

Program EVALPLOT  
(Version 2018-1)

by

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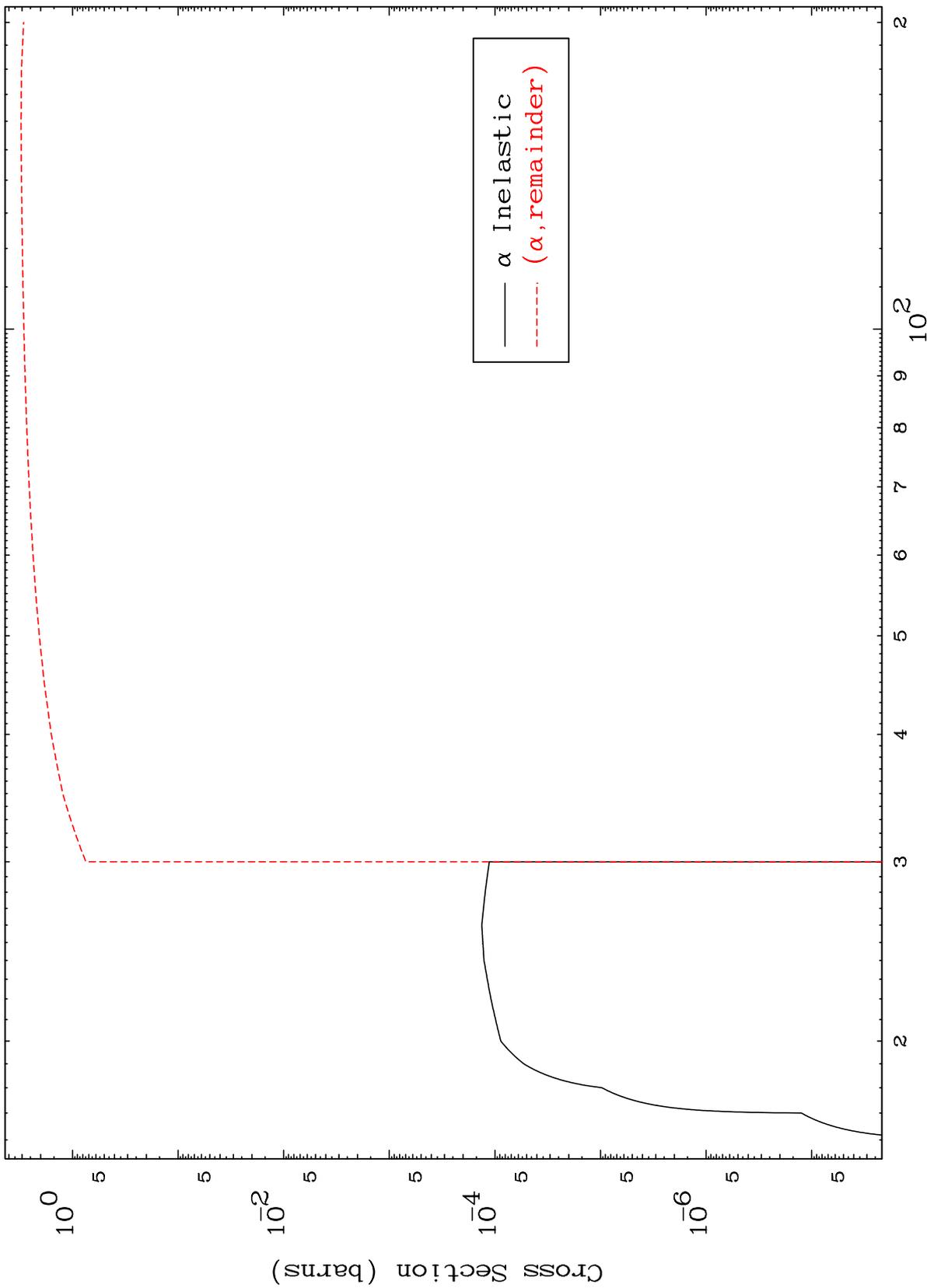
Web: [redcullen1.net/HOMEPAGE.NEW](http://redcullen1.net/HOMEPAGE.NEW)

Press Mouse Button to Start

MAT 9933

$\alpha$  Major  
0 Kelvin Cross Sections

100-Fm-252



1

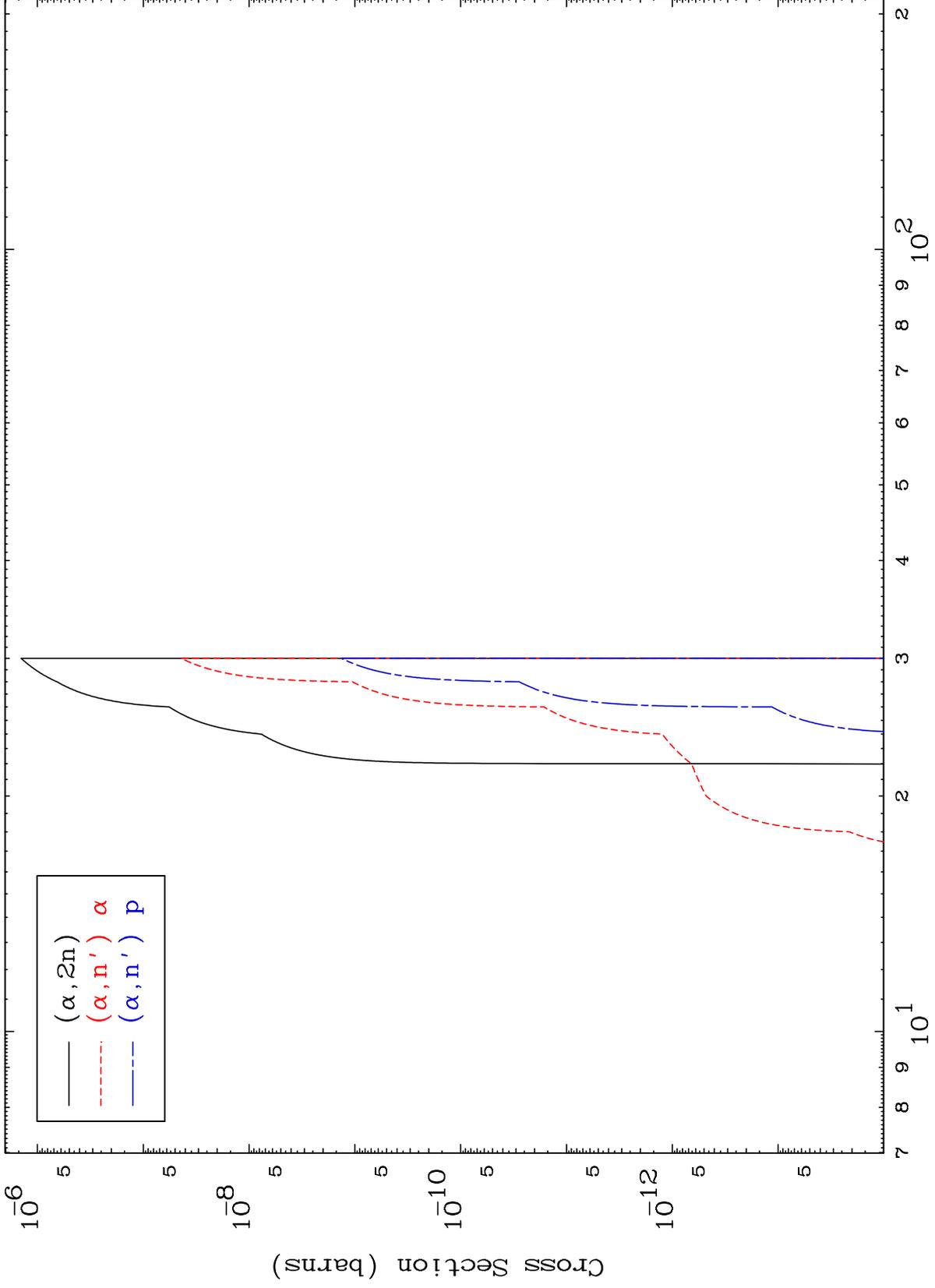
Incident Energy (MeV)

100-Fm-252

MAT 9933

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

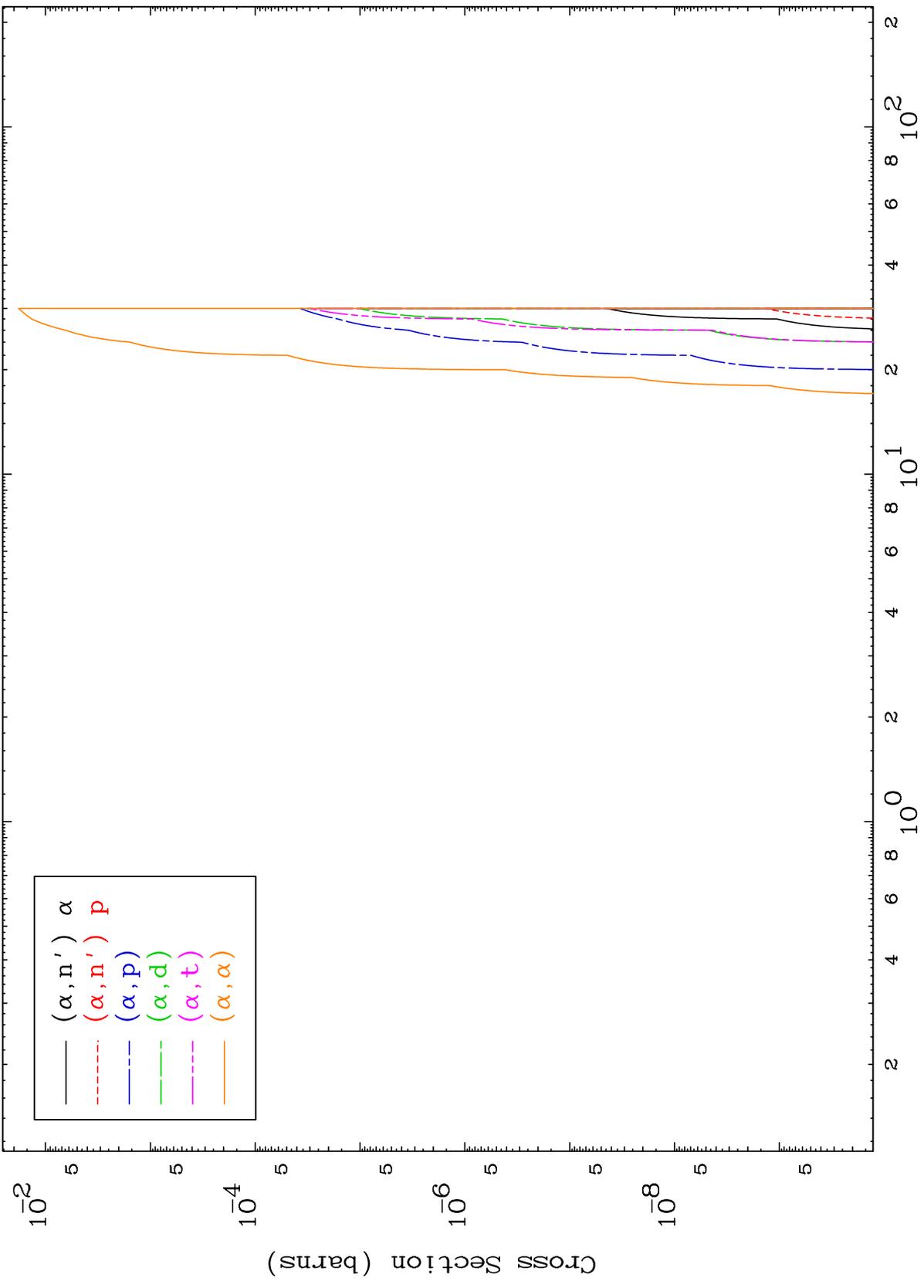
100-Fm-252



100-Fm-252

Incident Energy (MeV)

2

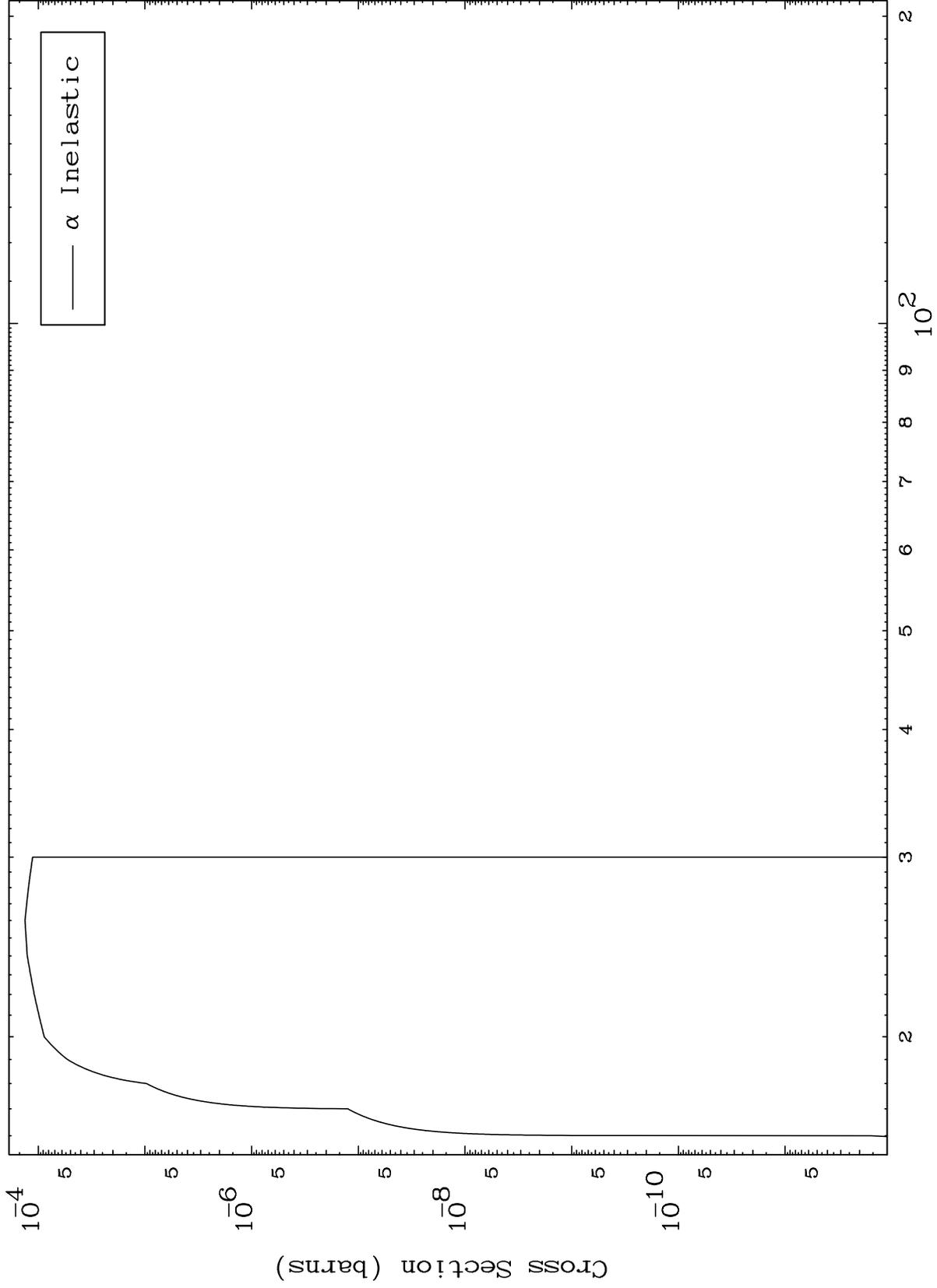


MAT 9933

( $\alpha, n'$ ) Level

100-Fm-252

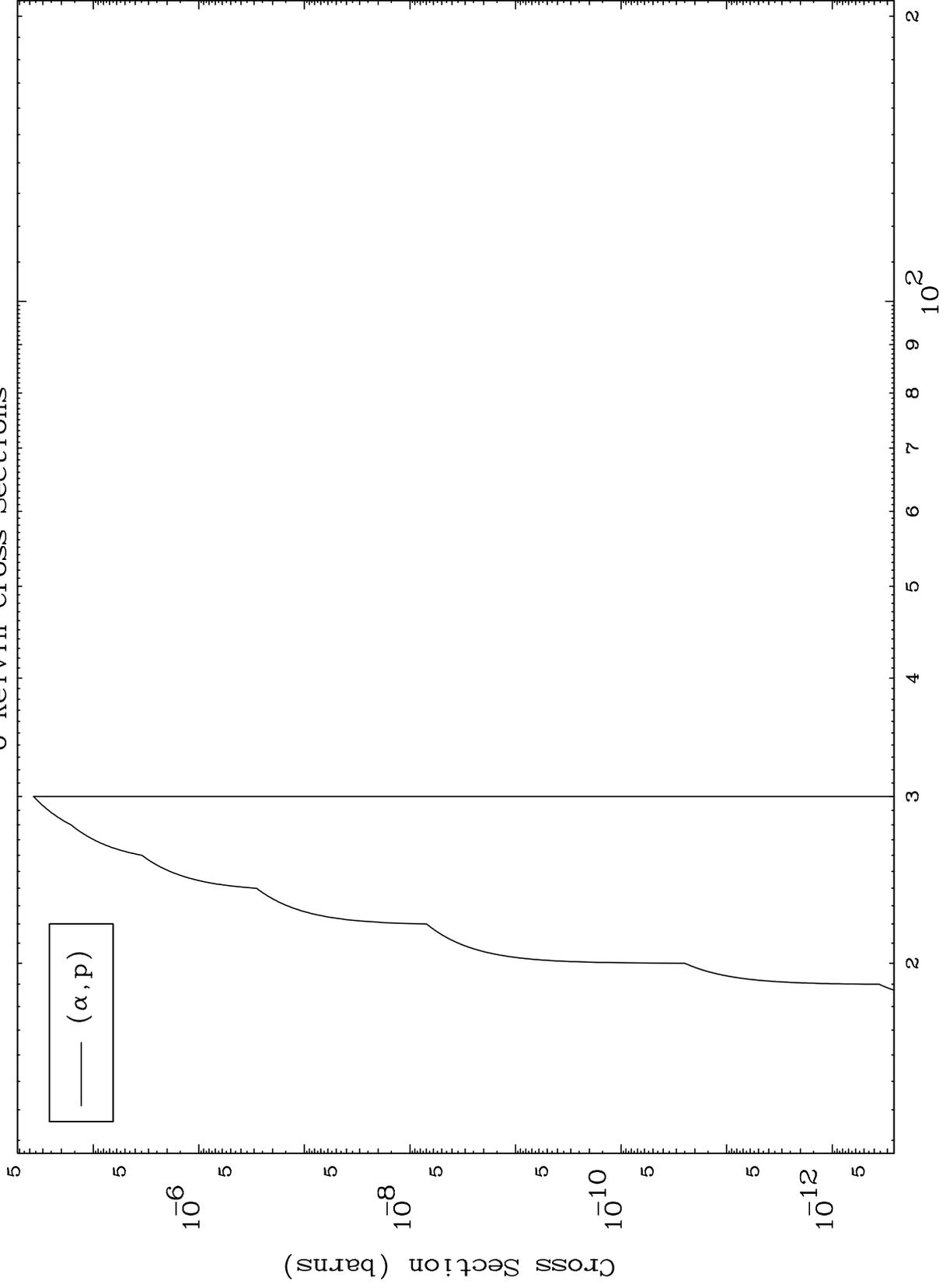
0 Kelvin Cross Sections



MAT 9933

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

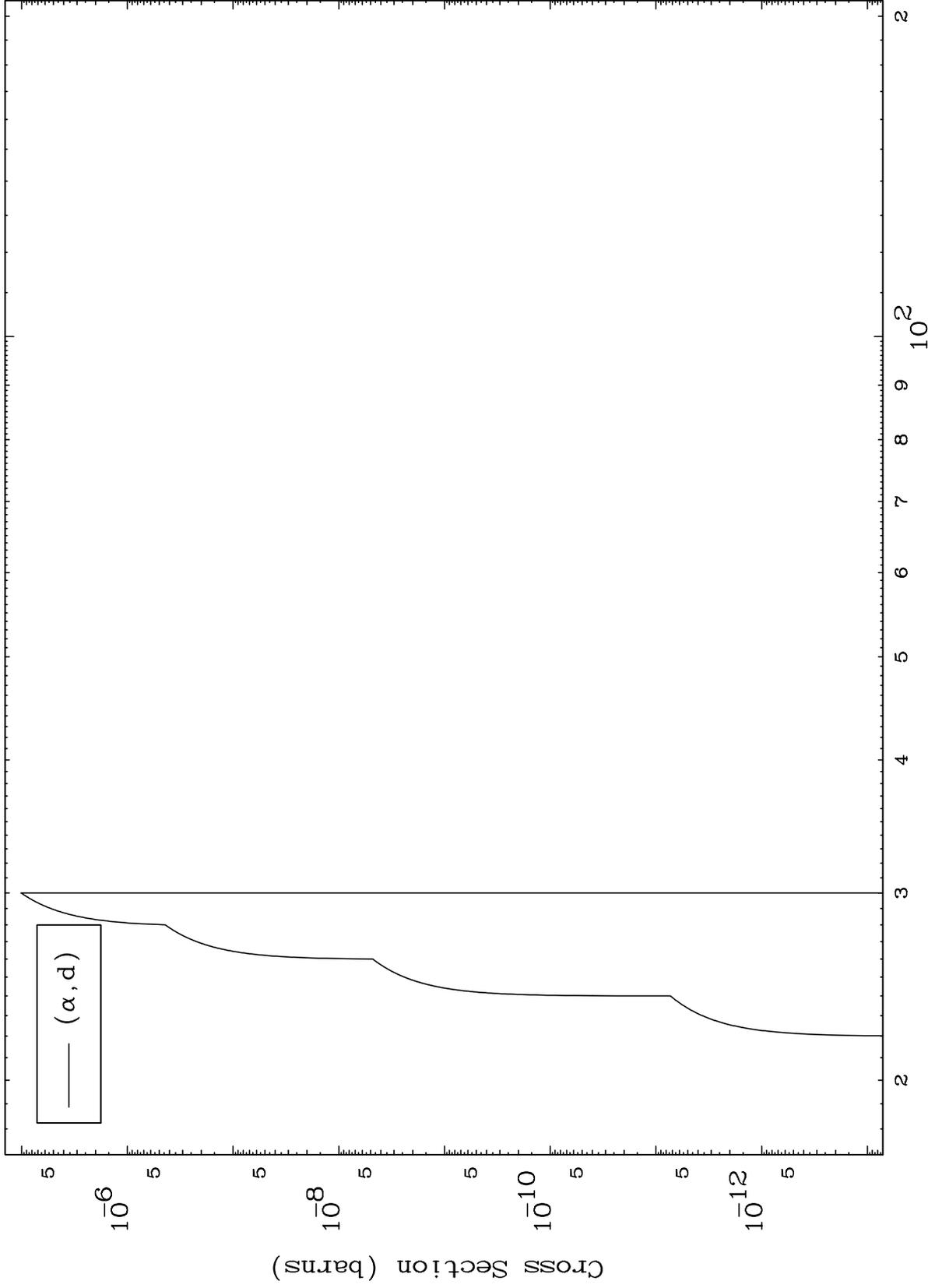
100-Fm-252



5

Incident Energy (MeV)

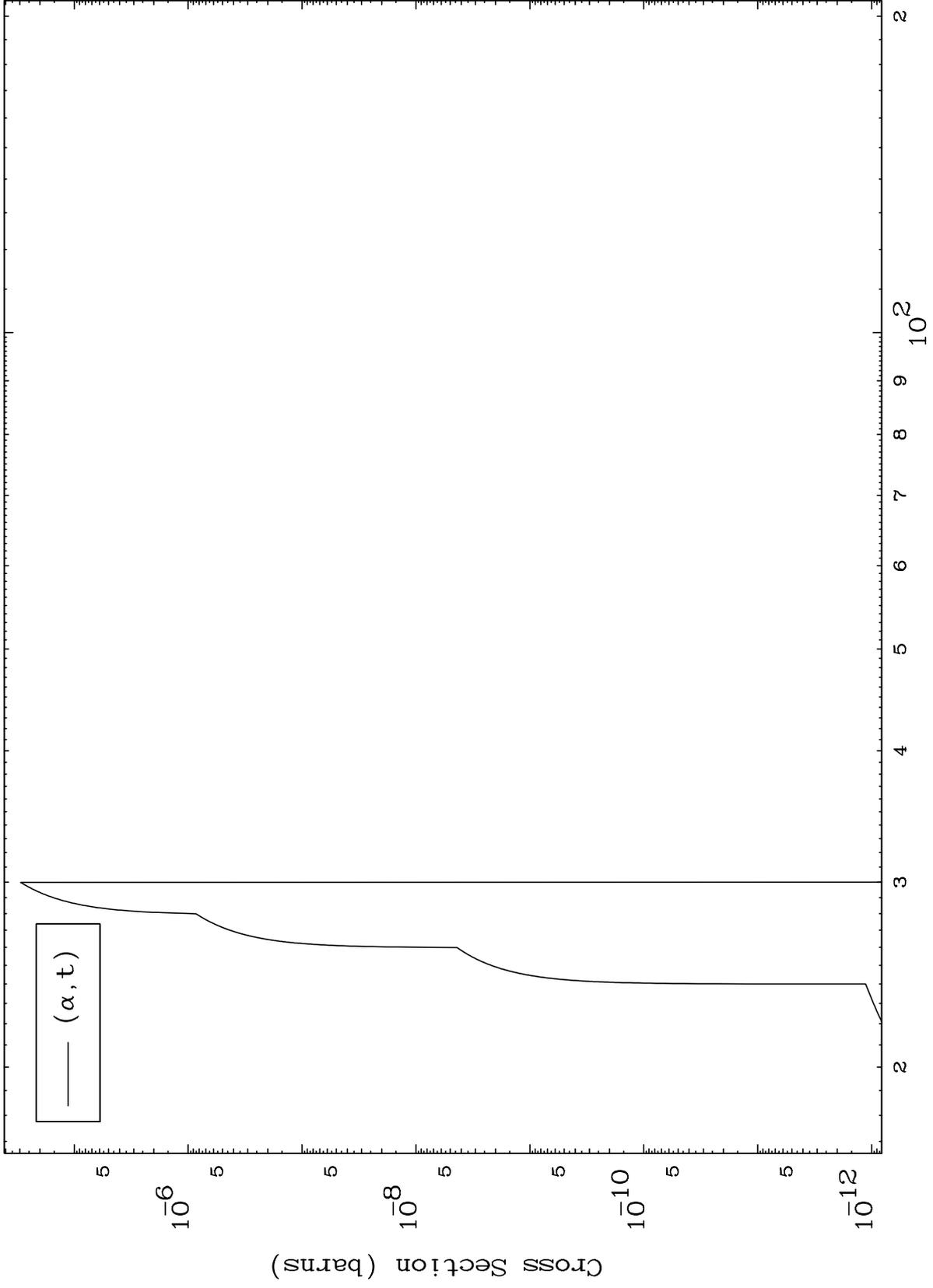
100-Fm-252



MAT 9933

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

100-Fm-252



7

Incident Energy (MeV)

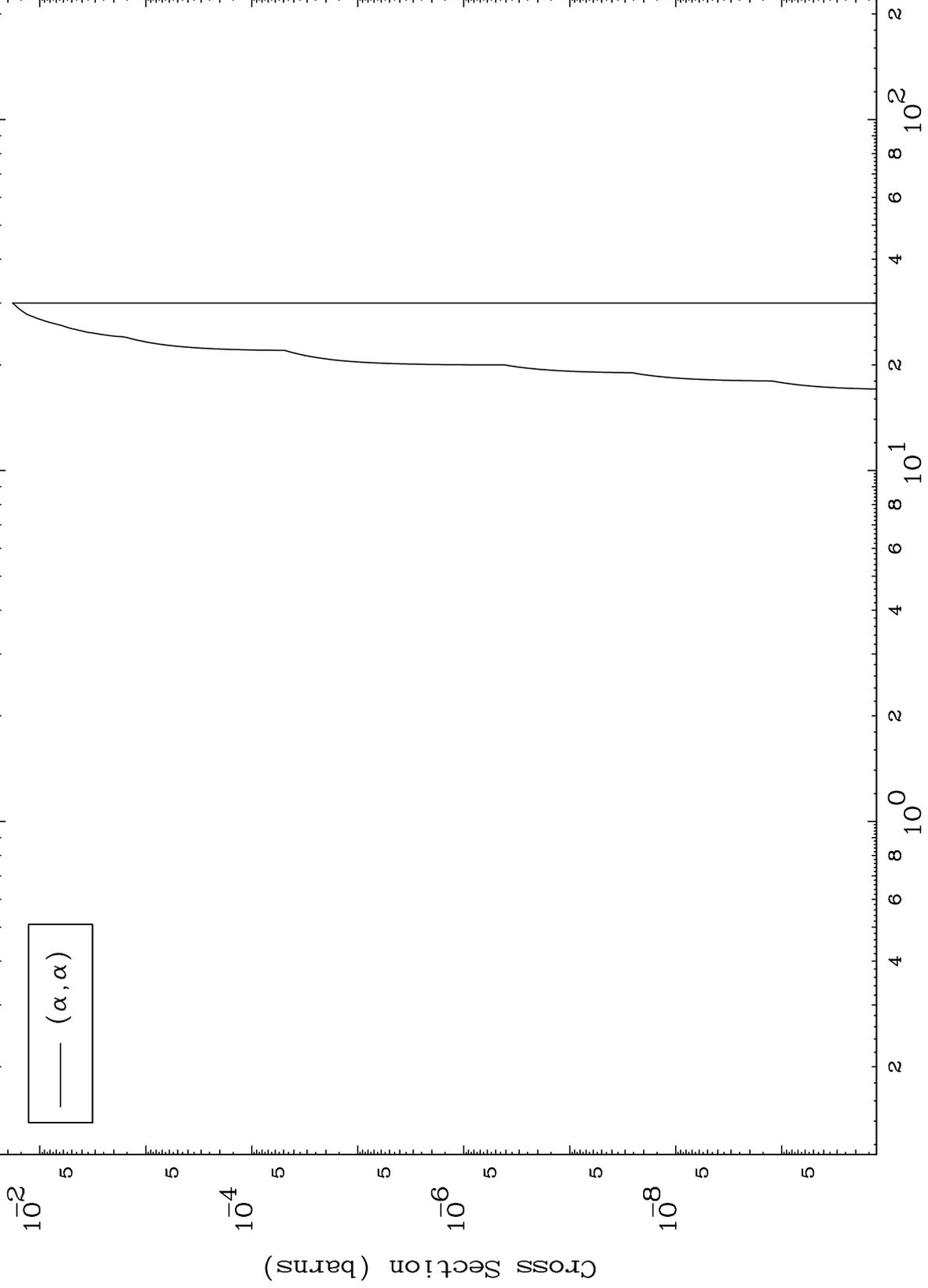
100-Fm-252

MAT 9933

( $\alpha, \alpha$ ) Levels

100-Fm-252

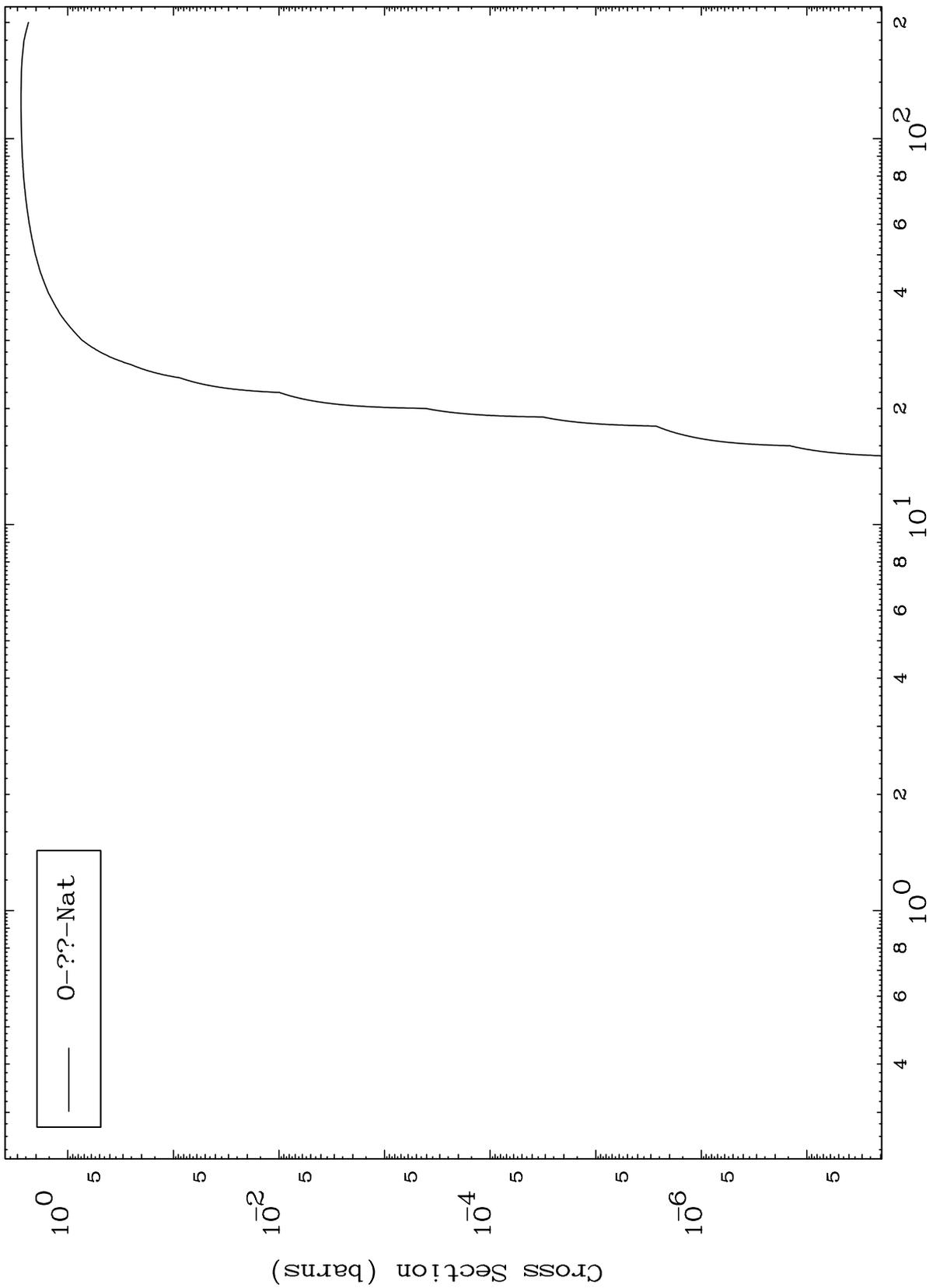
0 Kelvin Cross Sections



MAT 9933

100-Fm-252

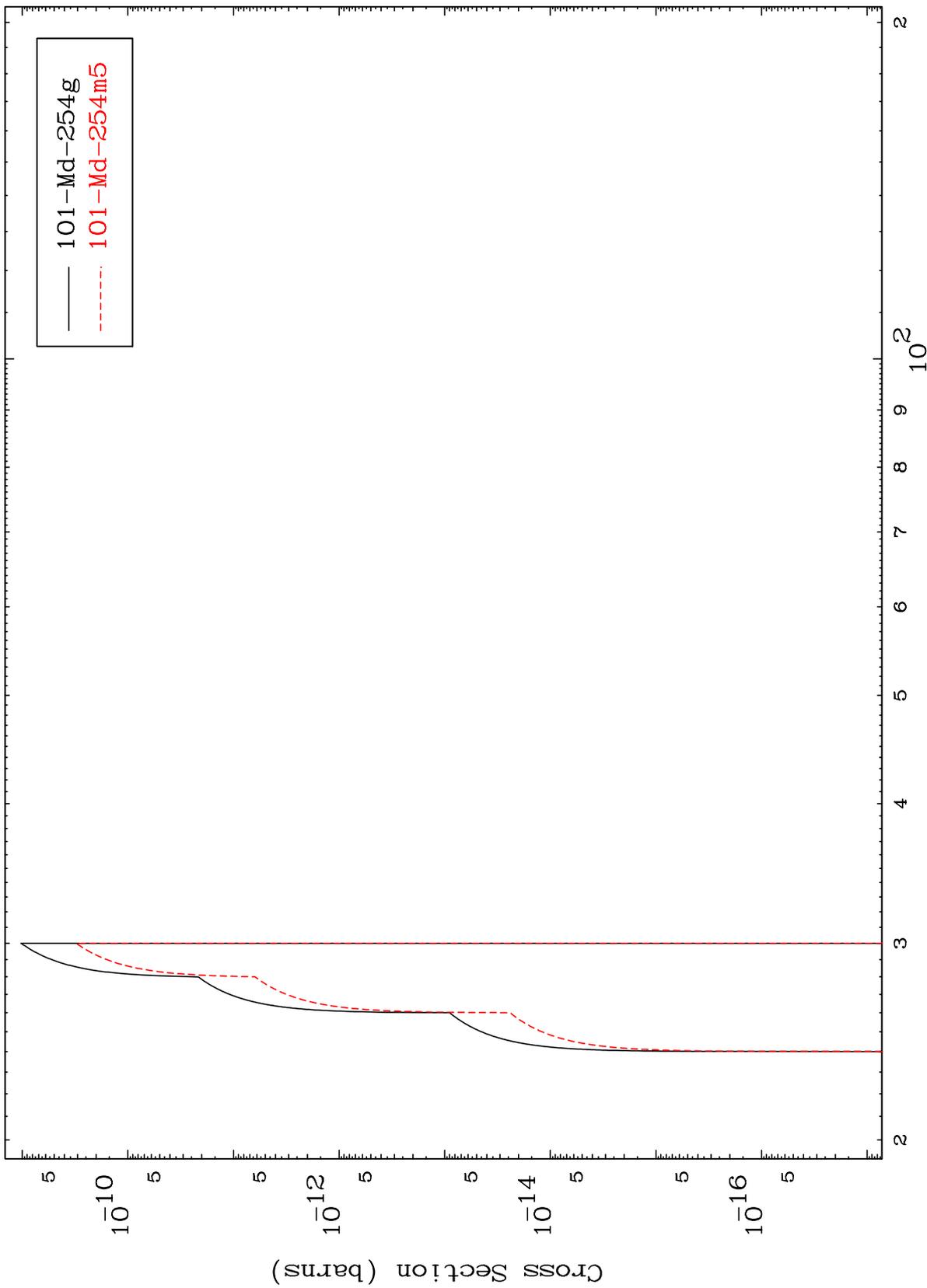
$\alpha$  Fission  
Radionuclide Production Cross Section



MAT 9933

100-Fm-252

$(\alpha, n')$  p  
Radionuclide Production Cross Section



100-Fm-252

Incident Energy (MeV)

10

Radionuclide Production Cross Section

