

Program EVALPLOT  
(Version 2018-1)

by

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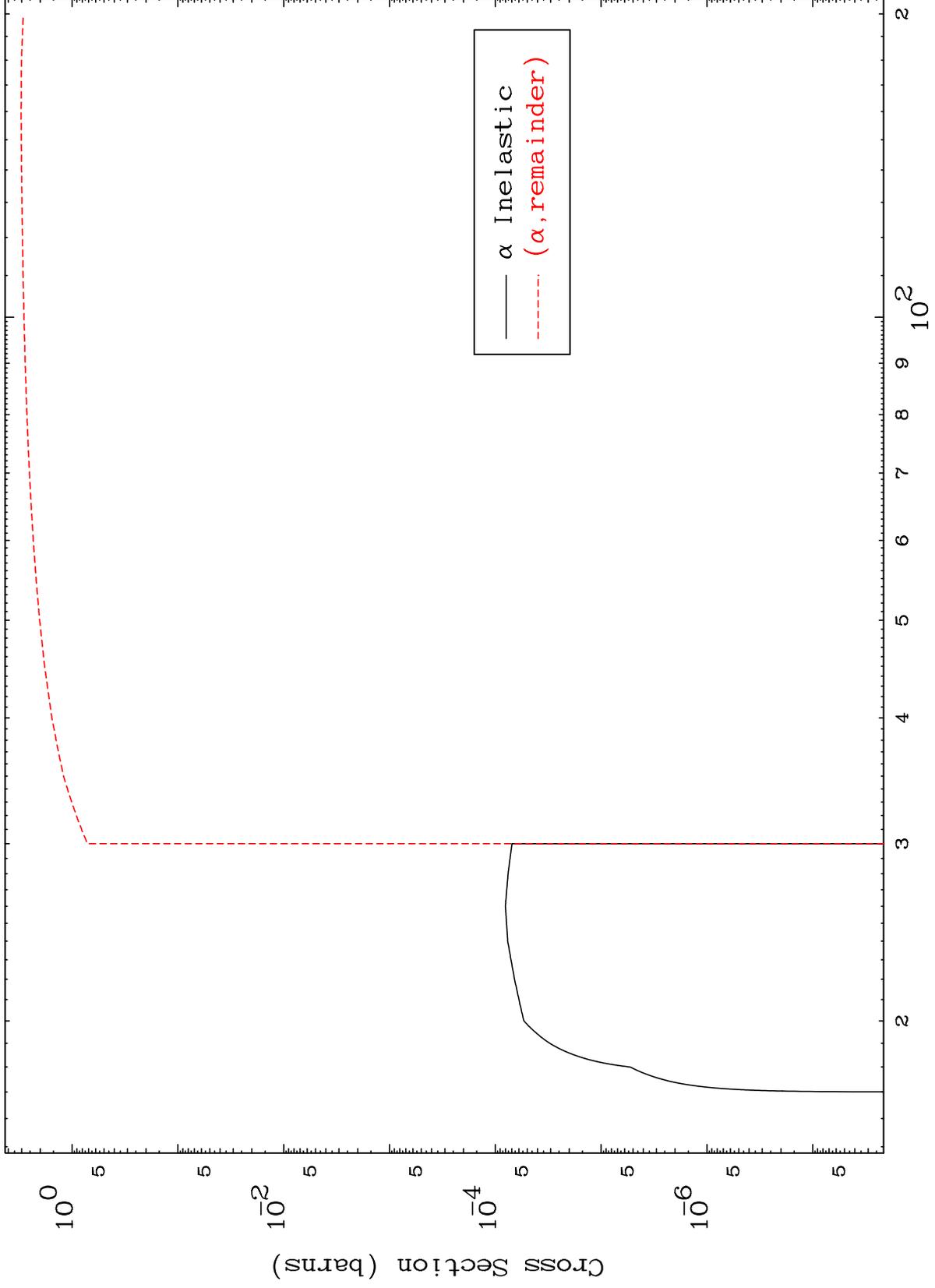
Web: [redcullen1.net/HOMEPAGE.NEW](http://redcullen1.net/HOMEPAGE.NEW)

Press Mouse Button to Start

MAT 9933

$\alpha$  Major  
0 Kelvin Cross Sections

100-Fm-247



1

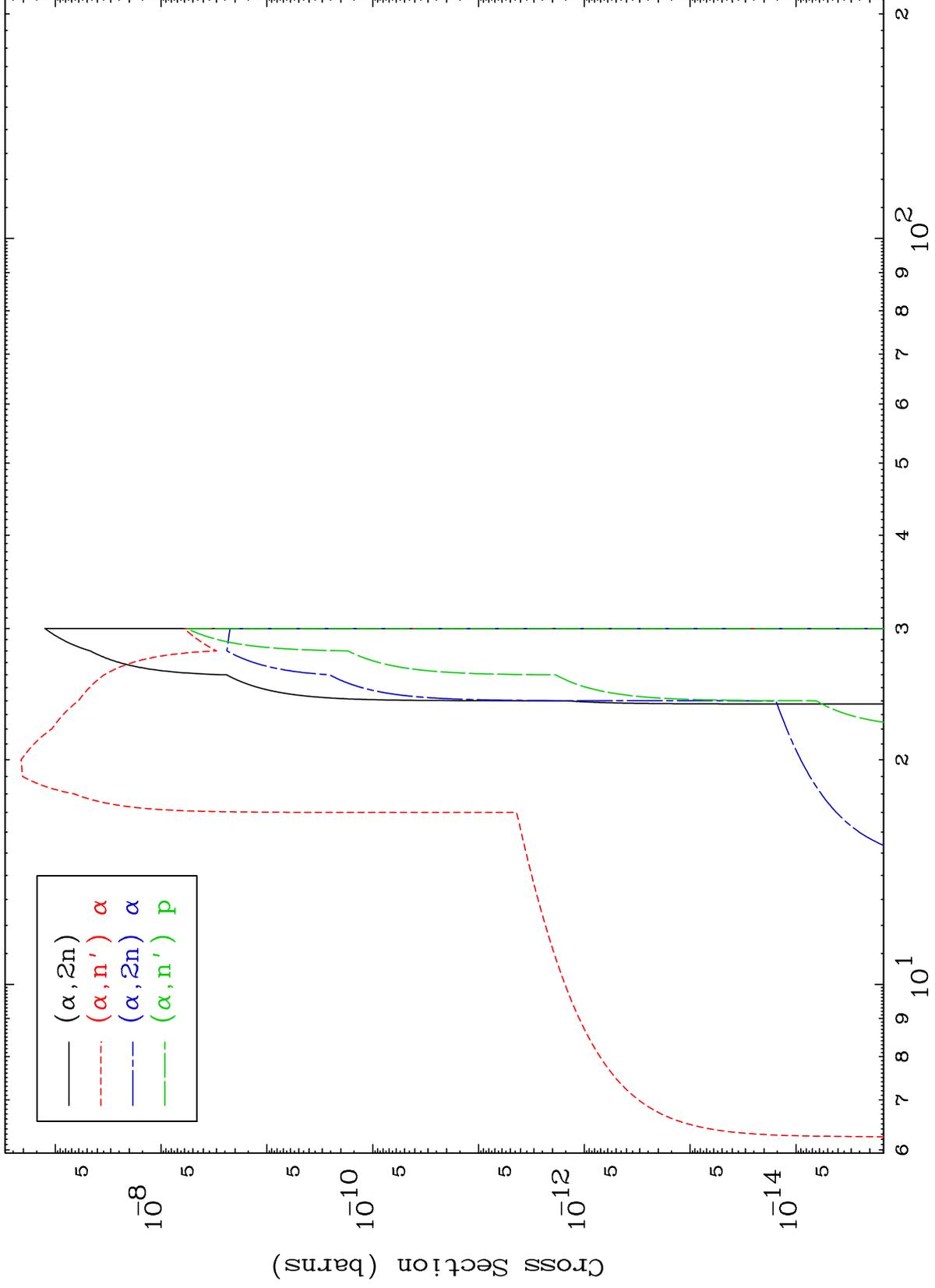
Incident Energy (MeV)

100-Fm-247

MAT 9933

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

100-Fm-247



2

Incident Energy (MeV)

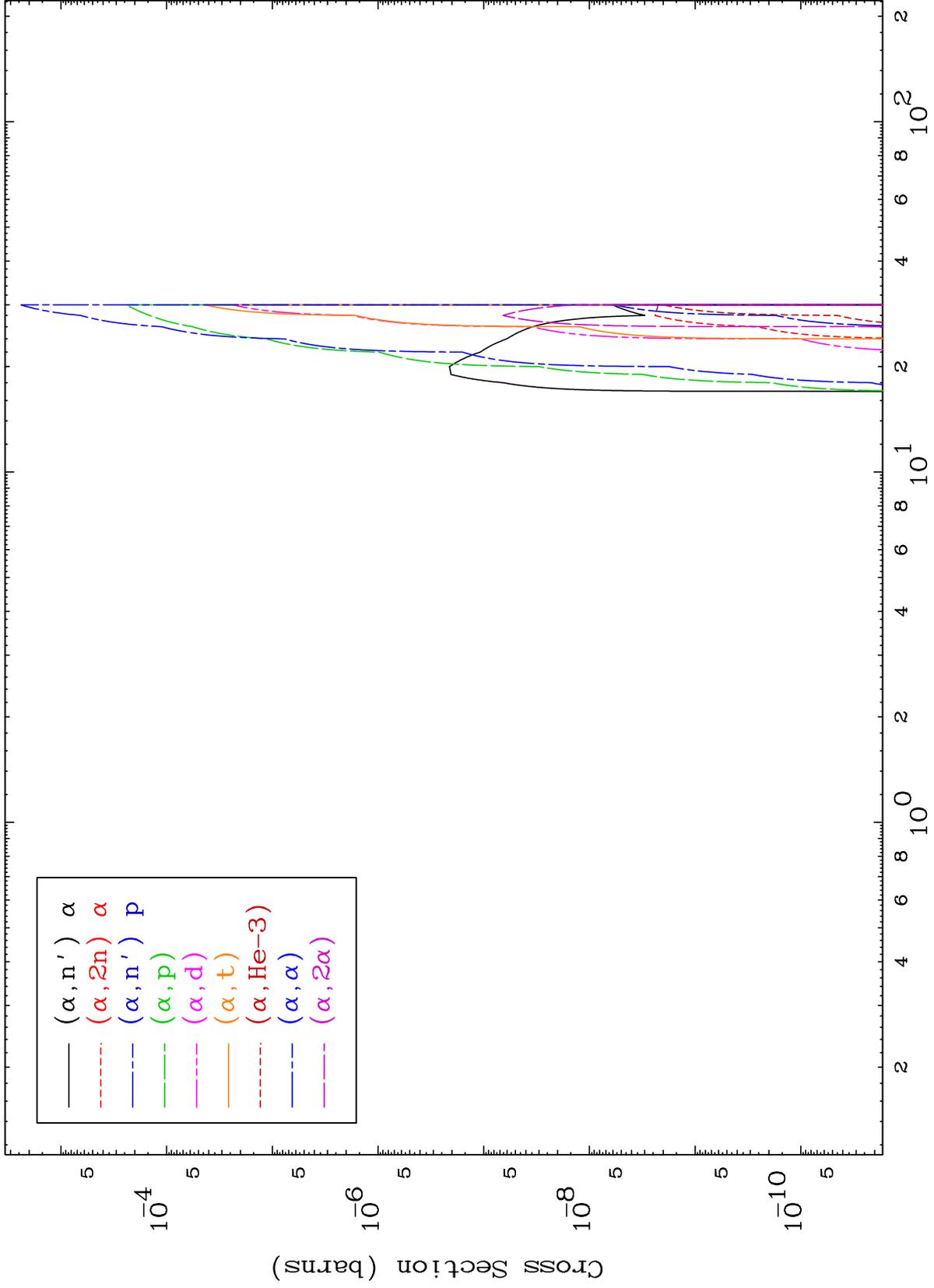
100-Fm-247

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$\alpha$  Charged Particle

100-Fm-247

0 Kelvin Cross Sections

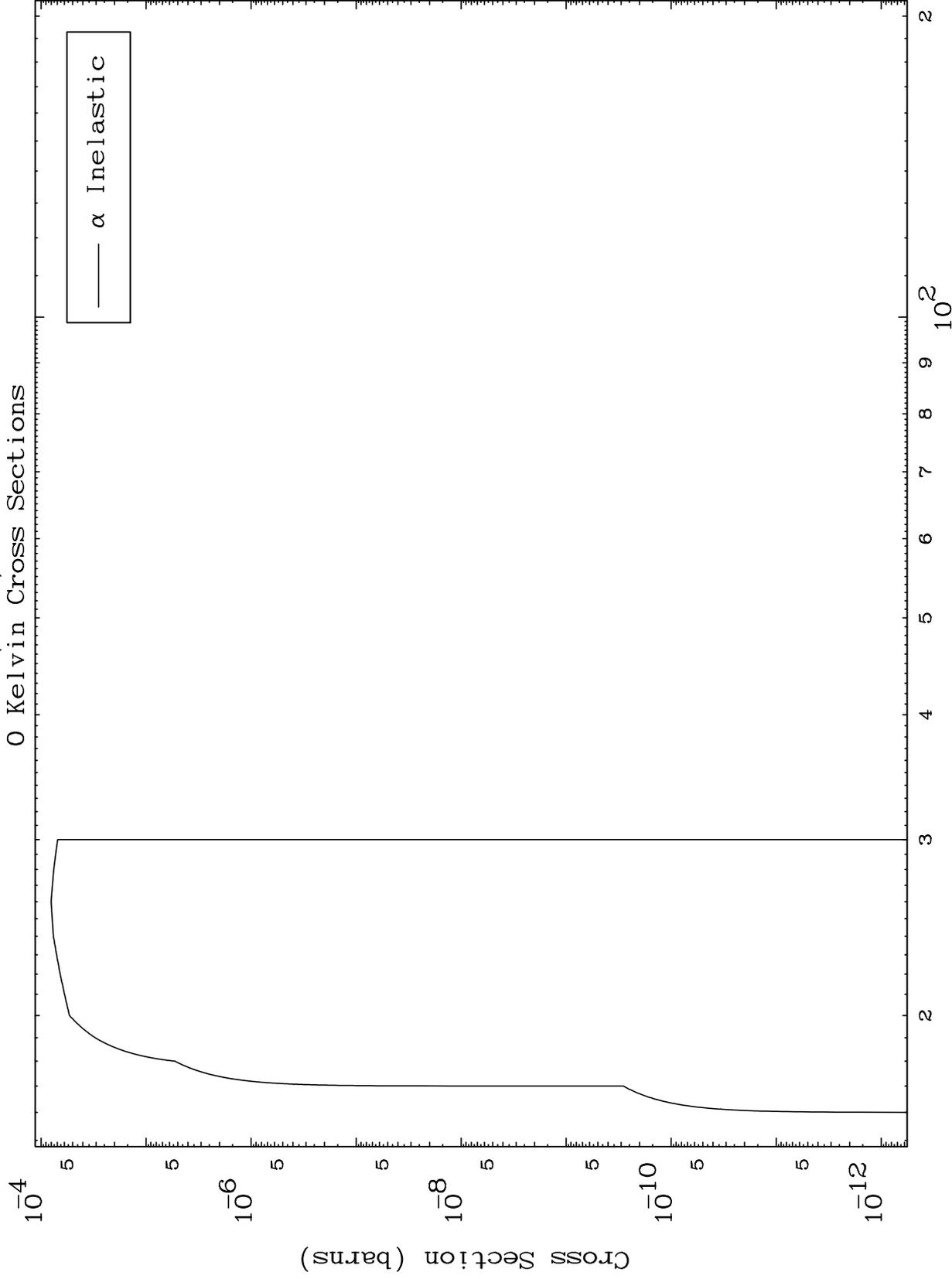


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( $\alpha, n'$ ) Level

100-Fm-247

0 Kelvin Cross Sections



Incident Energy (MeV)

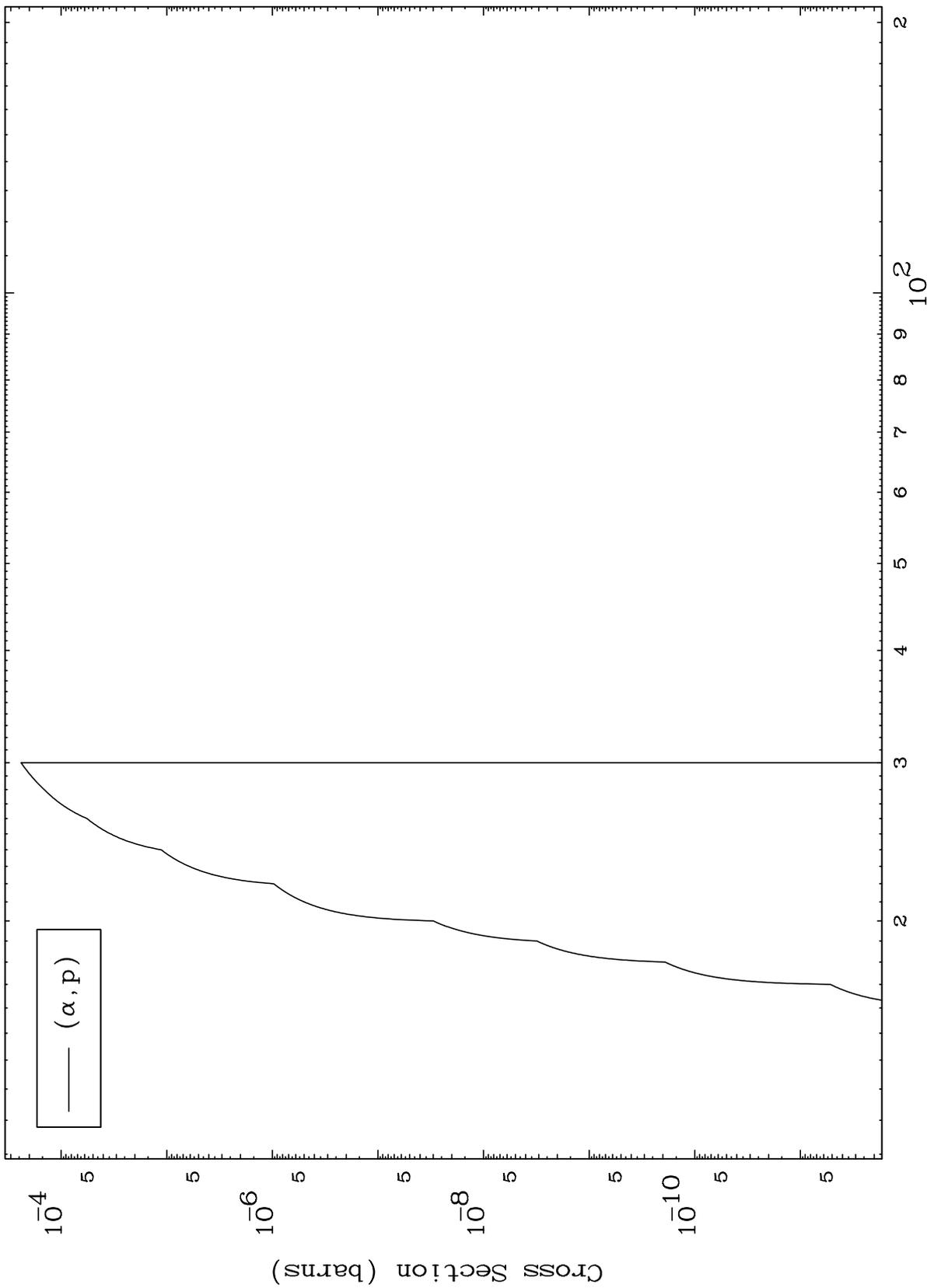
100-Fm-247

4

MAT 9933

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

100-Fm-247



5

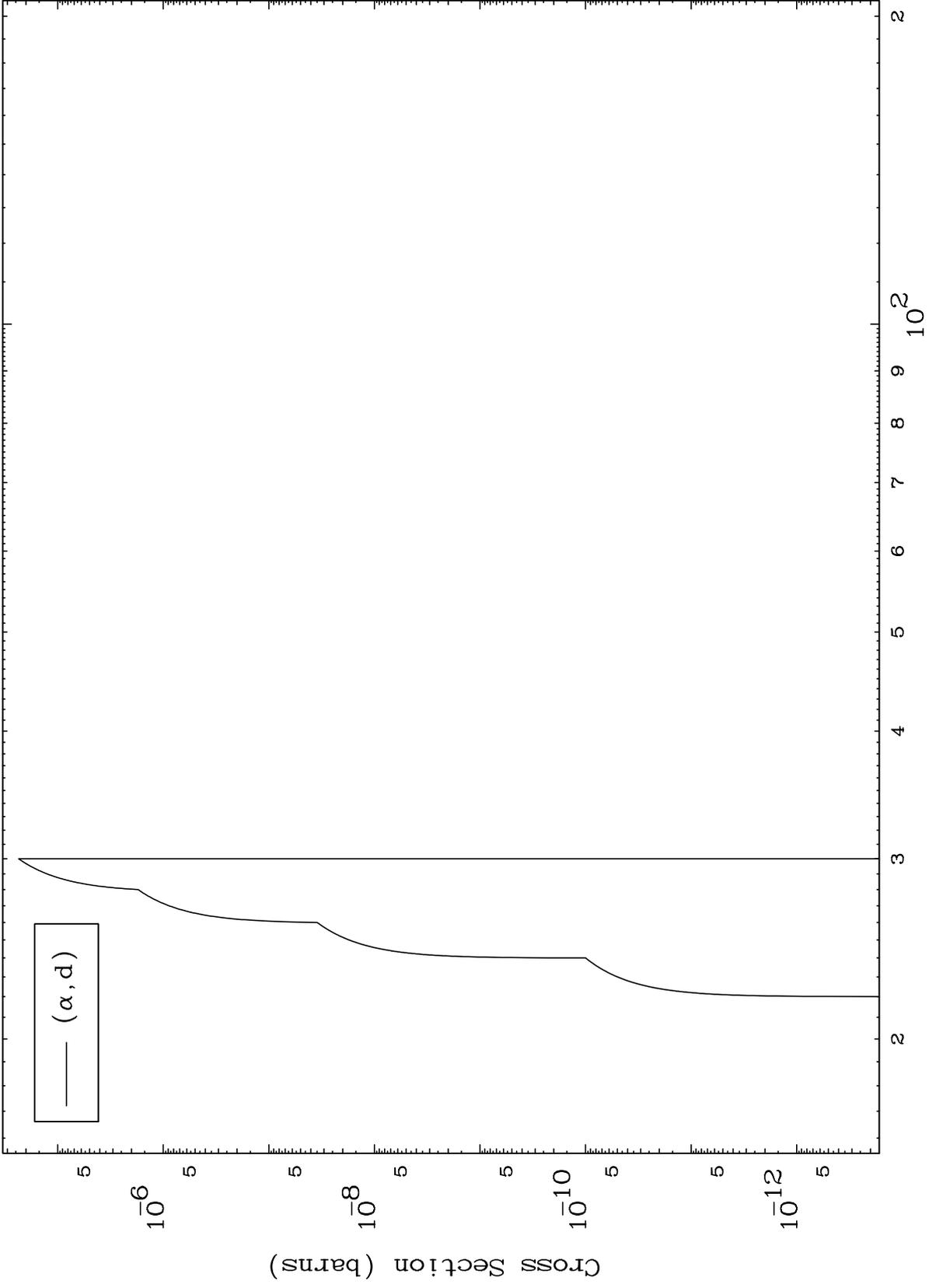
Incident Energy (MeV)

100-Fm-247

MAT 9933

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

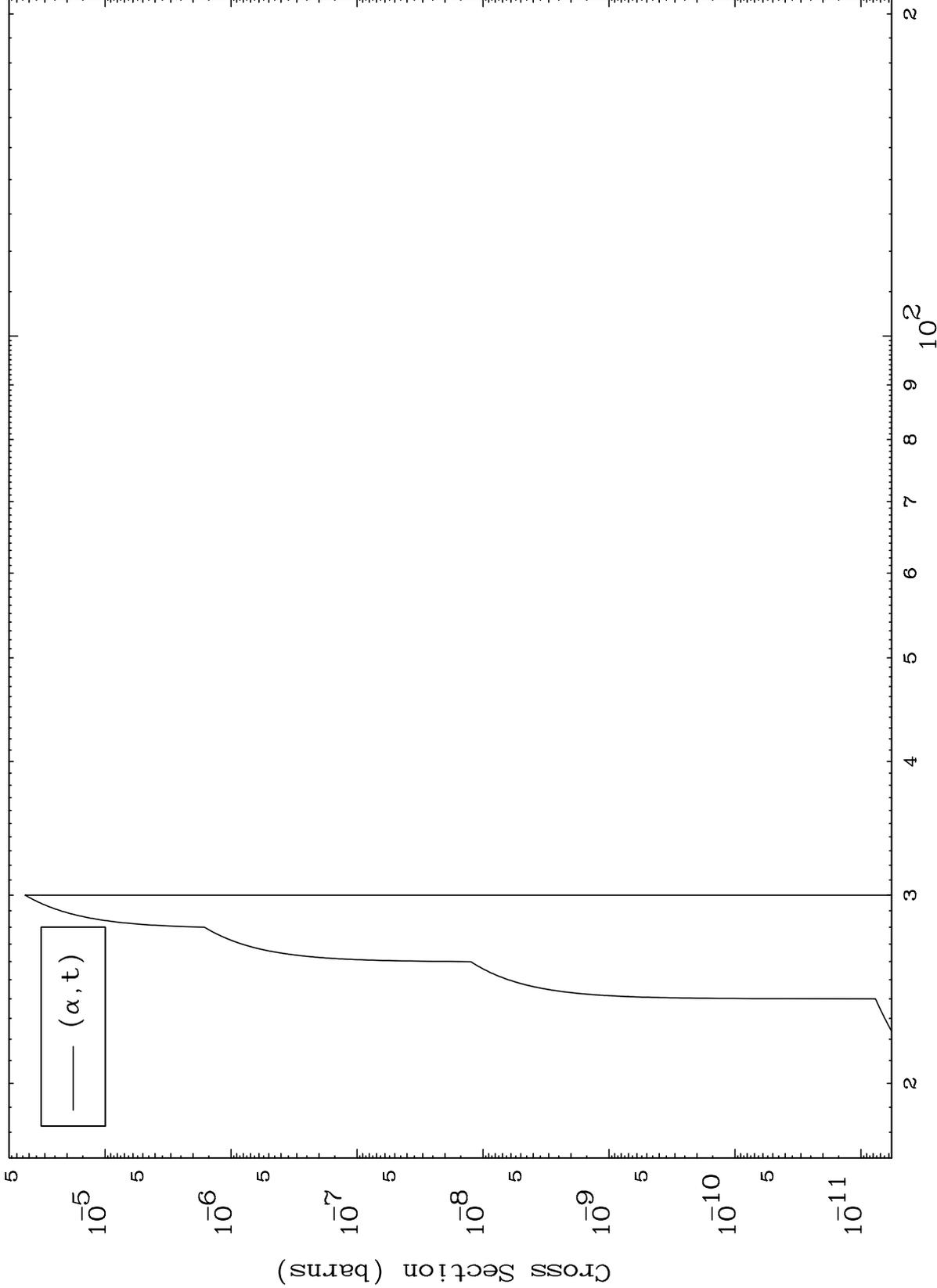
100-Fm-247



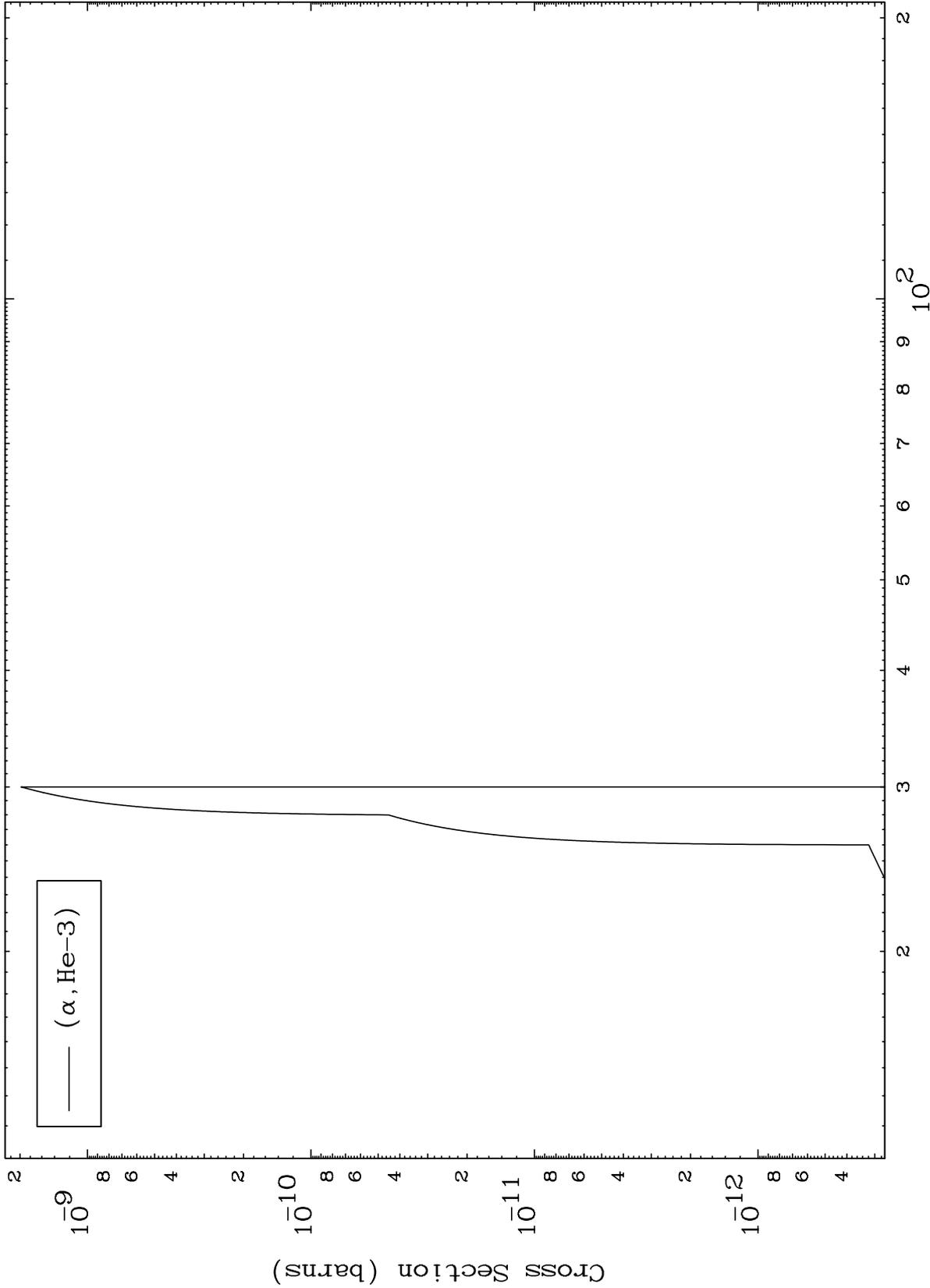
6

Incident Energy (MeV)

100-Fm-247



( $\alpha$ , He3) Levels  
0 Kelvin Cross Sections

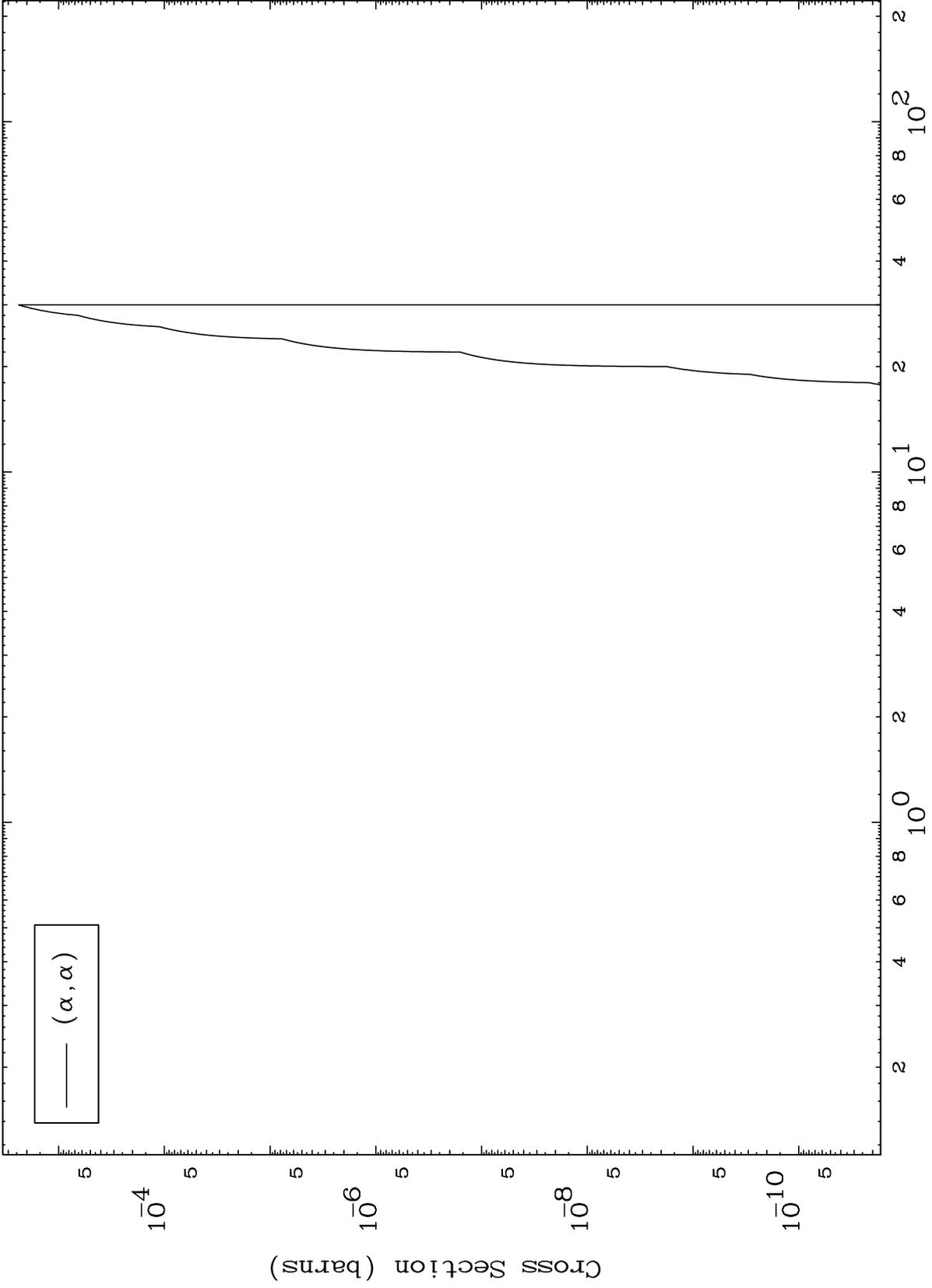


MAT 9933

( $\alpha, \alpha$ ) Levels

100-Fm-247

0 Kelvin Cross Sections

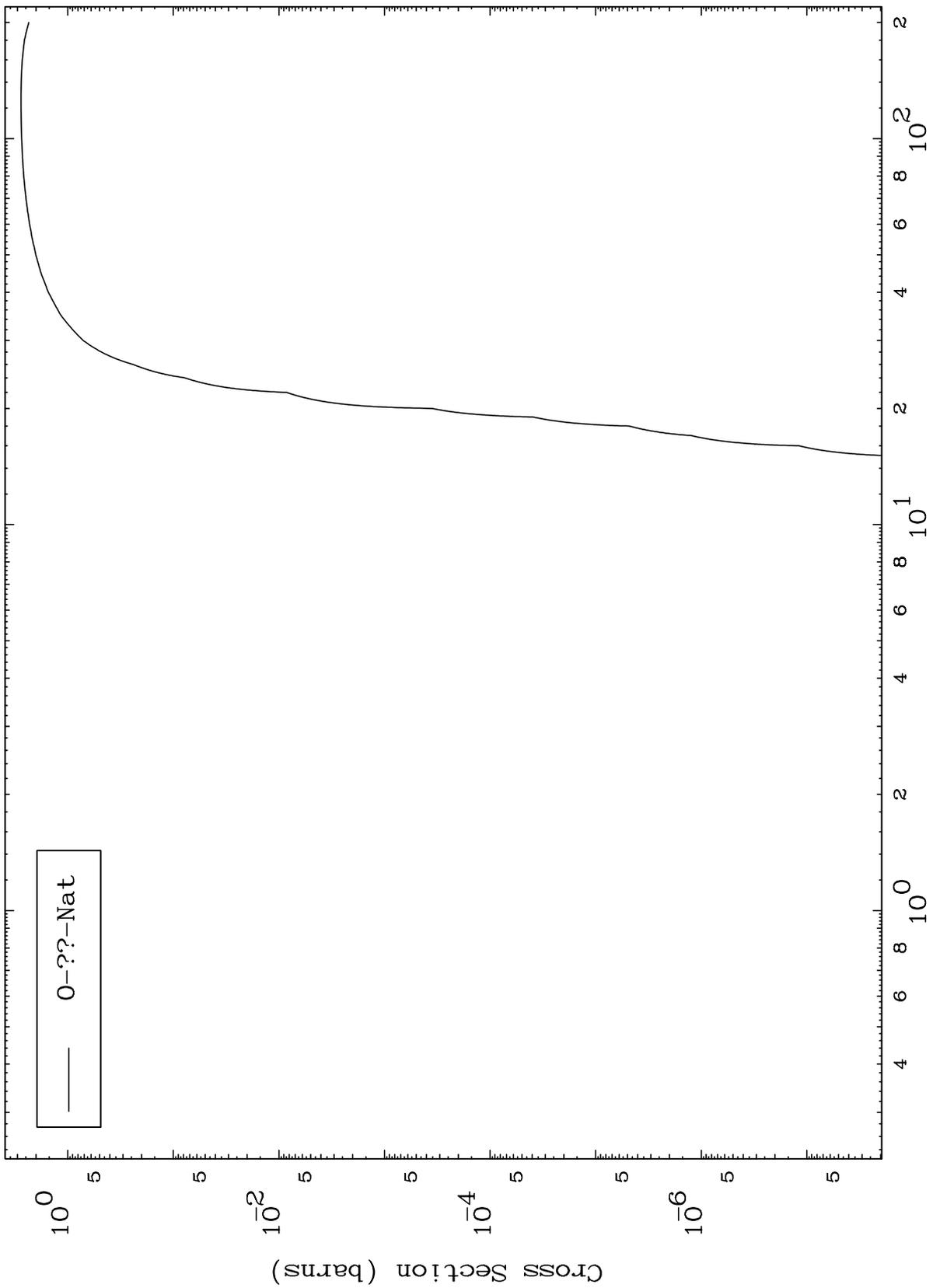


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100-Fm-247

$\alpha$  Fission

Radionuclide Production Cross Section

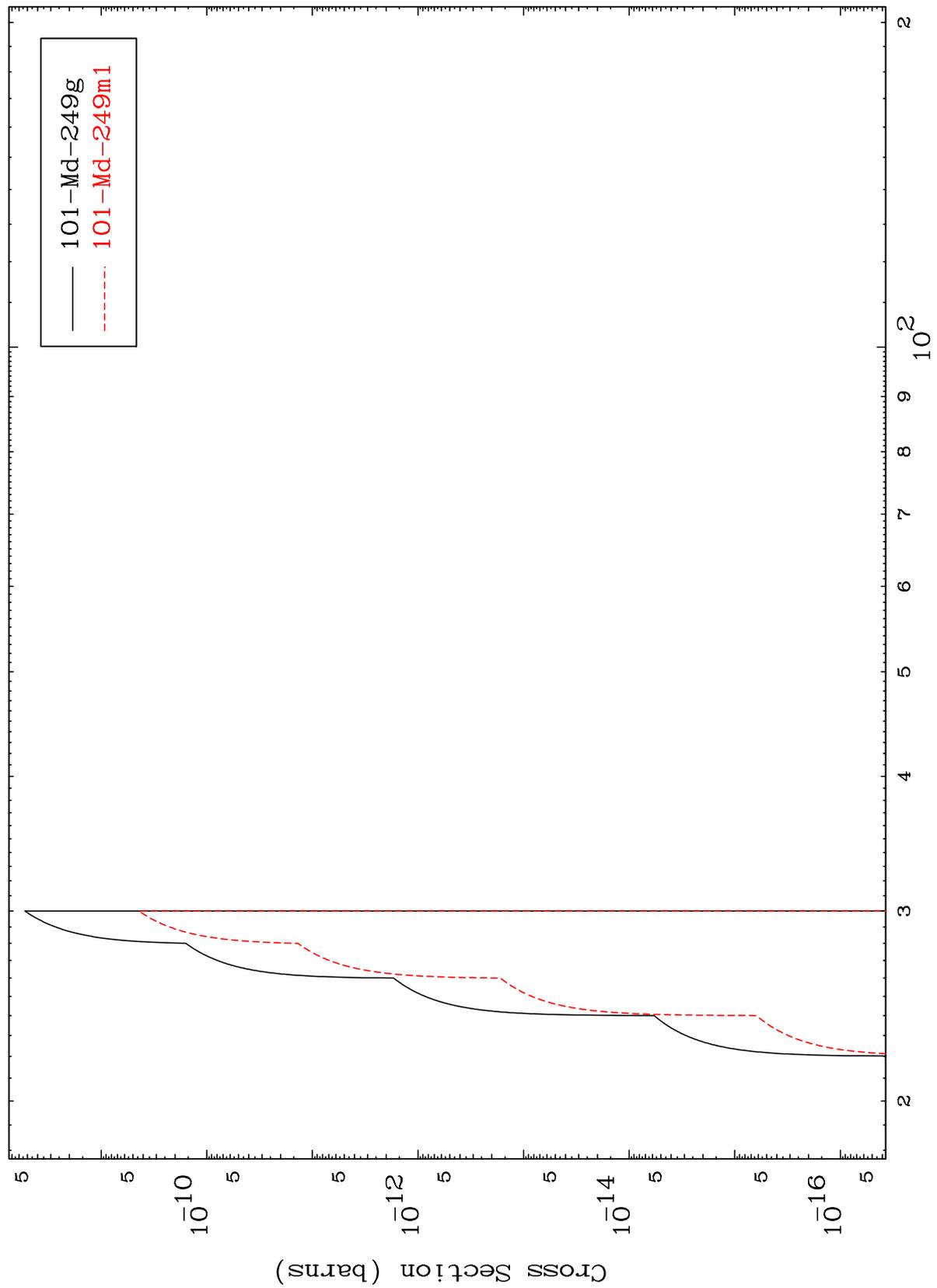


10

100-Fm-247

Incident Energy (MeV)

Radionuclide Production Cross Section

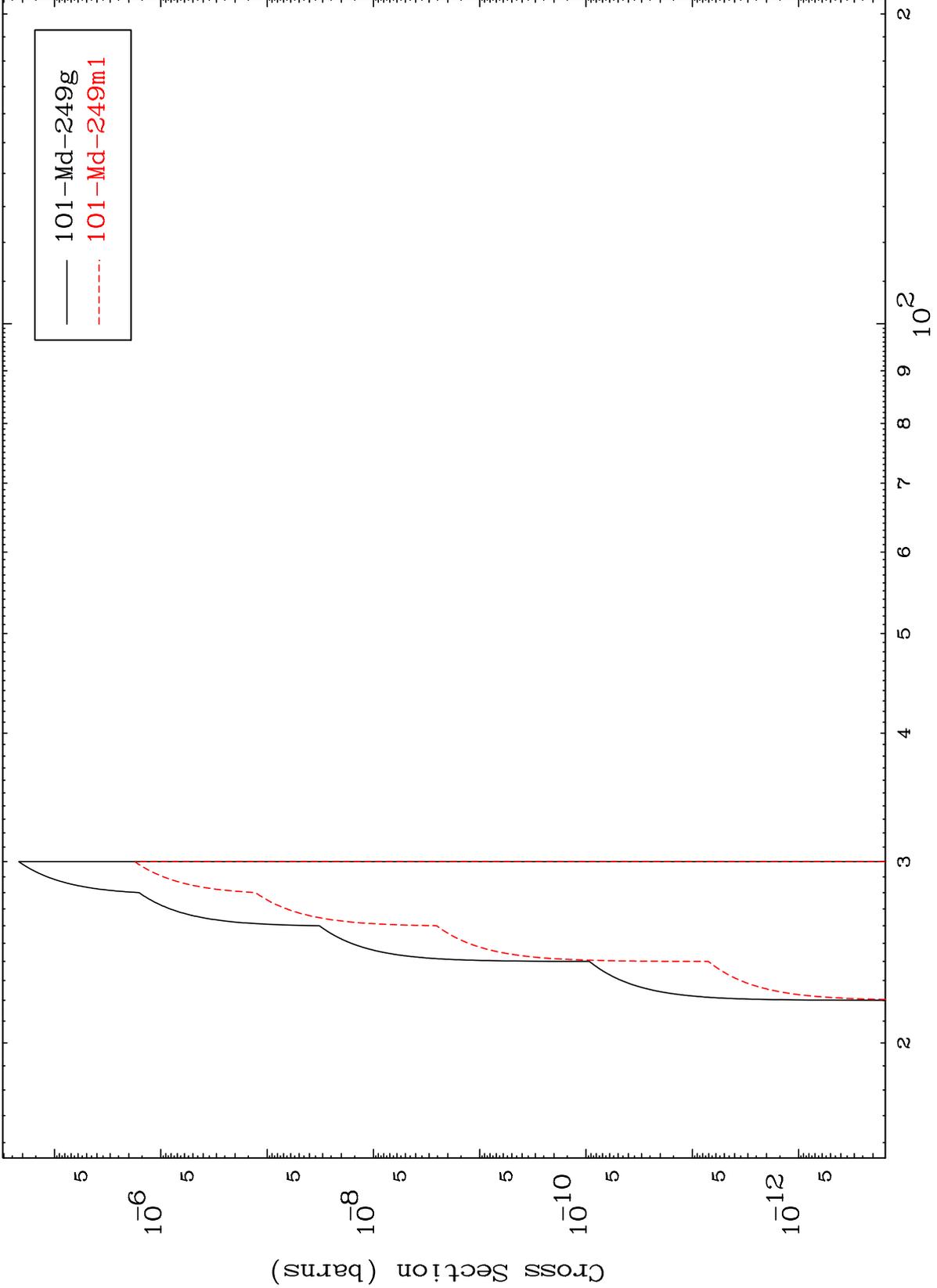


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( $\alpha, d$ )

100-Fm-247

Radionuclide Production Cross Section



12

Incident Energy (MeV)

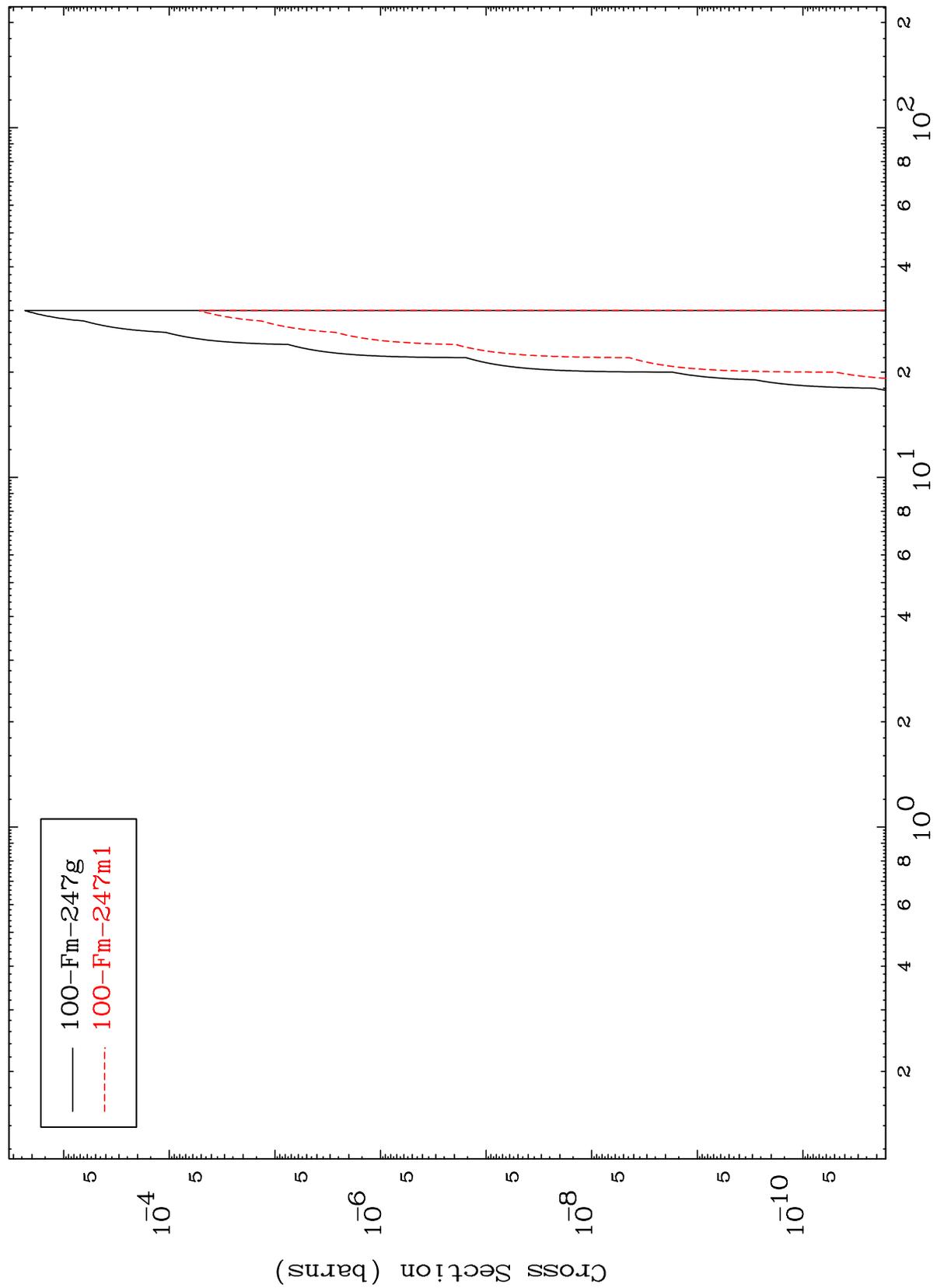
100-Fm-247

MAT 9933

100-Fm-247

( $\alpha, \alpha$ )

Radionuclide Production Cross Section



100-Fm-247

Incident Energy (MeV)