

Program EVALPLOT  
(Version 2018-1)

by

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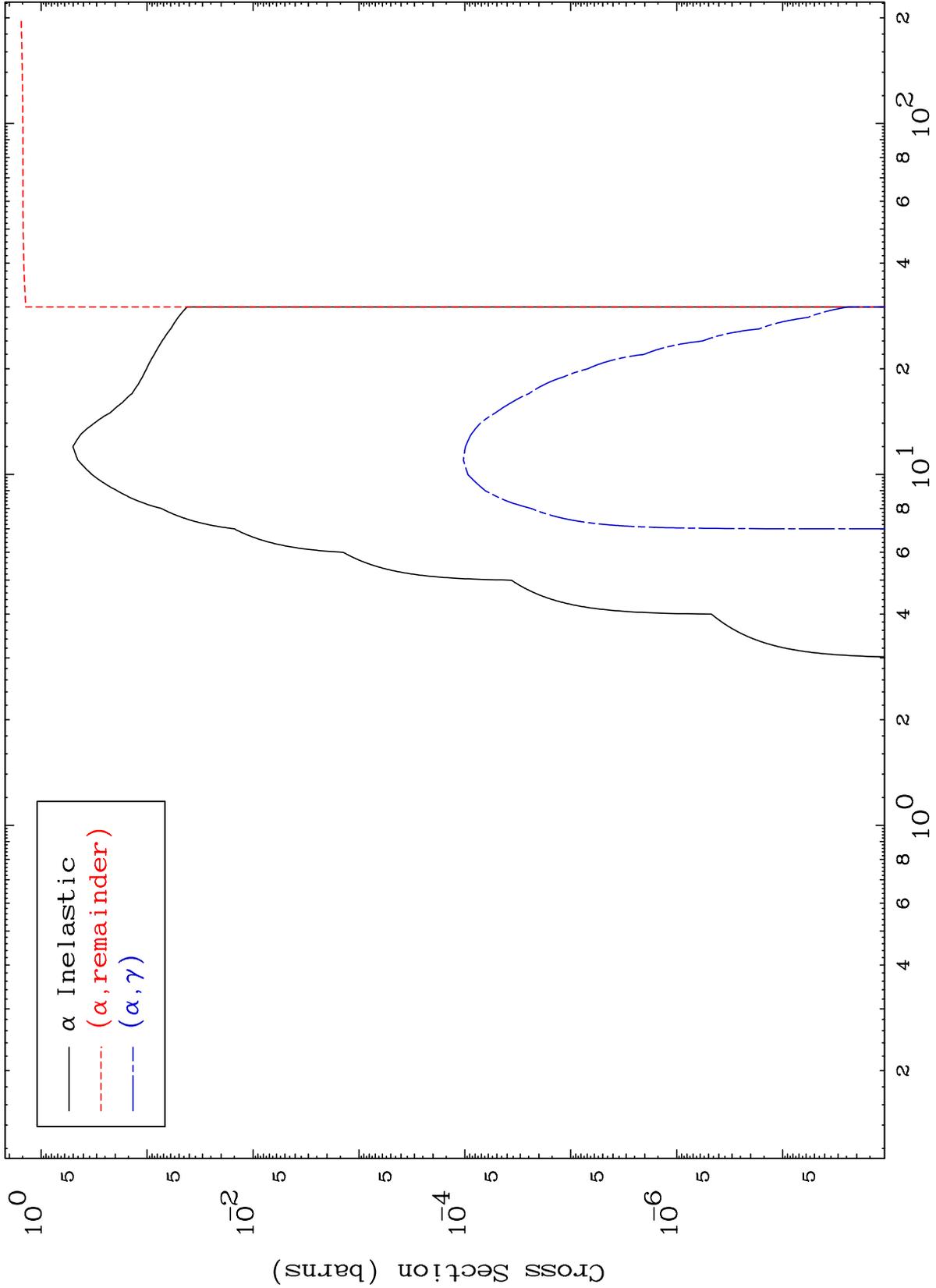
Press Mouse Button to Start

MAT 2528

$\alpha$  Major

25-Mn-56

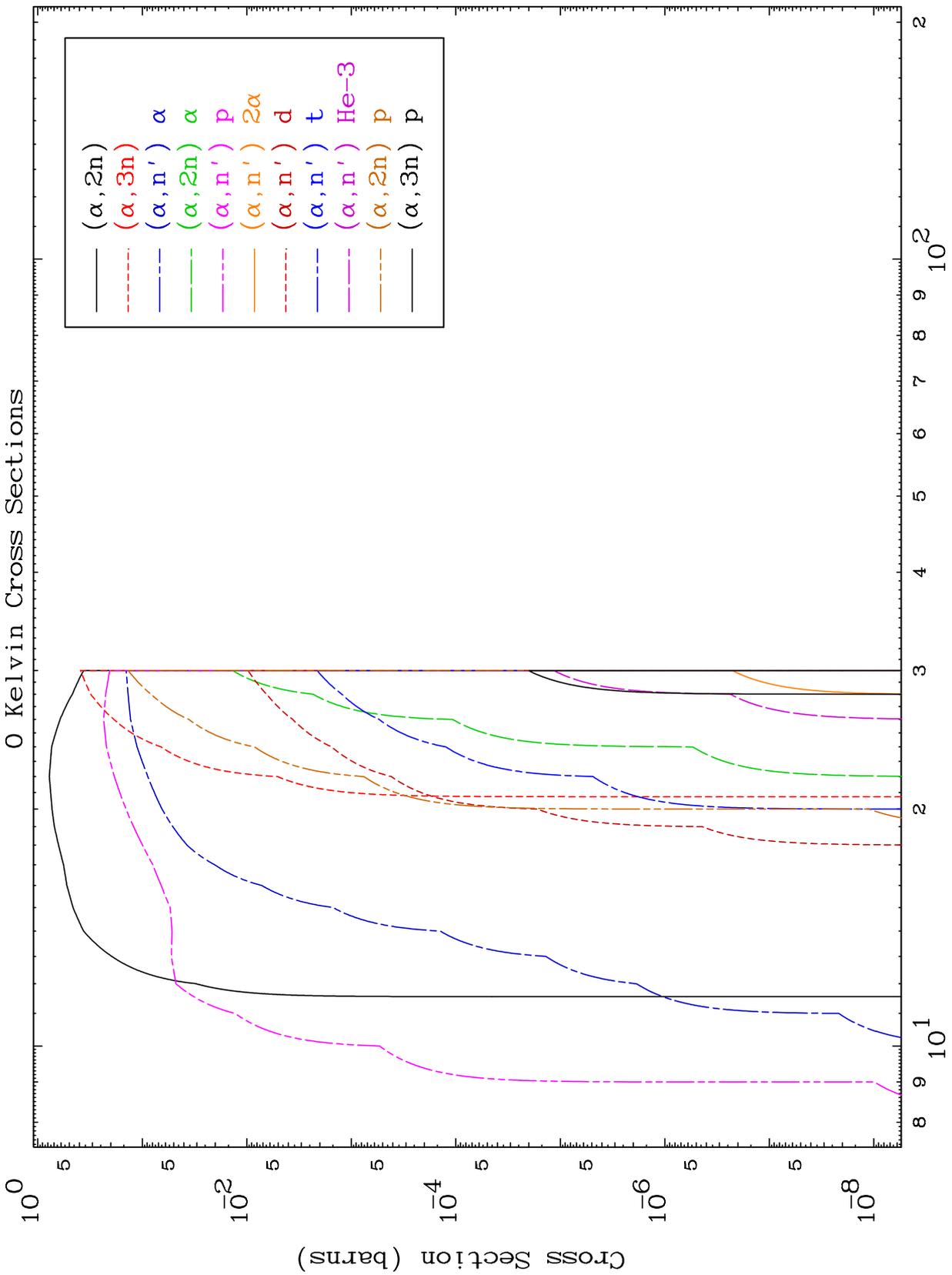
0 Kelvin Cross Sections



MAT 2528

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

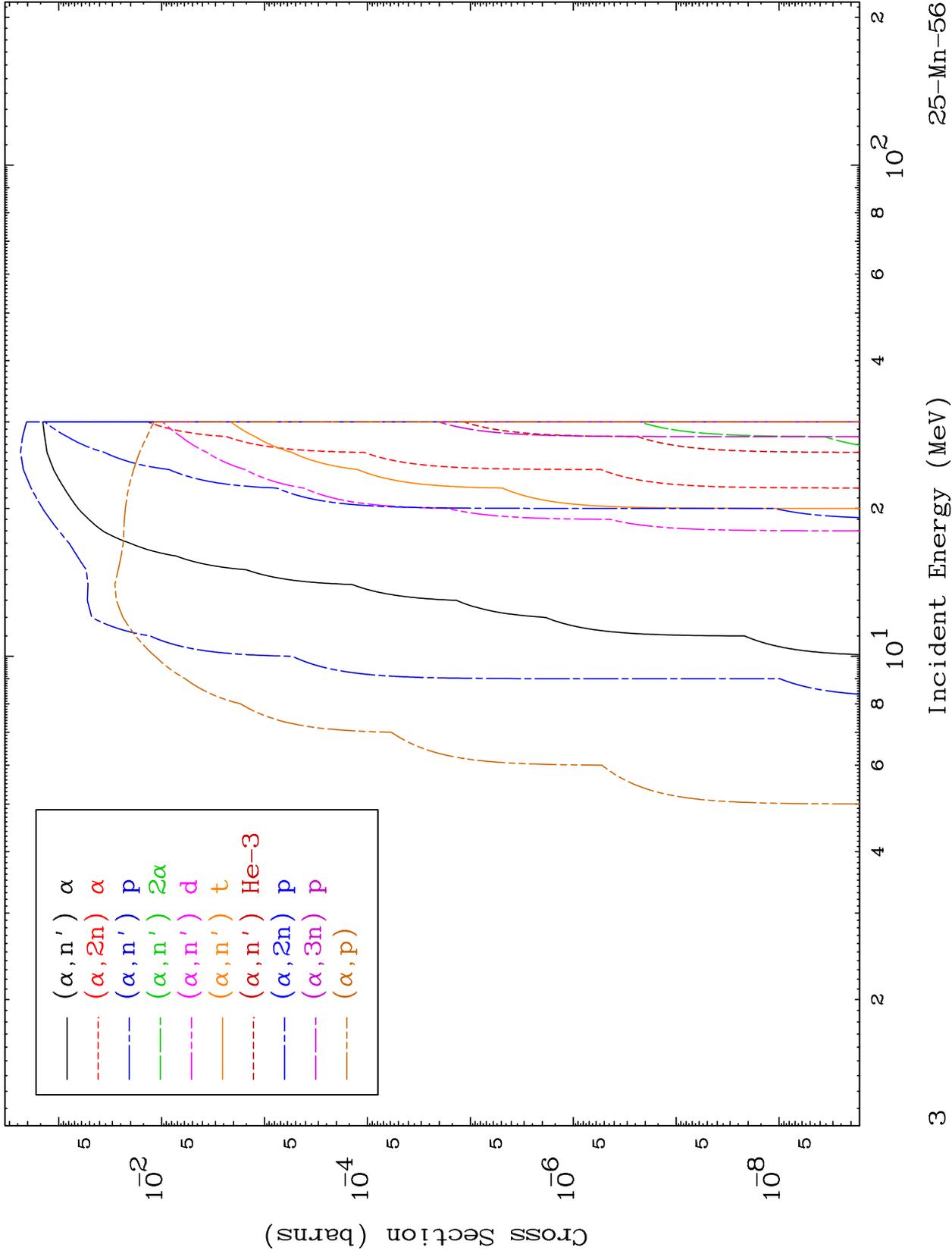
25-Mn-56

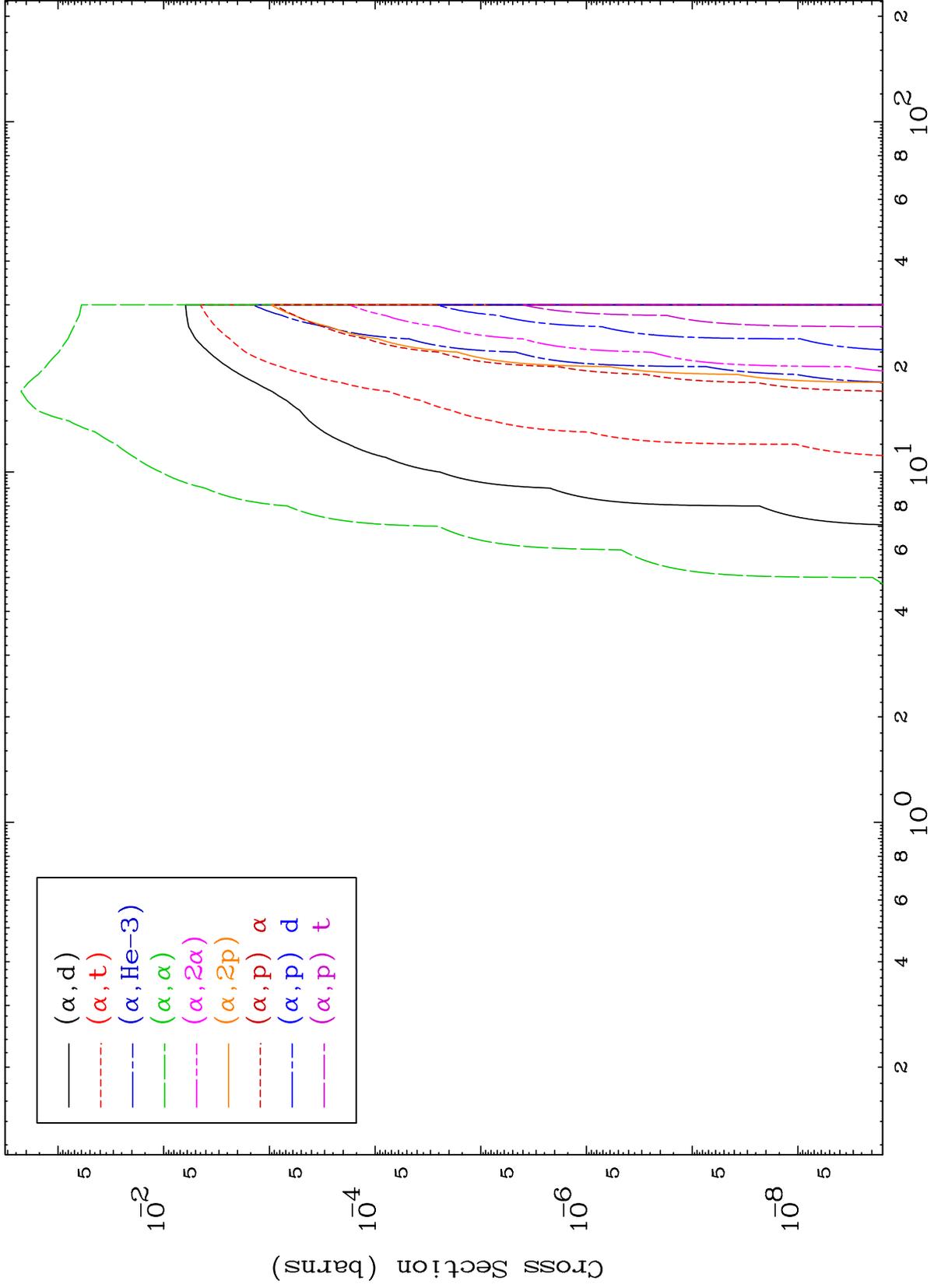


2

Incident Energy (MeV)

25-Mn-56



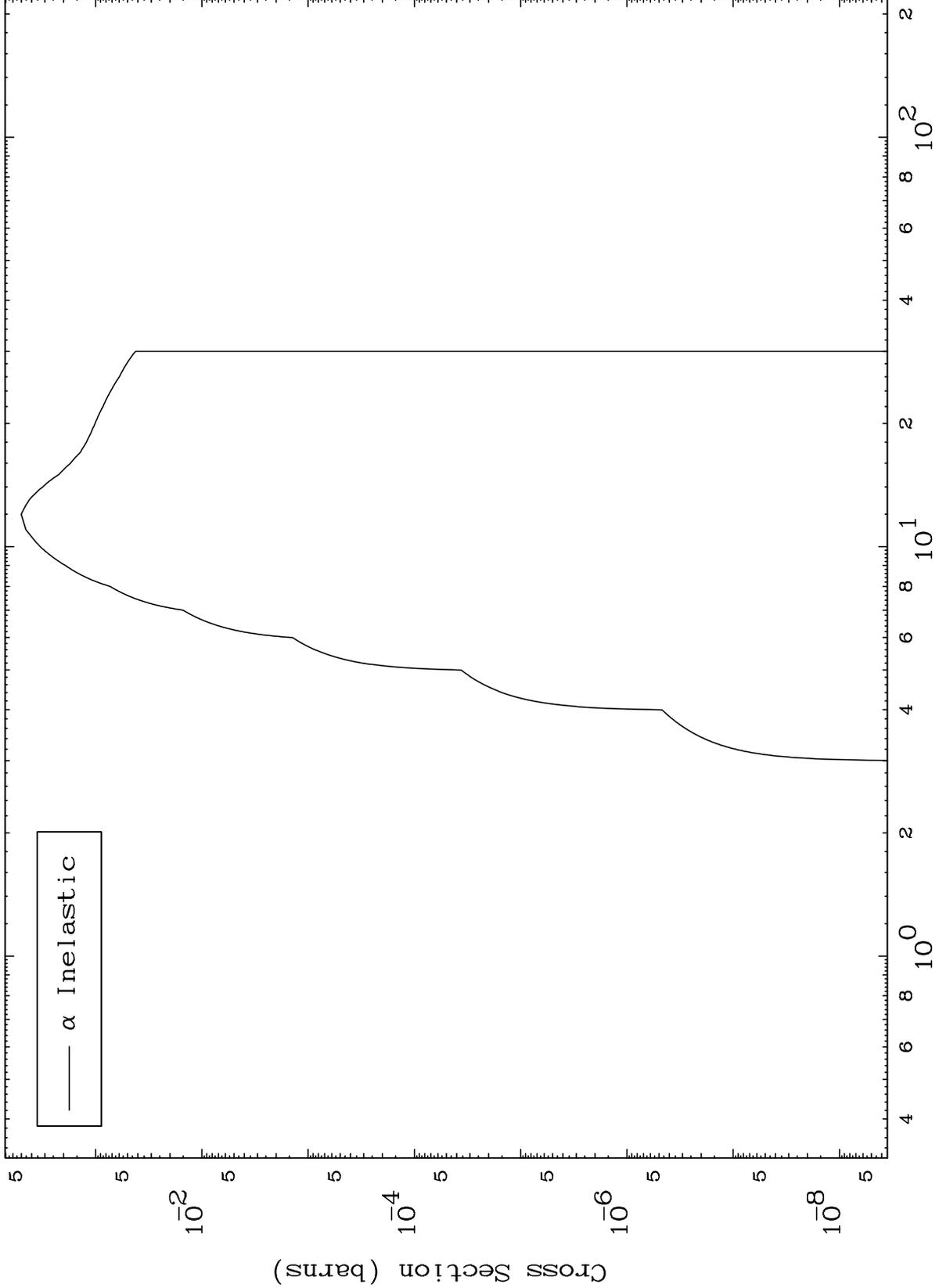


MAT 2528

( $\alpha, n'$ ) Level

25-Mn-56

0 Kelvin Cross Sections



5

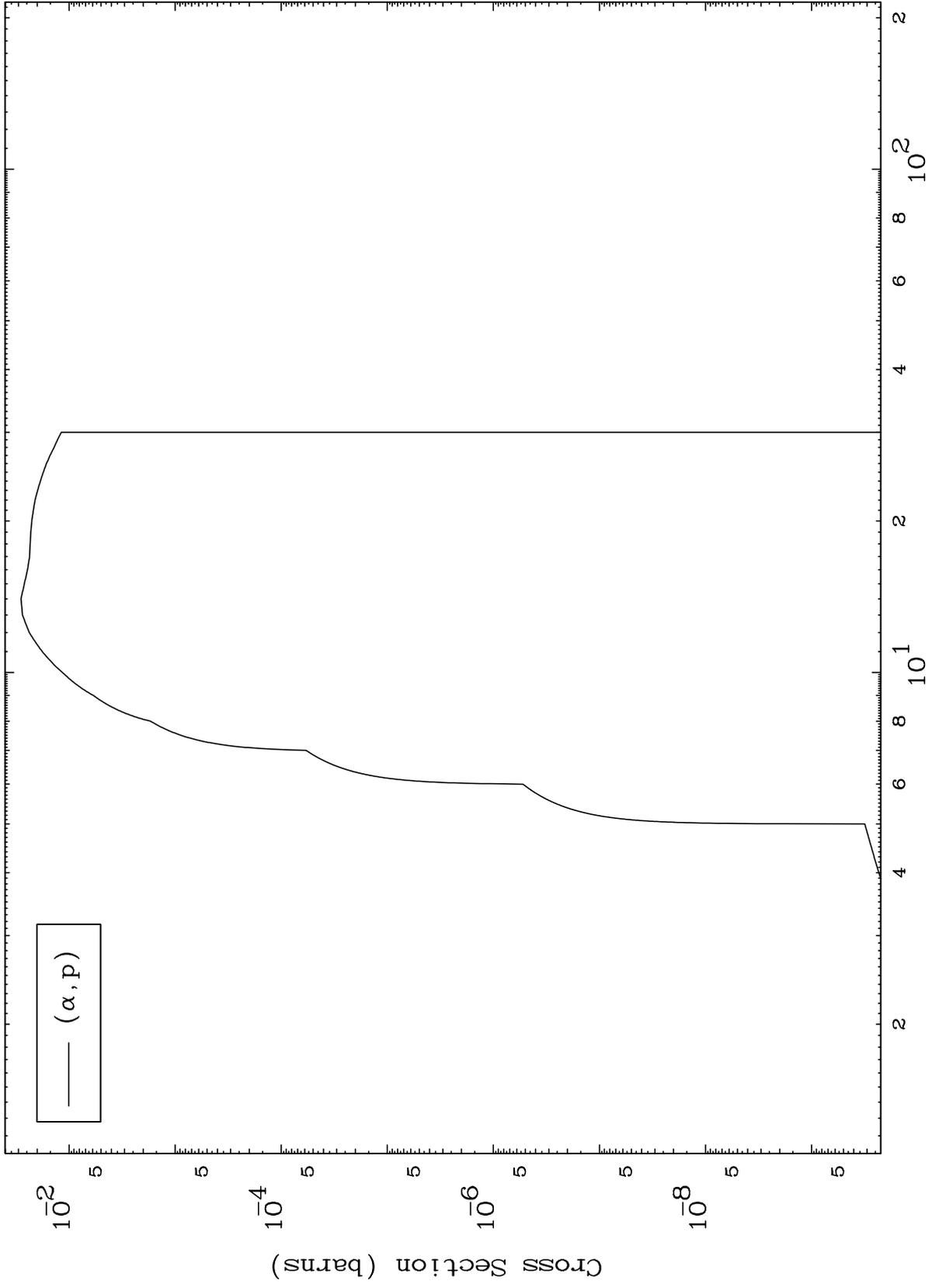
Incident Energy (MeV)

25-Mn-56

MAT 2528

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

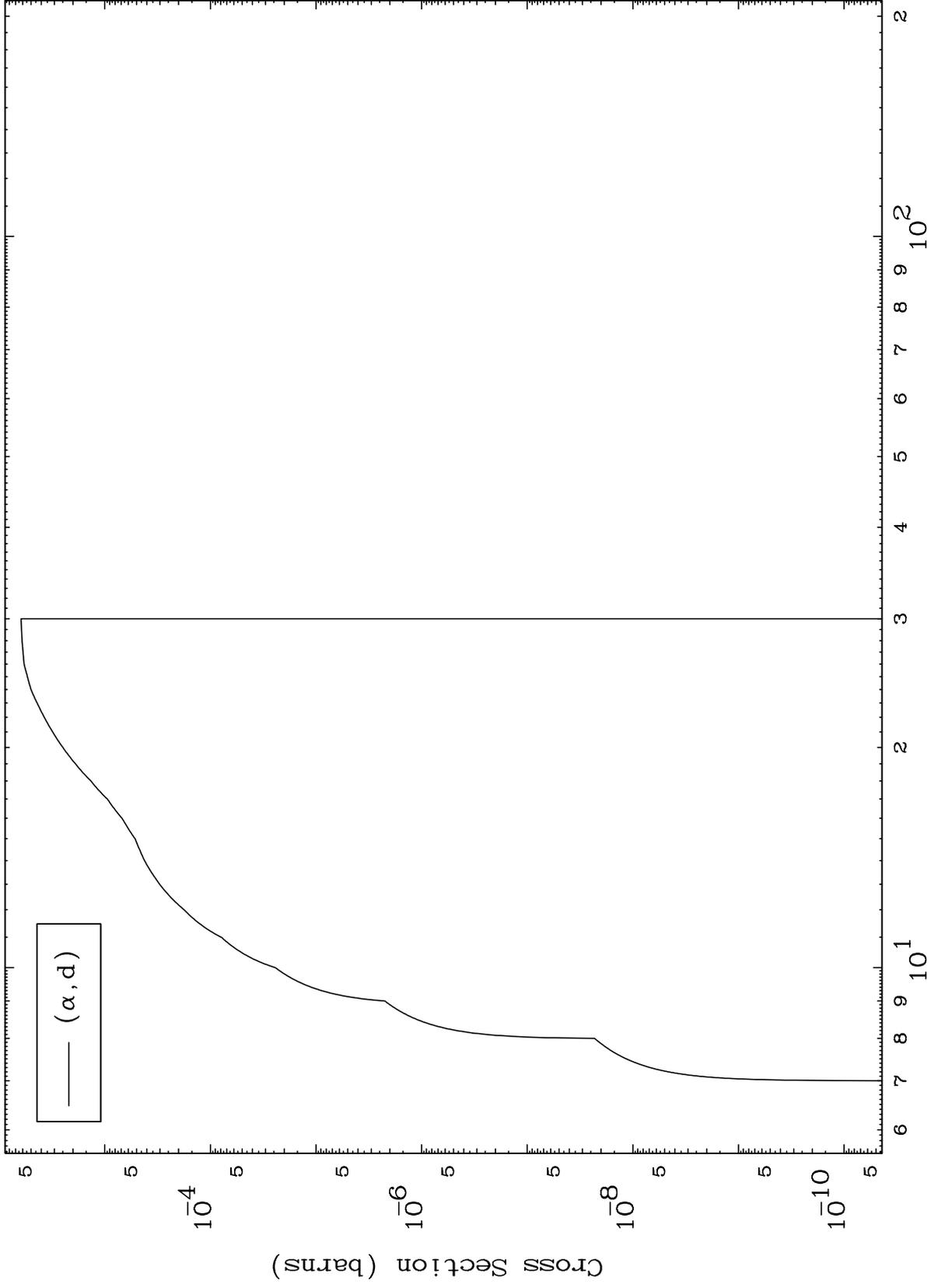
25-Mn-56



6

Incident Energy (MeV)

25-Mn-56

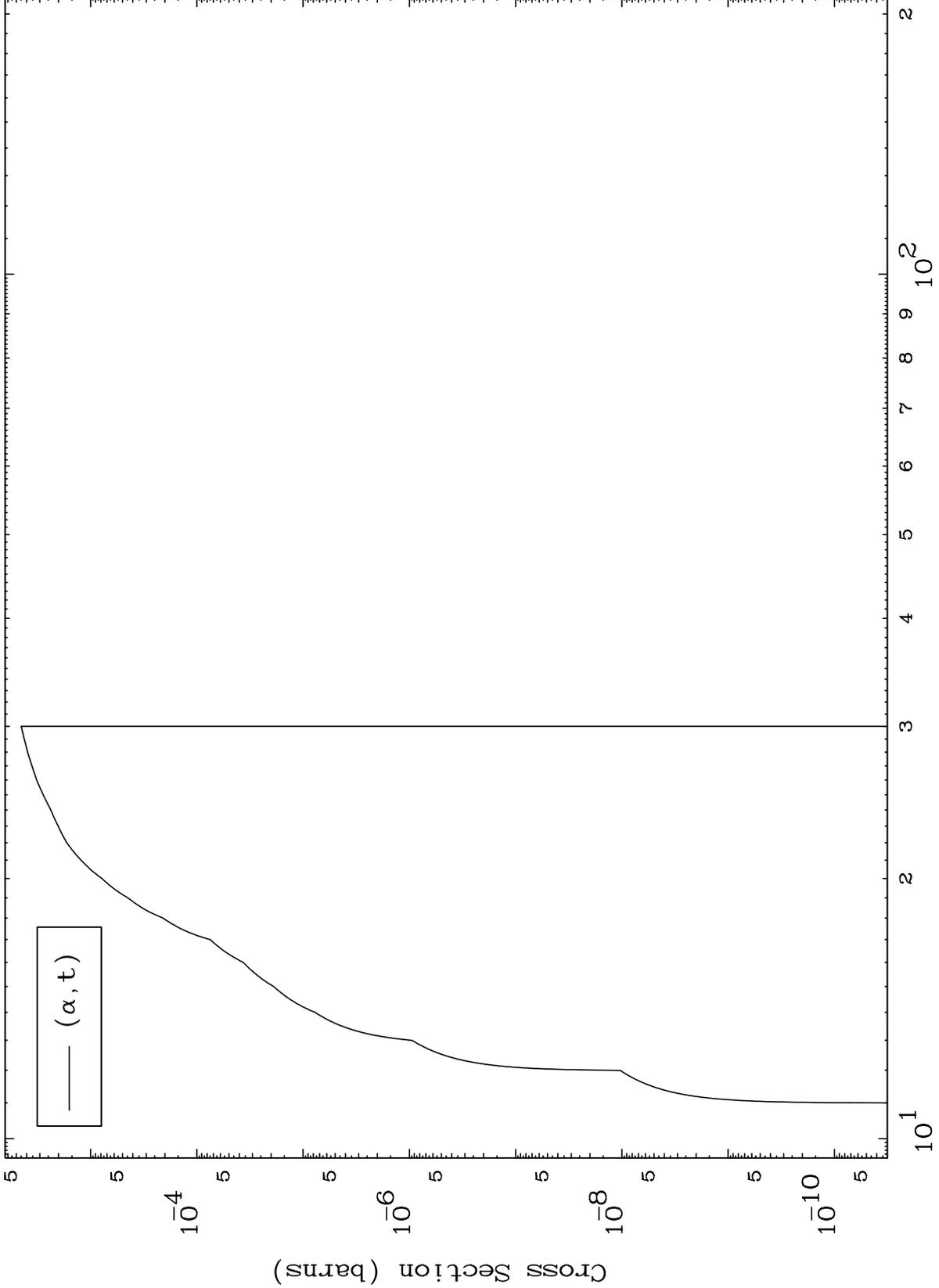


MAT 2528

( $\alpha, t$ ) Levels

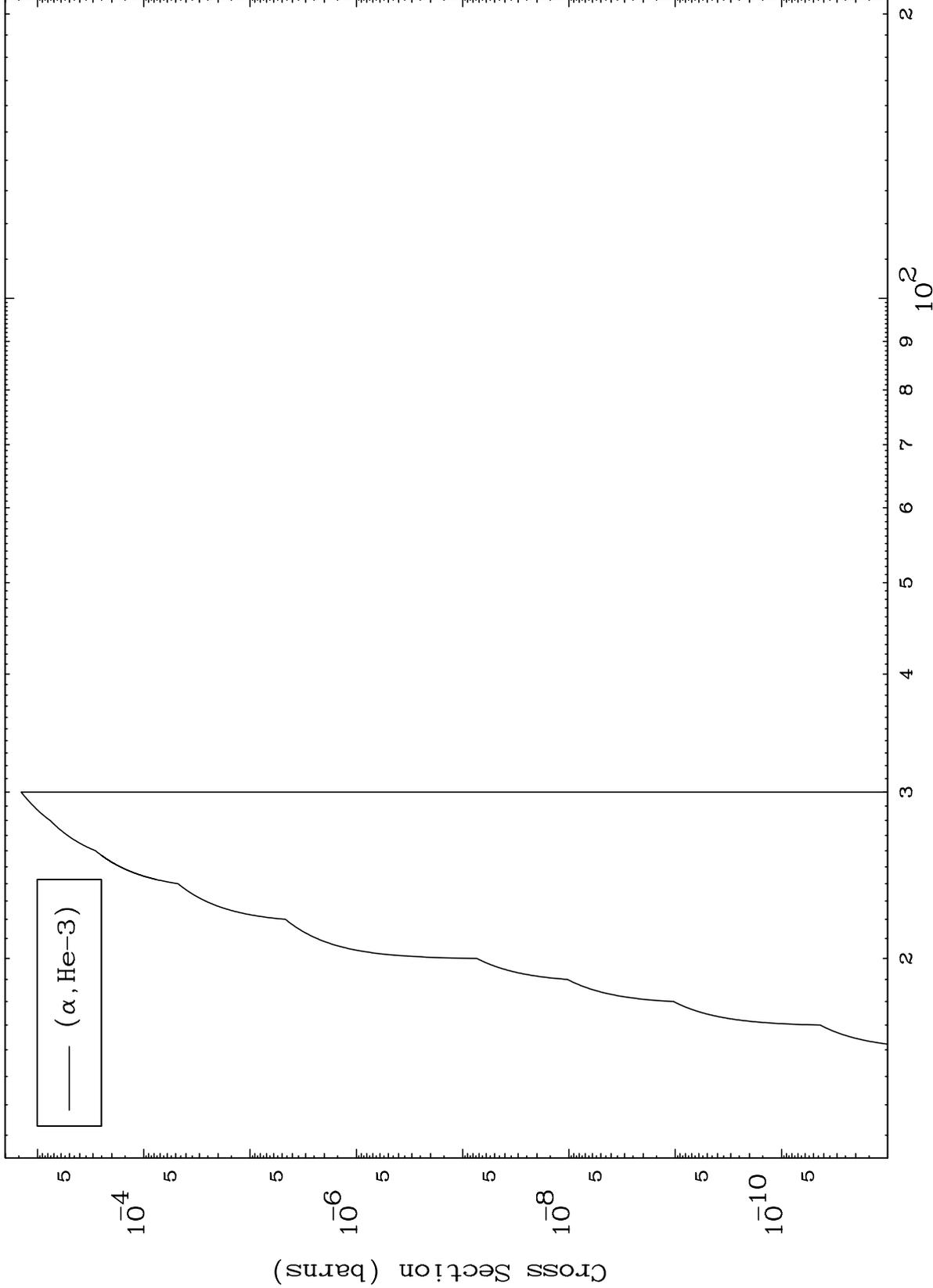
25-Mn-56

0 Kelvin Cross Sections



Incident Energy (MeV)

25-Mn-56

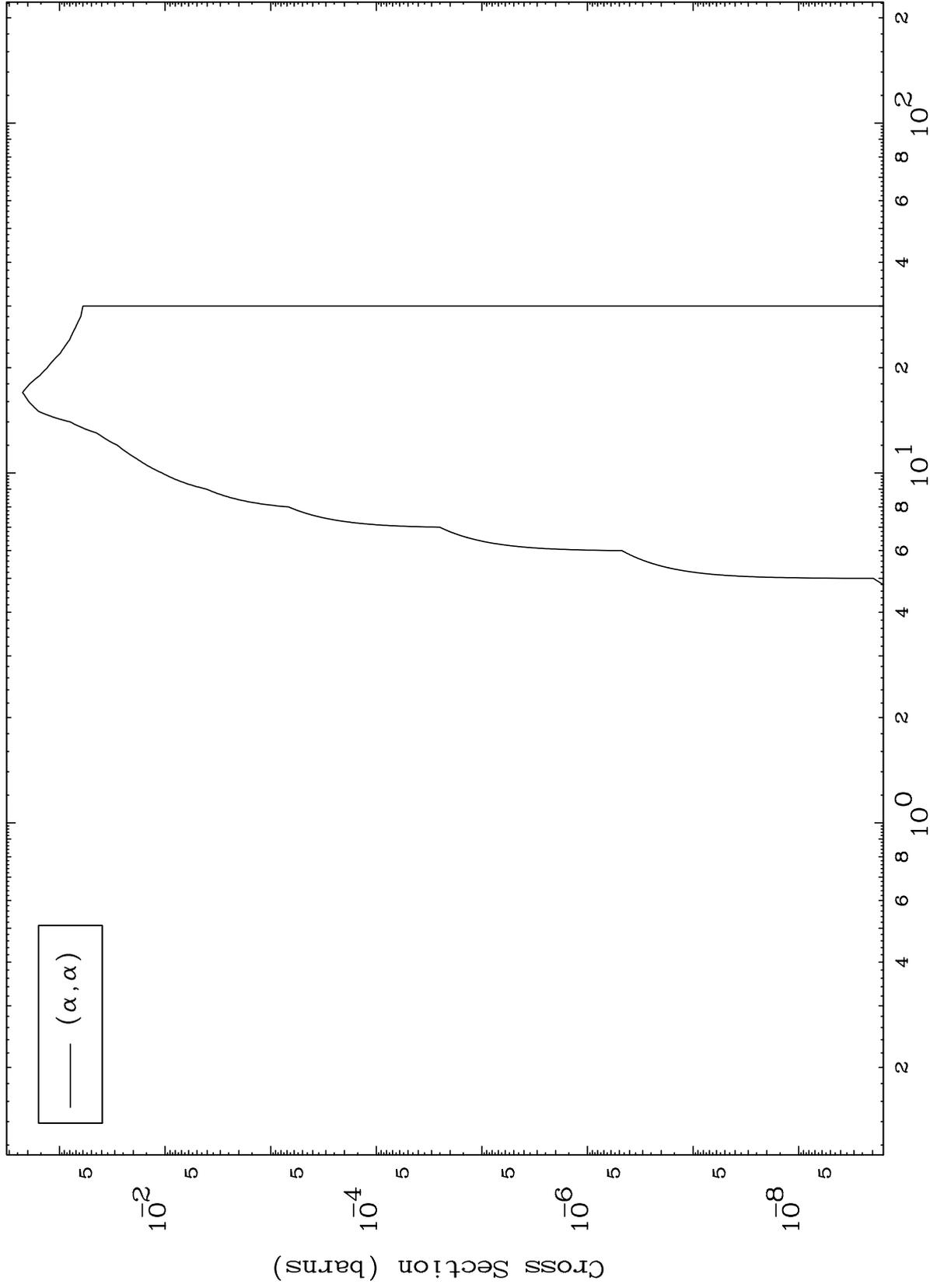


MAT 2528

( $\alpha, \alpha$ ) Levels

25-Mn-56

0 Kelvin Cross Sections



10

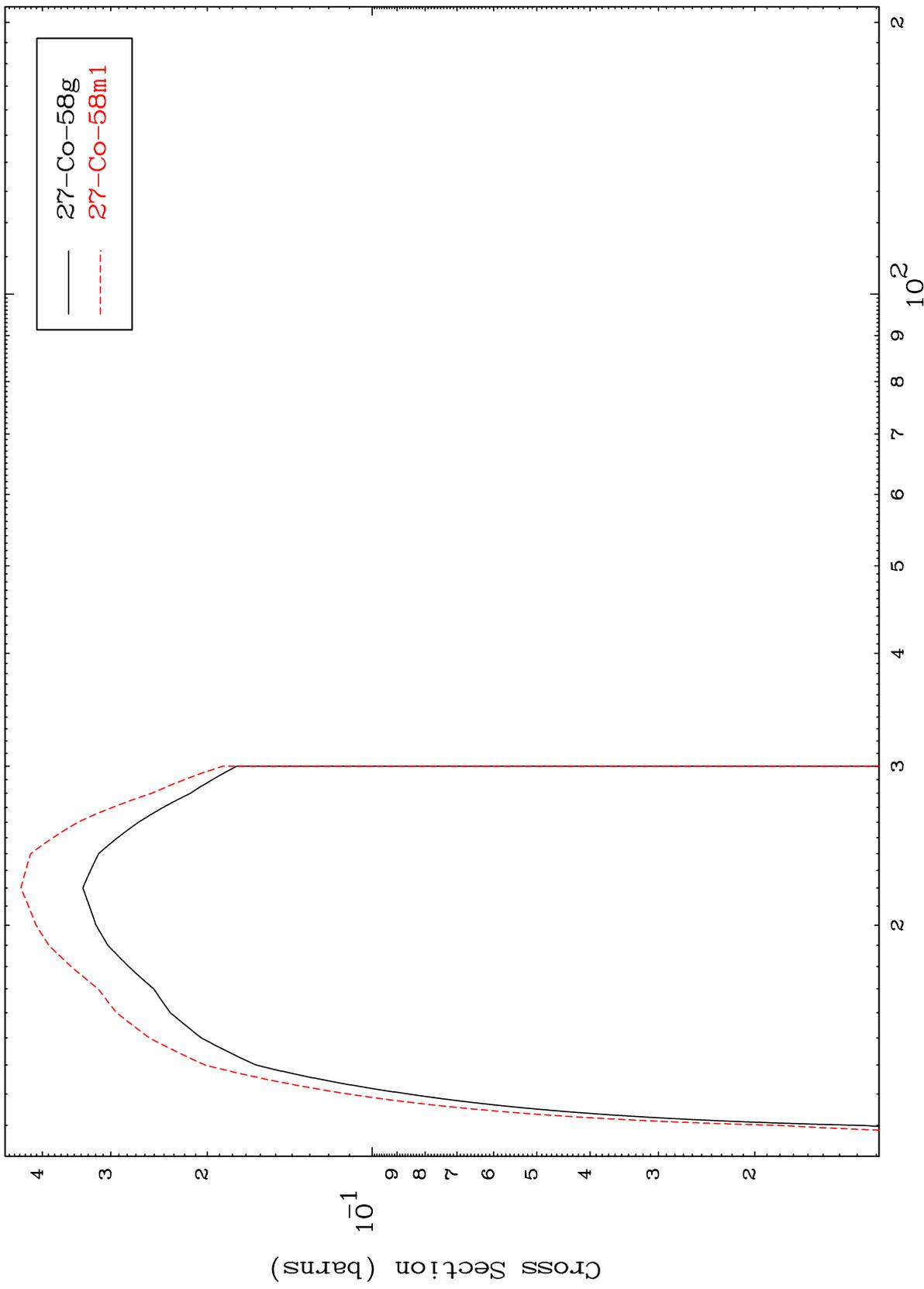
Incident Energy (MeV)

25-Mn-56

MAT 2528

25-Mn-56

$(\alpha, 2n)$   
Radionuclide Production Cross Section



11

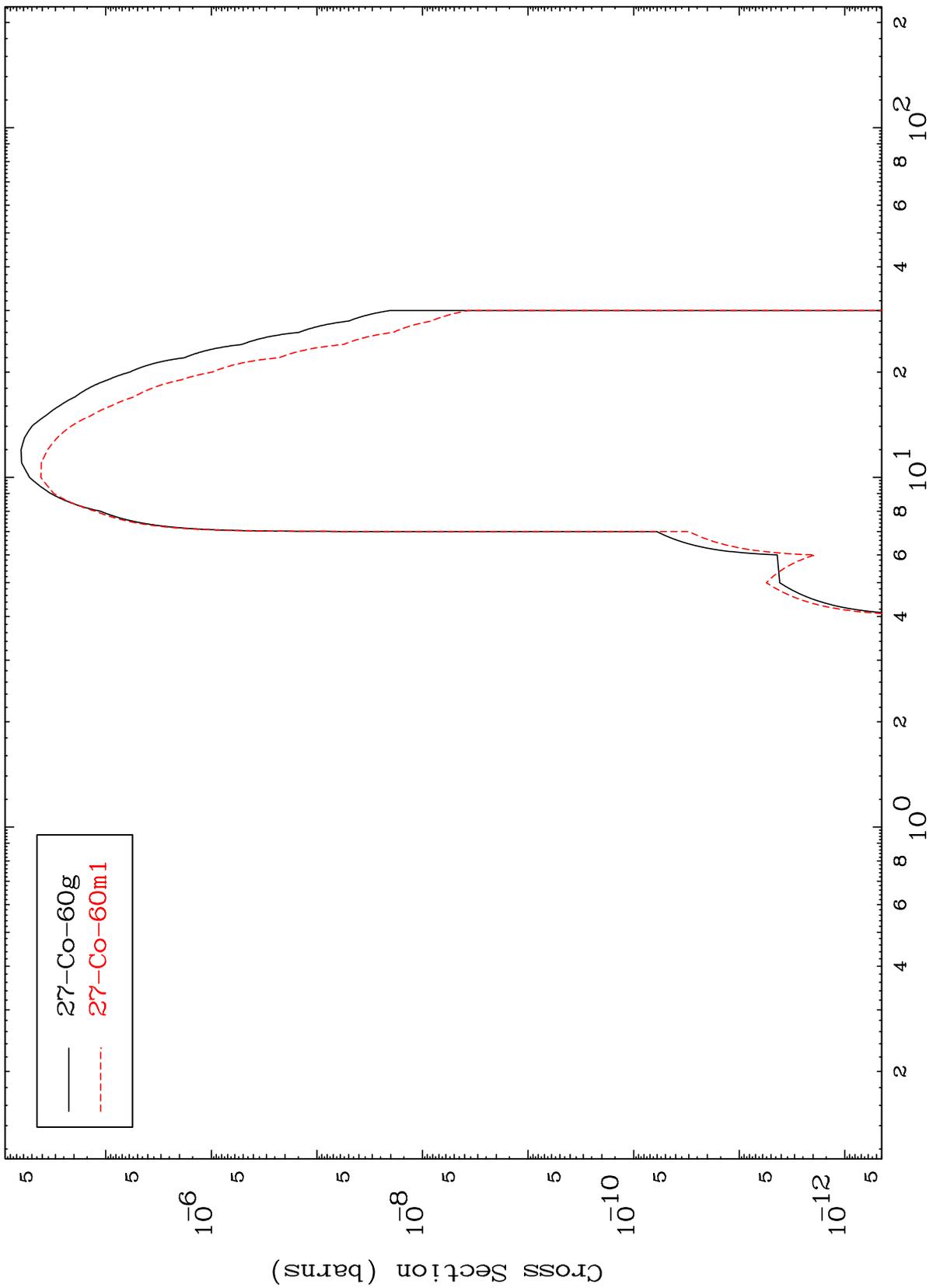
Incident Energy (MeV)

25-Mn-56

MAT 2528

25-Mn-56

$(\alpha, \gamma)$   
Radionuclide Production Cross Section



—  $^{27}\text{Co-60g}$   
- - -  $^{27}\text{Co-60m1}$

25-Mn-56

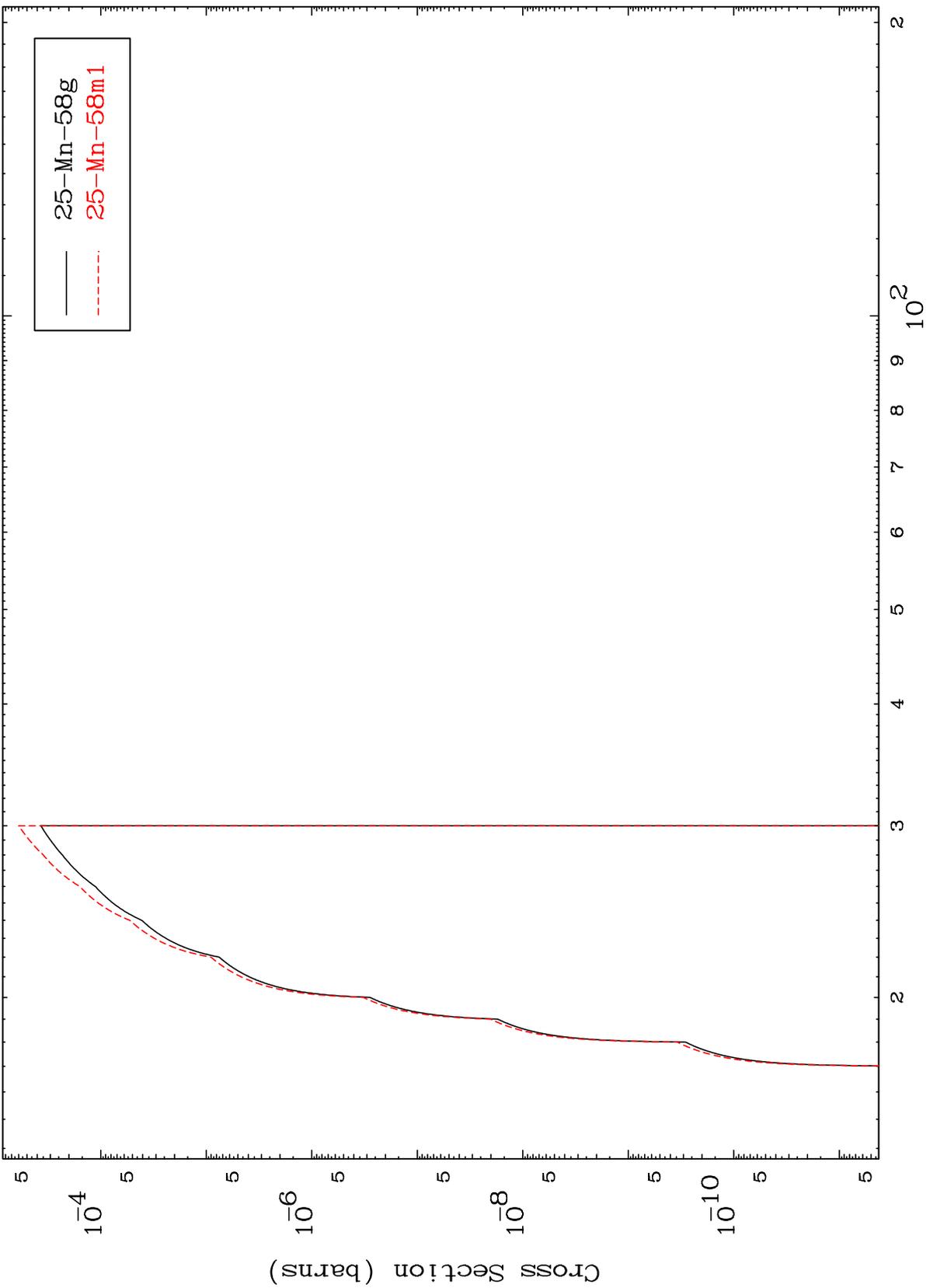
Incident Energy (MeV)

12

MAT 2528

25-Mn-56

( $\alpha, 2p$ )  
Radionuclide Production Cross Section



13

25-Mn-56

Incident Energy (MeV)