

Program EVALPLOT  
(Version 2018-1)

by

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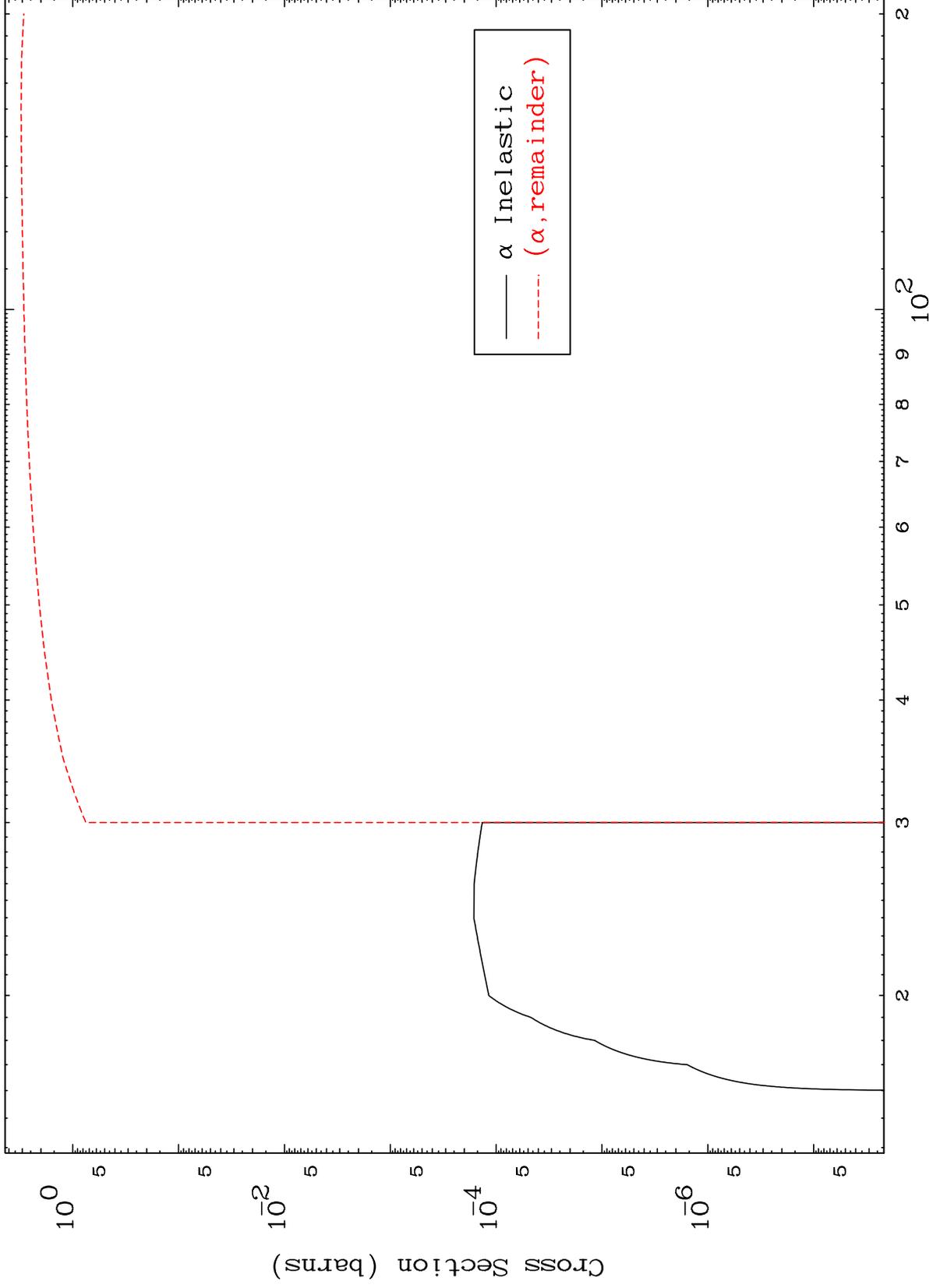
Web: [redcullen1.net/HOMEPAGE.NEW](http://redcullen1.net/HOMEPAGE.NEW)

Press Mouse Button to Start

MAT 9934

$\alpha$  Major  
0 Kelvin Cross Sections

100-Fm-253



1

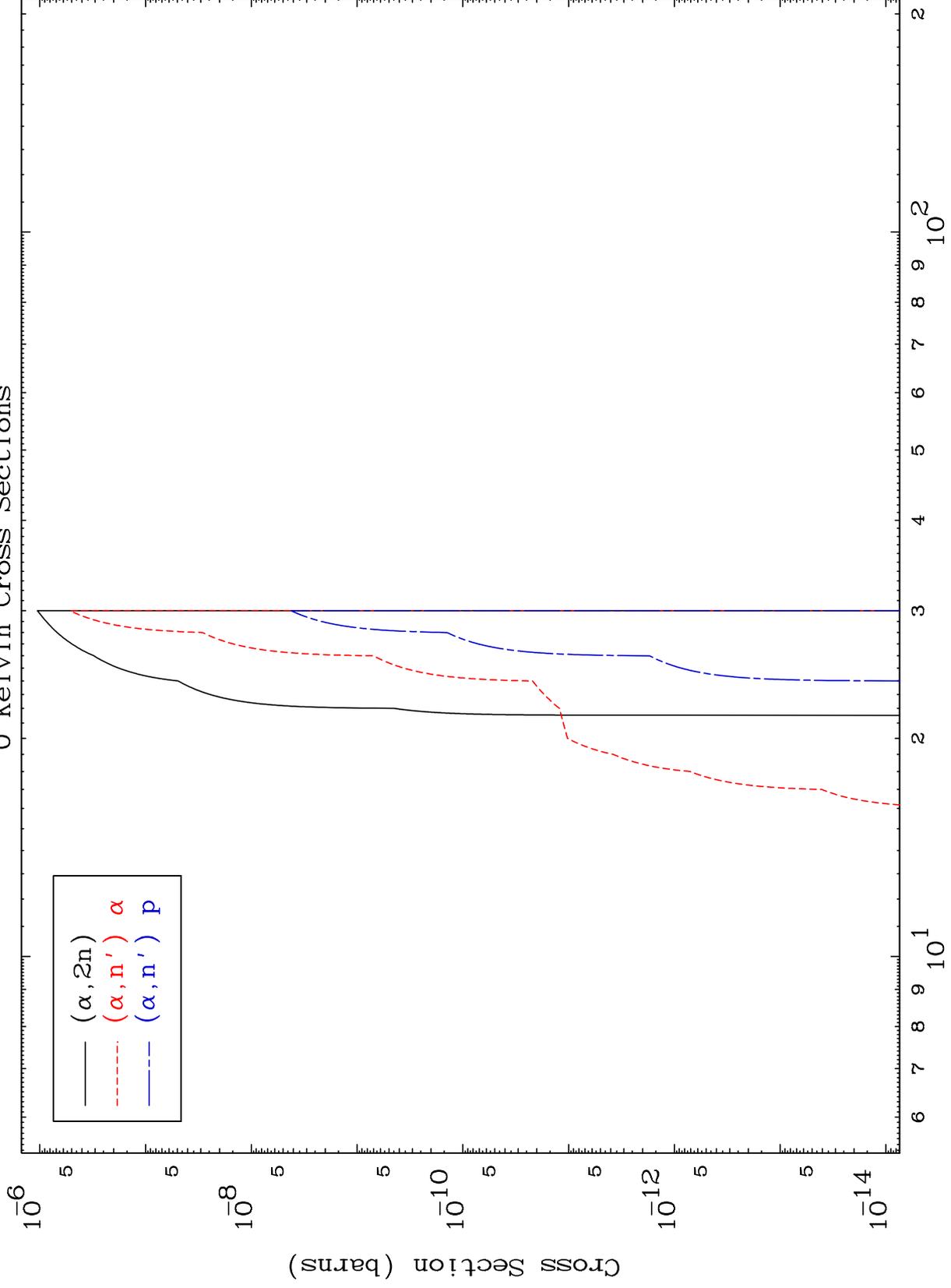
Incident Energy (MeV)

100-Fm-253

MAT 9934

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

100-Fm-253



2

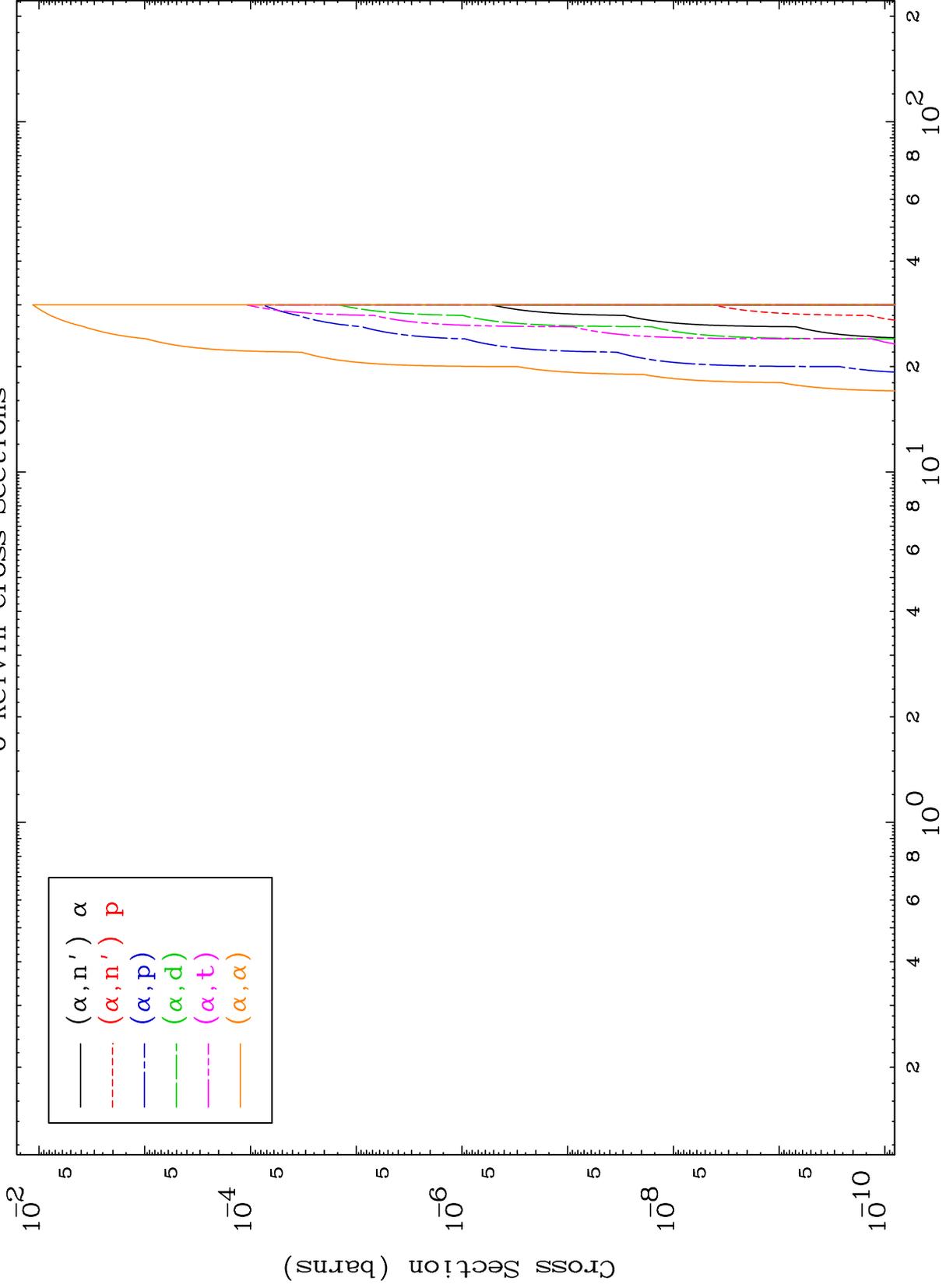
Incident Energy (MeV)

100-Fm-253

MAT 9934

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

100-Fm-253

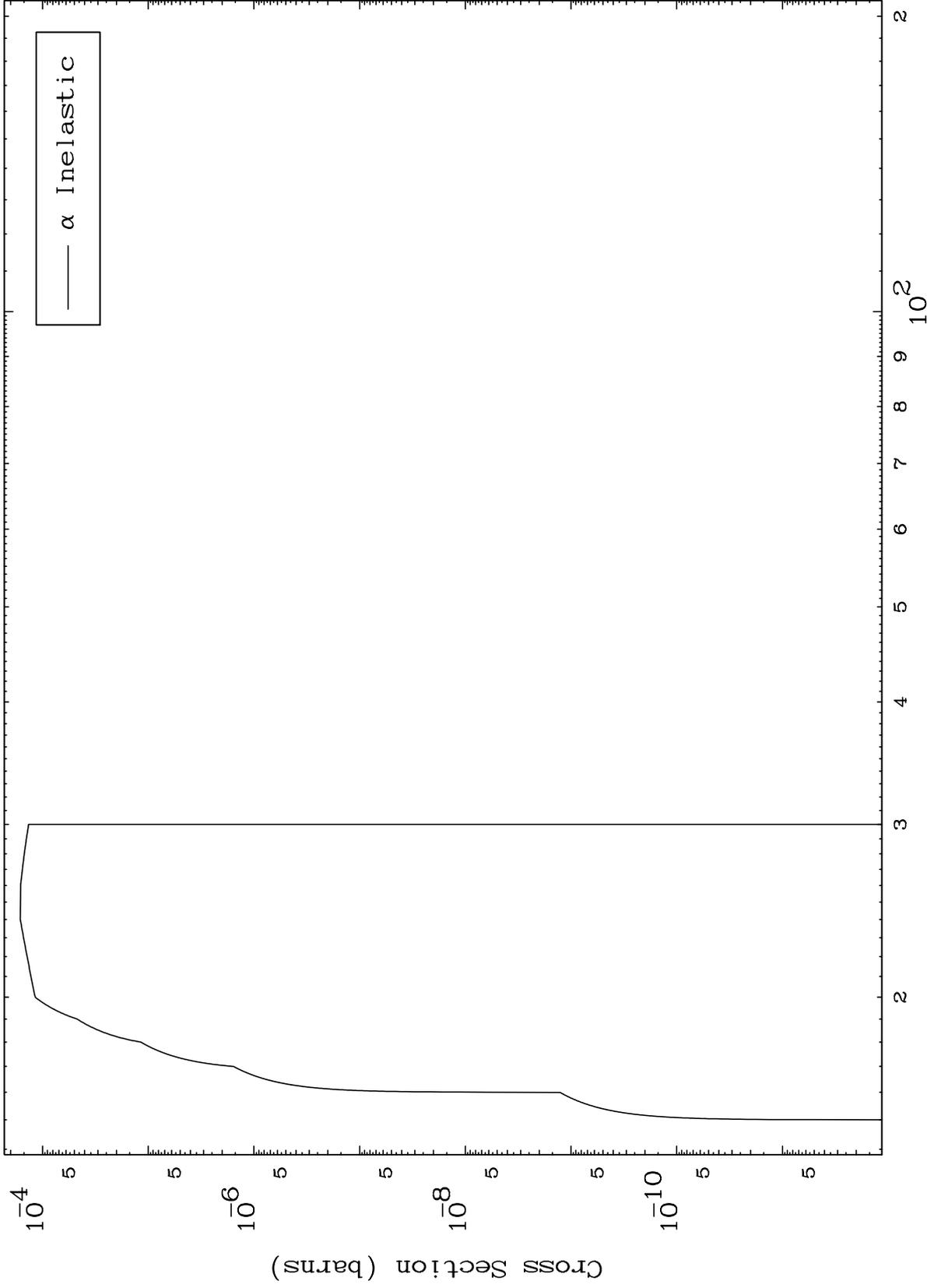


MAT 9934

( $\alpha, n'$ ) Level

100-Fm-253

0 Kelvin Cross Sections



100-Fm-253

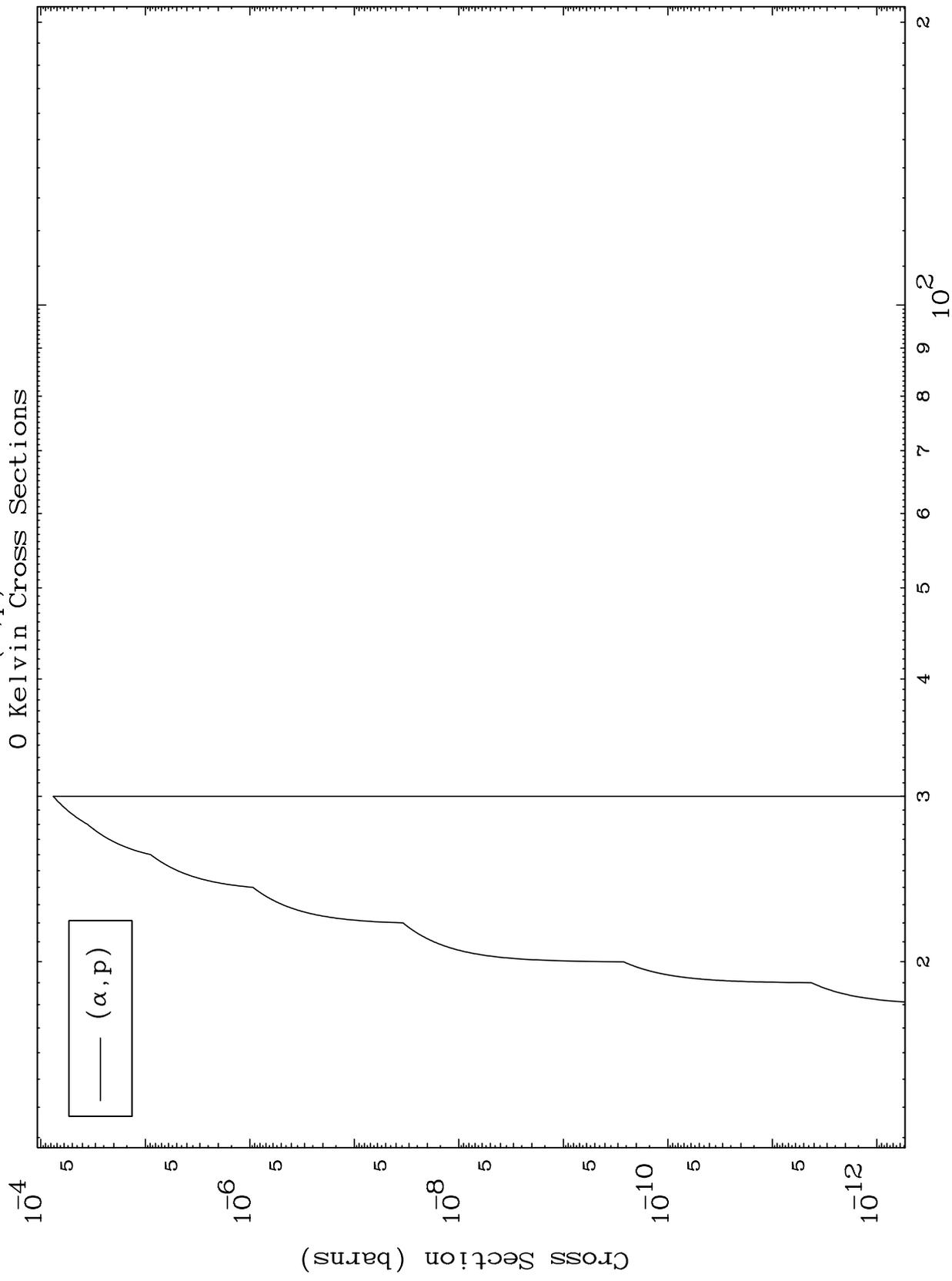
Incident Energy (MeV)

4

MAT 9934

100-Fm-253

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections



100-Fm-253

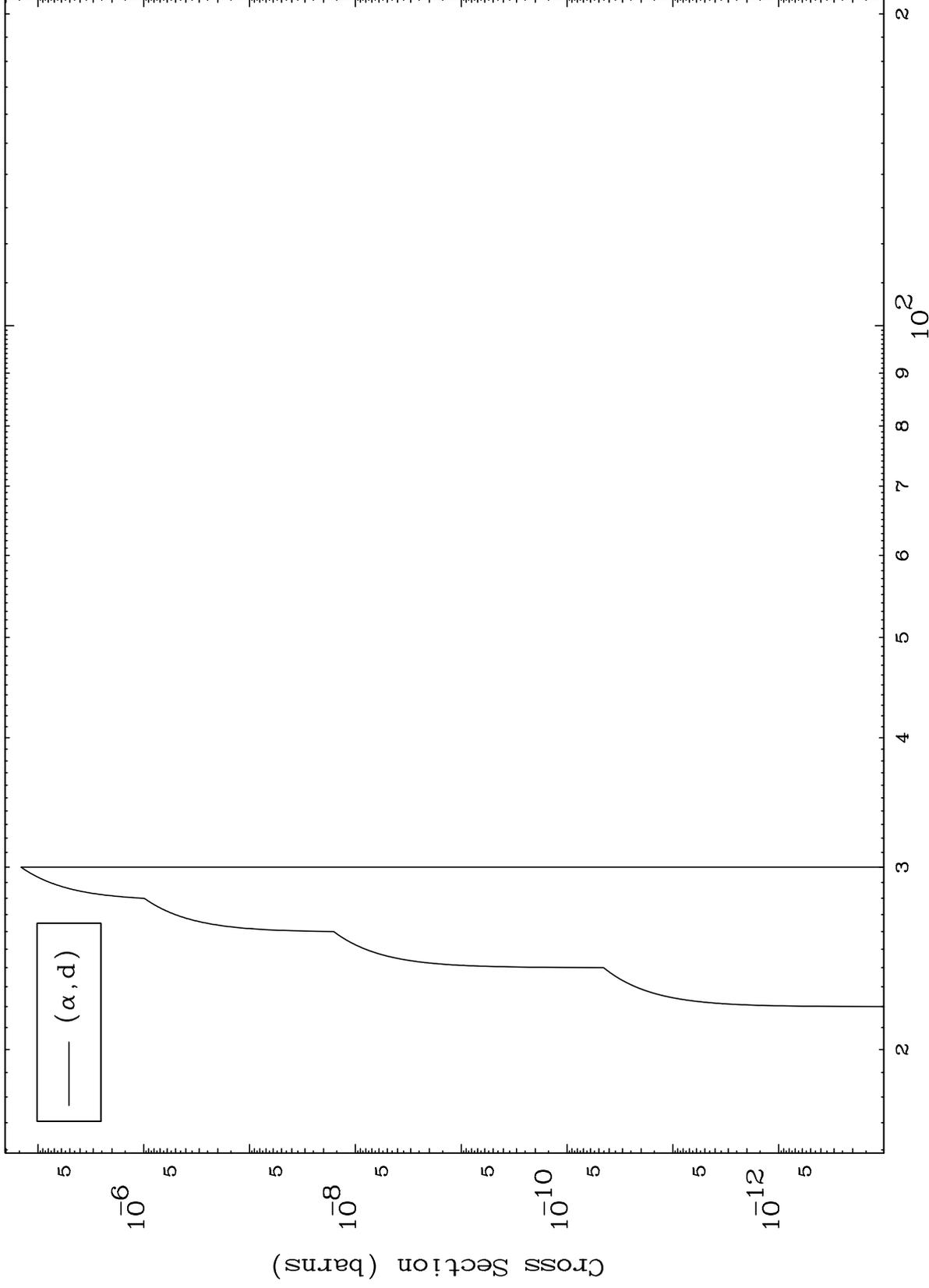
Incident Energy (MeV)

5

MAT 9934

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

100-Fm-253



6

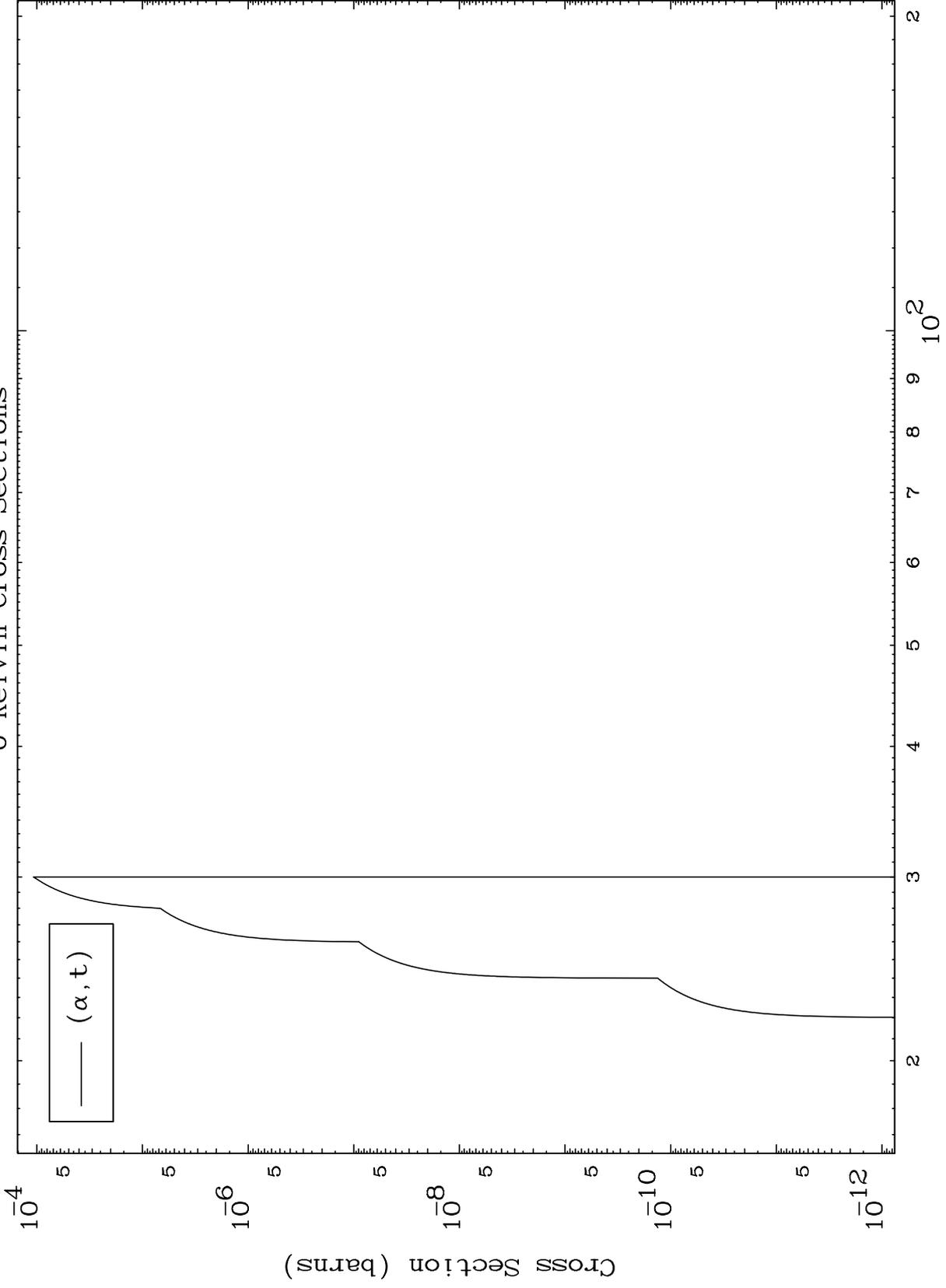
Incident Energy (MeV)

100-Fm-253

MAT 9934

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

100-Fm-253



7

Incident Energy (MeV)

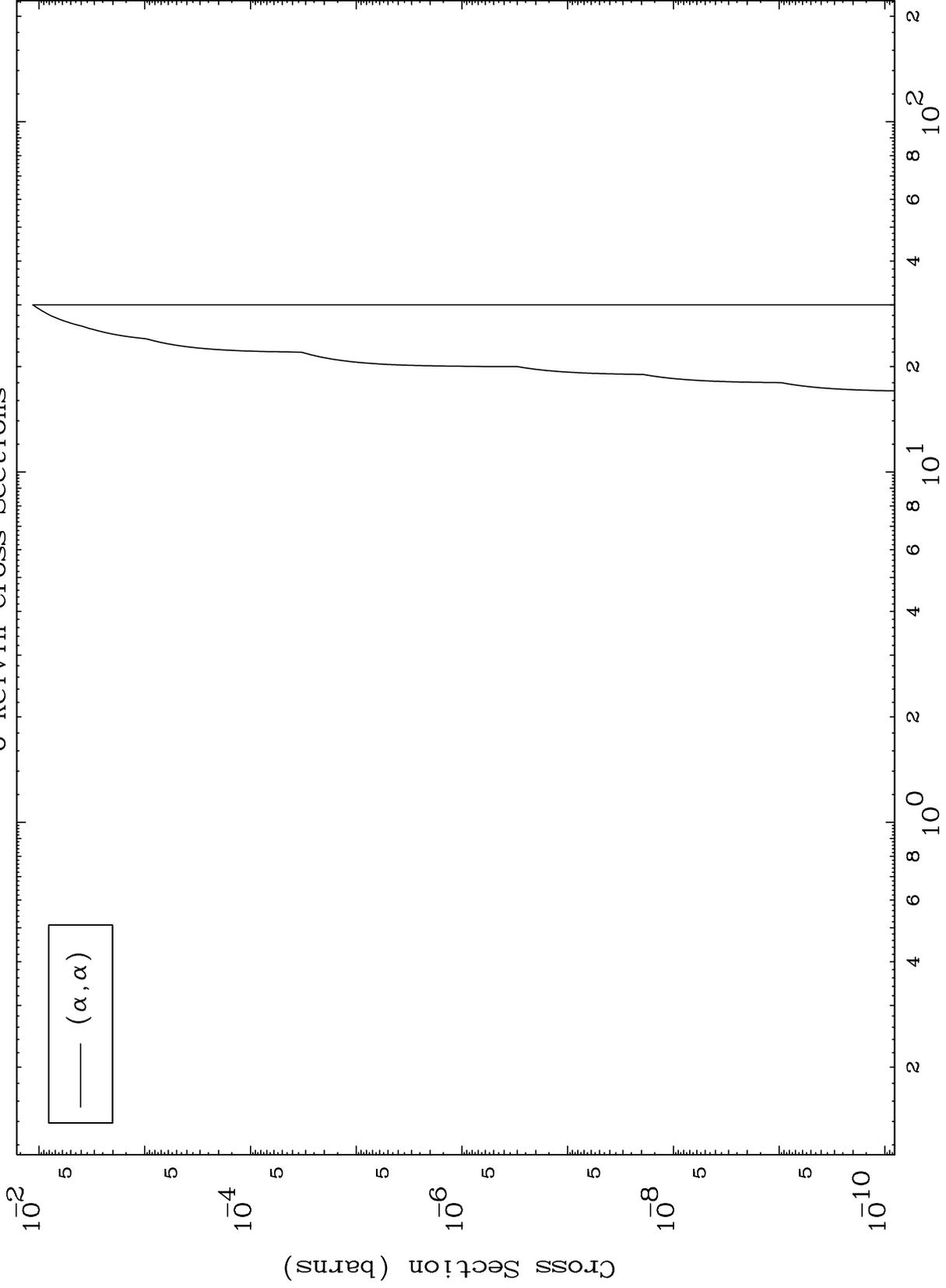
100-Fm-253

MAT 9934

( $\alpha, \alpha$ ) Levels

100-Fm-253

0 Kelvin Cross Sections

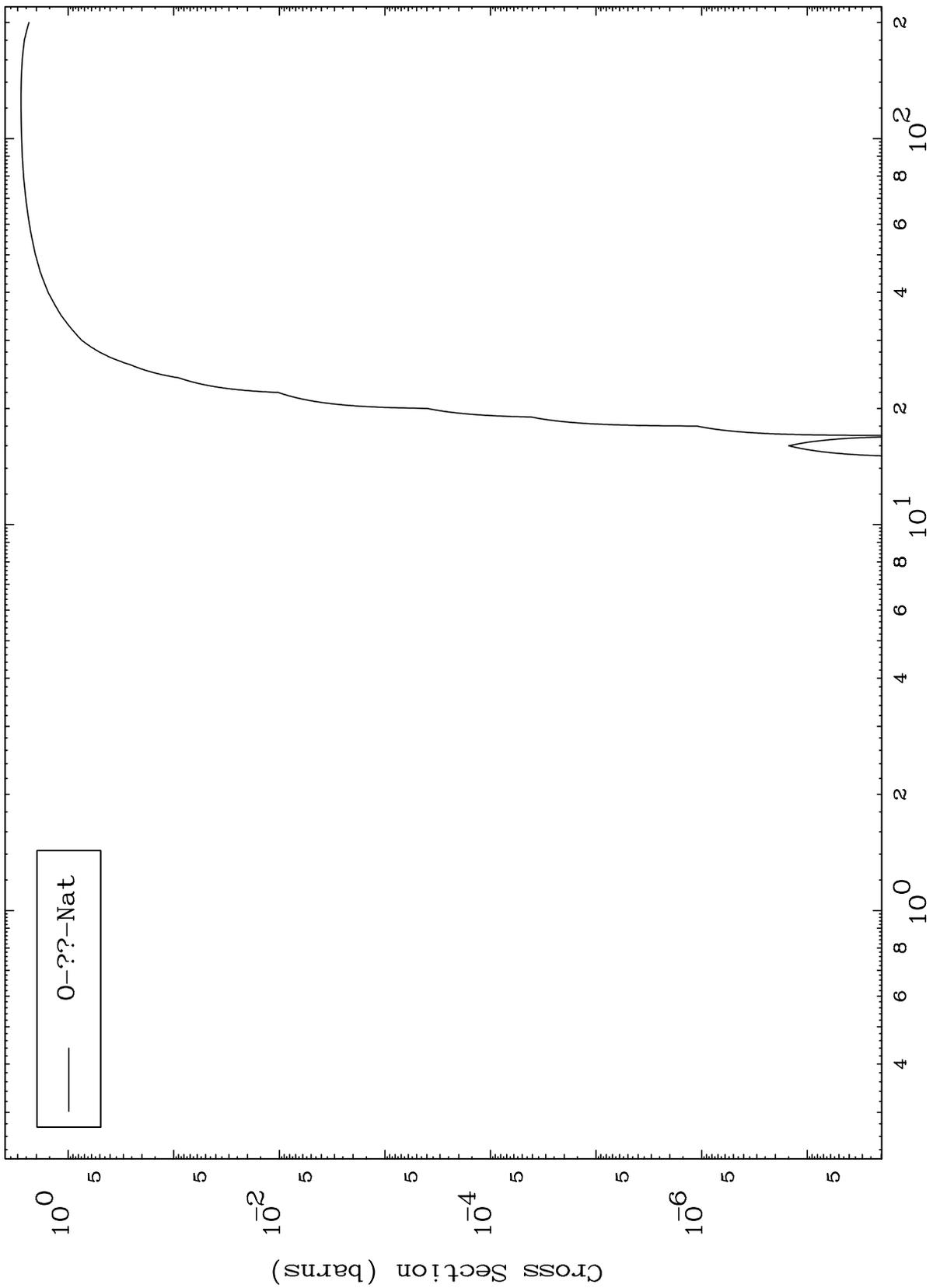


MAT 9934

100-Fm-253

$\alpha$  Fission

Radionuclide Production Cross Section



100-Fm-253

Incident Energy (MeV)

MAT 9934

( $\alpha, t$ )

100-Fm-253

