

Program EVALPLOT  
(Version 2018-1)

by

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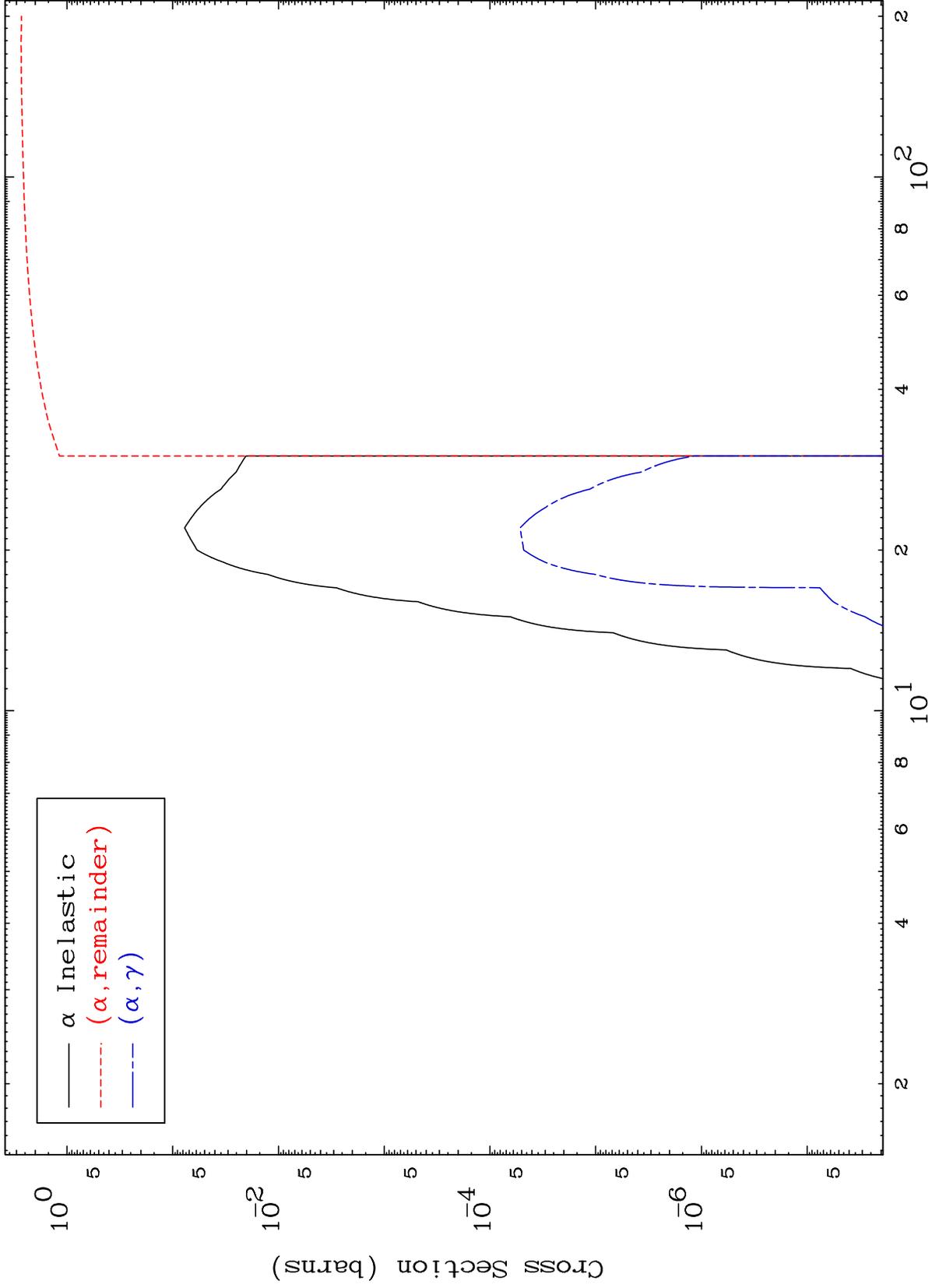
Press Mouse Button to Start

MAT 7726

$\alpha$  Major

77-Ir-191

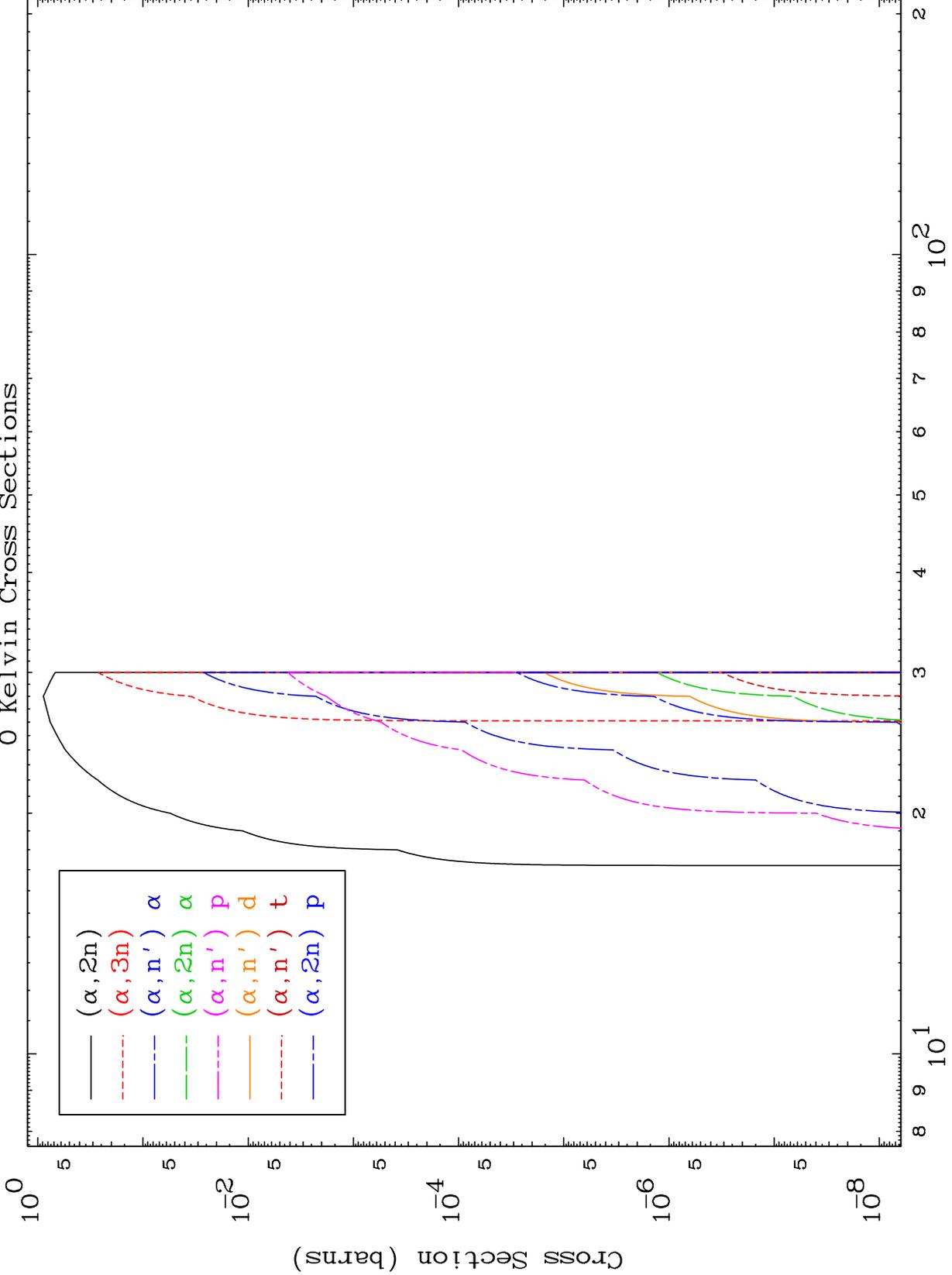
0 Kelvin Cross Sections



MAT 7726

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

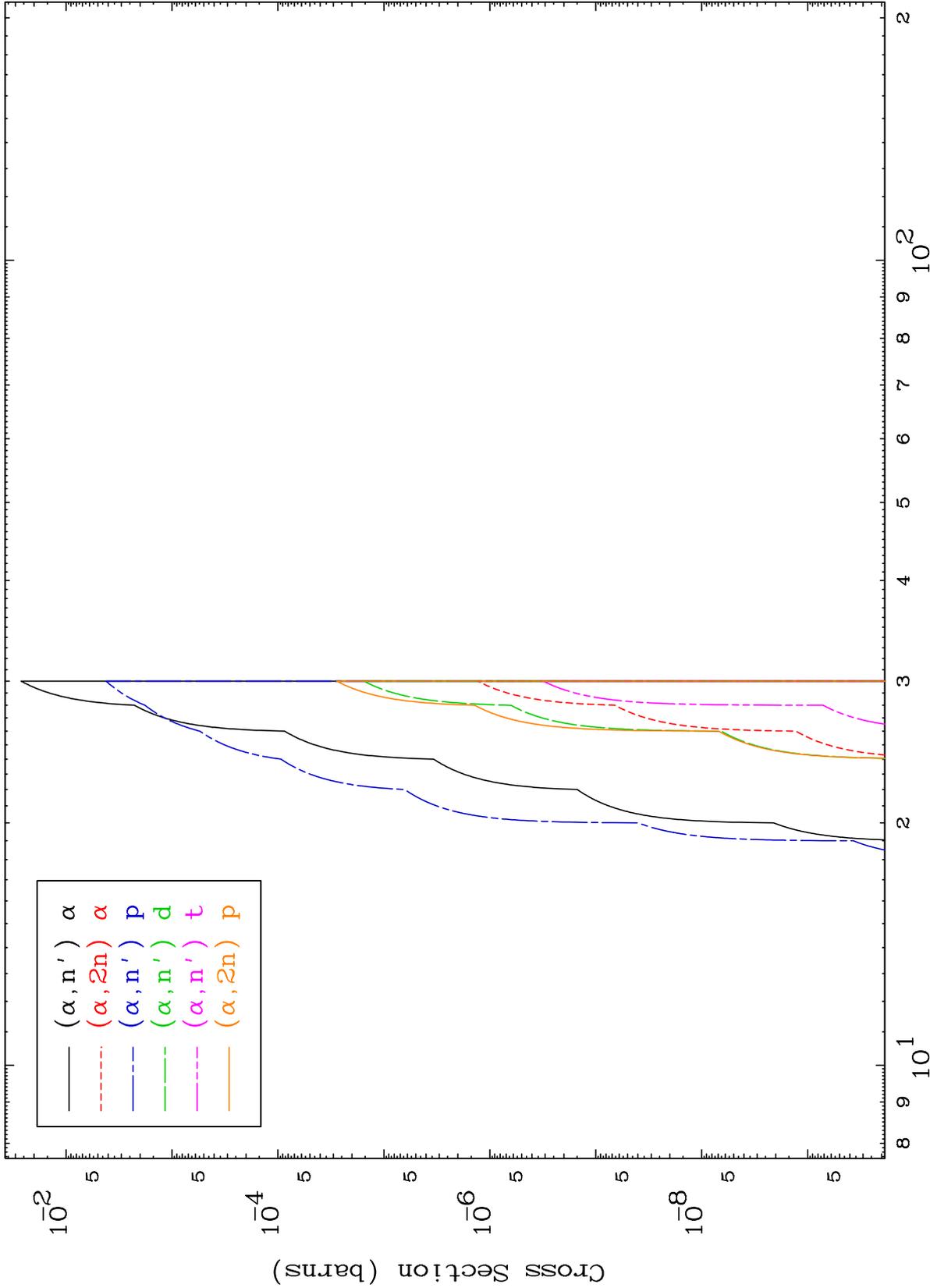
77-Ir-191

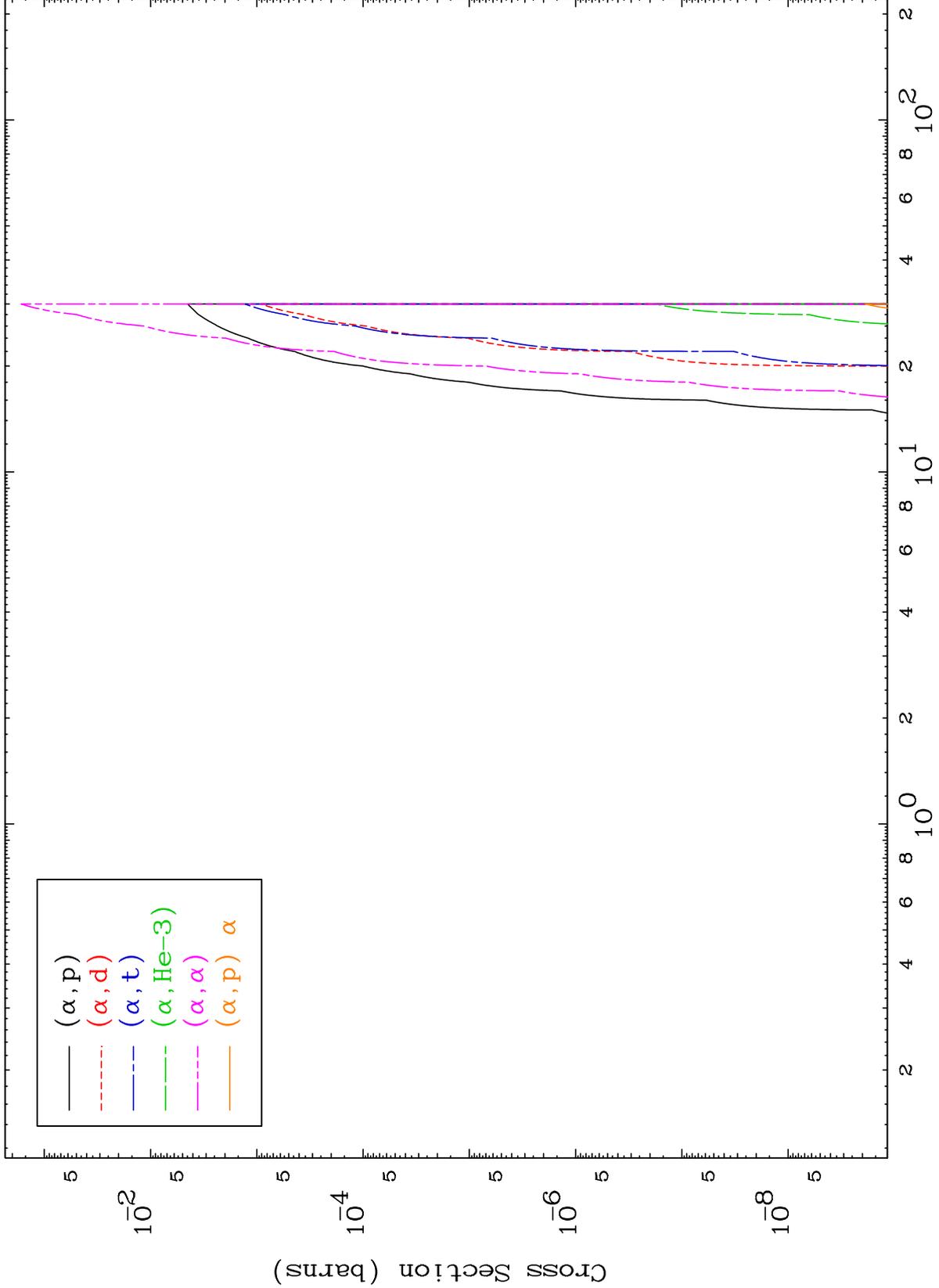


Incident Energy (MeV)

77-Ir-191

2



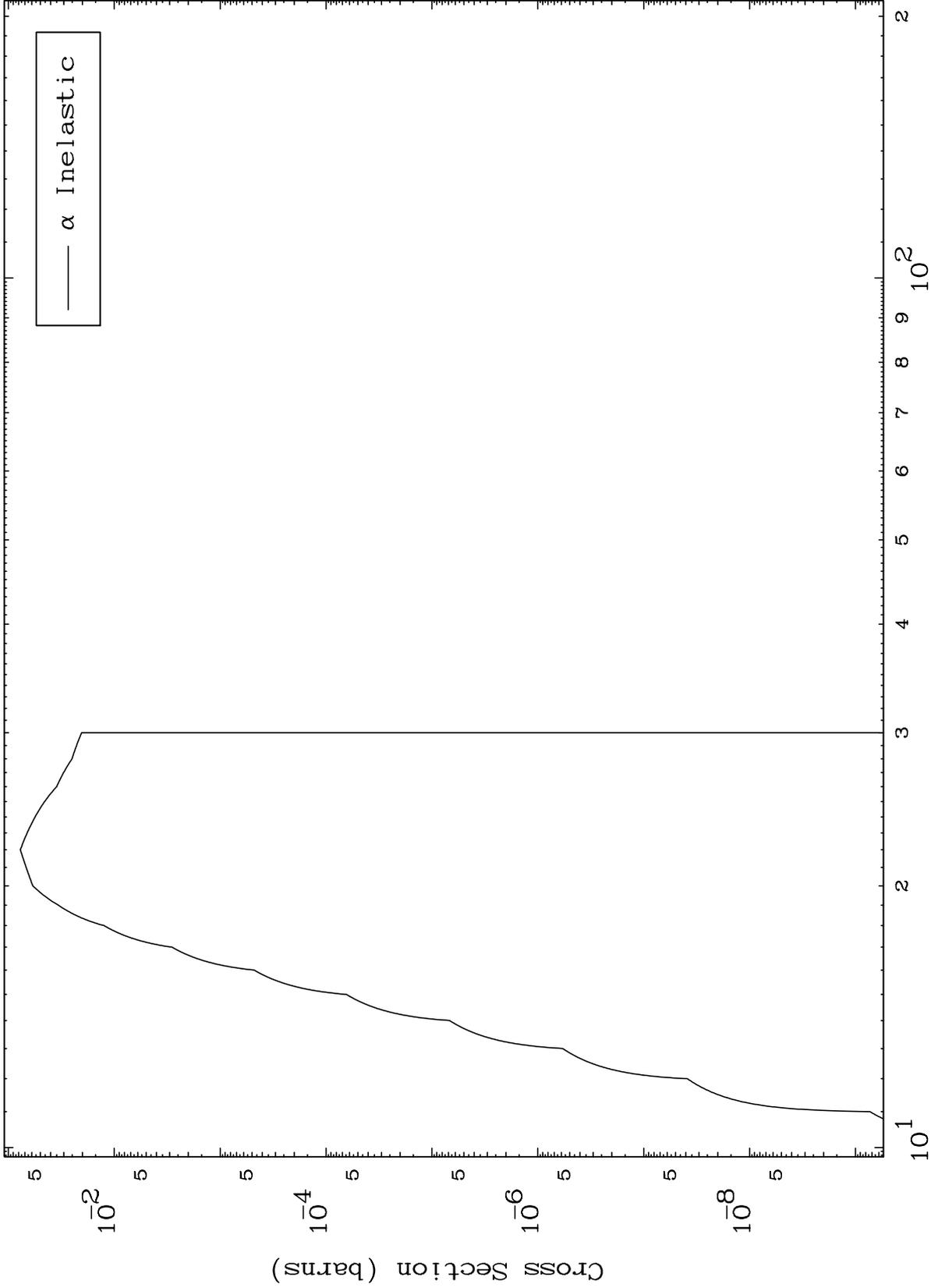


MAT 7726

( $\alpha, n'$ ) Level

77-Ir-191

0 Kelvin Cross Sections



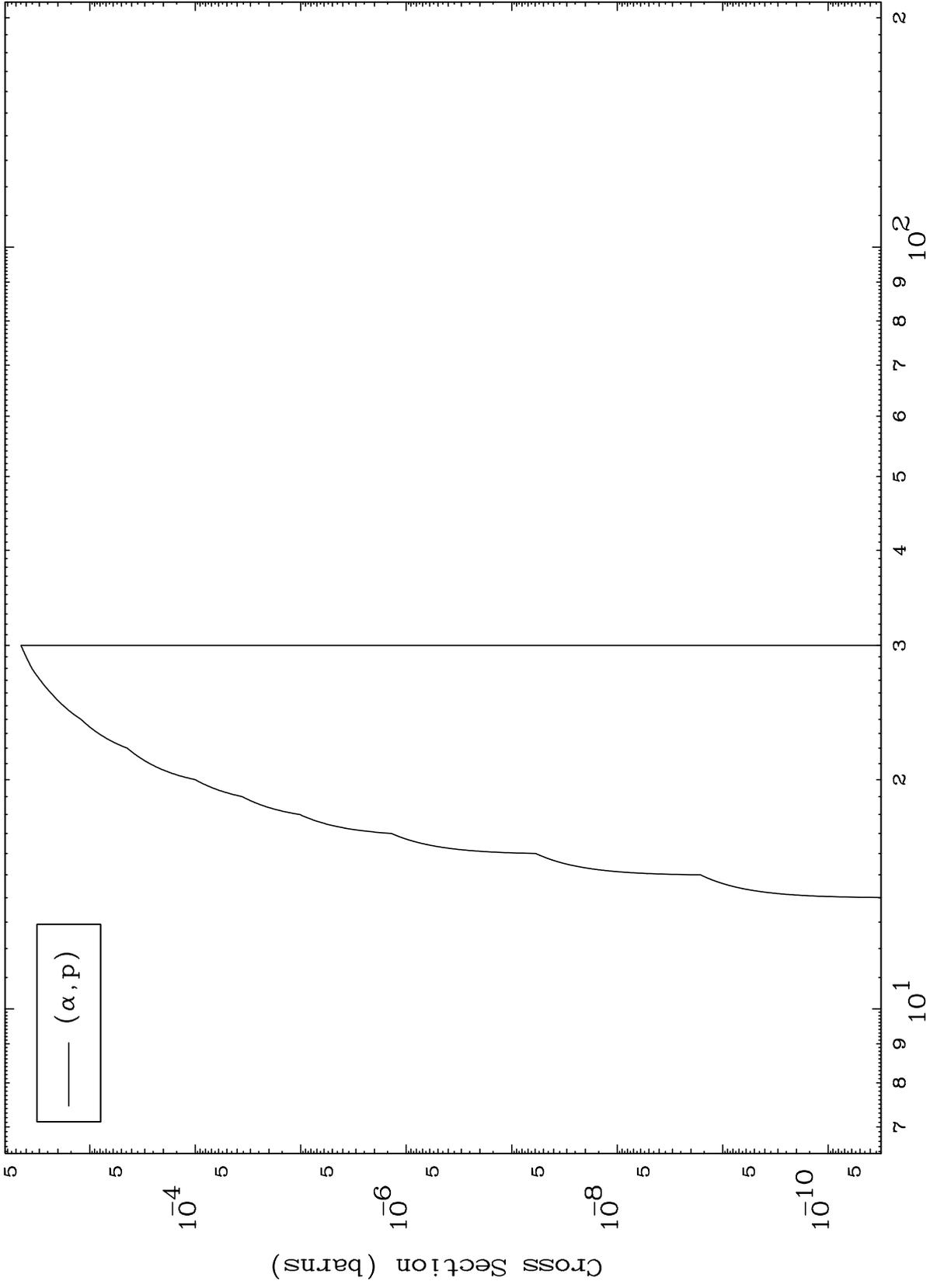
Incident Energy (MeV)

77-Ir-191

MAT 7726

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

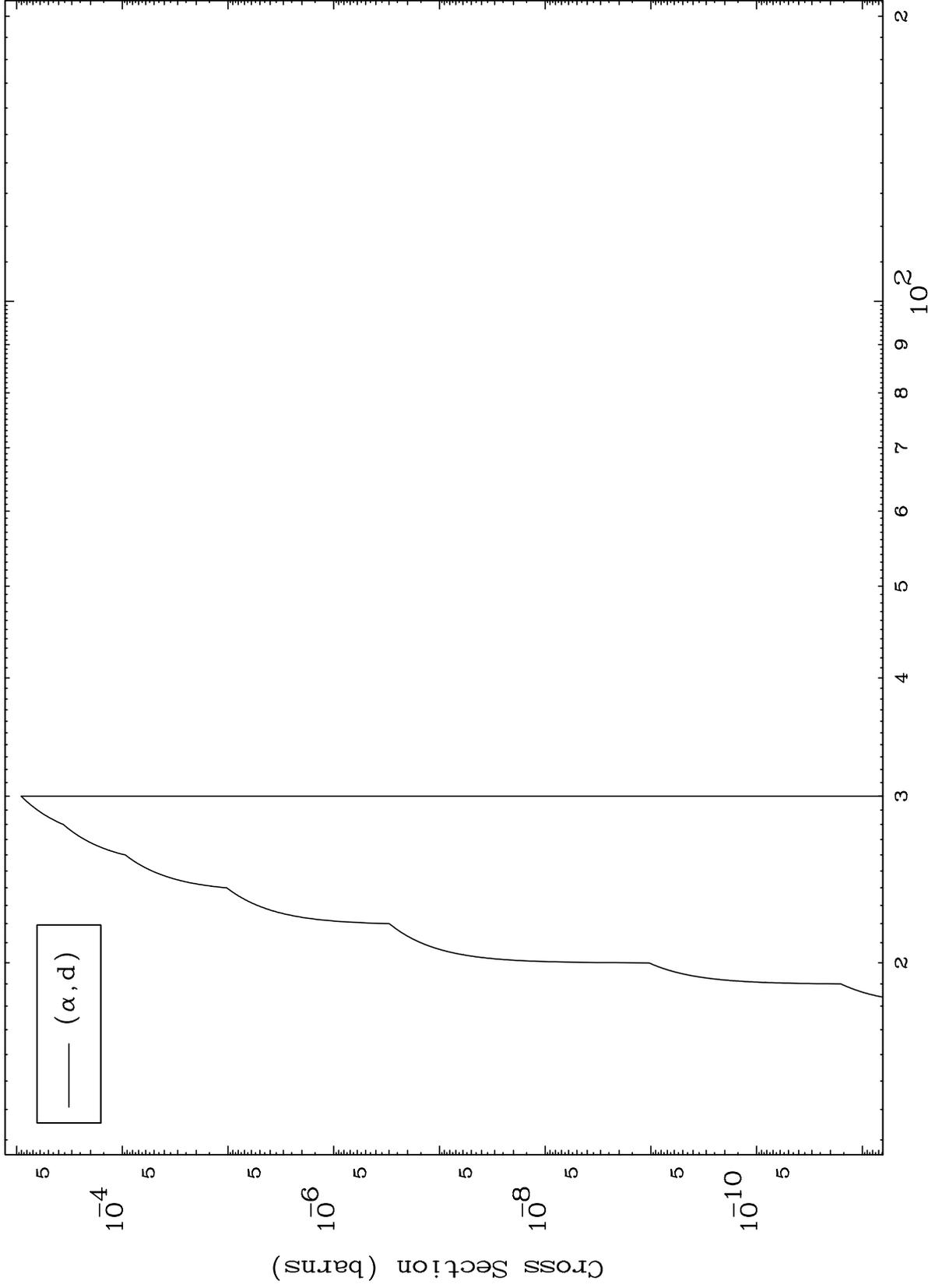
77-Ir-191

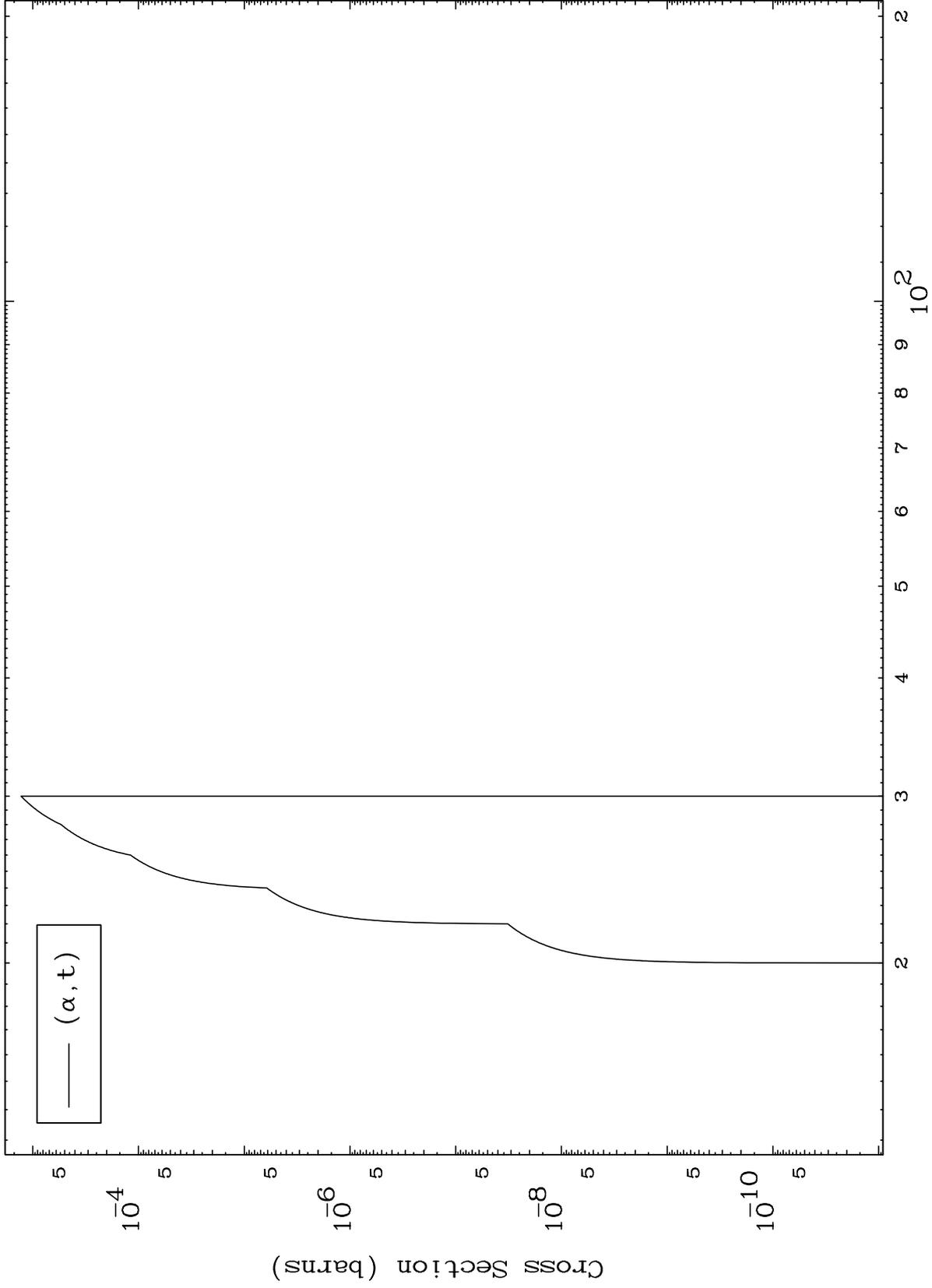


6

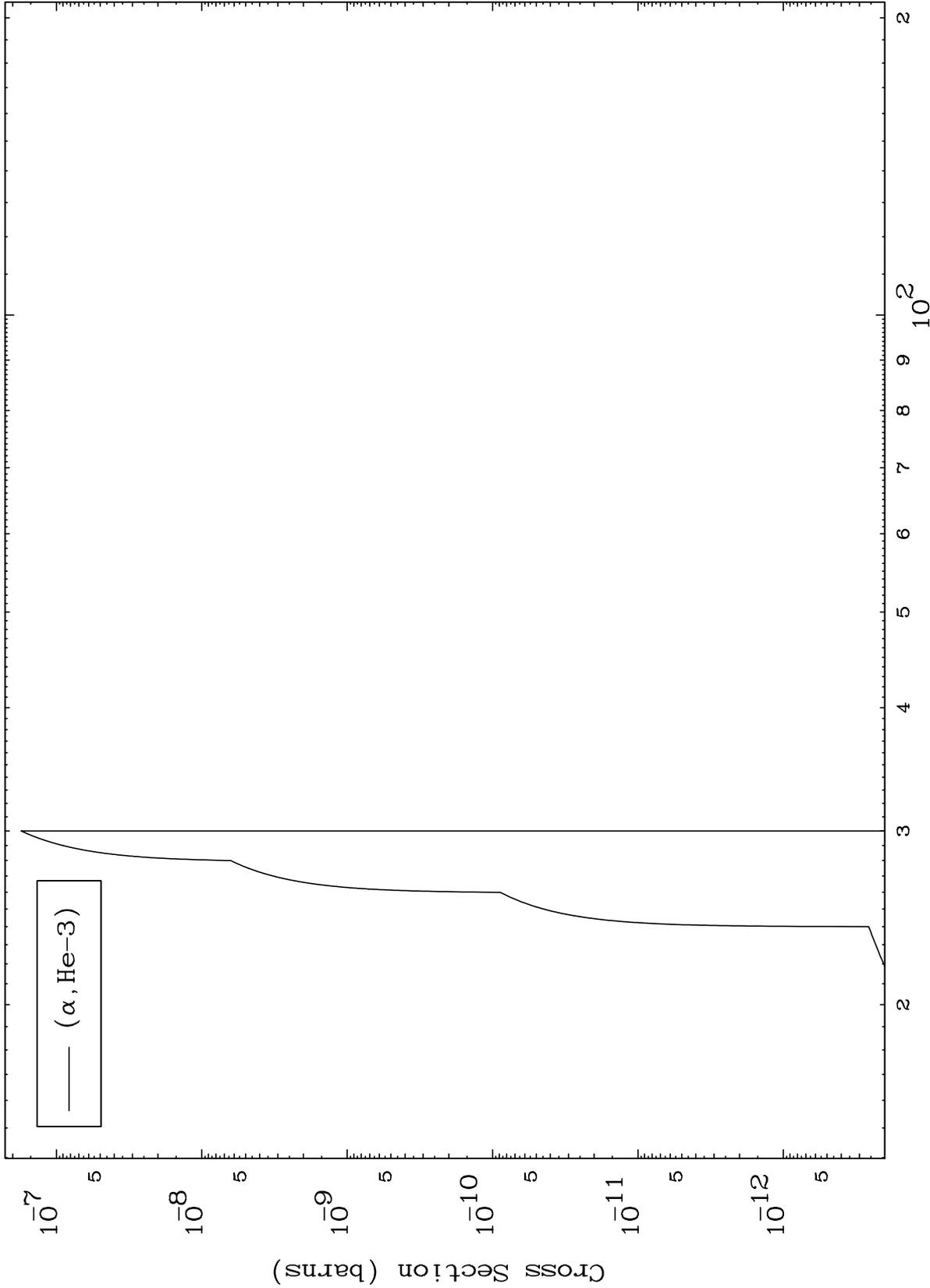
Incident Energy (MeV)

77-Ir-191





( $\alpha$ ,He3) Levels  
0 Kelvin Cross Sections

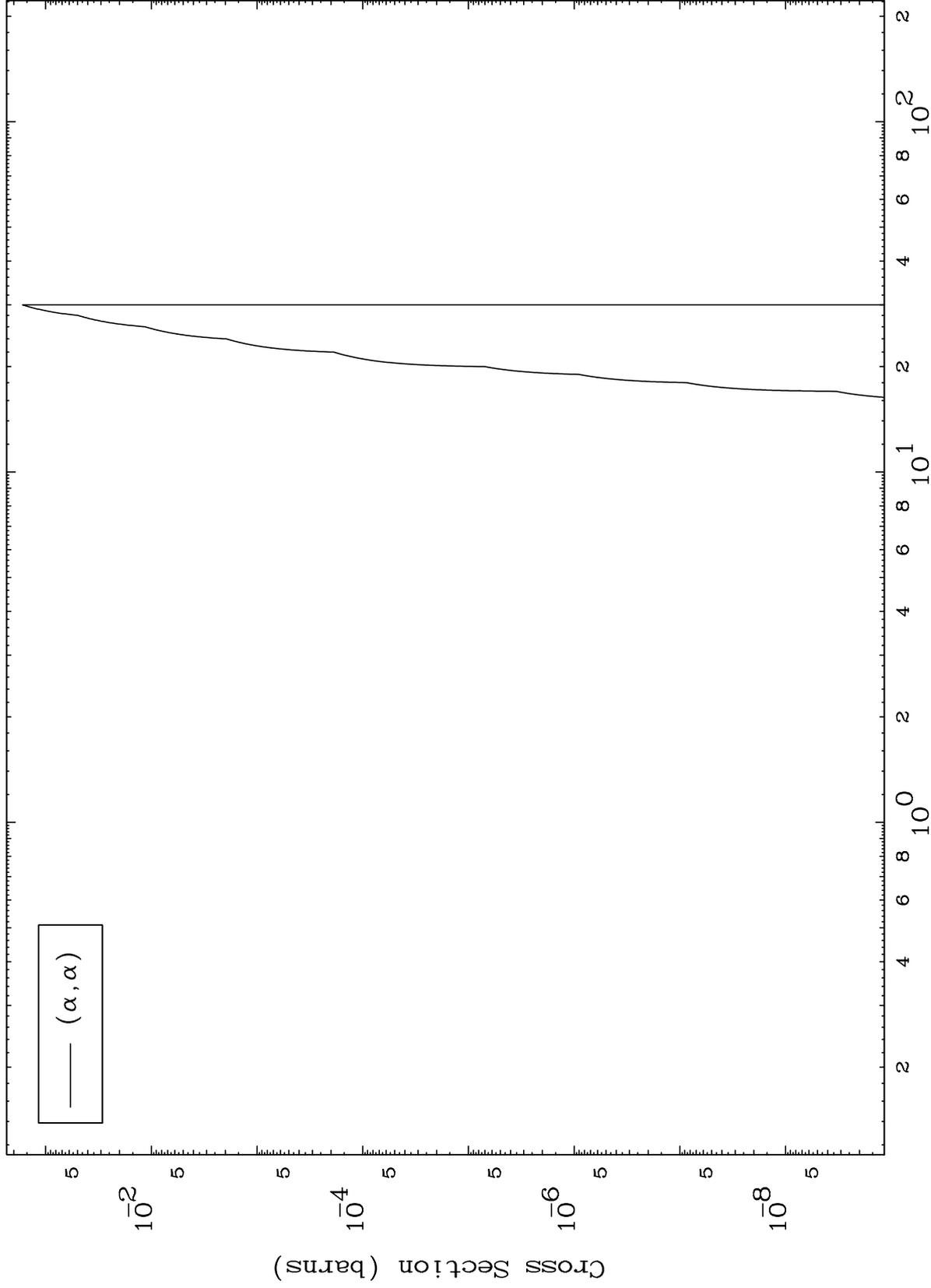


MAT 7726

( $\alpha, \alpha$ ) Levels

77-Ir-191

0 Kelvin Cross Sections

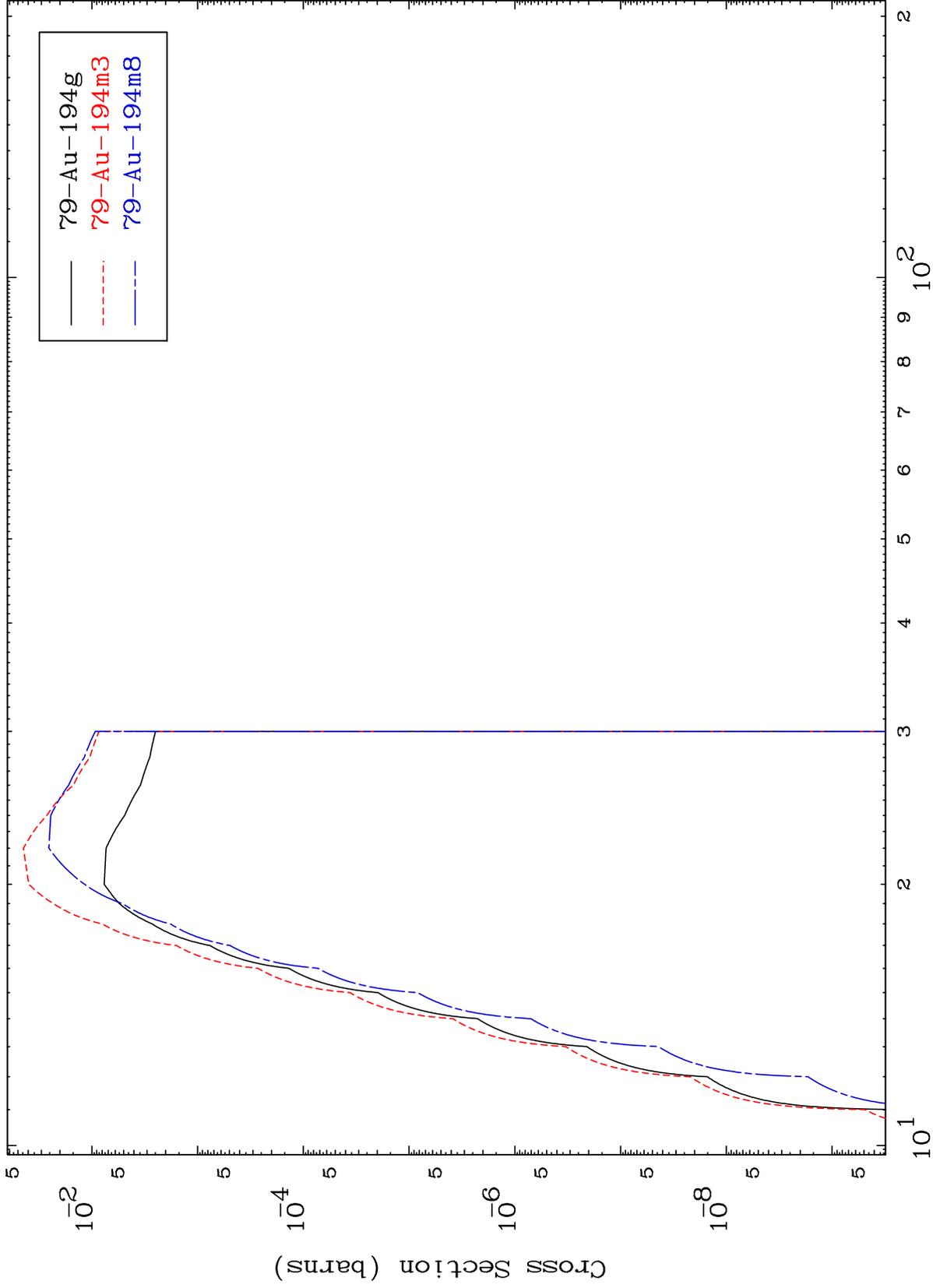


10

Incident Energy (MeV)

77-Ir-191

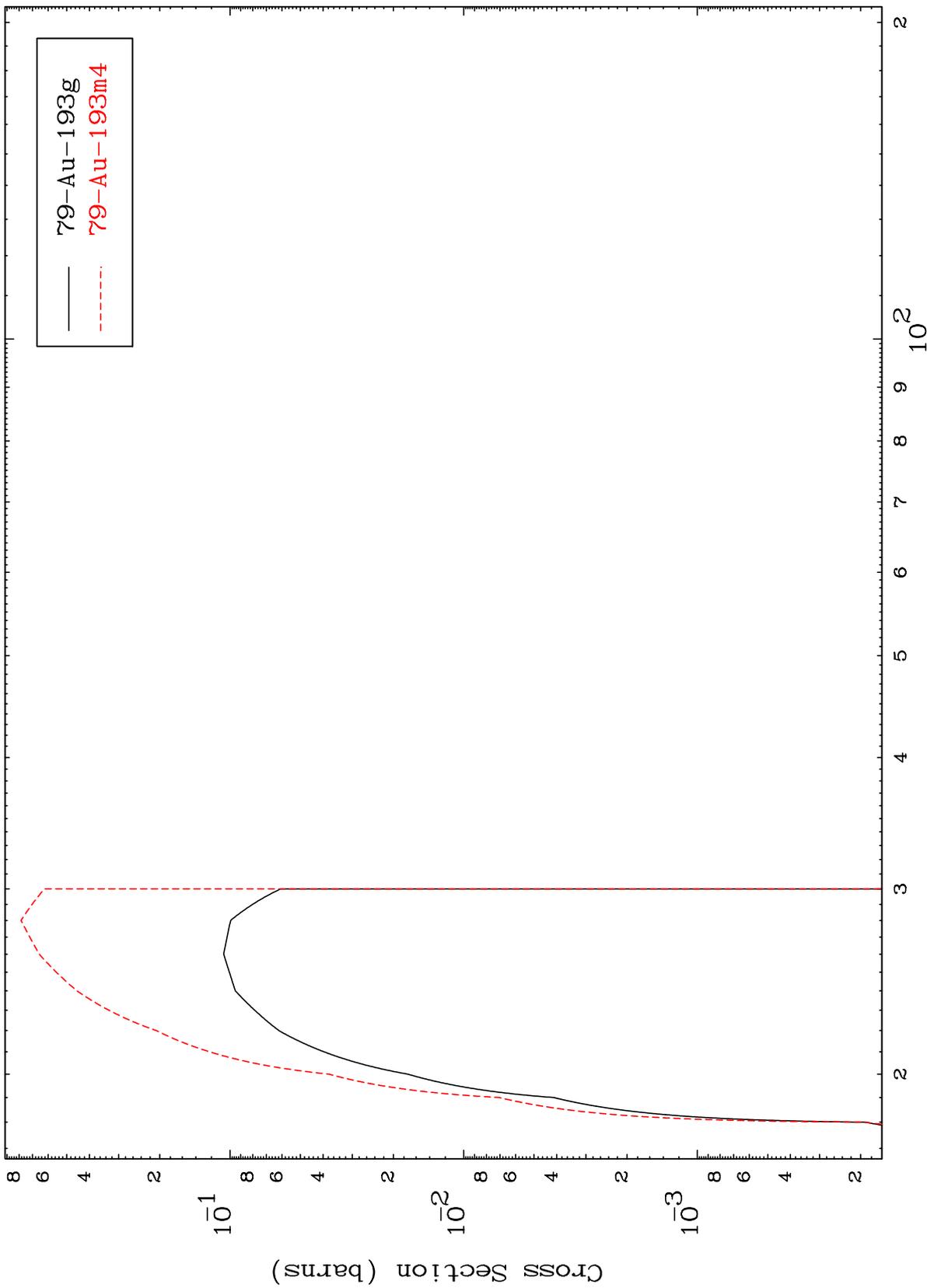
Radionuclide Production Cross Section



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77-Ir-191

( $\alpha, 2n$ )  
Radionuclide Production Cross Section



12

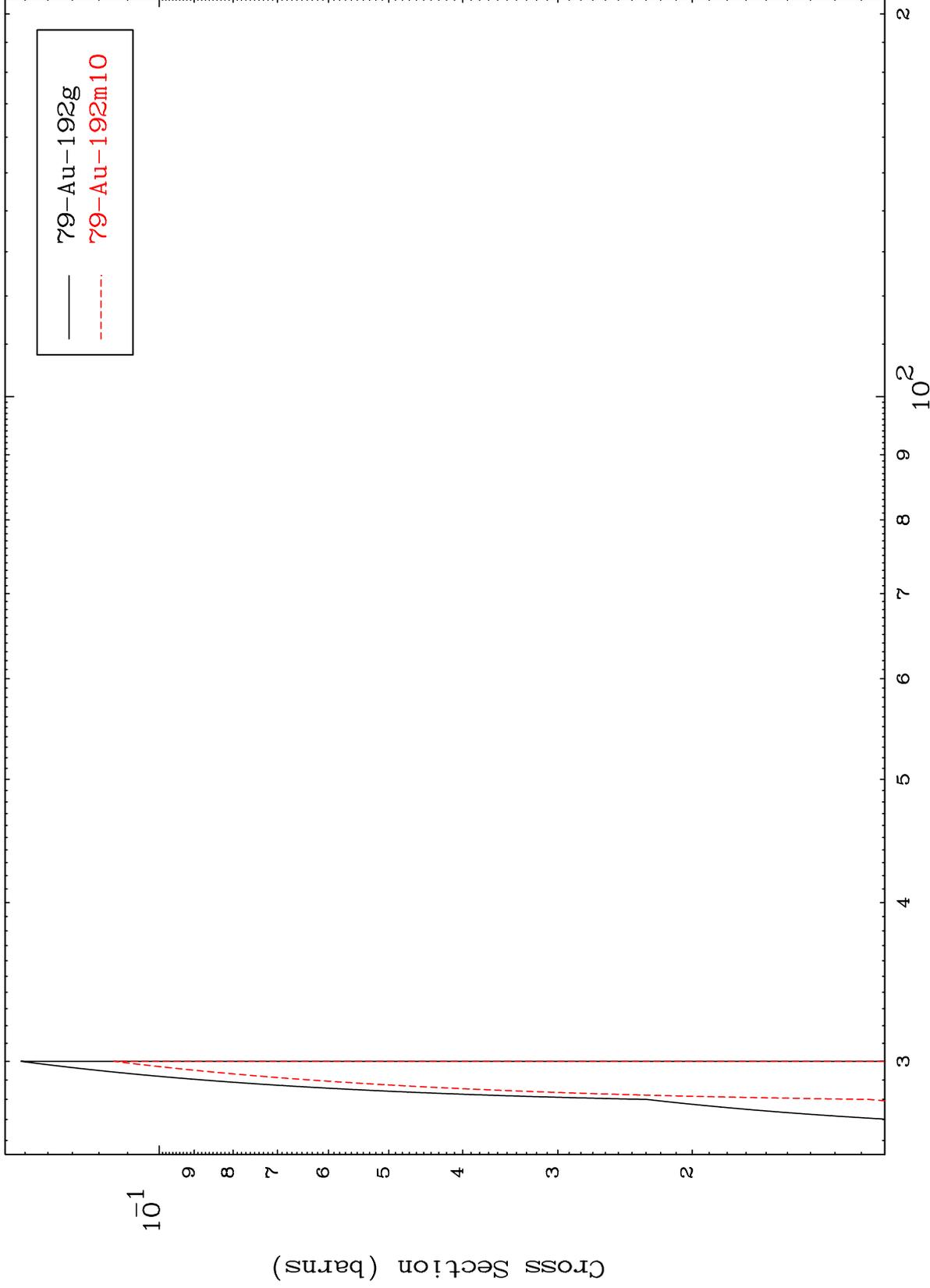
Incident Energy (MeV)

77-Ir-191

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<sup>77</sup>Ir-191

( $\alpha, 3n$ )  
Radionuclide Production Cross Section



13

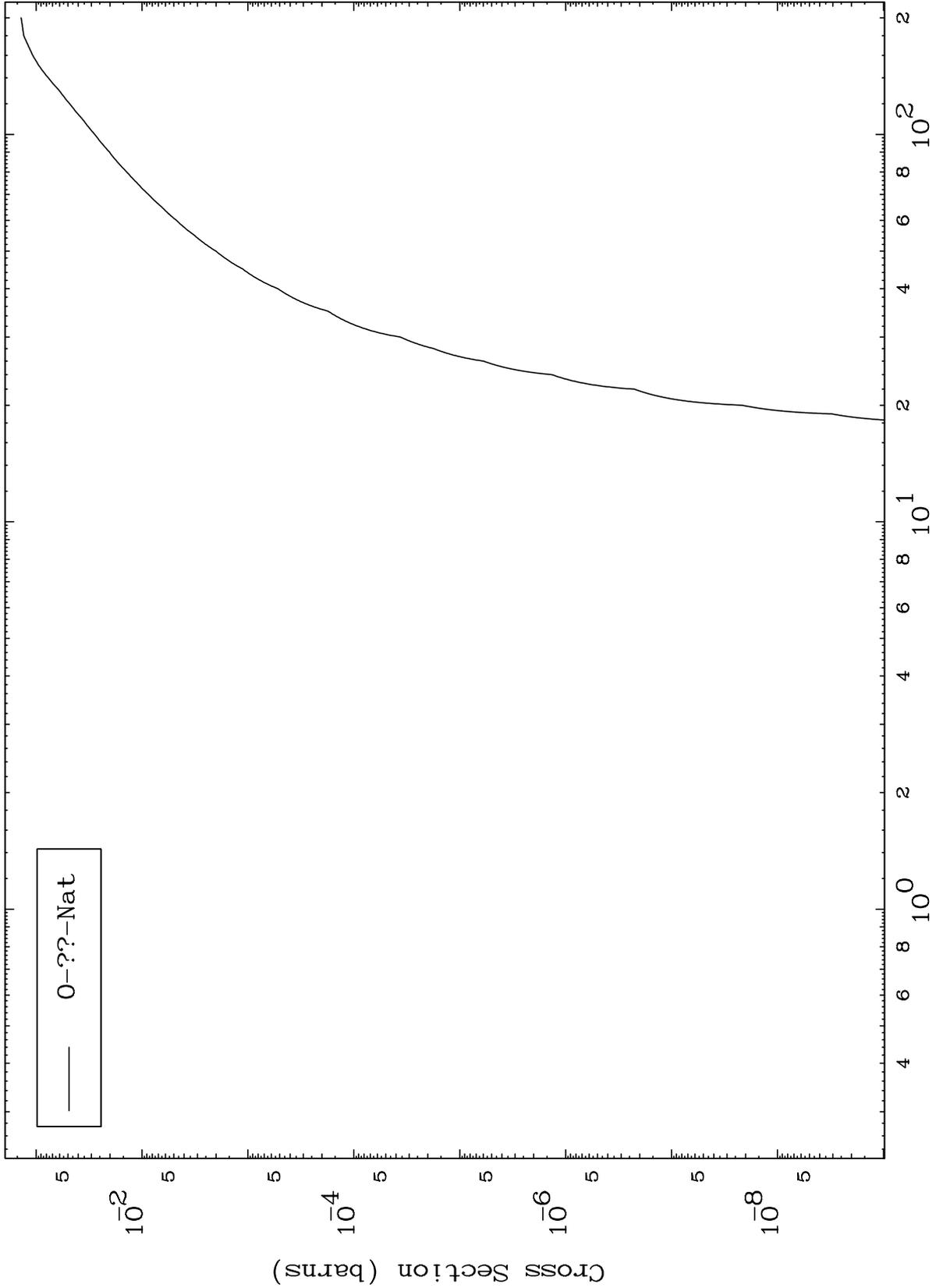
Incident Energy (MeV)

<sup>77</sup>Ir-191

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77-Ir-191

$\alpha$  Fission  
Radionuclide Production Cross Section



77-Ir-191

Incident Energy (MeV)

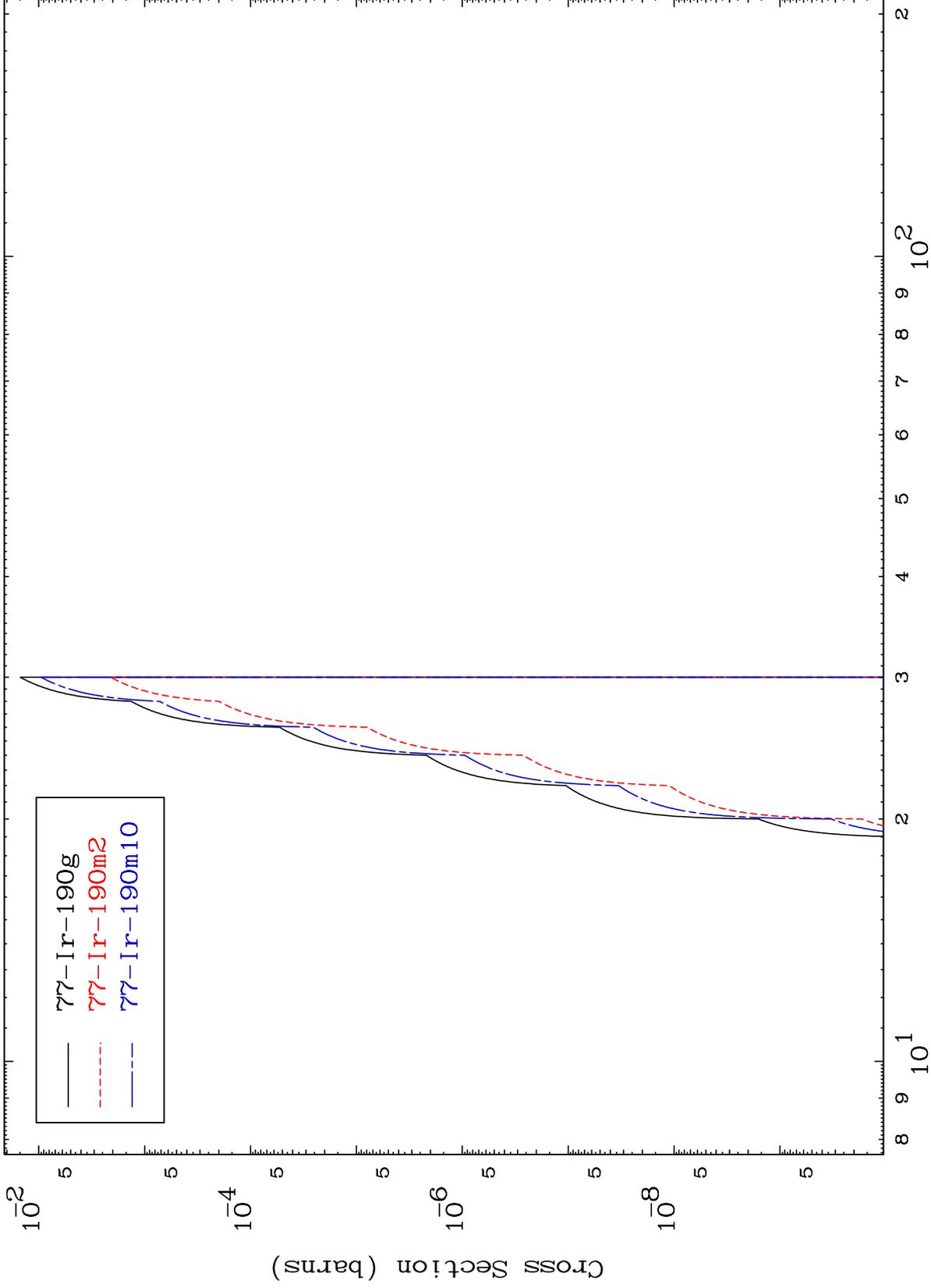
14

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$(\alpha, n')$   $\alpha$

$^{77}\text{Ir-191}$

Radionuclide Production Cross Section

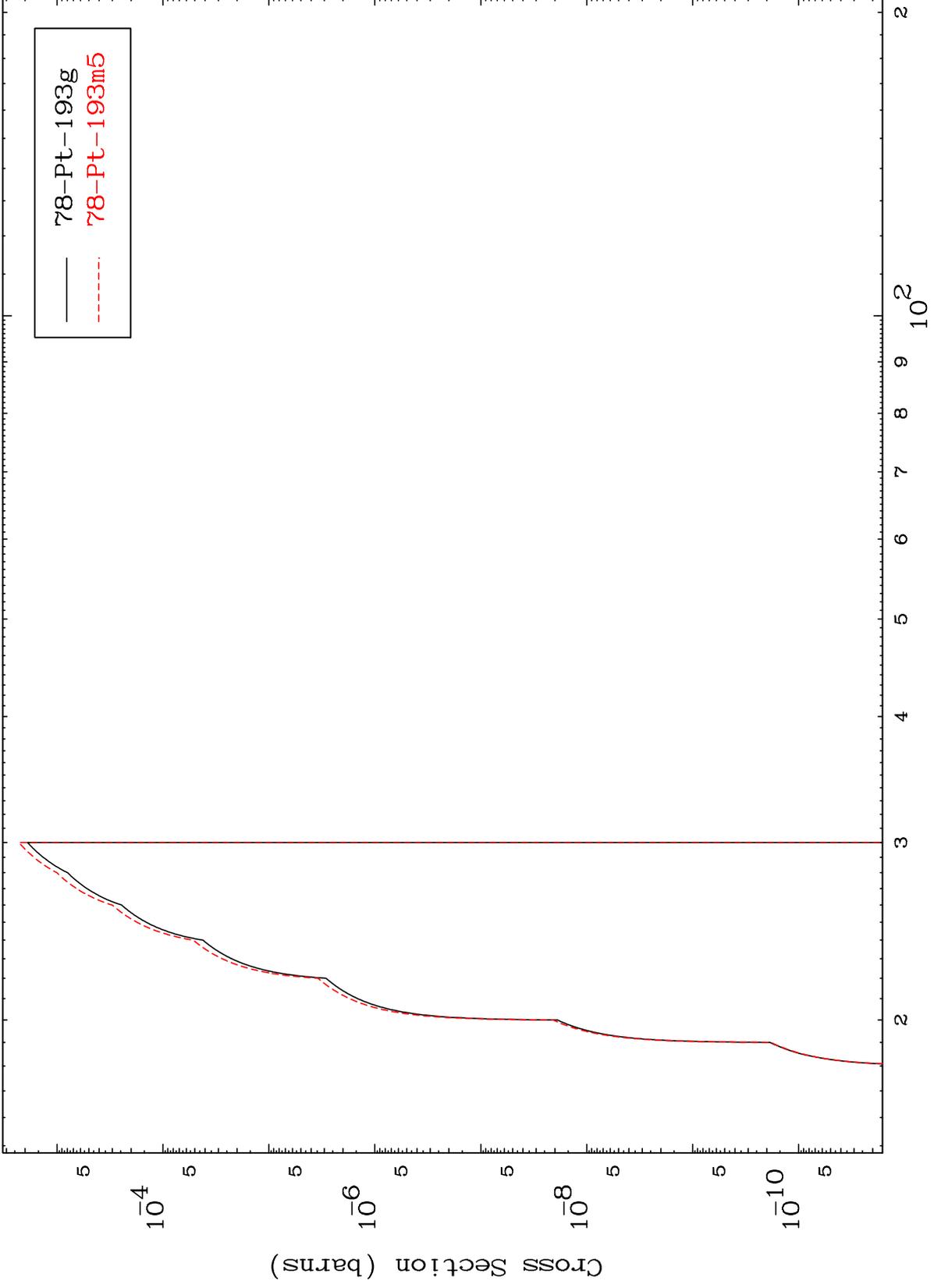


15

Incident Energy (MeV)

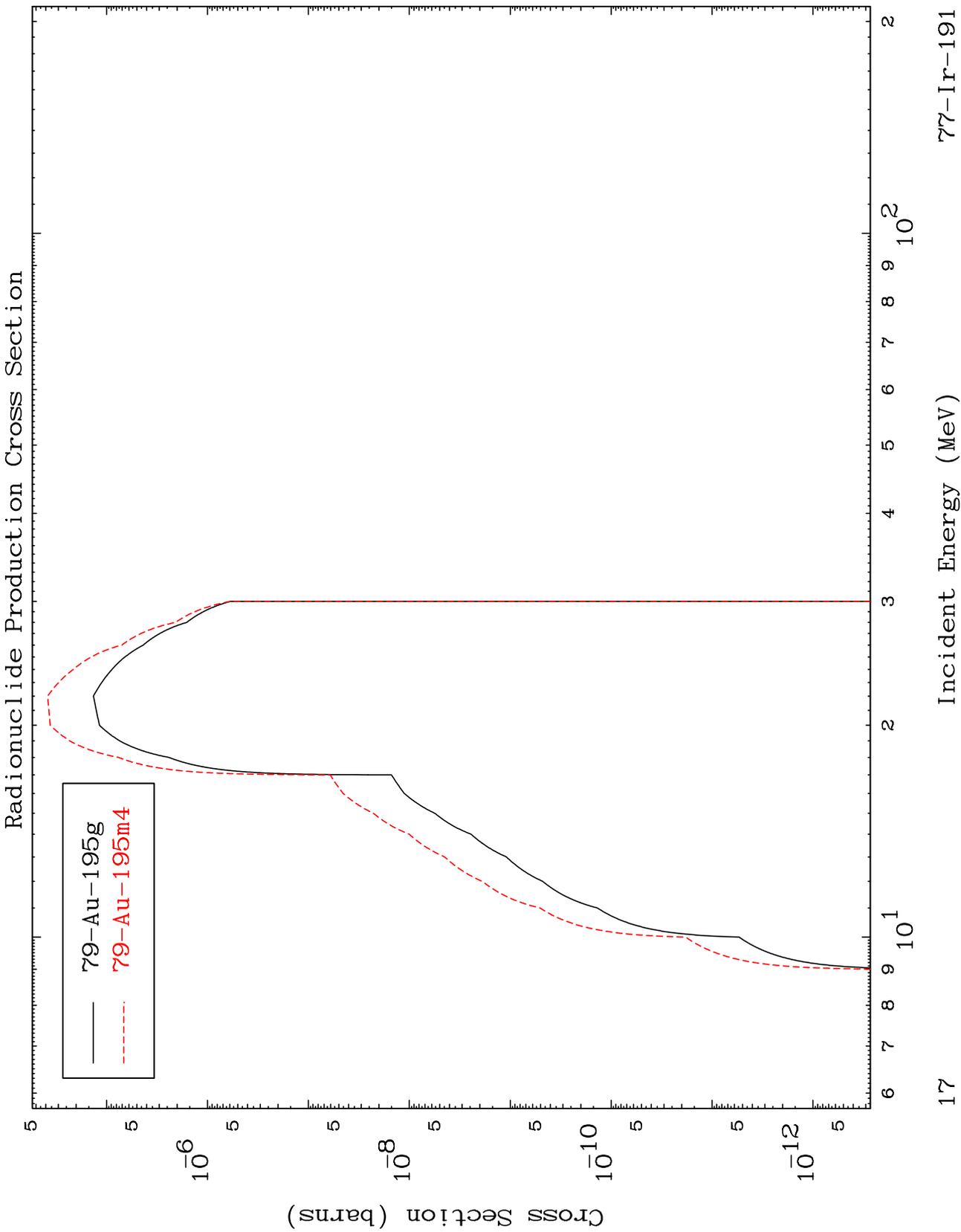
$^{77}\text{Ir-191}$

Radionuclide Production Cross Section



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<sup>77</sup>Ir-191



<sup>77</sup>Ir-191

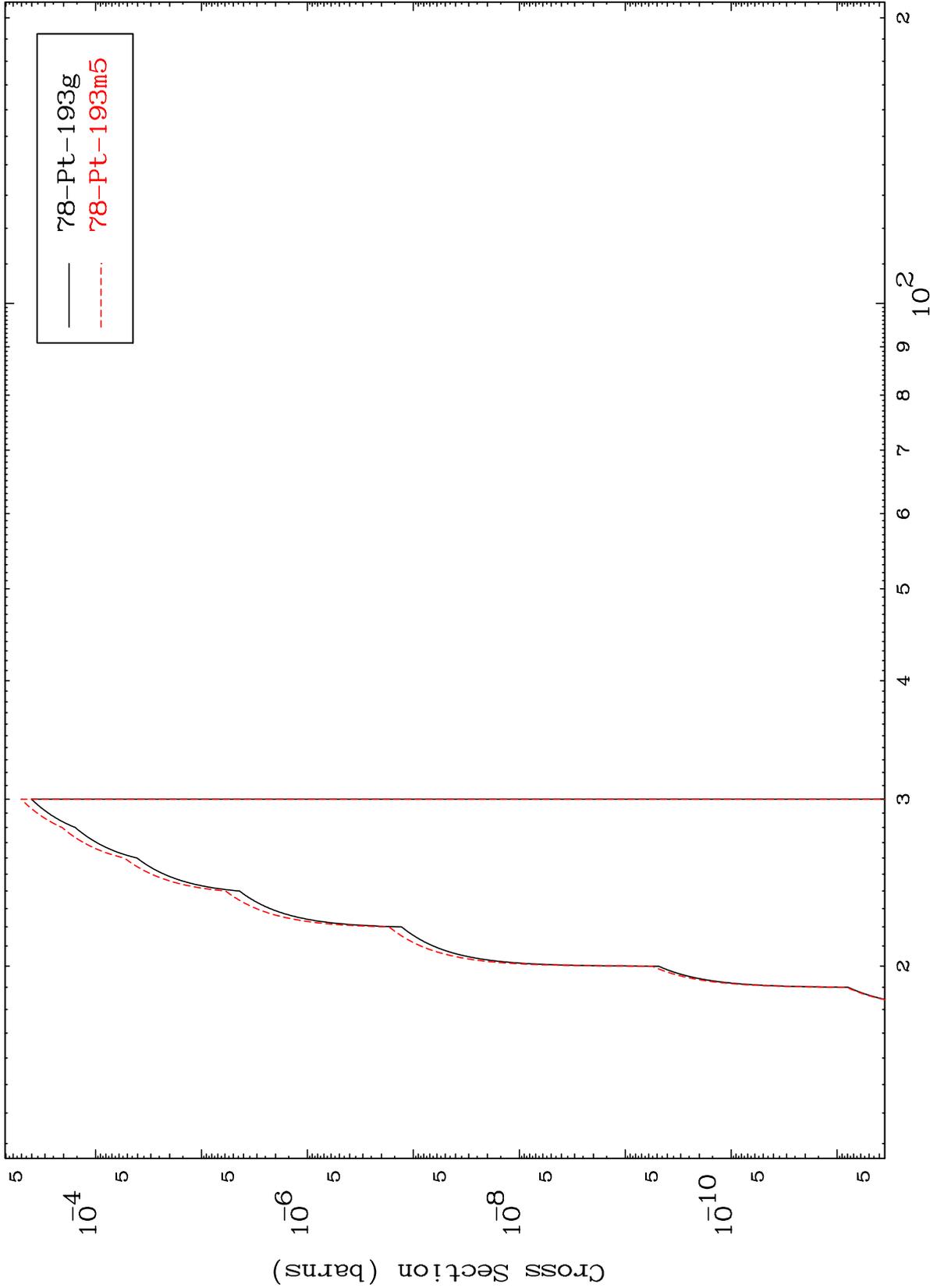
Incident Energy (MeV)

17

MAT 7726

77-Ir-191

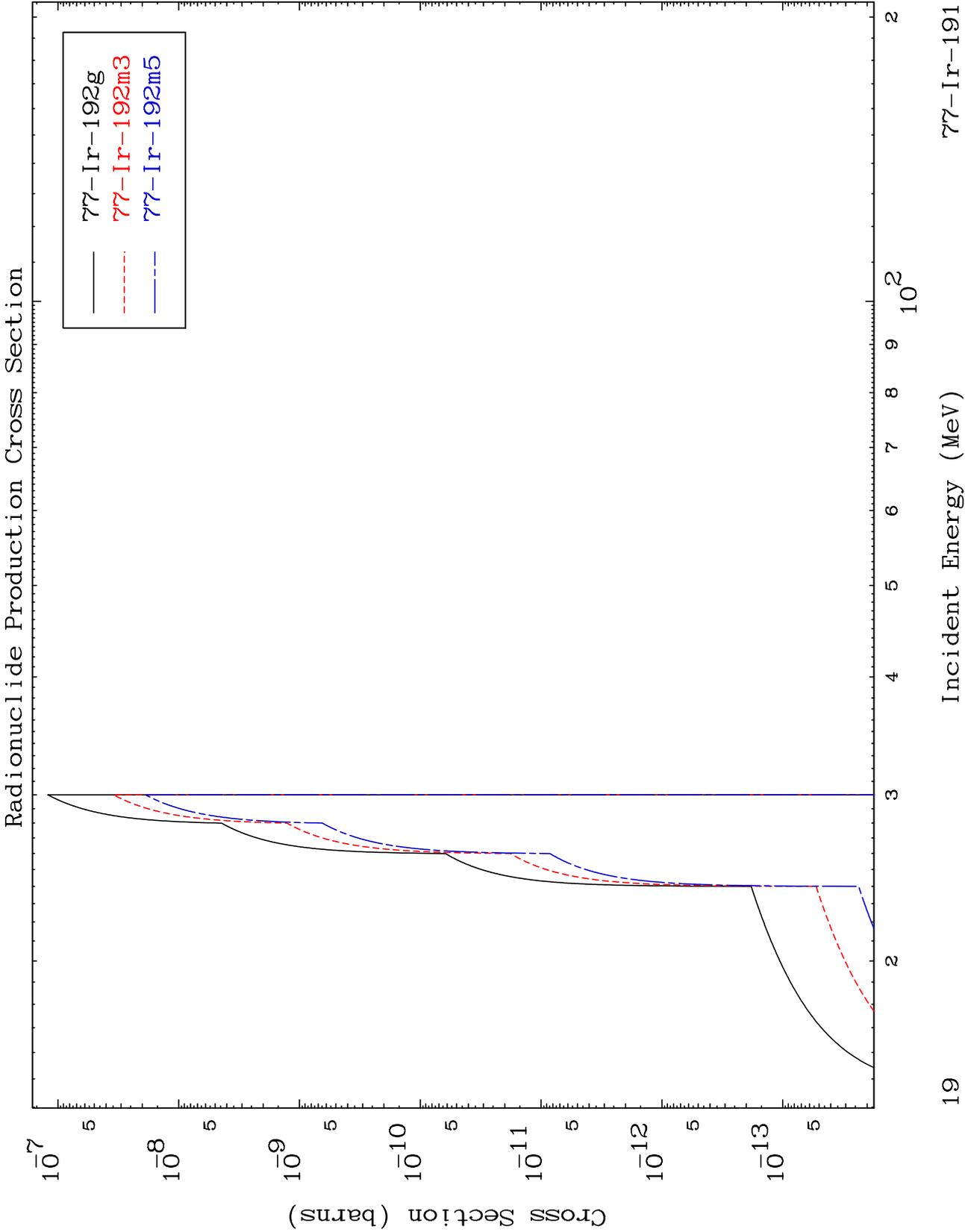
( $\alpha, d$ )  
Radionuclide Production Cross Section



18

Incident Energy (MeV)

77-Ir-191

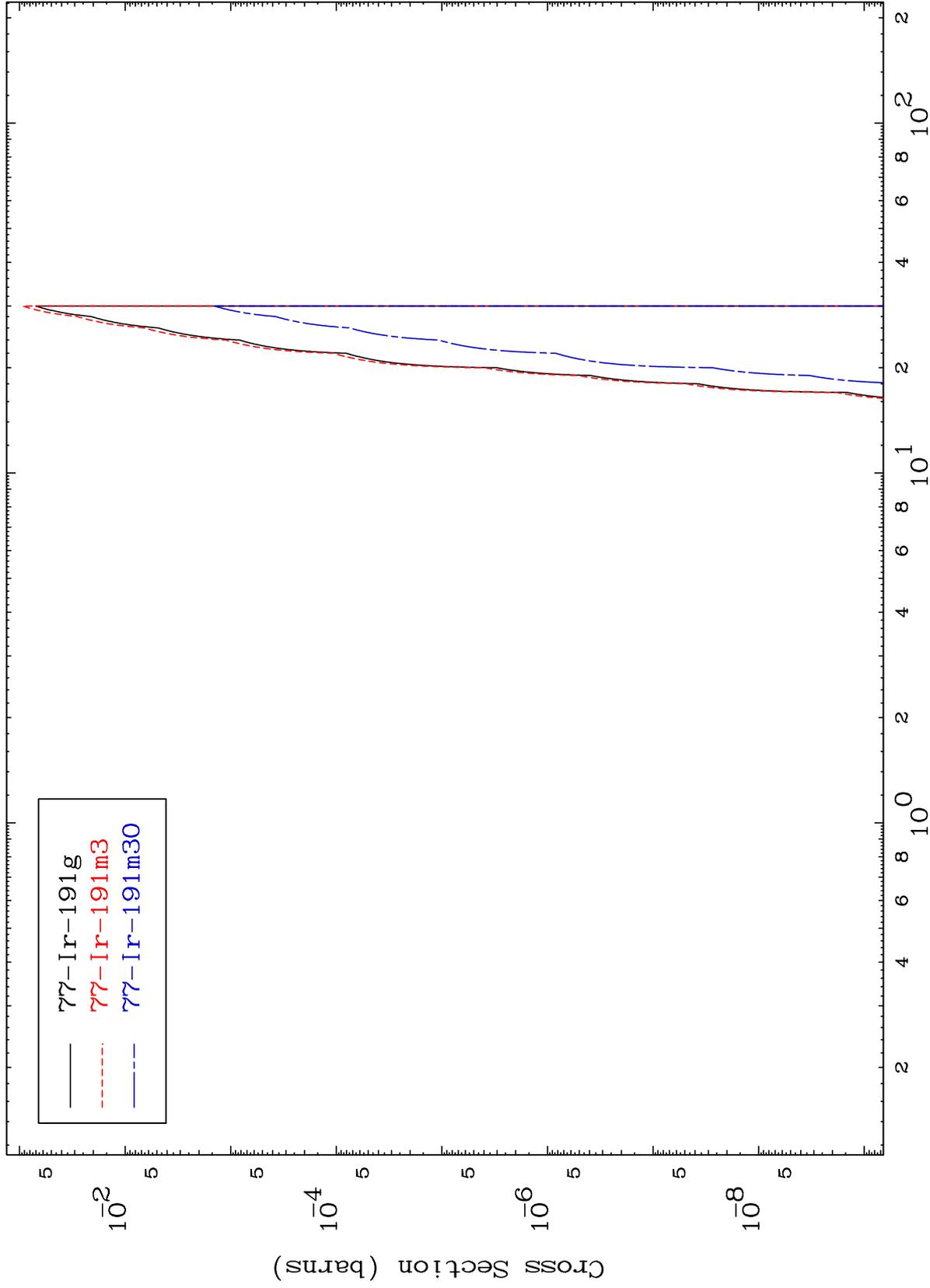


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<sup>77</sup>Ir-191

( $\alpha, \alpha$ )

Radionuclide Production Cross Section



20

Incident Energy (MeV)

<sup>77</sup>Ir-191

MAT 7726

$^{77}\text{Ir-191}$   $\alpha$

$^{77}\text{Ir-191}$

Radionuclide Production Cross Section

