

Program EVALPLOT  
(Version 2018-1)

by

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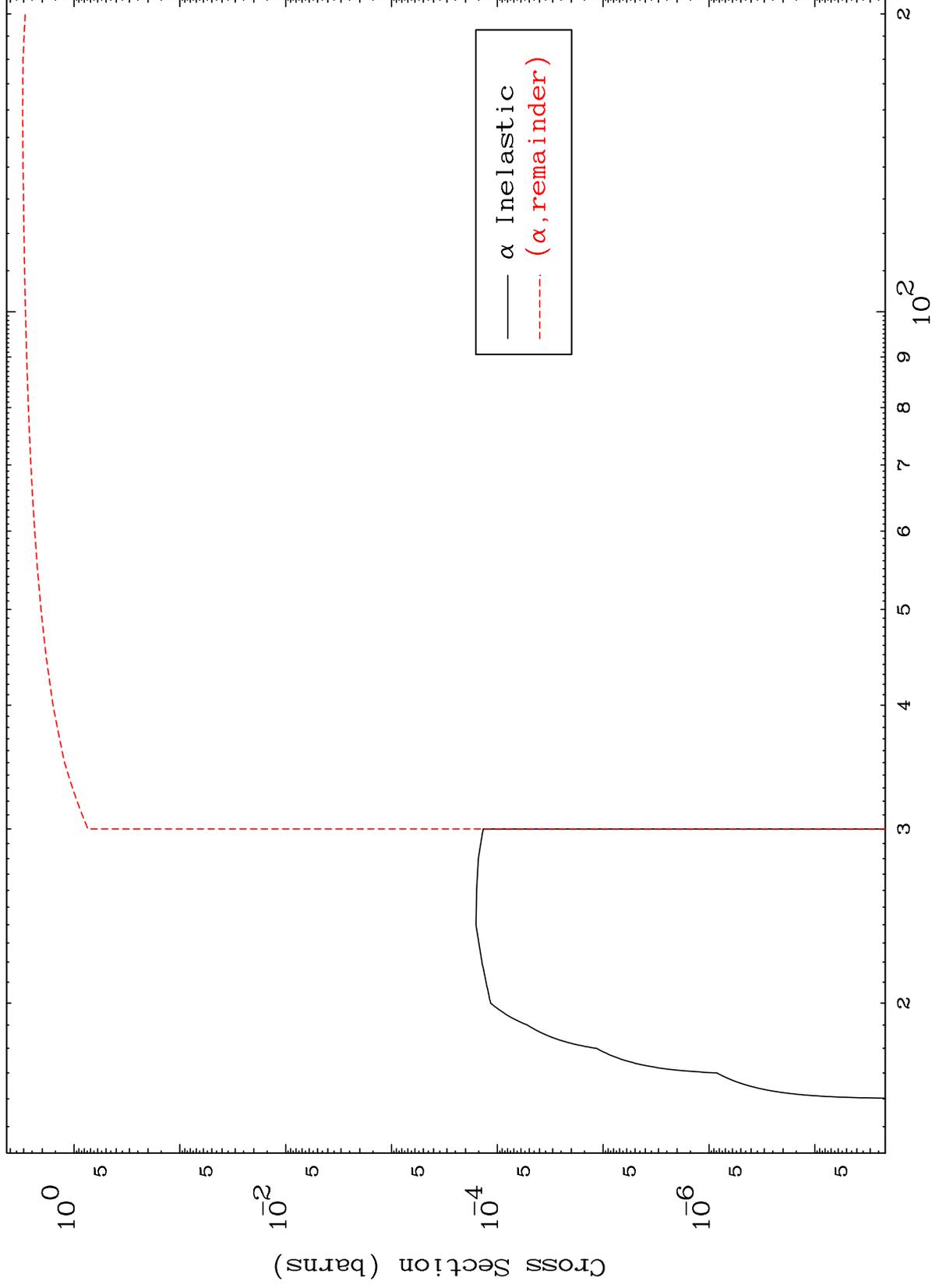
Web: [redcullen1.net/HOMEPAGE.NEW](http://redcullen1.net/HOMEPAGE.NEW)

Press Mouse Button to Start

MAT 9932

$\alpha$  Major  
0 Kelvin Cross Sections

100-Fm-251



1

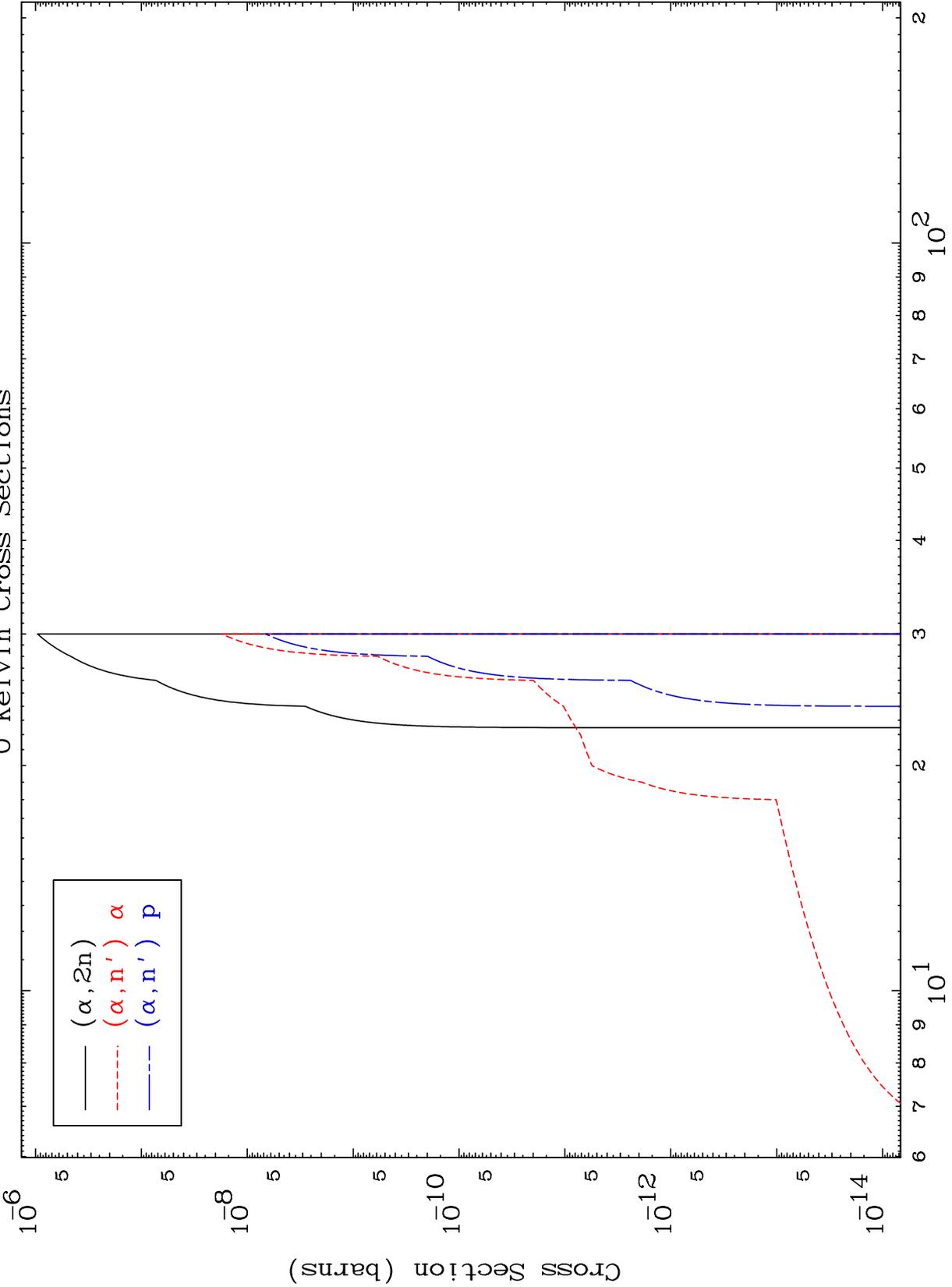
Incident Energy (MeV)

100-Fm-251

MAT 9932

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

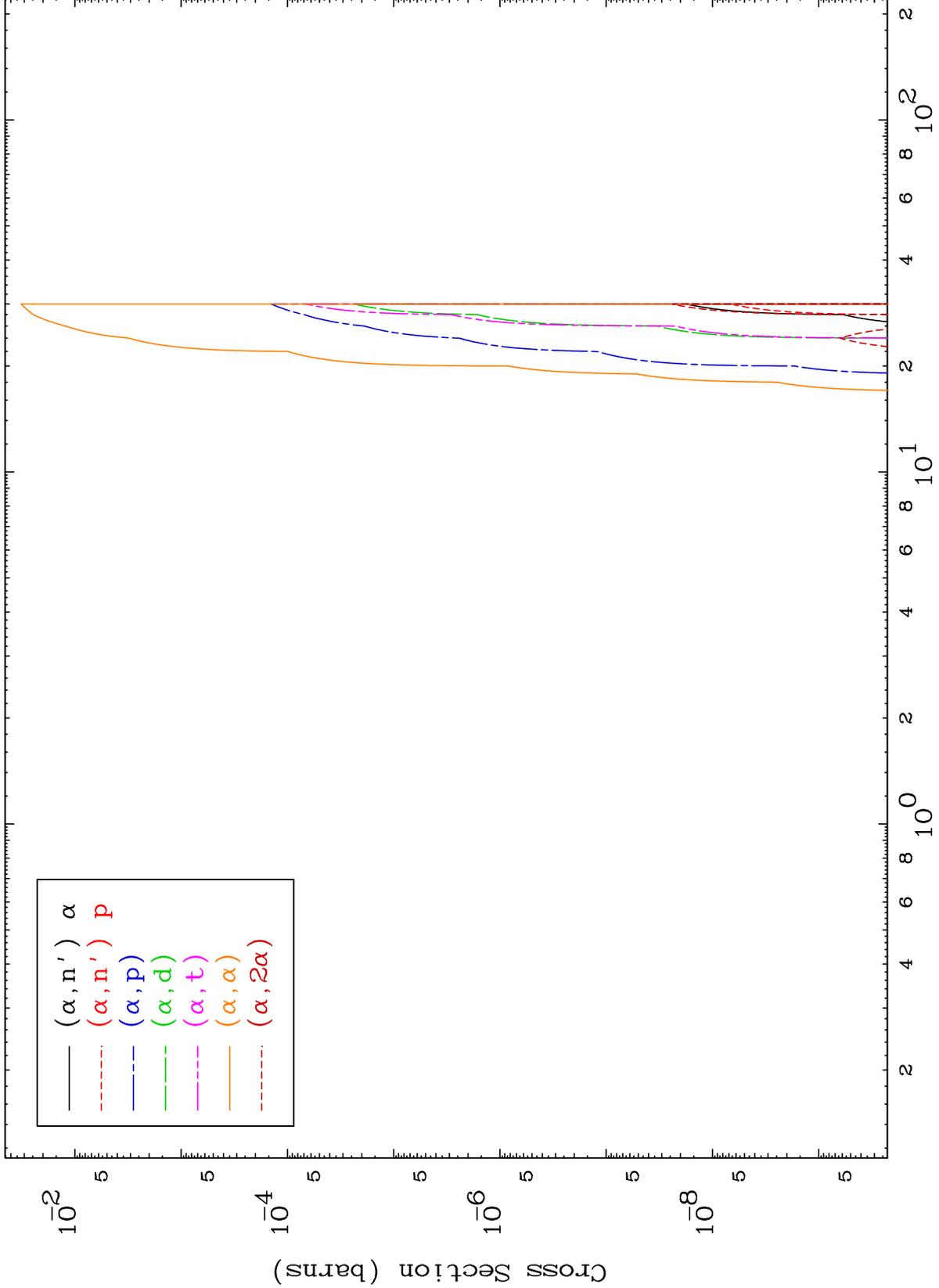
100-Fm-251



2

Incident Energy (MeV)

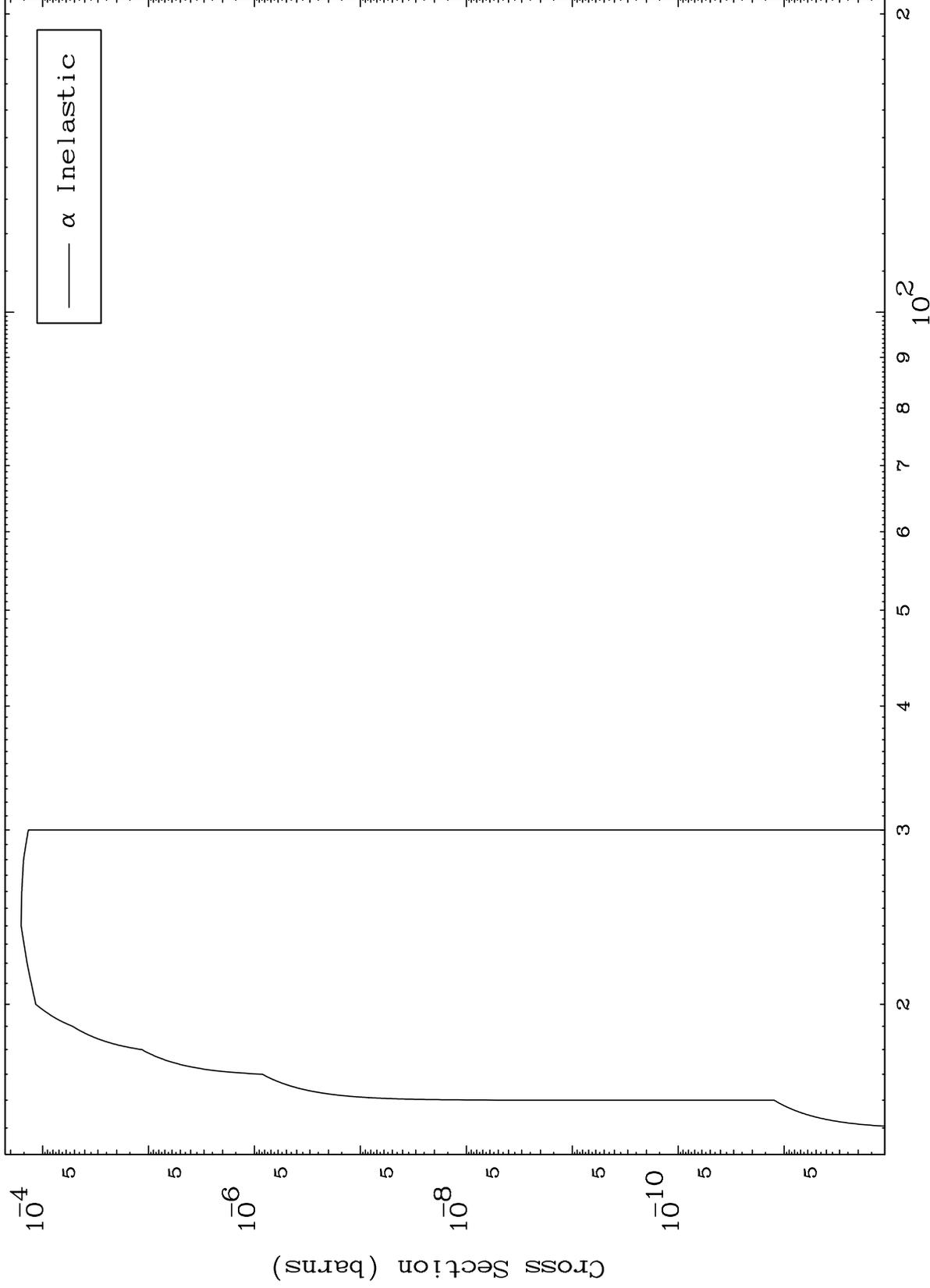
100-Fm-251



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100-Fm-251

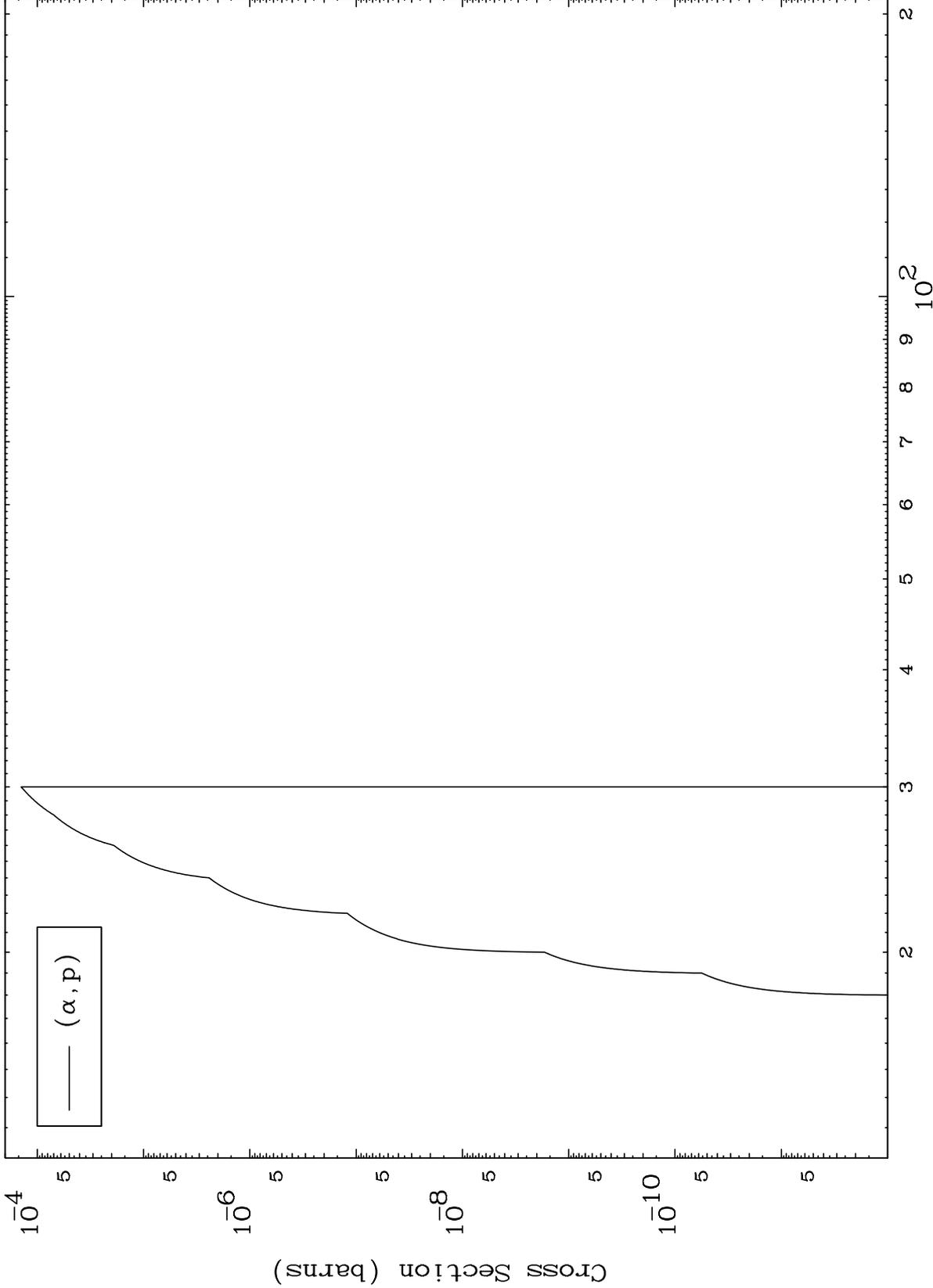
( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections



100-Fm-251

Incident Energy (MeV)

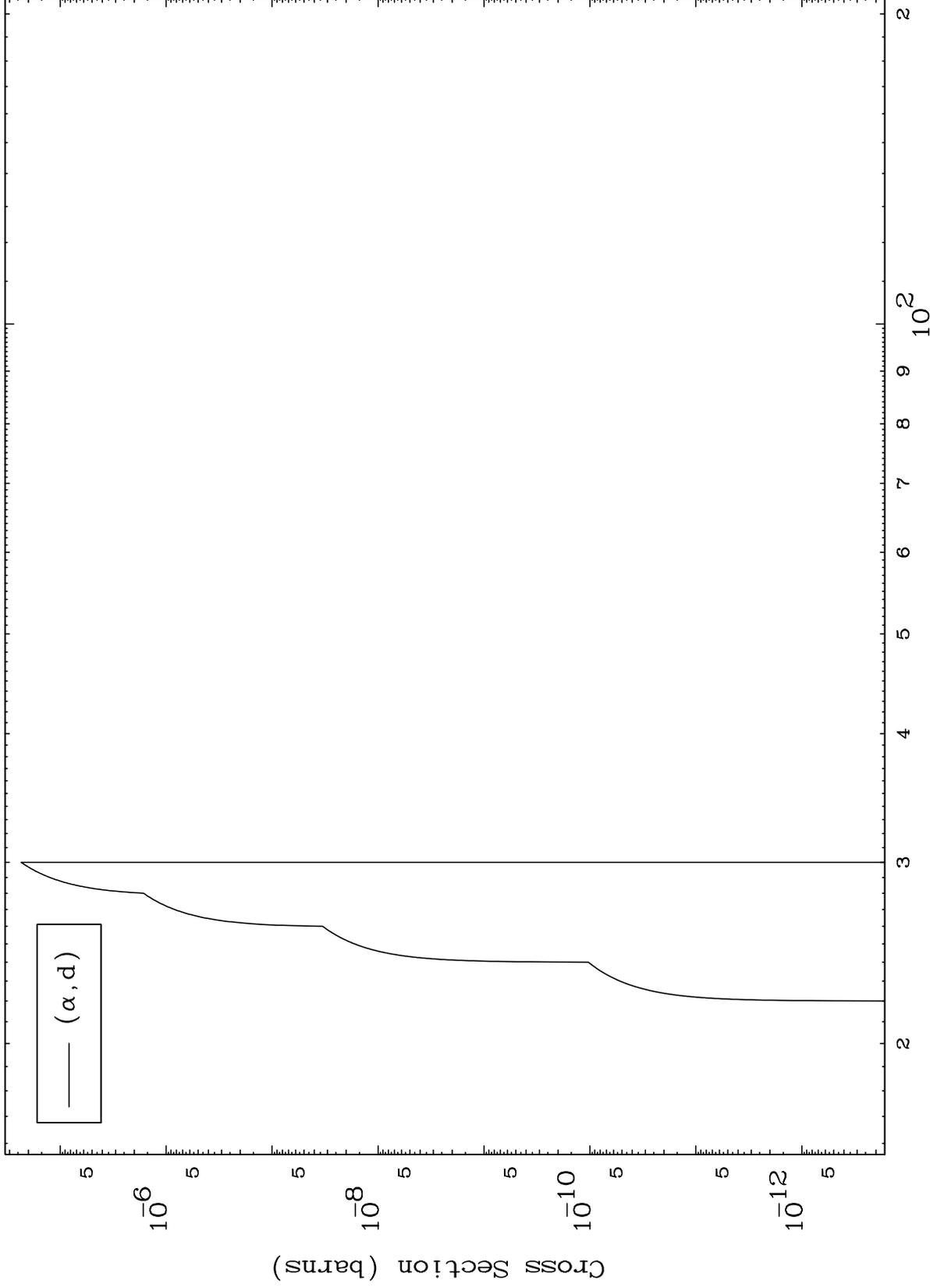
4



MAT 9932

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

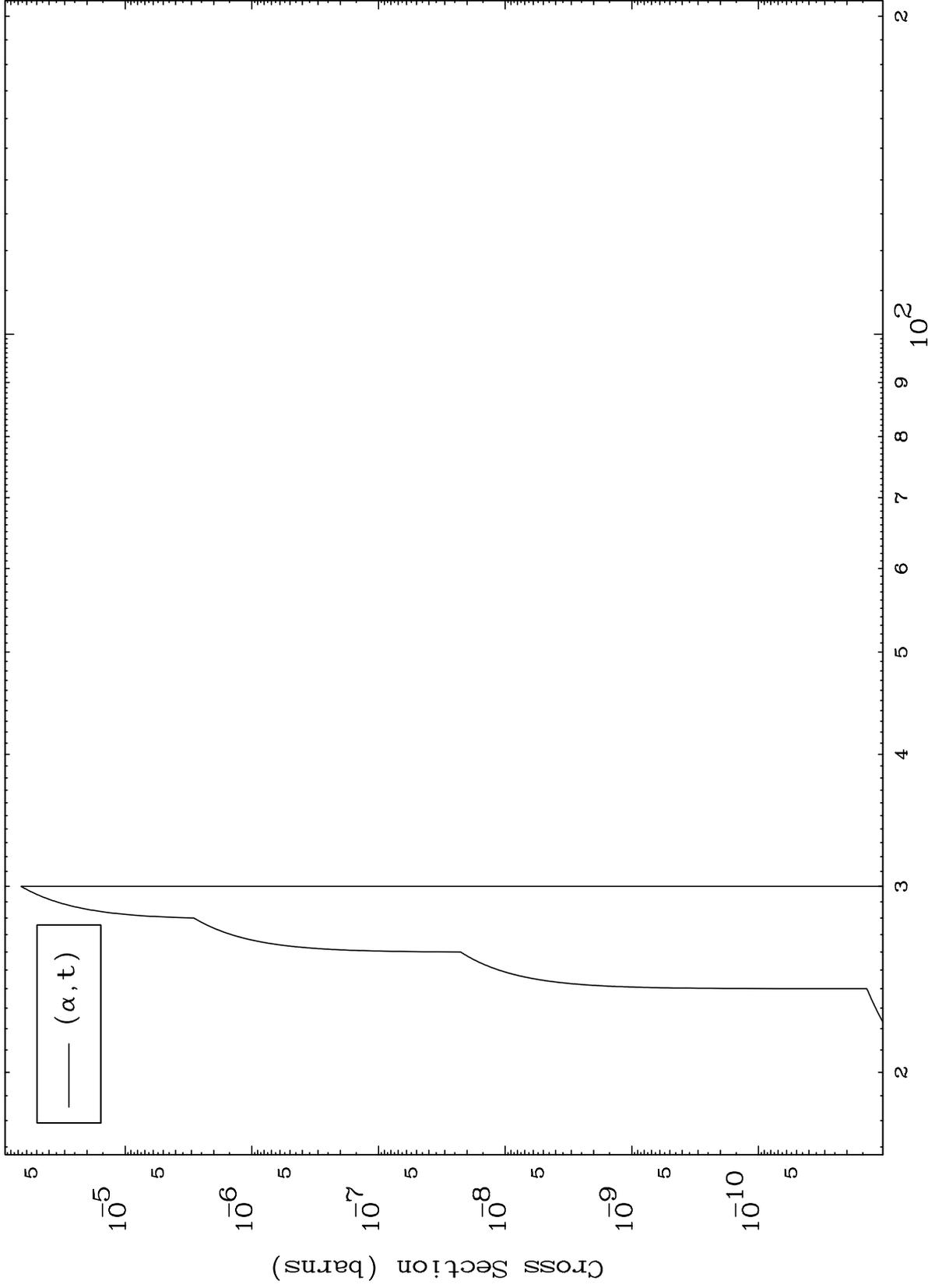
100-Fm-251



6

Incident Energy (MeV)

100-Fm-251

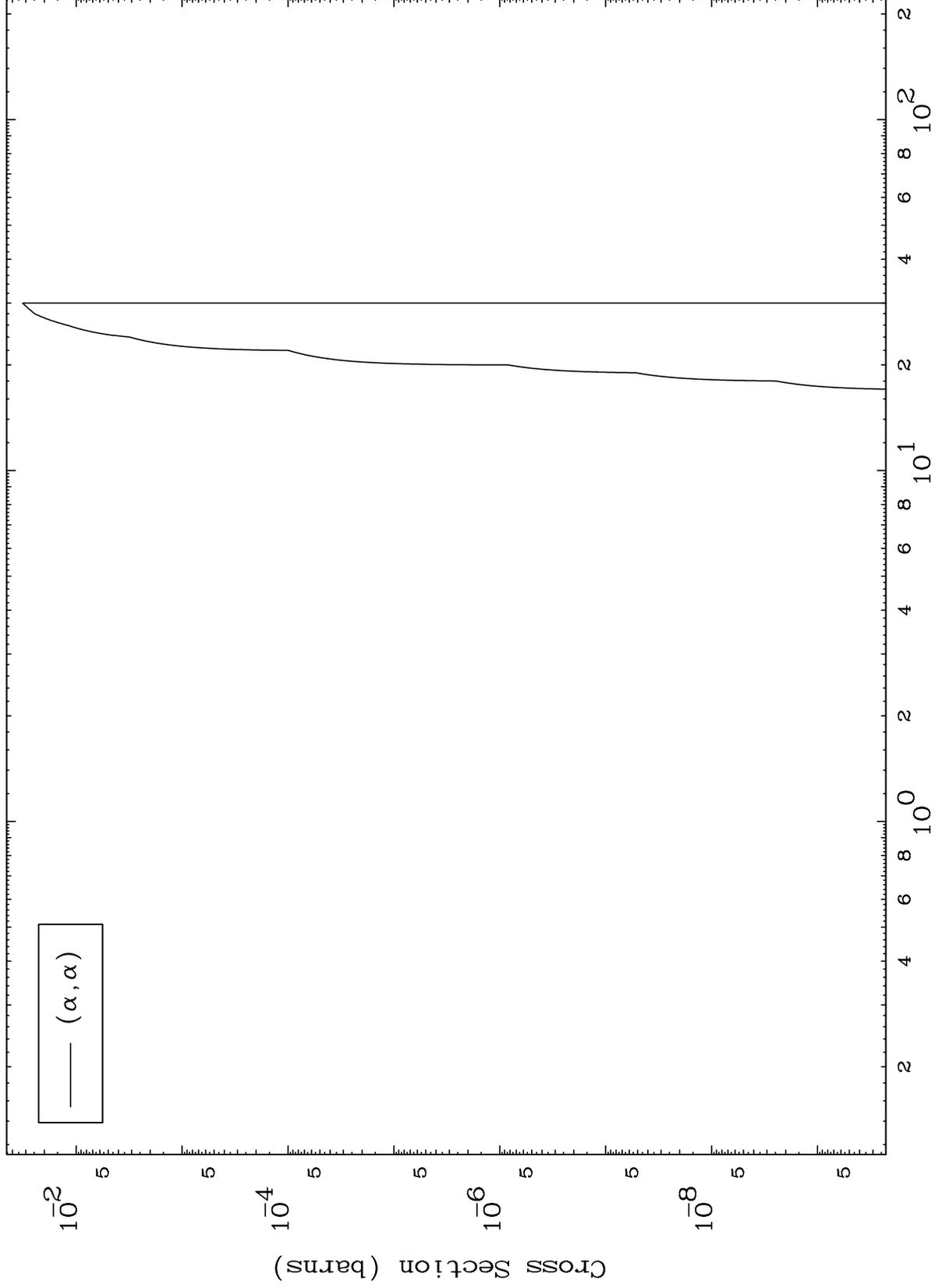


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( $\alpha, \alpha$ ) Levels

100-Fm-251

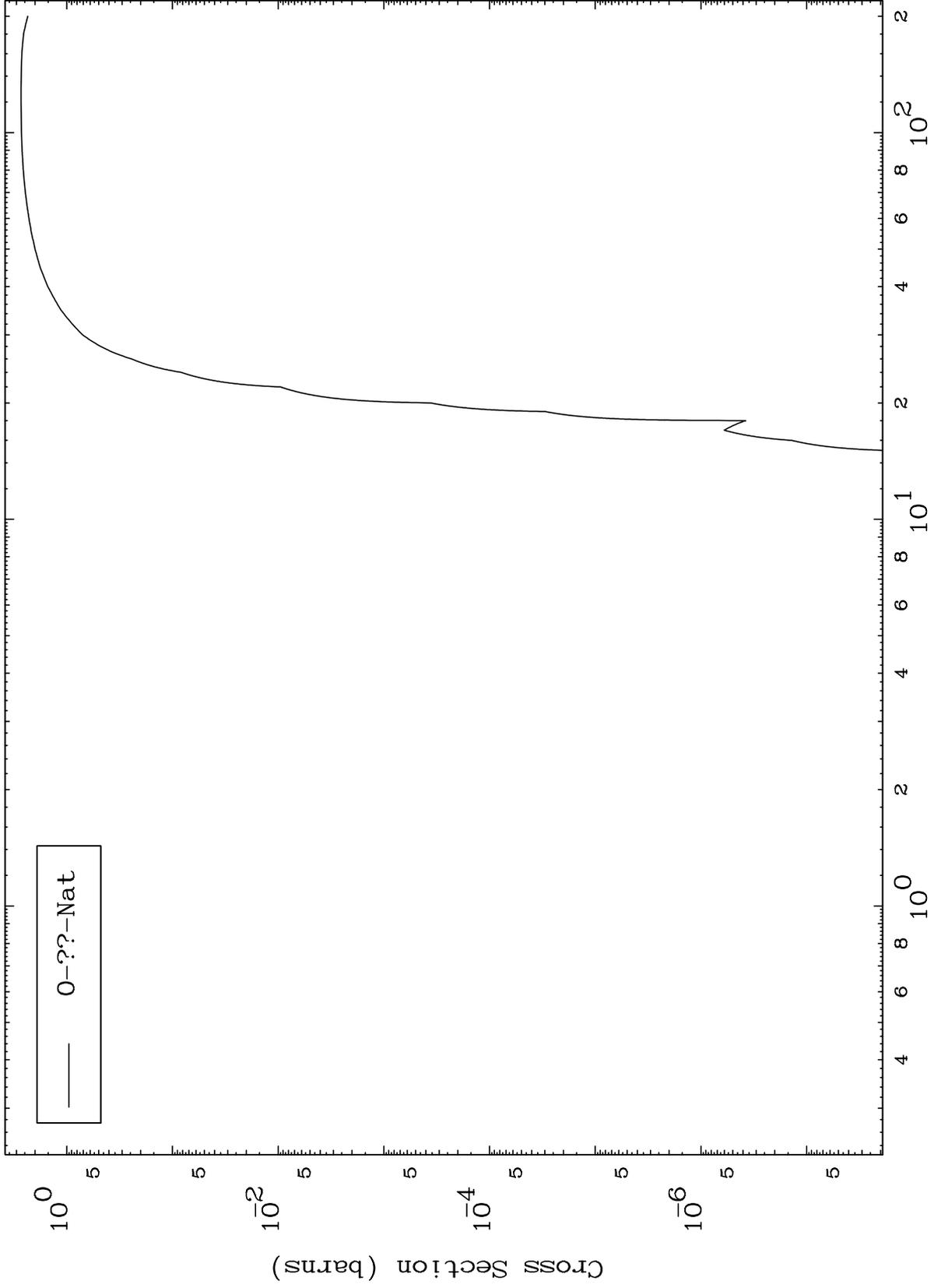
0 Kelvin Cross Sections



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100-Fm-251

$\alpha$  Fission  
Radionuclide Production Cross Section

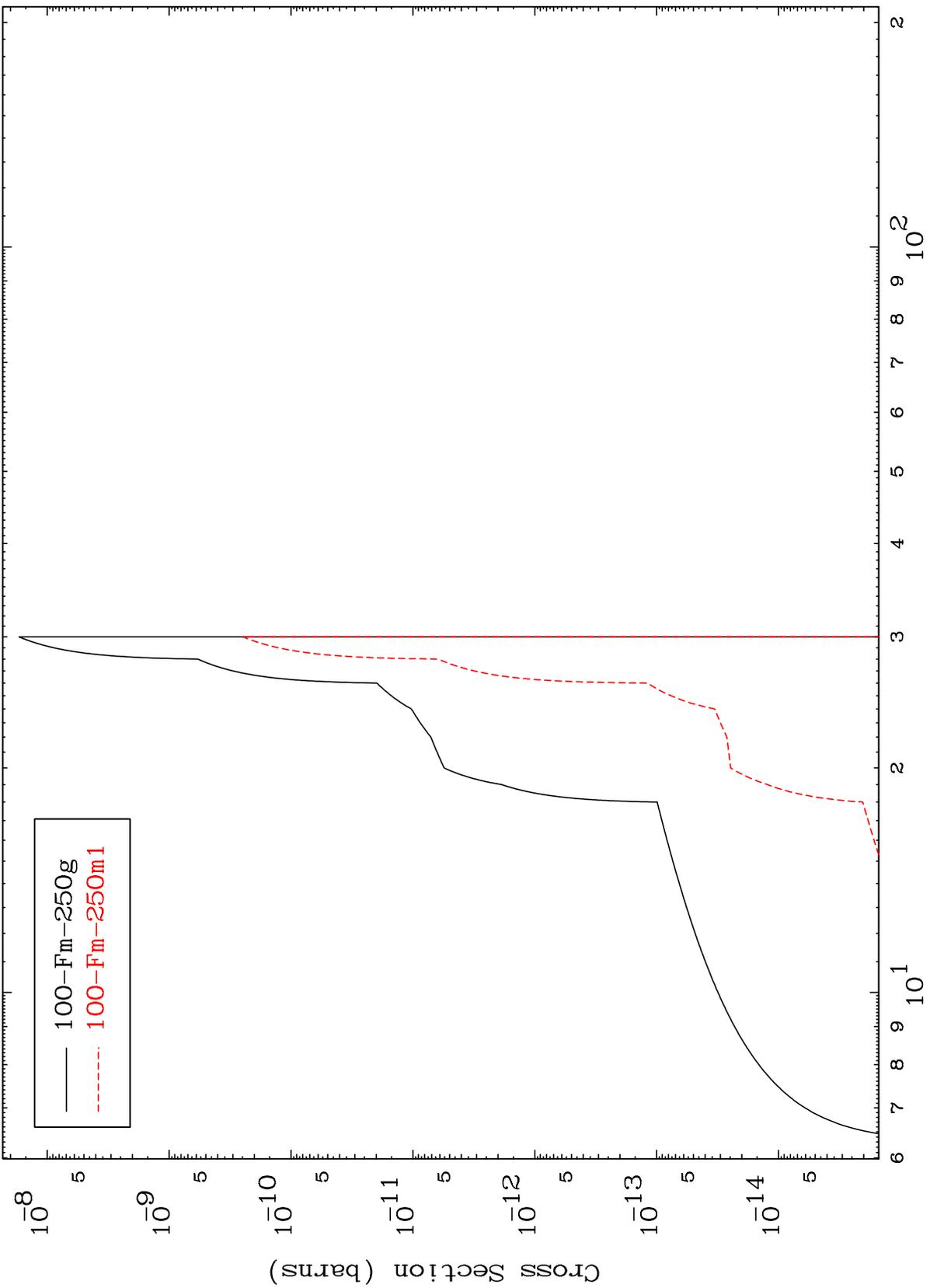


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$(\alpha, n')$   $\alpha$

100-Fm-251

Radionuclide Production Cross Section



100-Fm-251

Incident Energy (MeV)

10

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100-Fm-251

( $\alpha, p$ )  
Radionuclide Production Cross Section

