

Program EVALPLOT  
(Version 2018-1)

by

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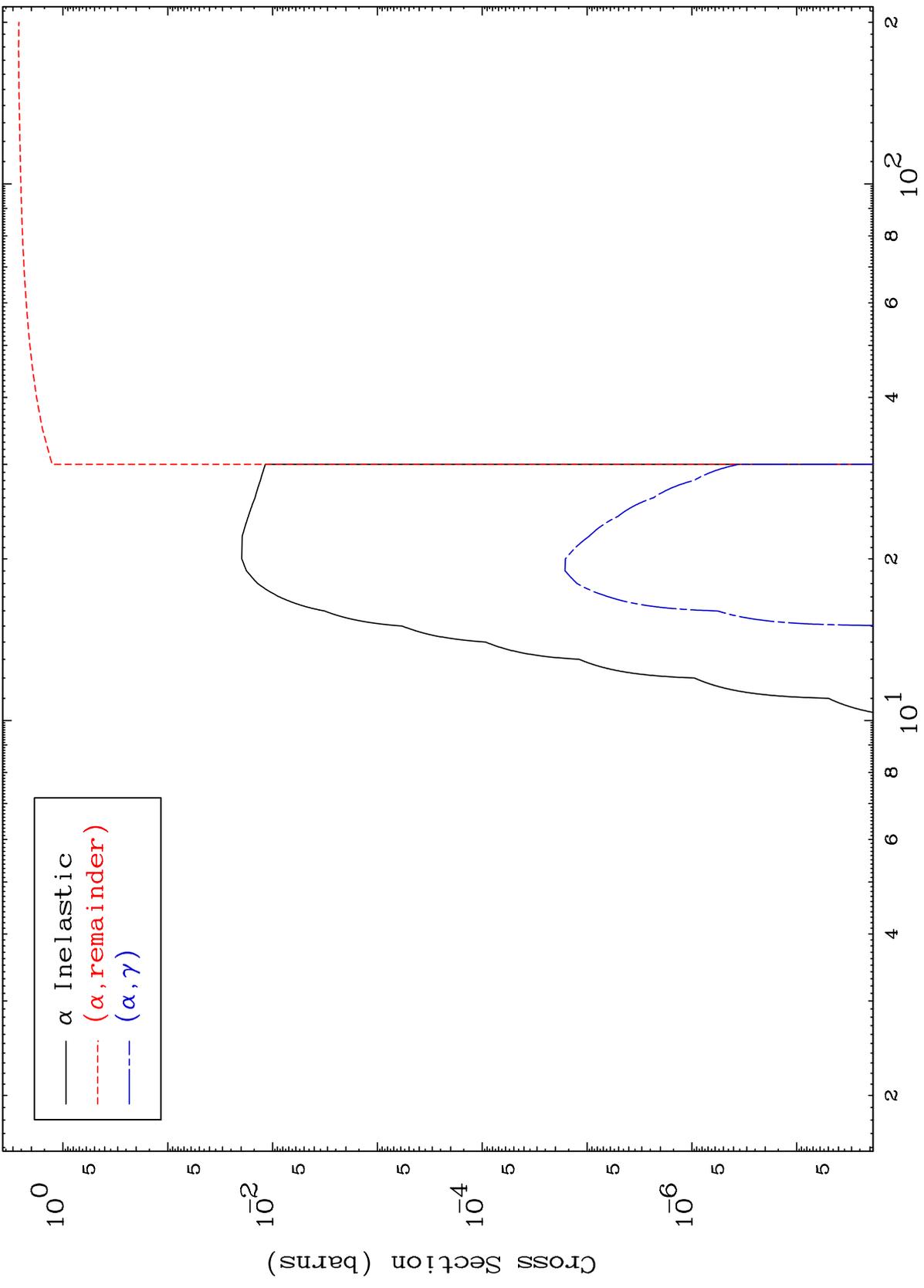
Press Mouse Button to Start

MAT 7243

$\alpha$  Major

72-Hf-180

0 Kelvin Cross Sections

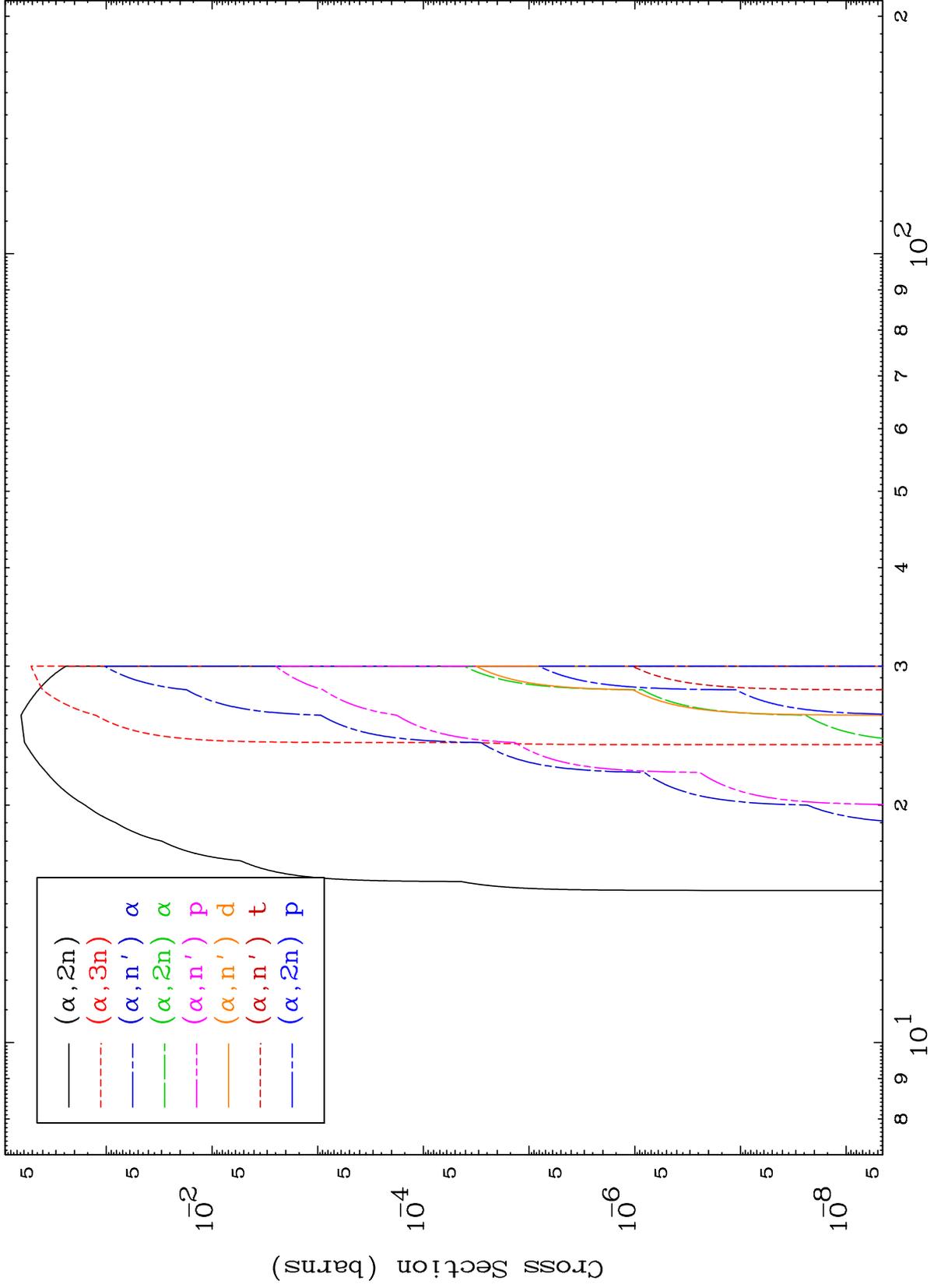


$\alpha$  Inelastic  
 $(\alpha, \text{remainder})$   
 $(\alpha, \gamma)$

MAT 7243

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

72-Hf-180



2

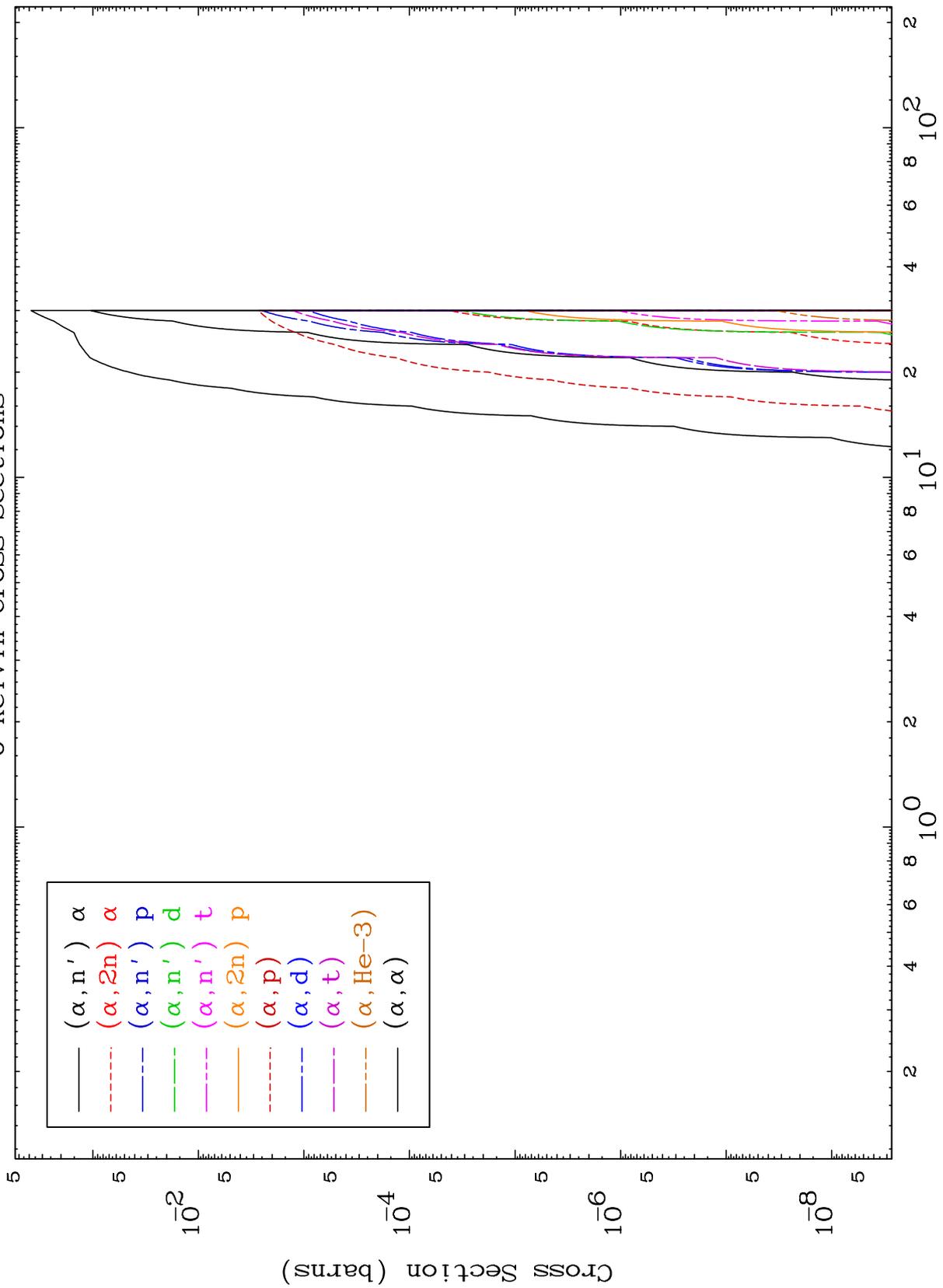
Incident Energy (MeV)

72-Hf-180

MAT 7243

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

72-Hf-180

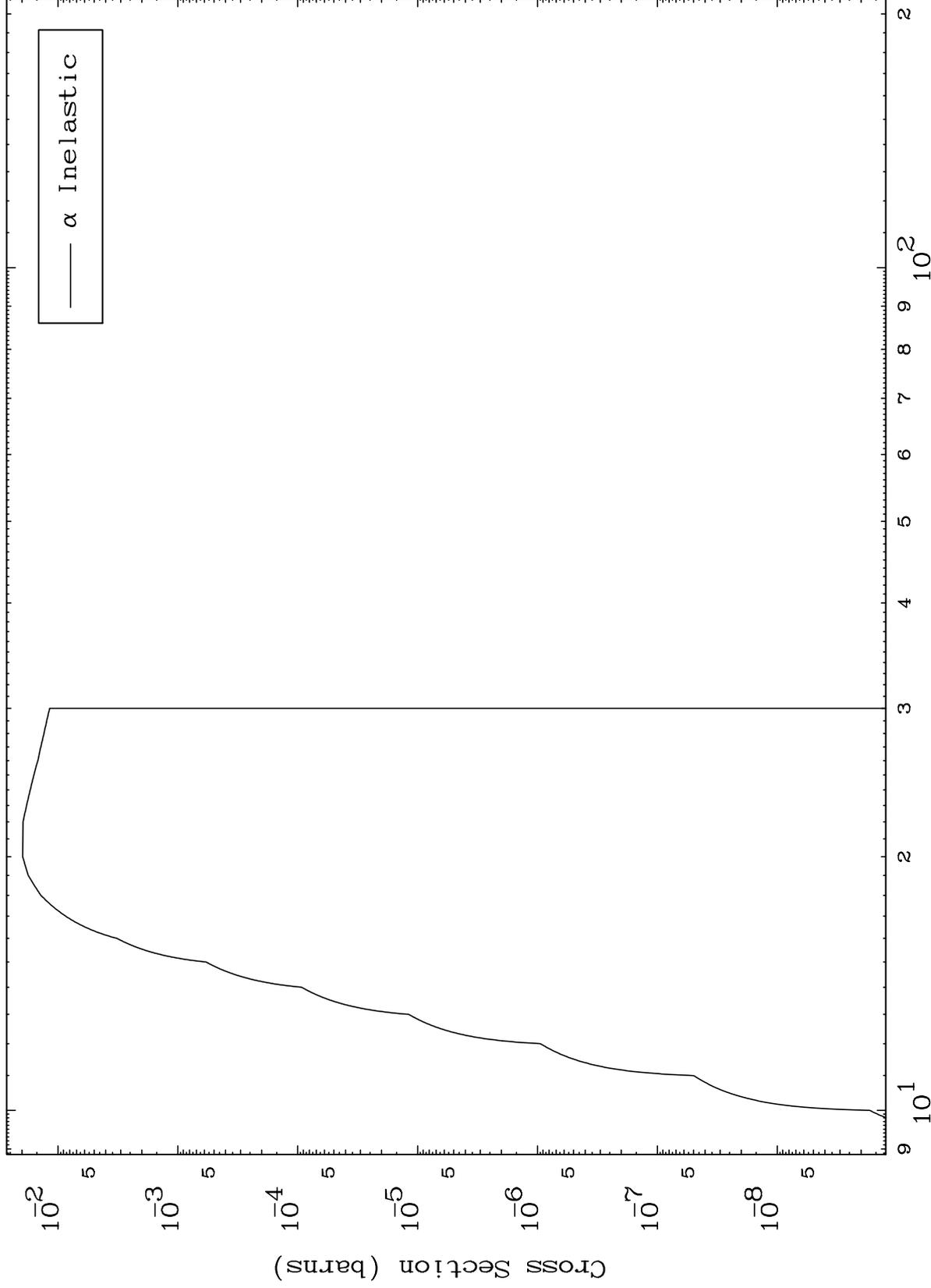


MAT 7243

( $\alpha, n'$ ) Level

72-Hf-180

0 Kelvin Cross Sections



Incident Energy (MeV)

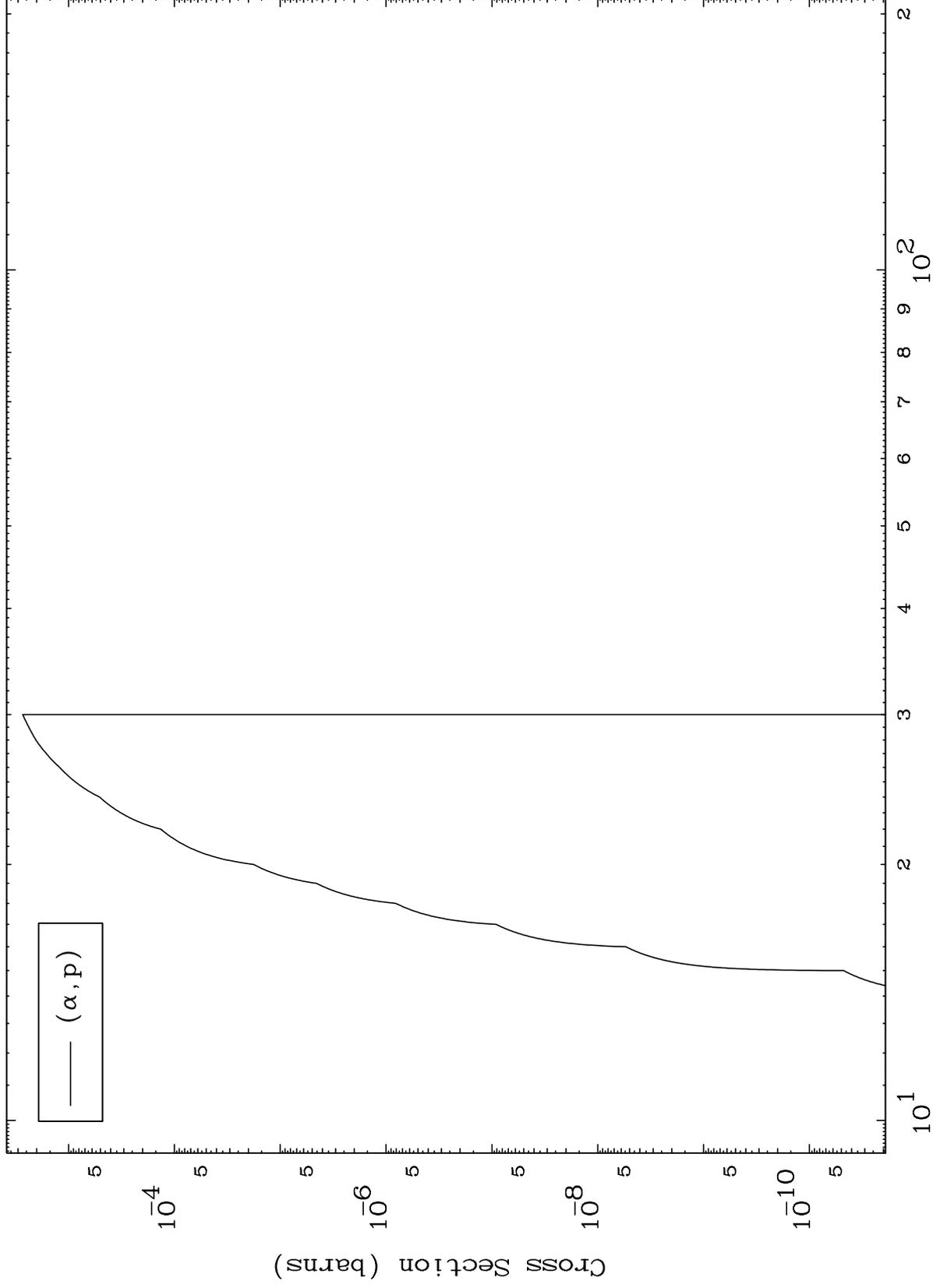
72-Hf-180

MAT 7243

( $\alpha, p$ ) Levels

72-Hf-180

0 Kelvin Cross Sections



Incident Energy (MeV)

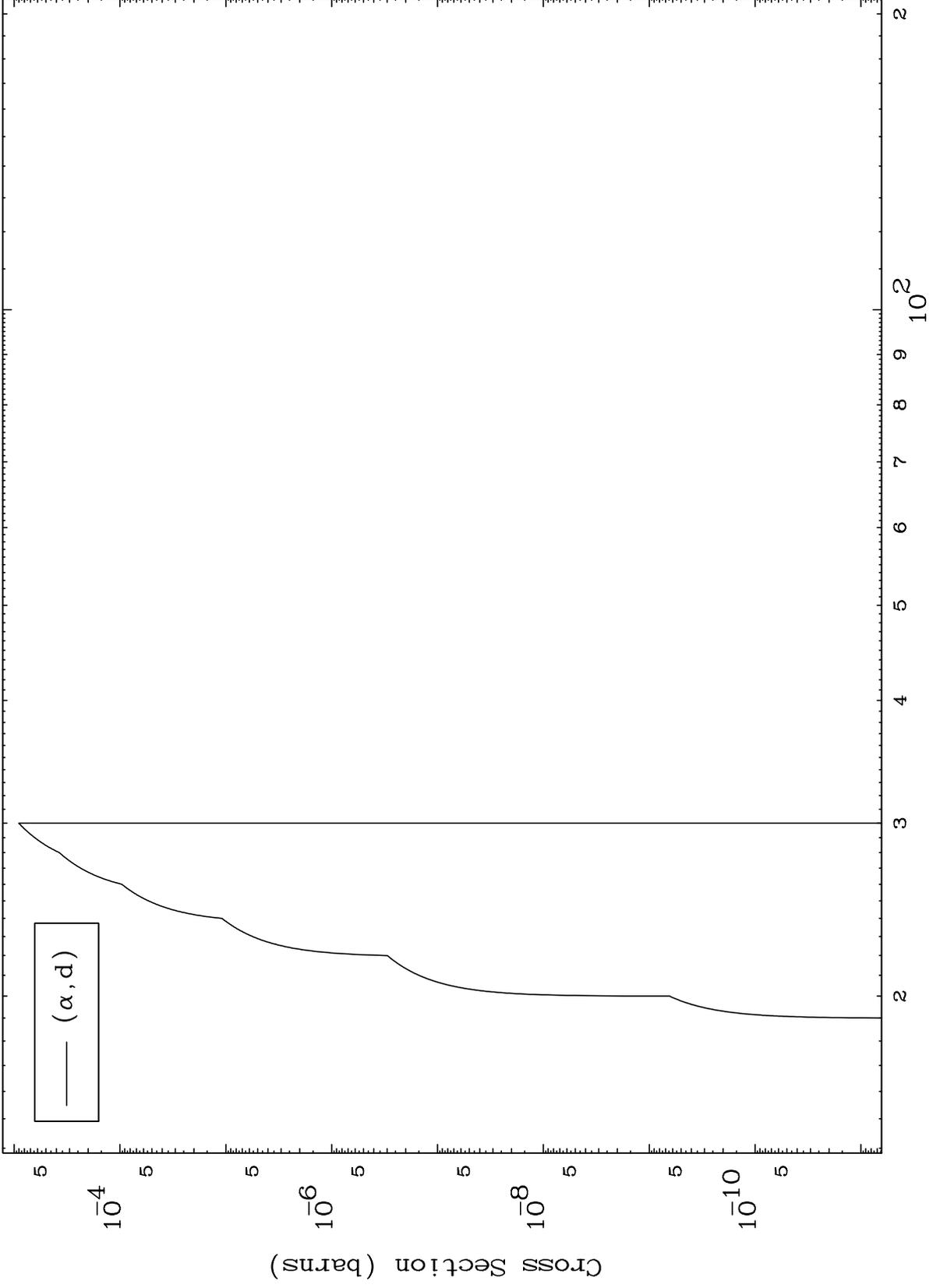
72-Hf-180

MAT 7243

( $\alpha, d$ ) Levels

72-Hf-180

0 Kelvin Cross Sections



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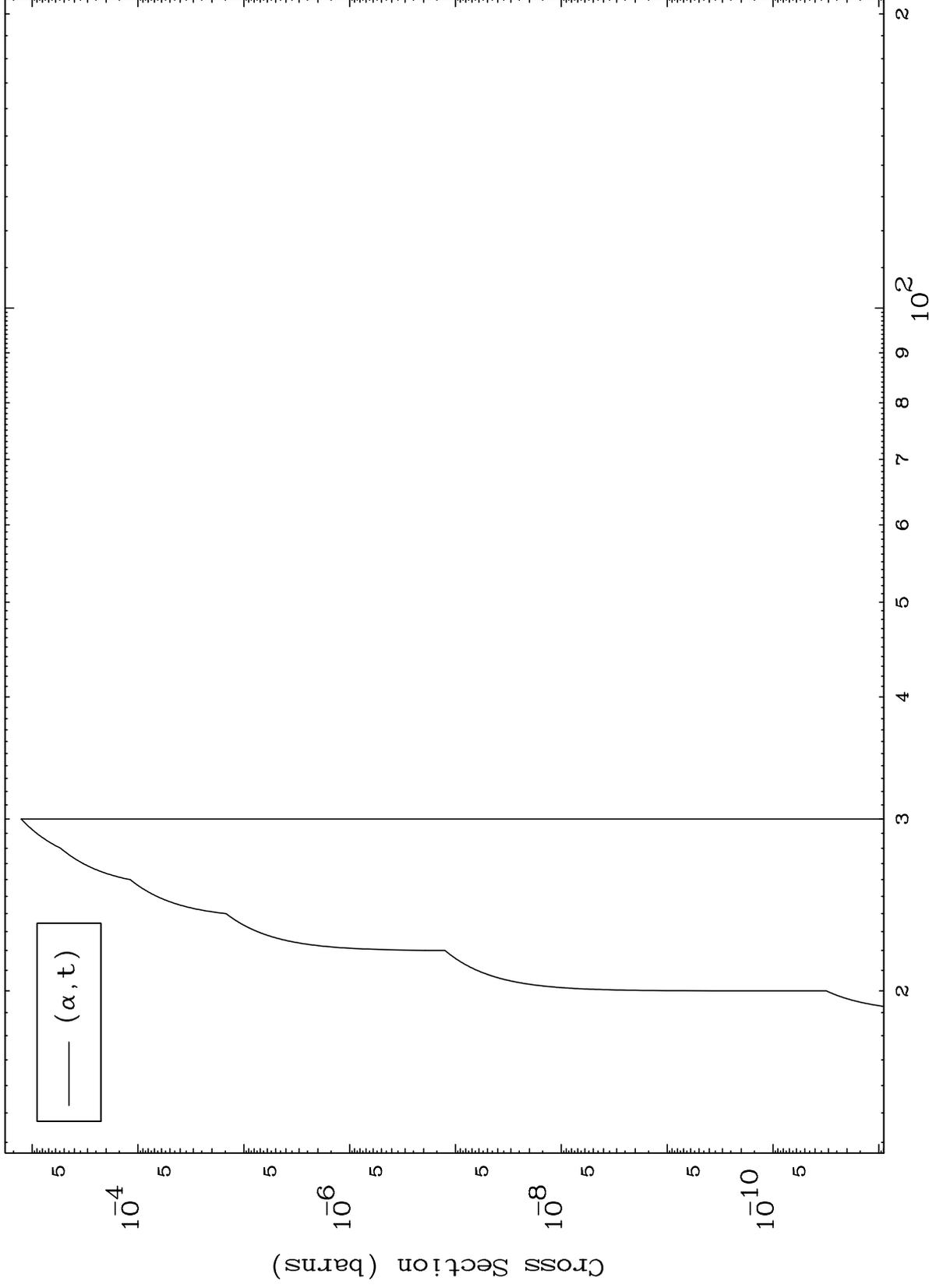
Incident Energy (MeV)

72-Hf-180

MAT 7243

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

72-Hf-180

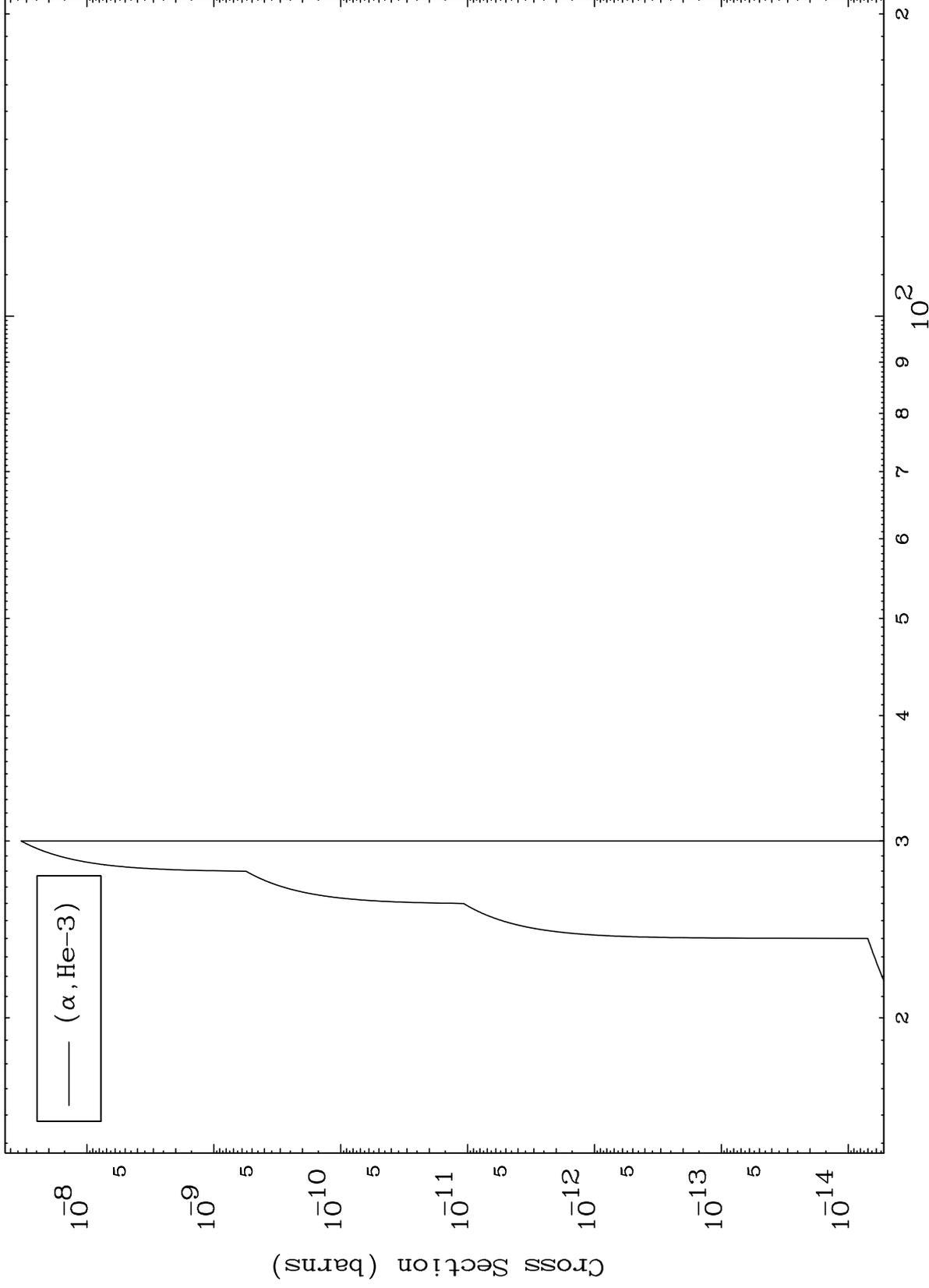


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Incident Energy (MeV)

72-Hf-180

0 Kelvin Cross Sections

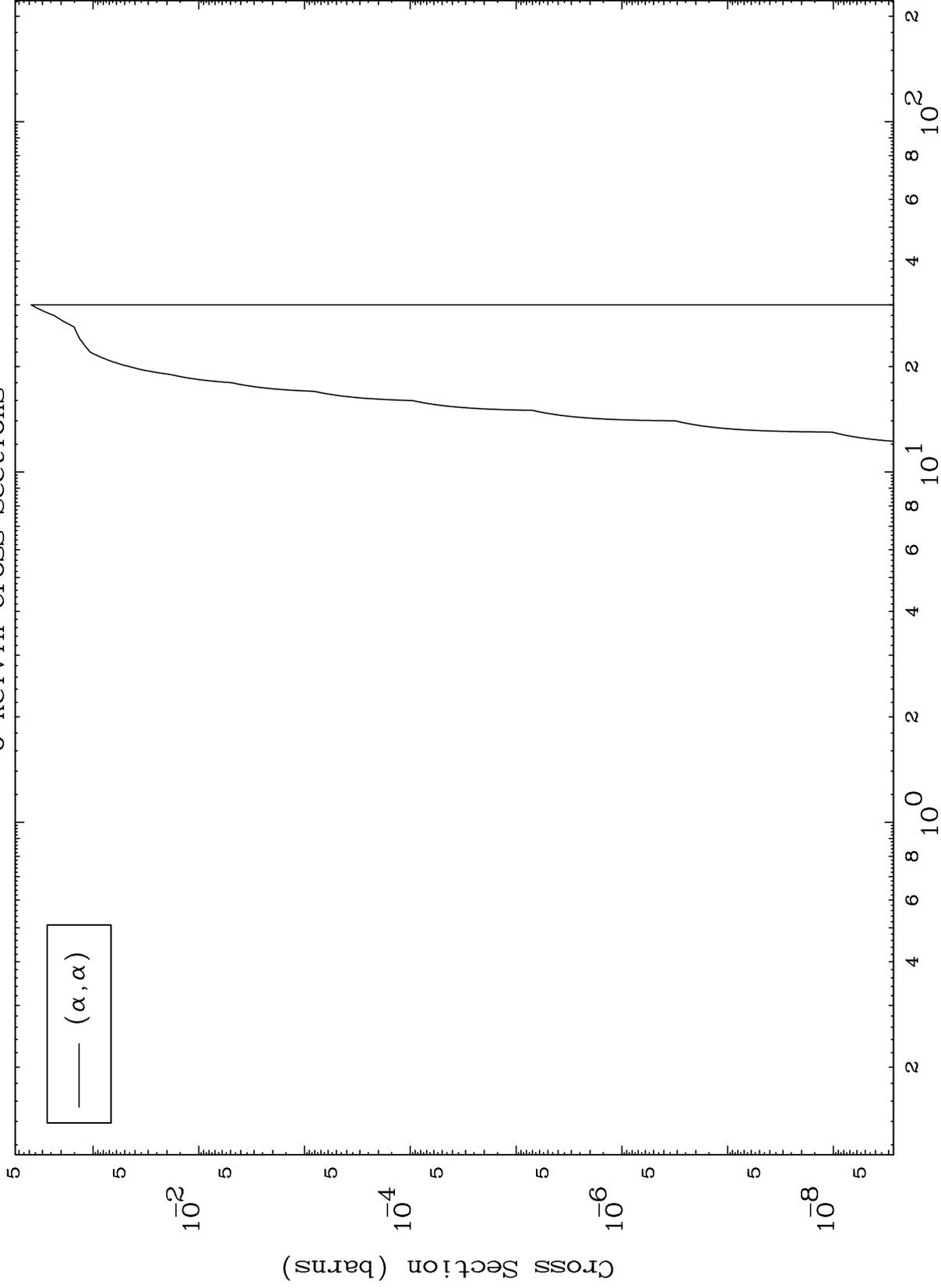


MAT 7243

( $\alpha, \alpha$ ) Levels

72-Hf-180

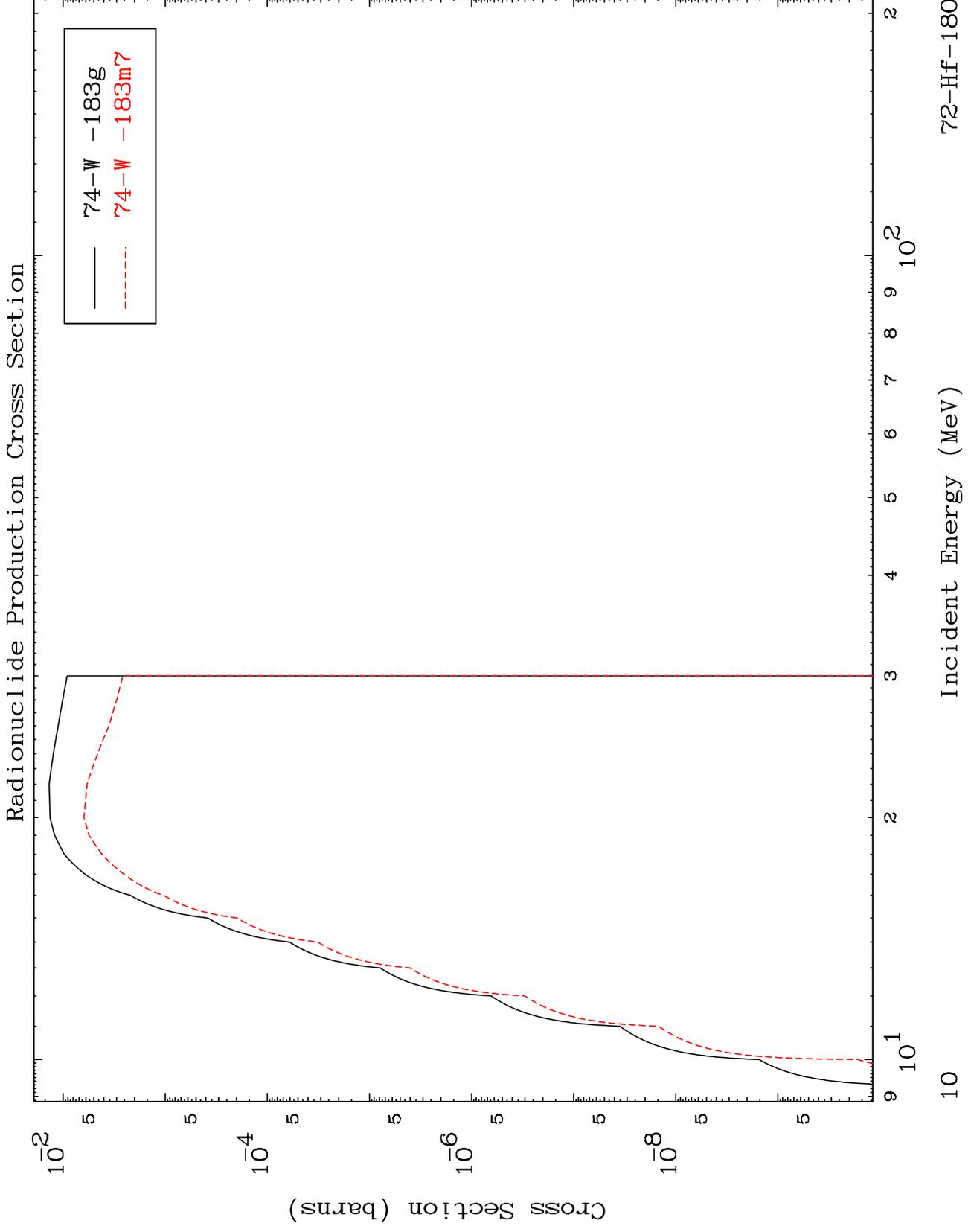
0 Kelvin Cross Sections



MAT 7243

$\alpha$  Inelastic

72-Hf-180

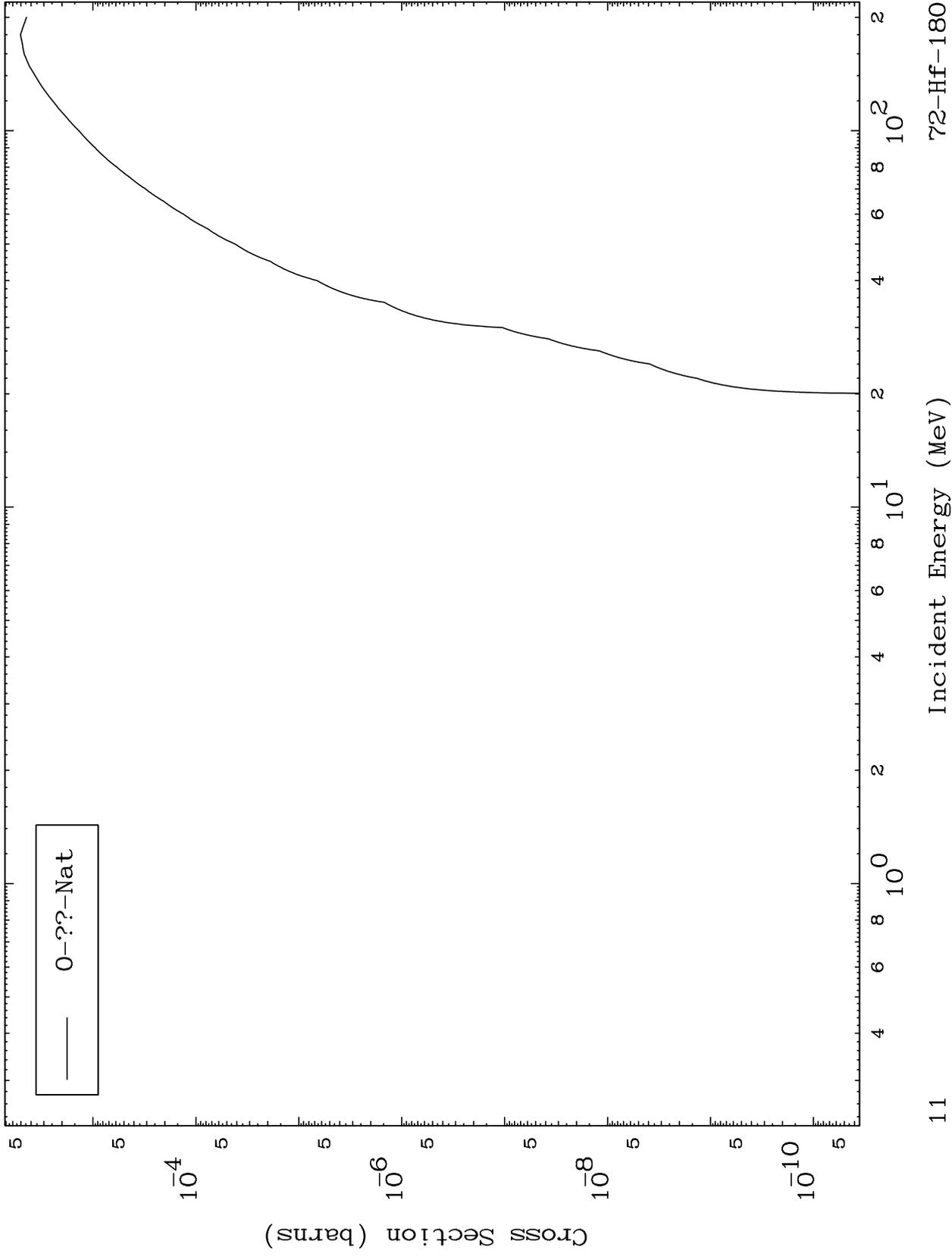


MAT 7243

$\alpha$  Fission

72-Hf-180

Radionuclide Production Cross Section

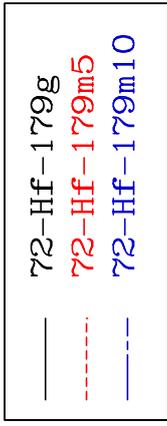
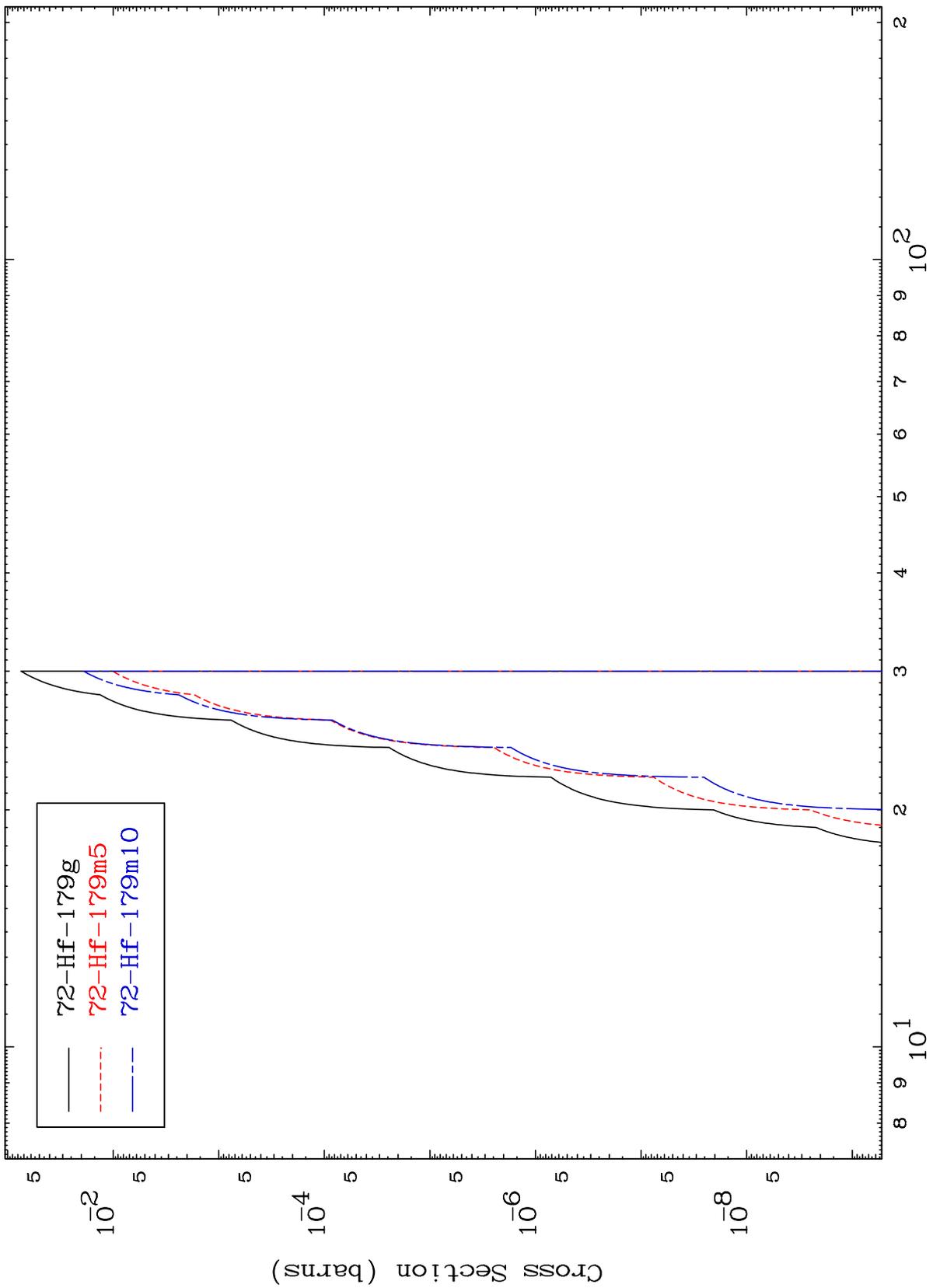


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$(\alpha, n')$   $\alpha$

72-Hf-180

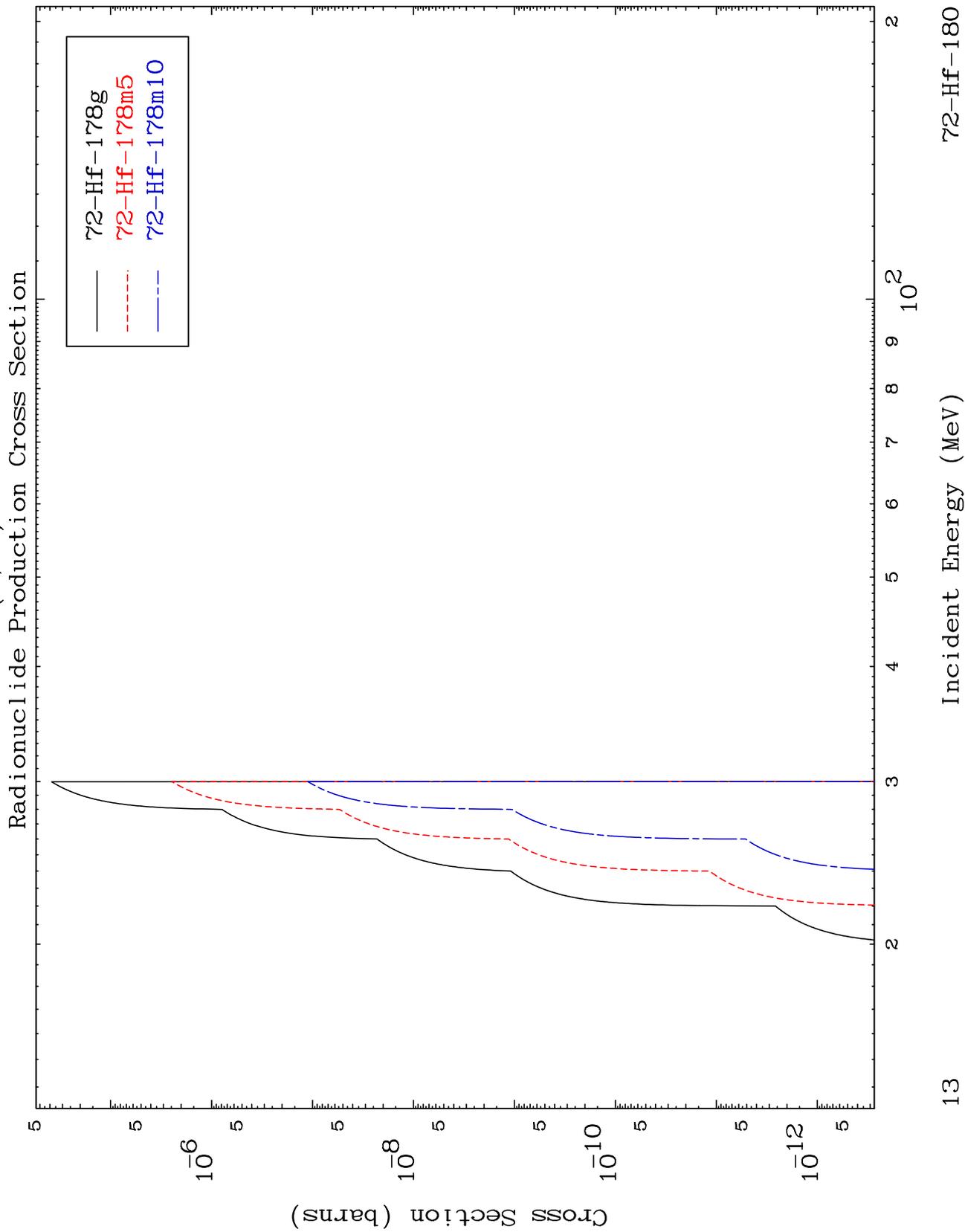
Radionuclide Production Cross Section



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Incident Energy (MeV)

72-Hf-180

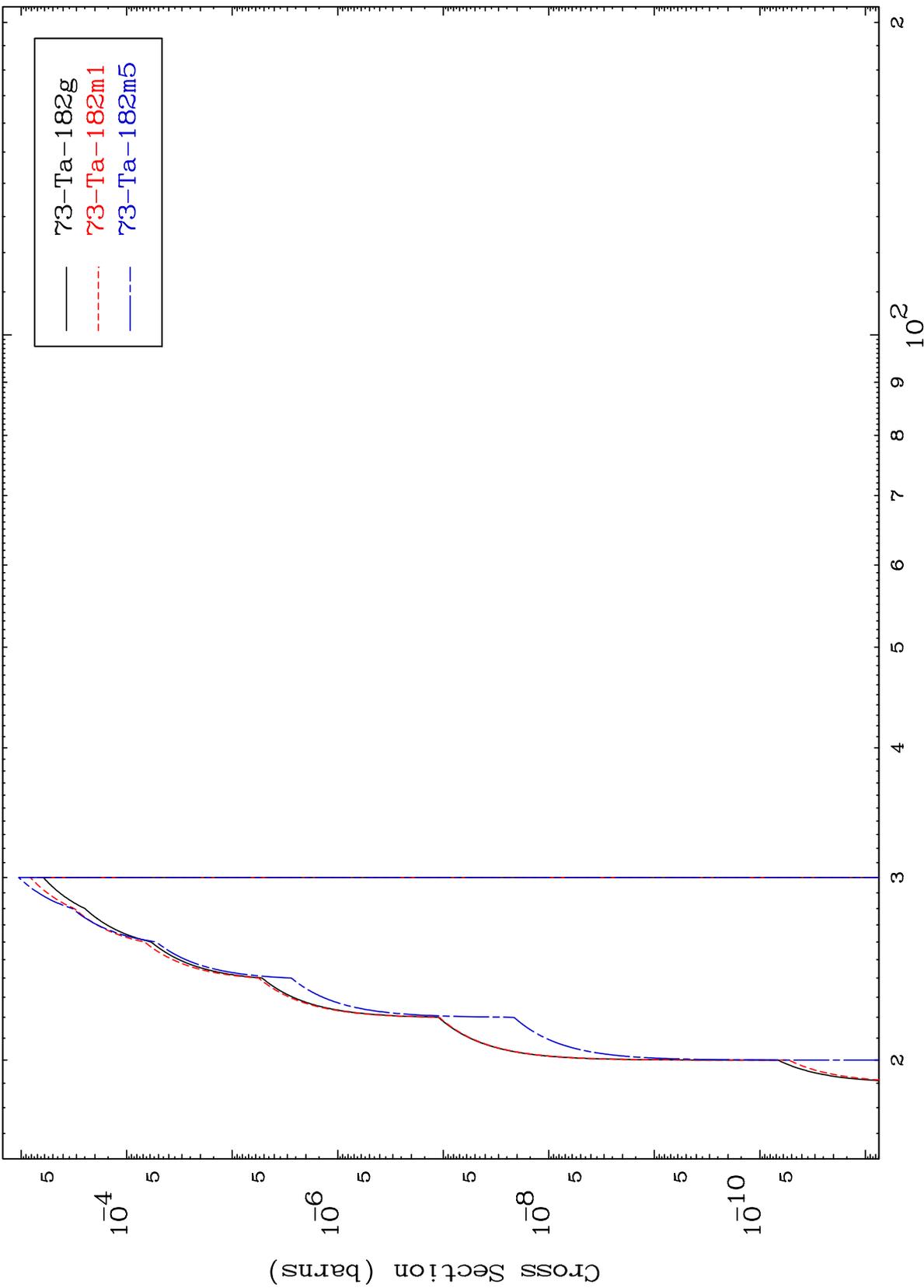


MAT 7243

$(\alpha, n')$  p

72-Hf-180

Radionuclide Production Cross Section



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Incident Energy (MeV)

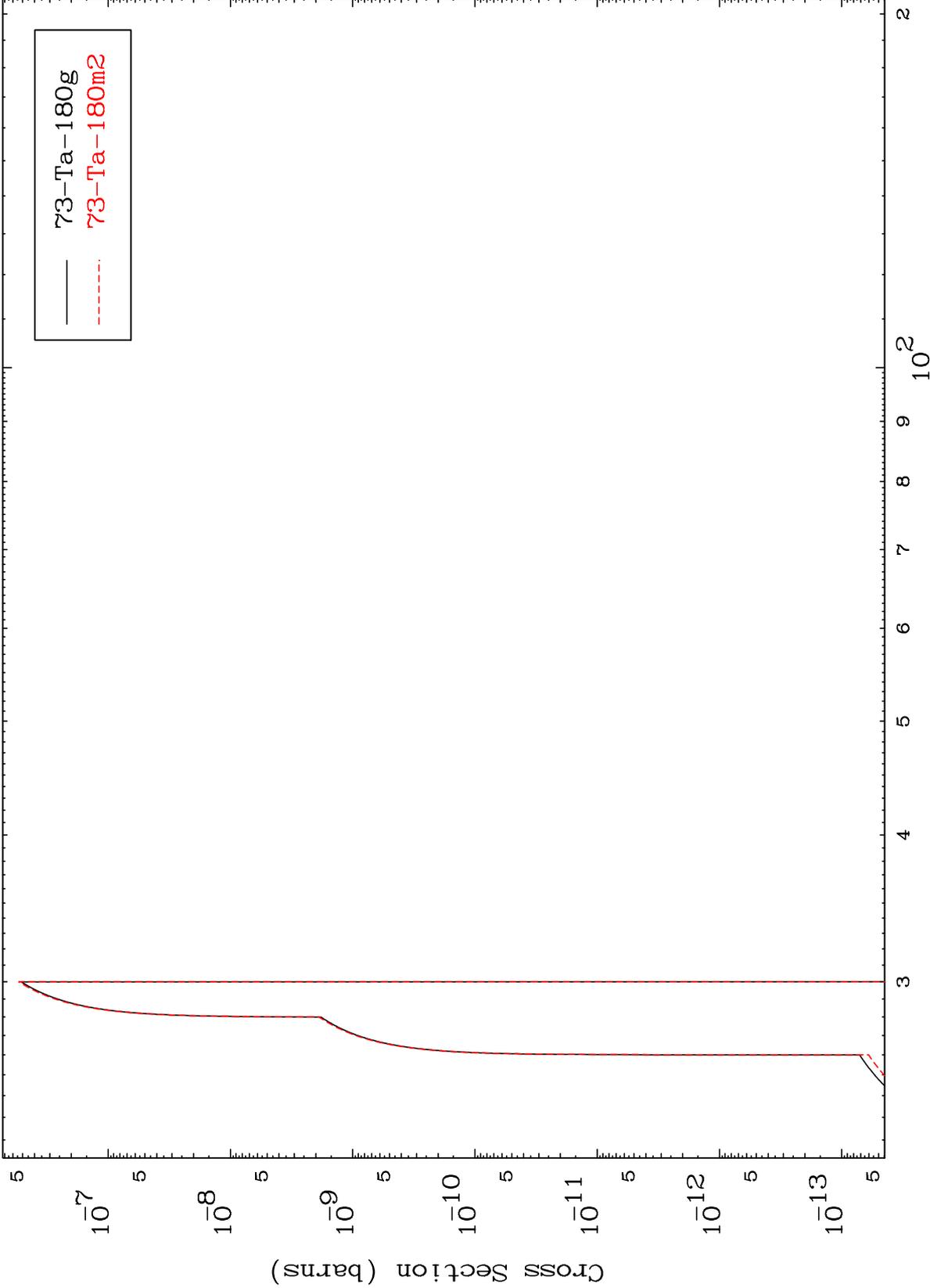
72-Hf-180

MAT 7243

$(\alpha, n')$  t

72-Hf-180

Radionuclide Production Cross Section



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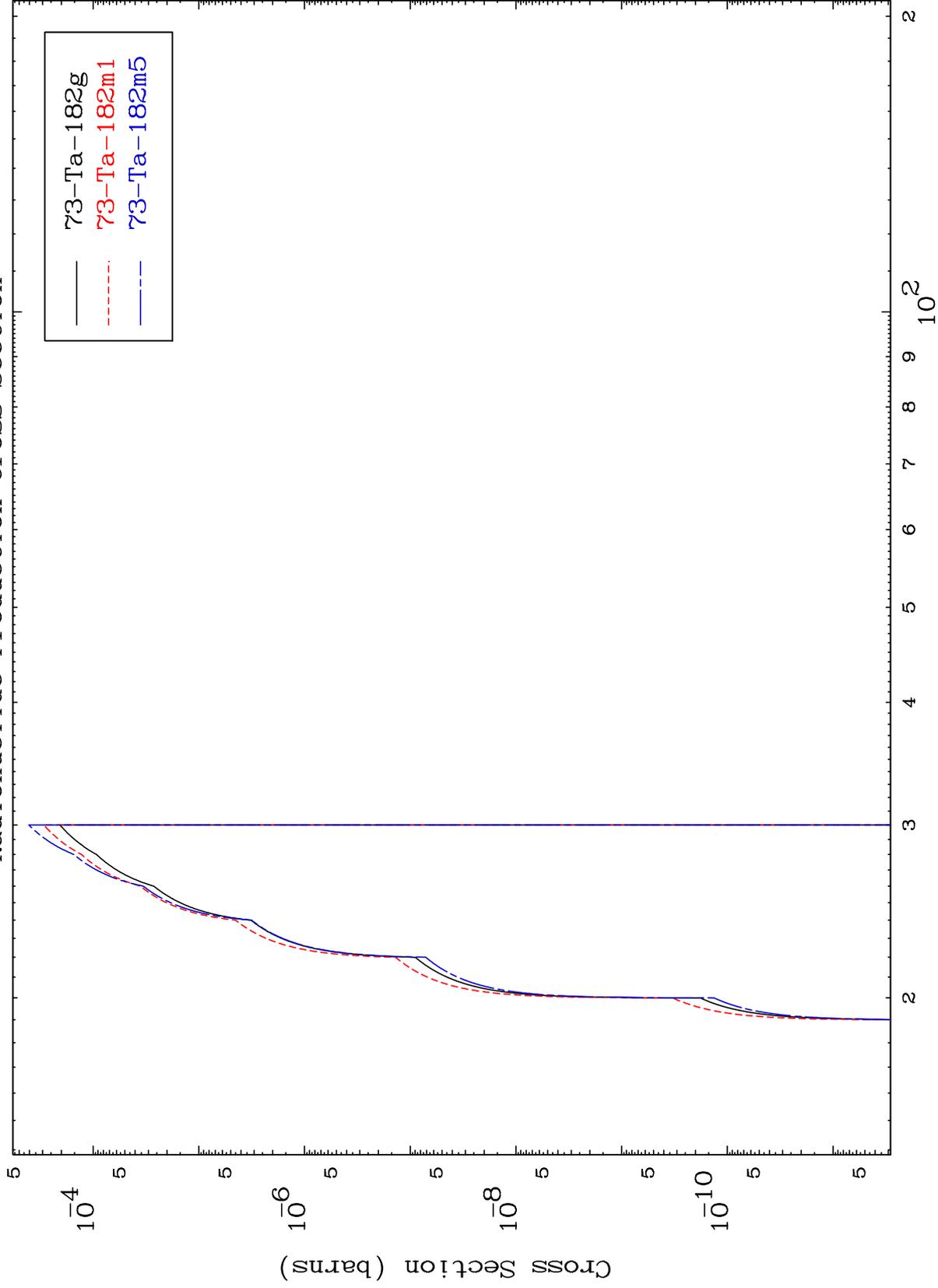
Incident Energy (MeV)

72-Hf-180

MAT 7243

72-Hf-180

( $\alpha, d$ )  
Radionuclide Production Cross Section



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Incident Energy (MeV)

72-Hf-180

MAT 7243

( $\alpha, \alpha$ )

72-Hf-180

Radionuclide Production Cross Section

