

Program EVALPLOT  
(Version 2018-1)

by

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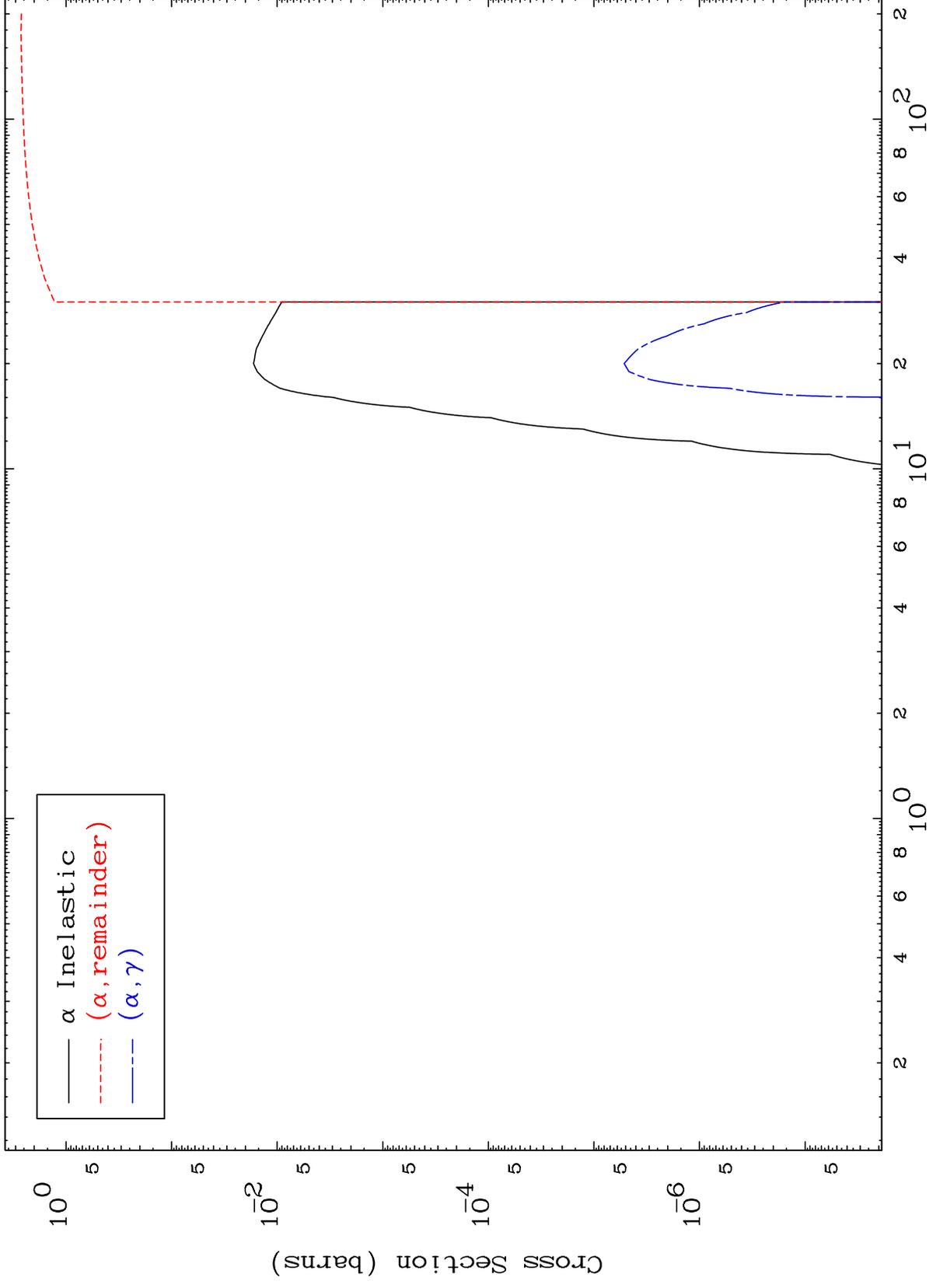
Press Mouse Button to Start

MAT 7250

$\alpha$  Major

72-Hf-182

0 Kelvin Cross Sections



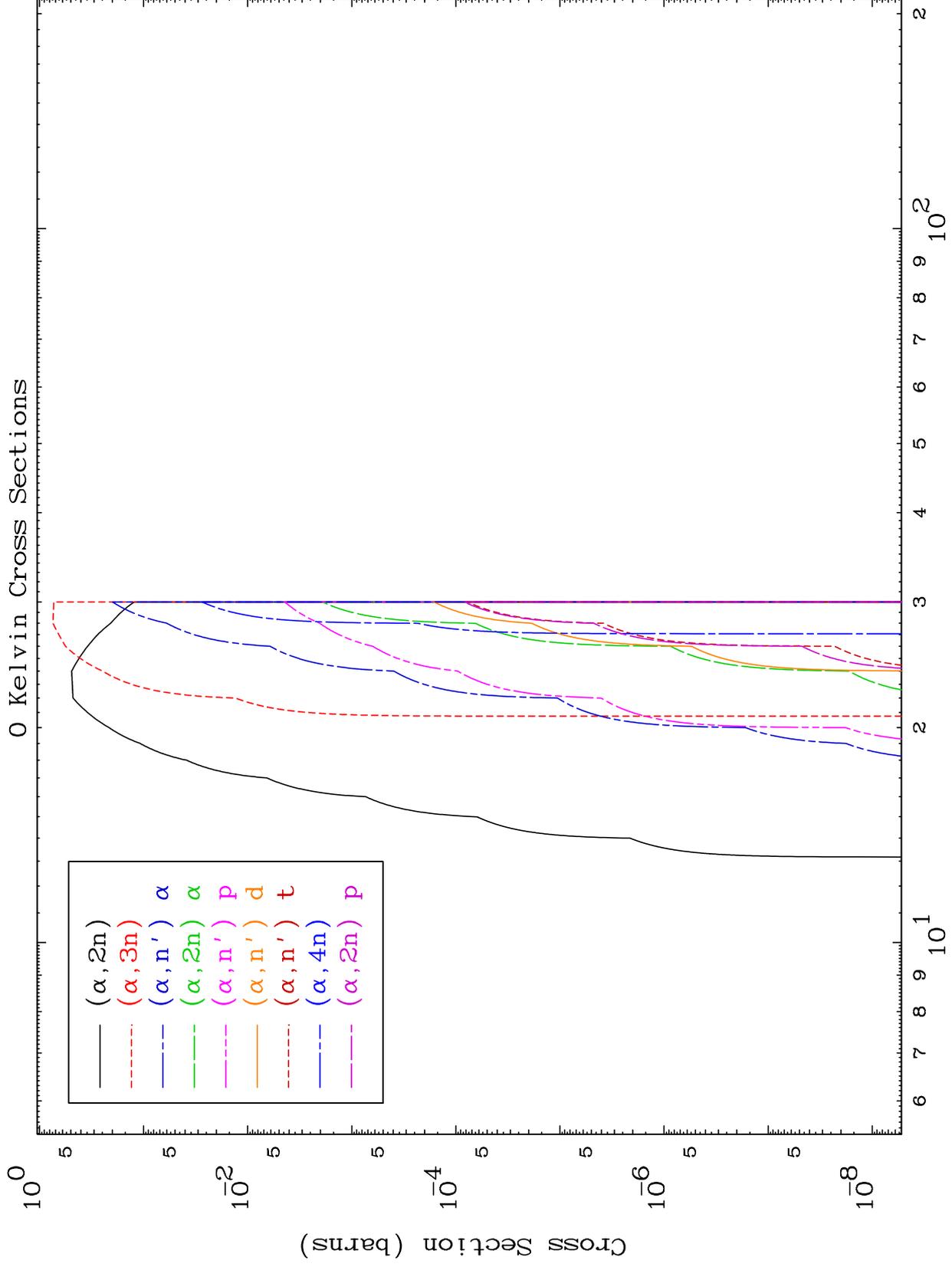
Incident Energy (MeV)

72-Hf-182

MAT 7250

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

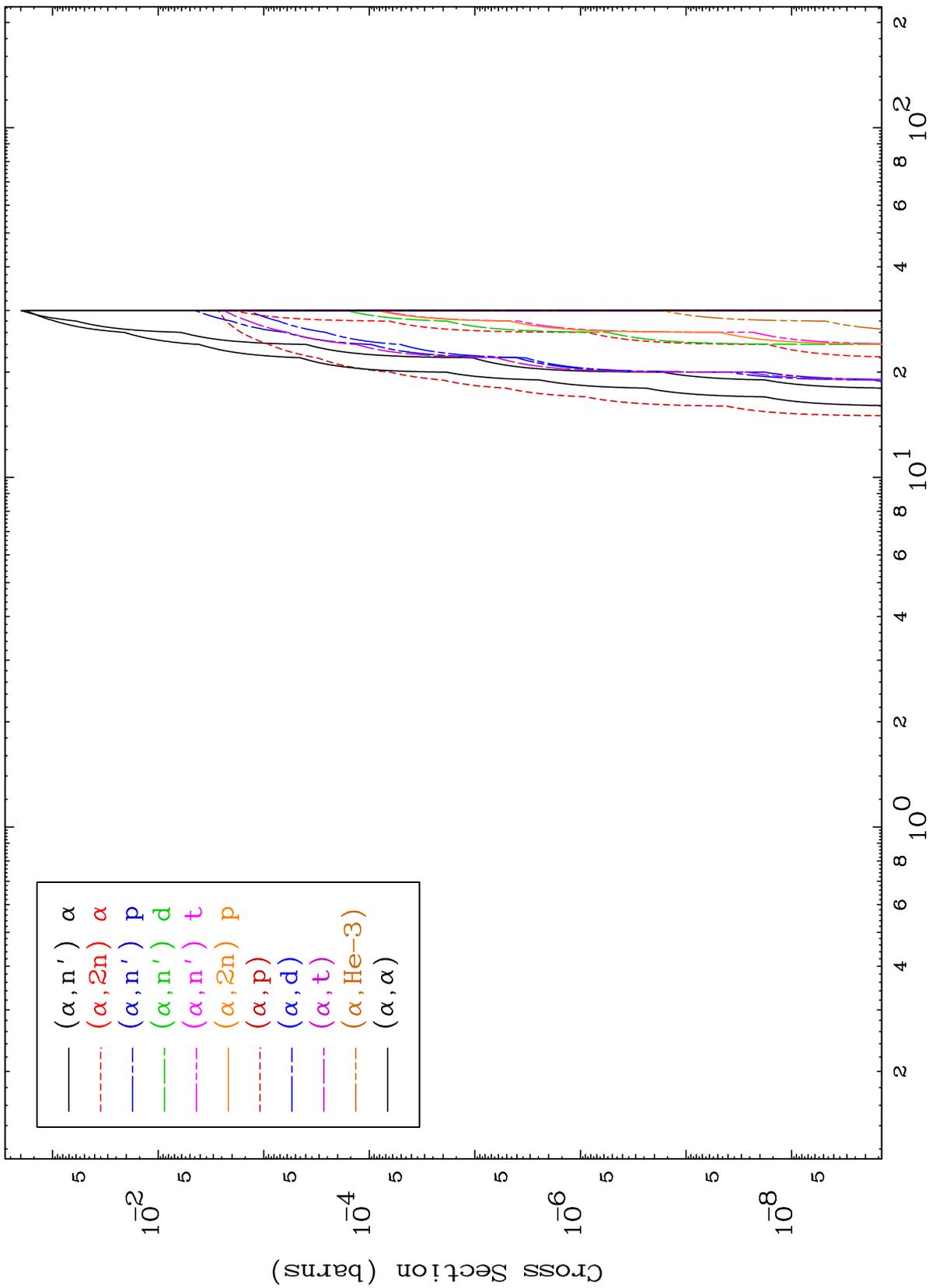
72-Hf-182



MAT 7250

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

72-Hf-182



72-Hf-182

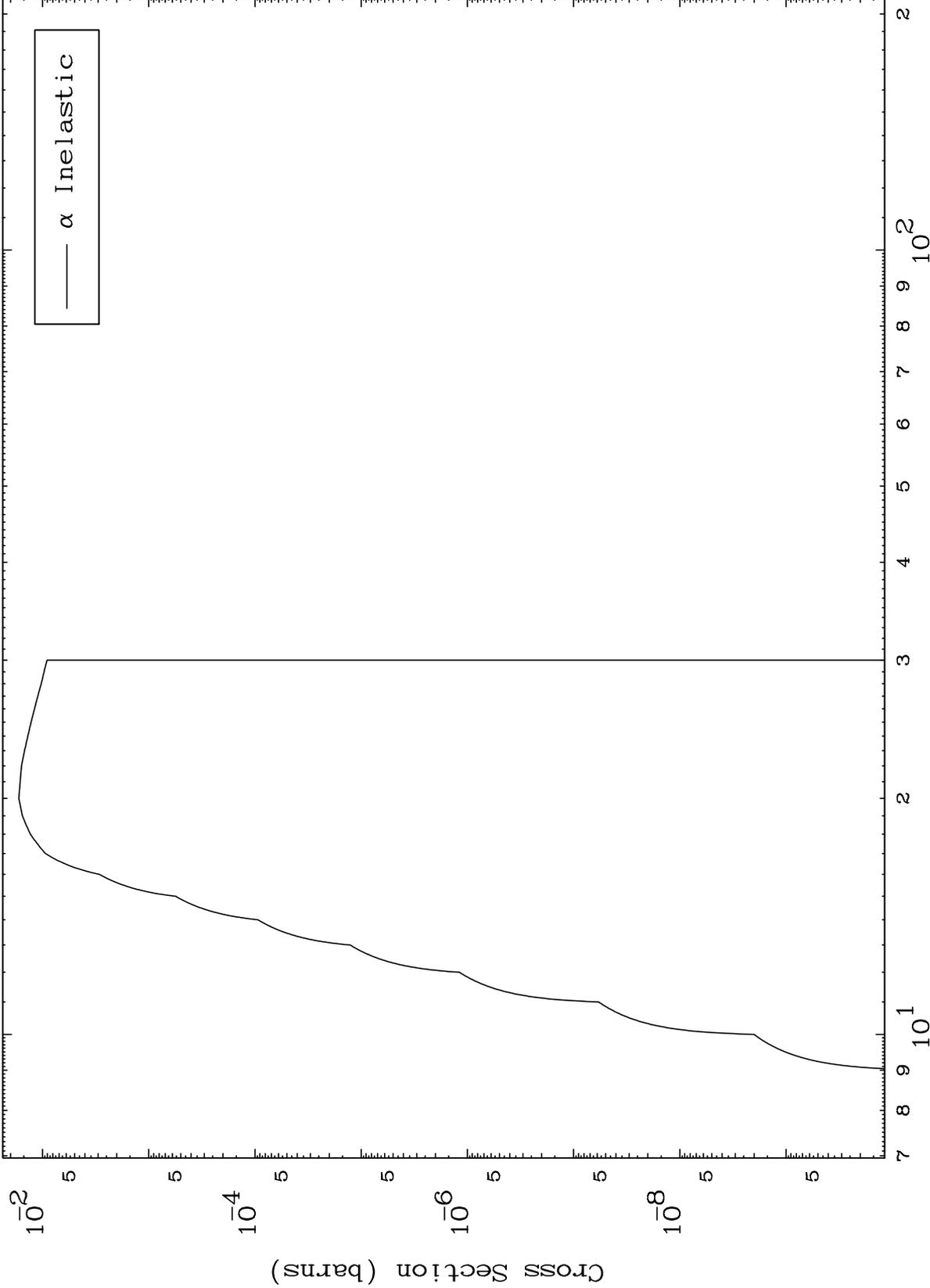
Incident Energy (MeV)

MAT 7250

( $\alpha, n'$ ) Level

72-Hf-182

0 Kelvin Cross Sections

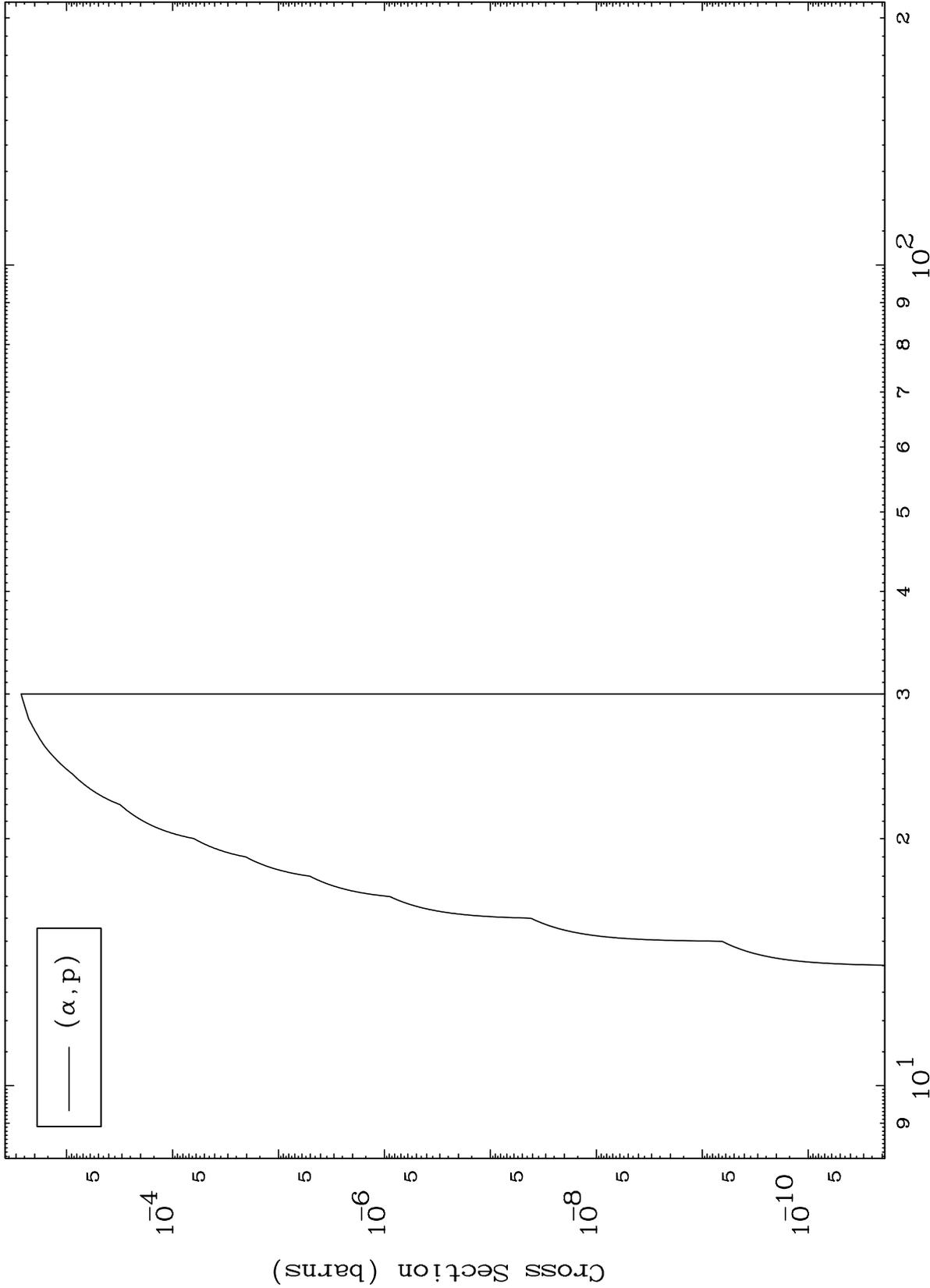


MAT 7250

( $\alpha, p$ ) Levels

72-Hf-182

0 Kelvin Cross Sections



Incident Energy (MeV)

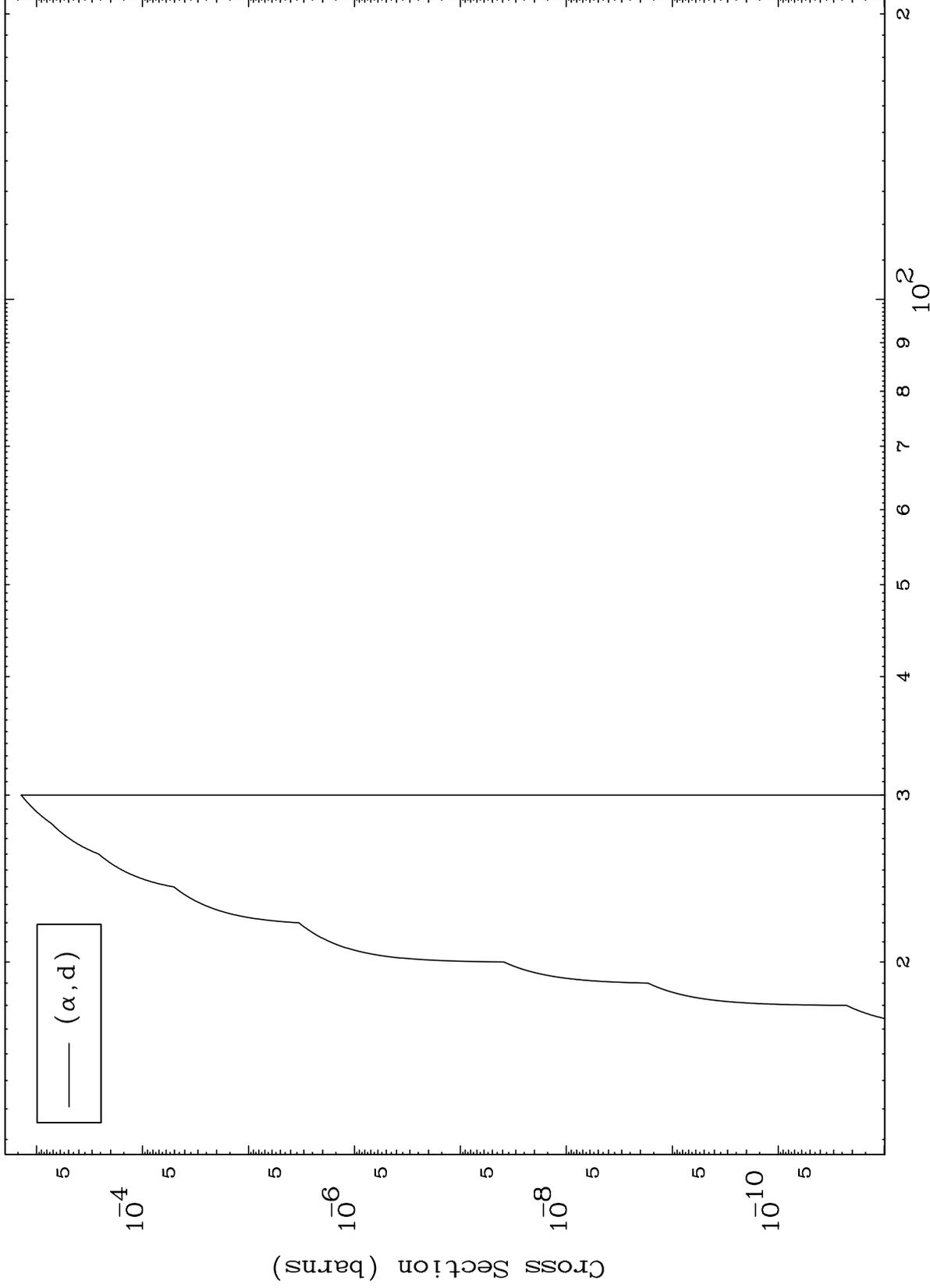
72-Hf-182

5

MAT 7250

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

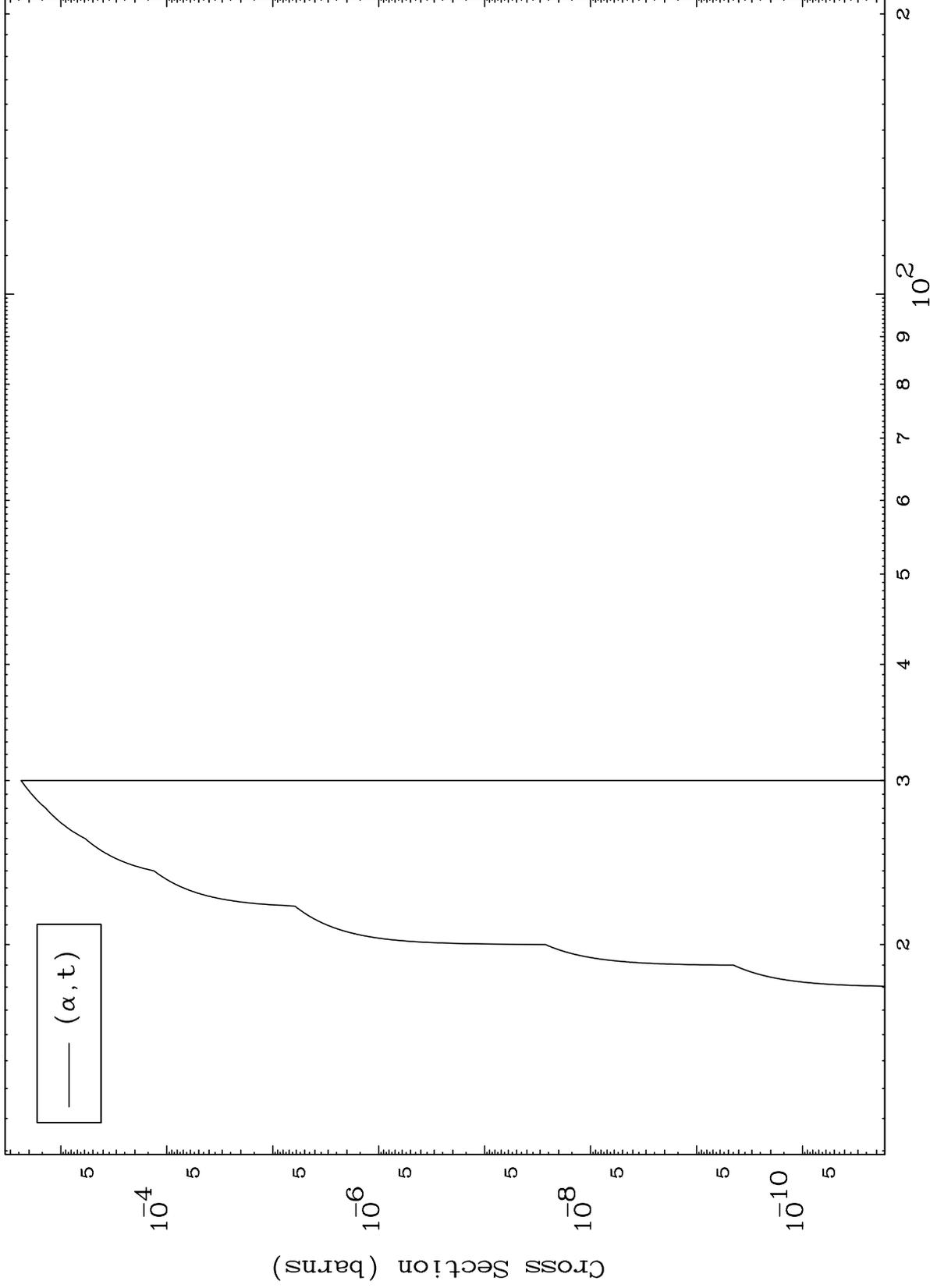
72-Hf-182

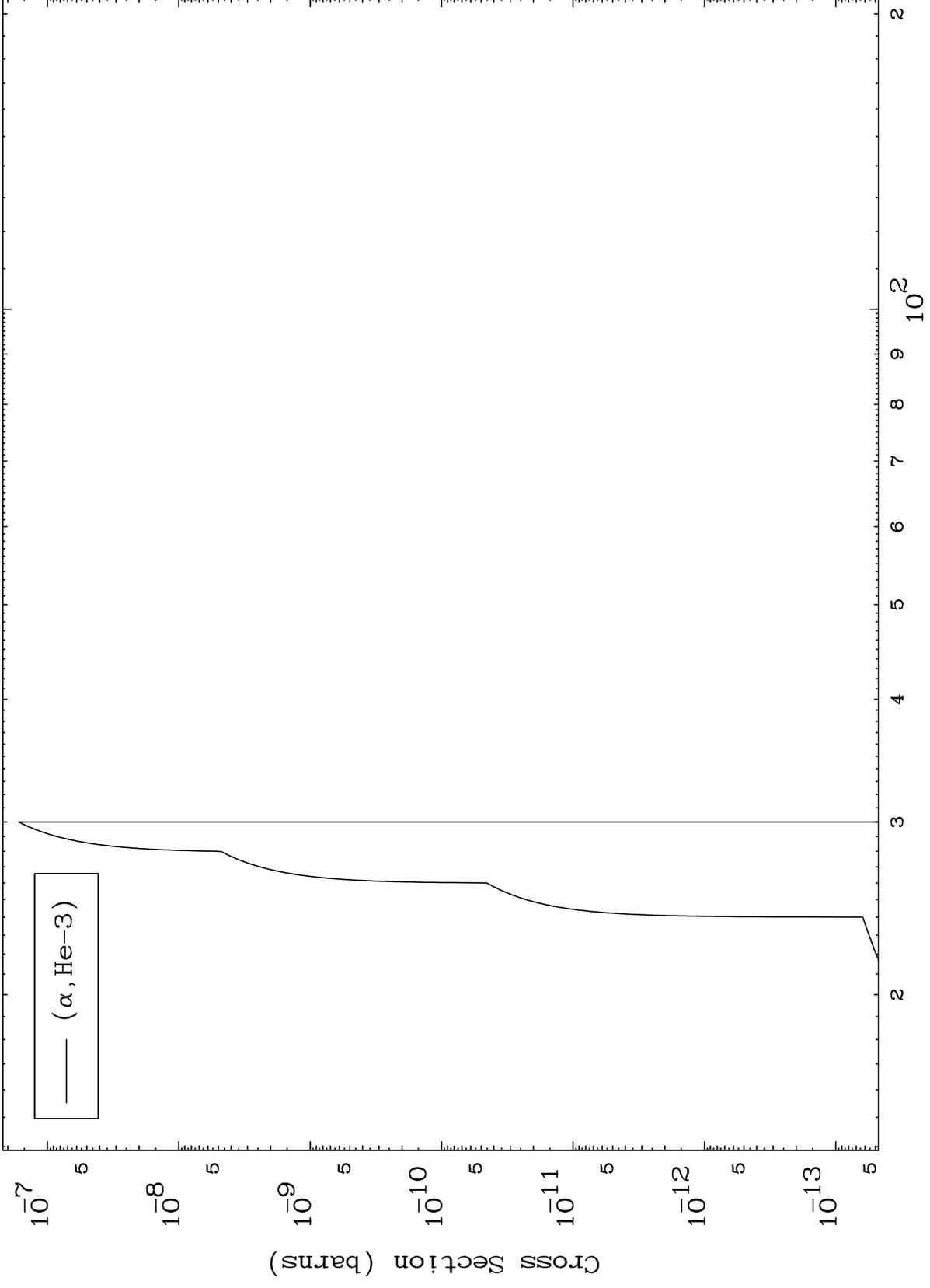


Incident Energy (MeV)

72-Hf-182

6



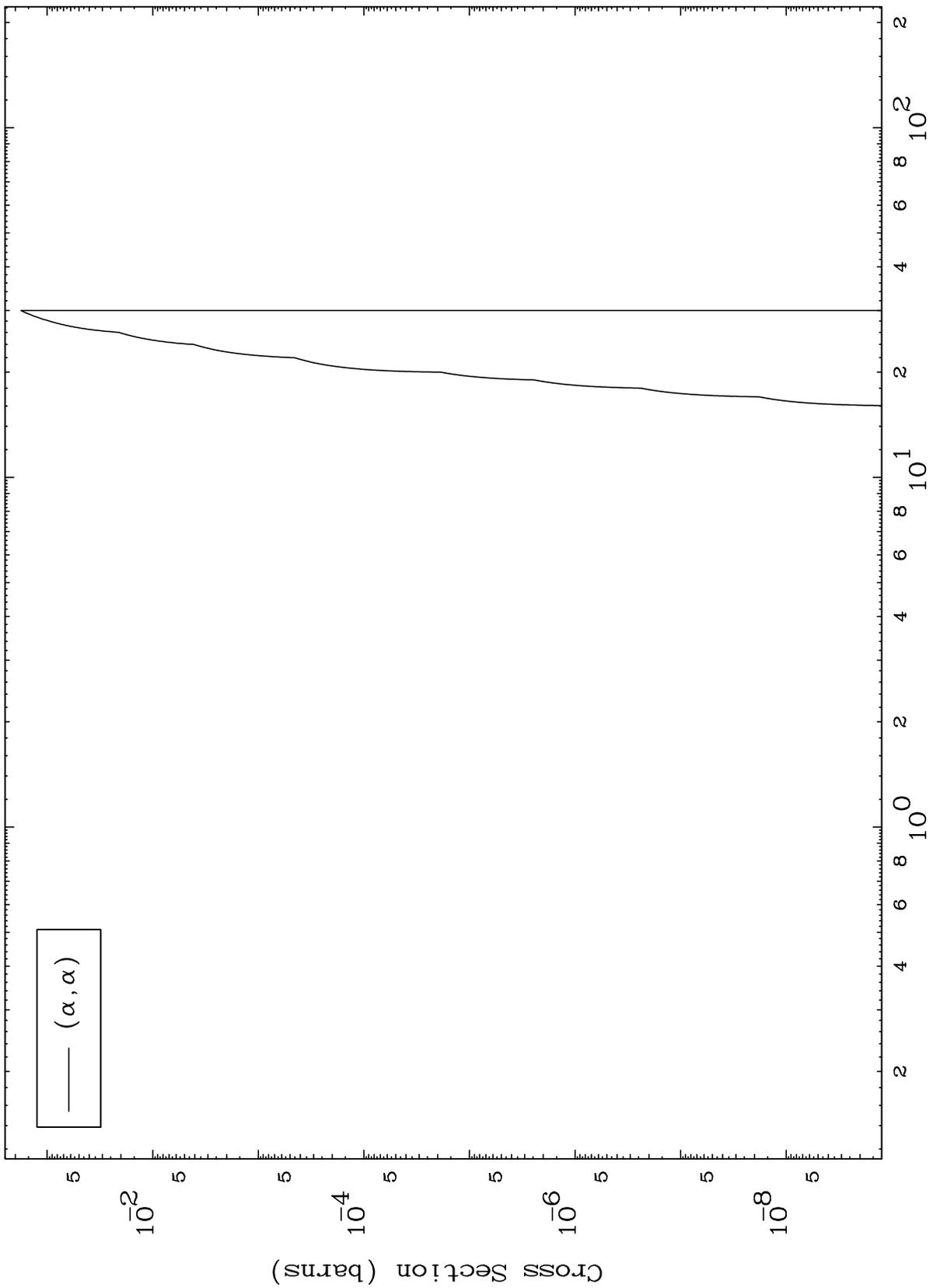


MAT 7250

( $\alpha, \alpha$ ) Levels

72-Hf-182

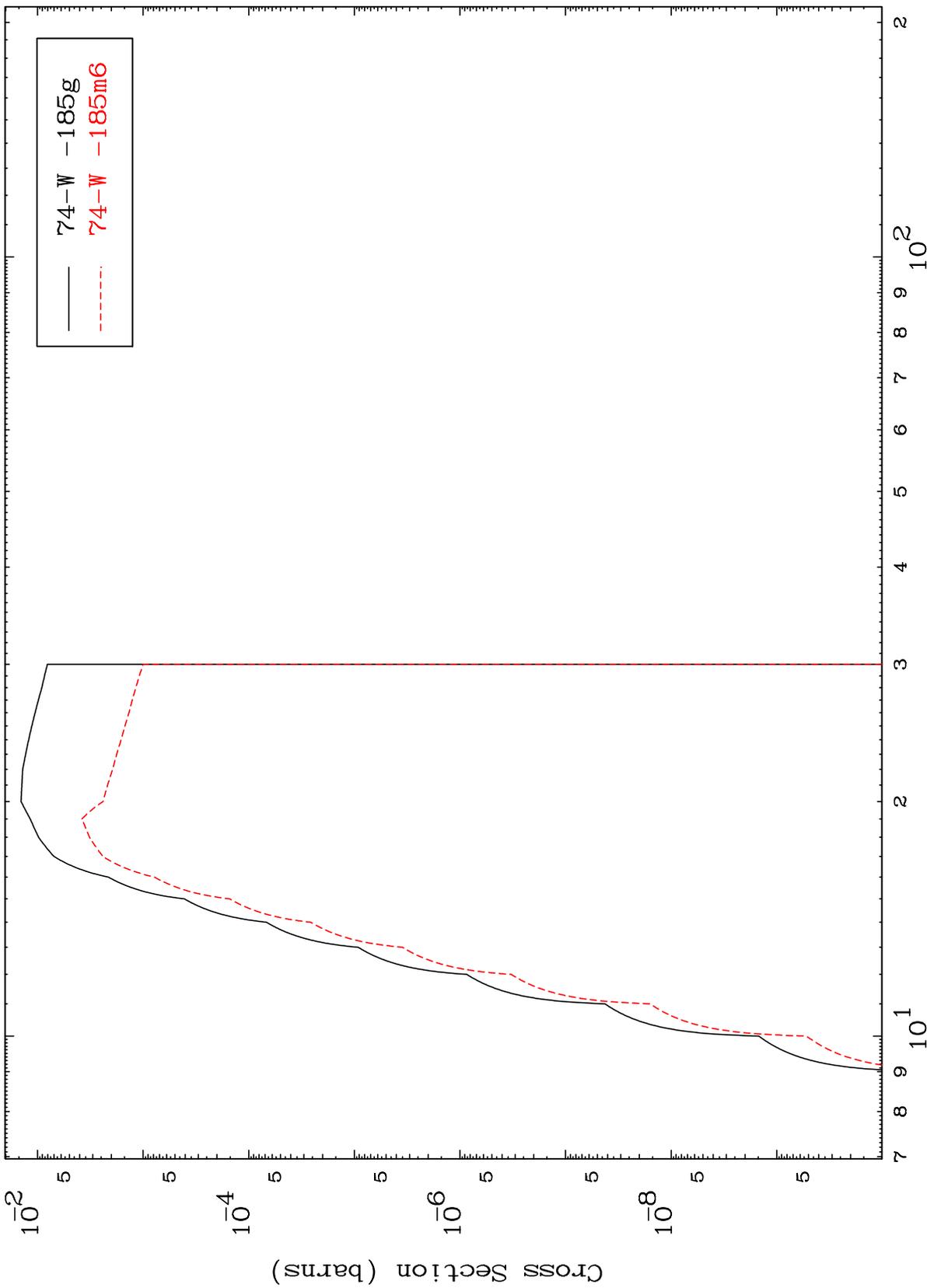
0 Kelvin Cross Sections



MAT 7250

72-Hf-182

Radionuclide Production Cross Section  
 $\alpha$  Inelastic



72-Hf-182

Incident Energy (MeV)

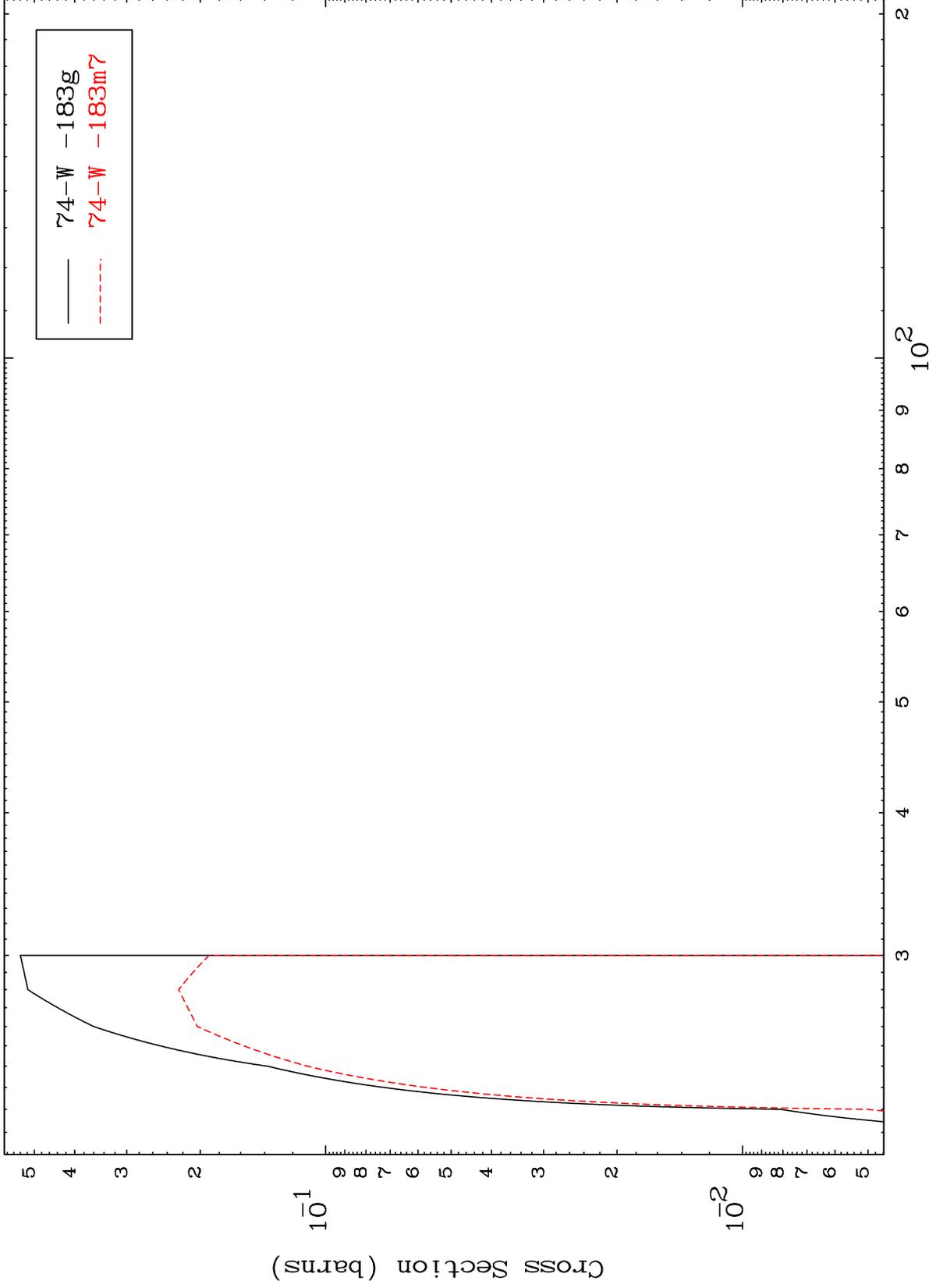
10

MAT 7250

( $\alpha, 3n$ )

<sup>72</sup>Hf-182

Radionuclide Production Cross Section



11

Incident Energy (MeV)

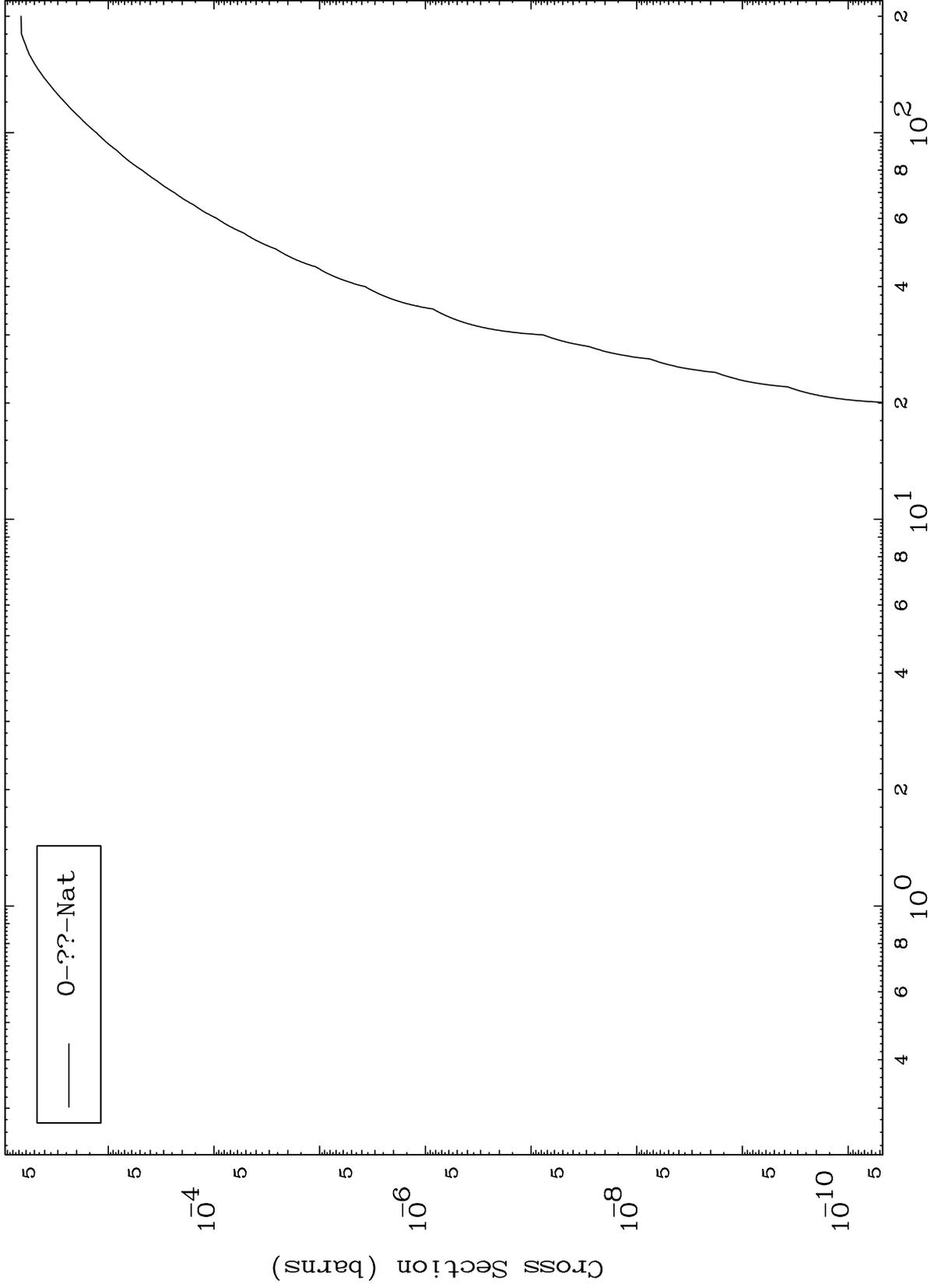
<sup>72</sup>Hf-182

MAT 7250

$\alpha$  Fission

72-Hf-182

Radionuclide Production Cross Section



12

Incident Energy (MeV)

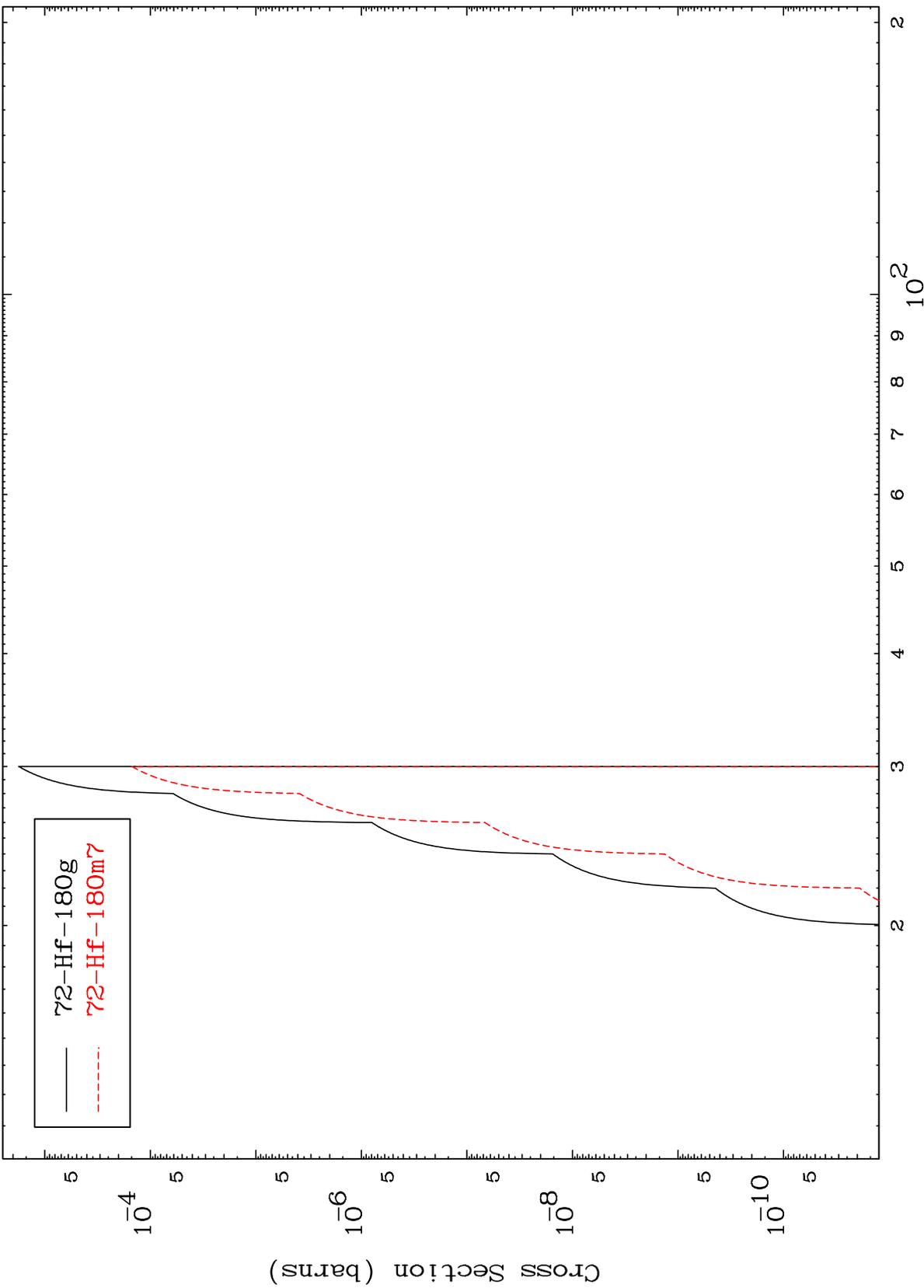
72-Hf-182

MAT 7250

( $\alpha, 2n$ )  $\alpha$

$^{72}\text{Hf}-182$

Radionuclide Production Cross Section

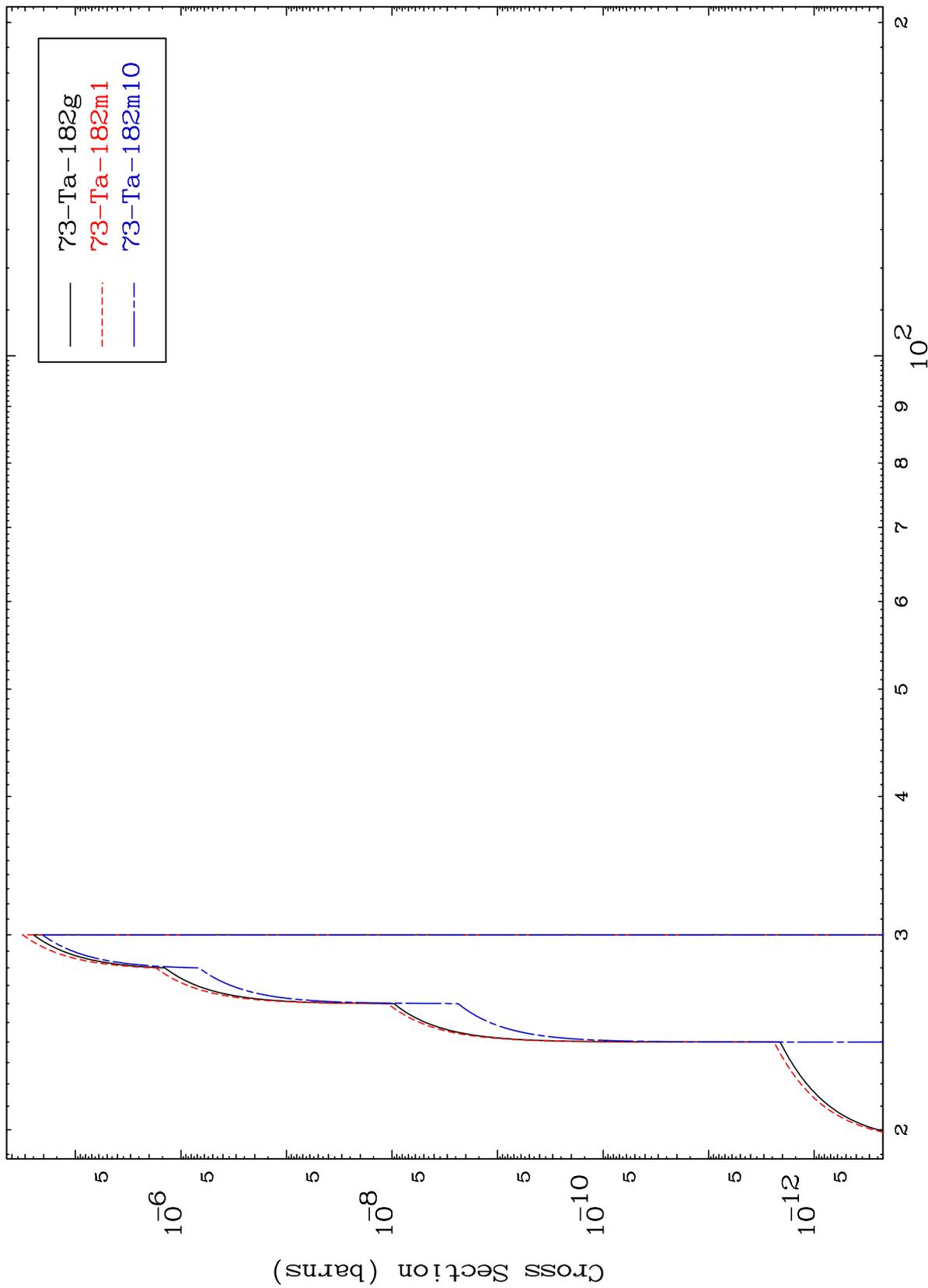


MAT 7250

$(\alpha, n')$  t

72-Hf-182

Radionuclide Production Cross Section



14

Incident Energy (MeV)

72-Hf-182

MAT 7250

( $\alpha, \alpha$ )

<sup>72</sup>Hf-182

Radionuclide Production Cross Section

