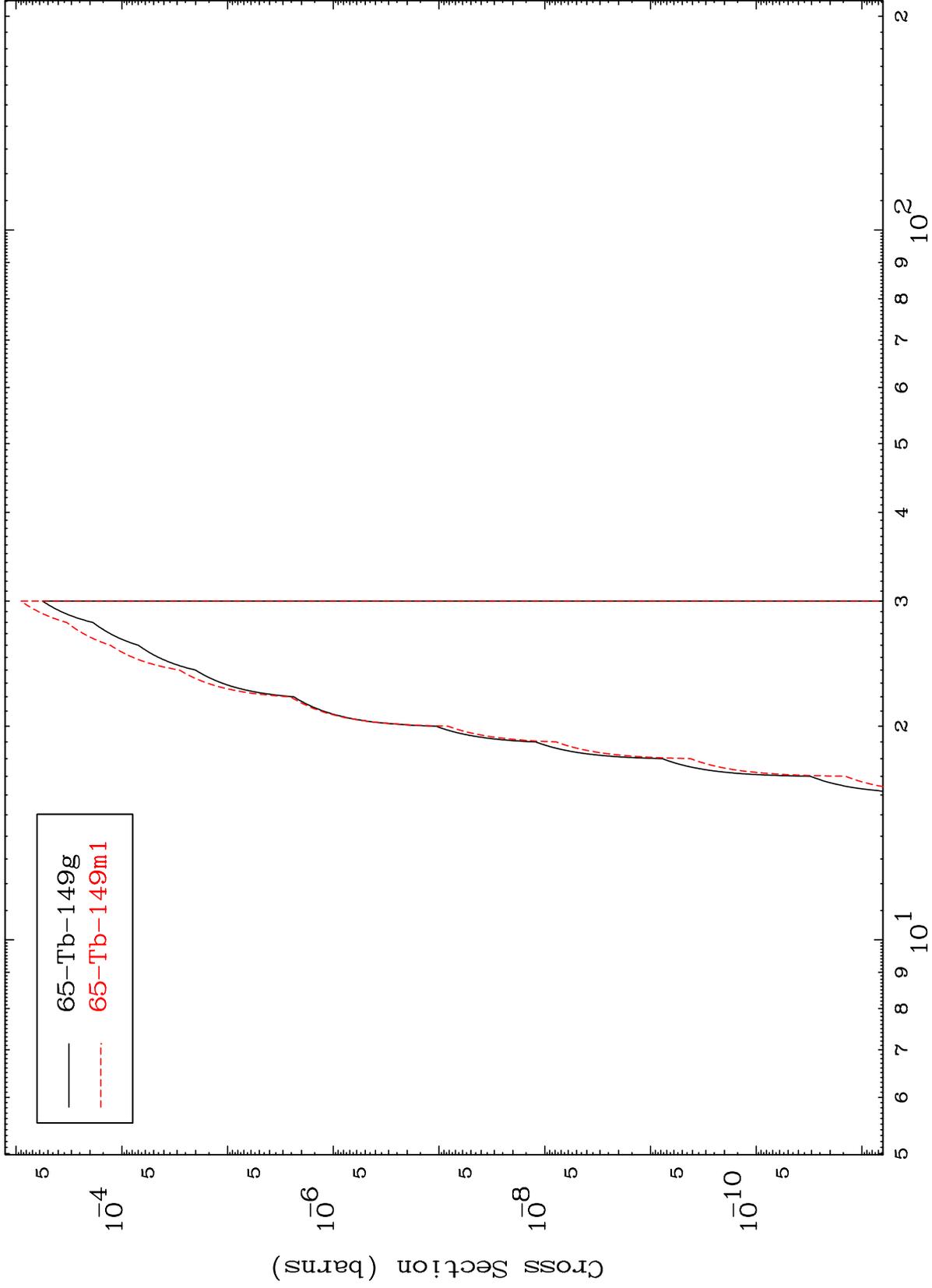
66-Dy-150

MAT 6607

( $\alpha, p$ )  $\alpha$

66-Dy-150

Radionuclide Production Cross Section



20

Incident Energy (MeV)

66-Dy-150

MAT 6607

( $\alpha, d$ )  $\alpha$

66-Dy-150

Radionuclide Production Cross Section

