

Program EVALPLOT  
(Version 2018-1)

by

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(Present Contact Information)

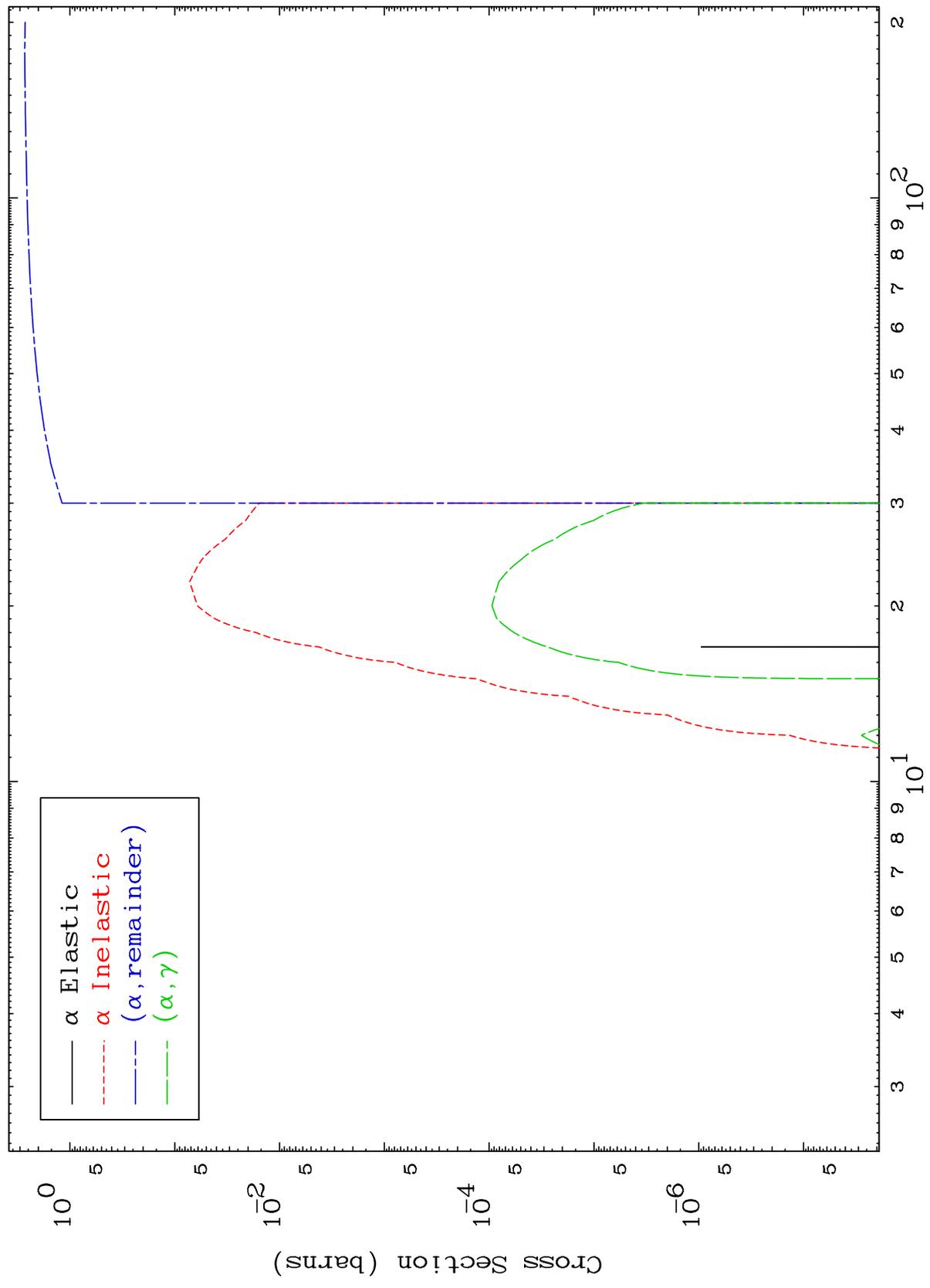
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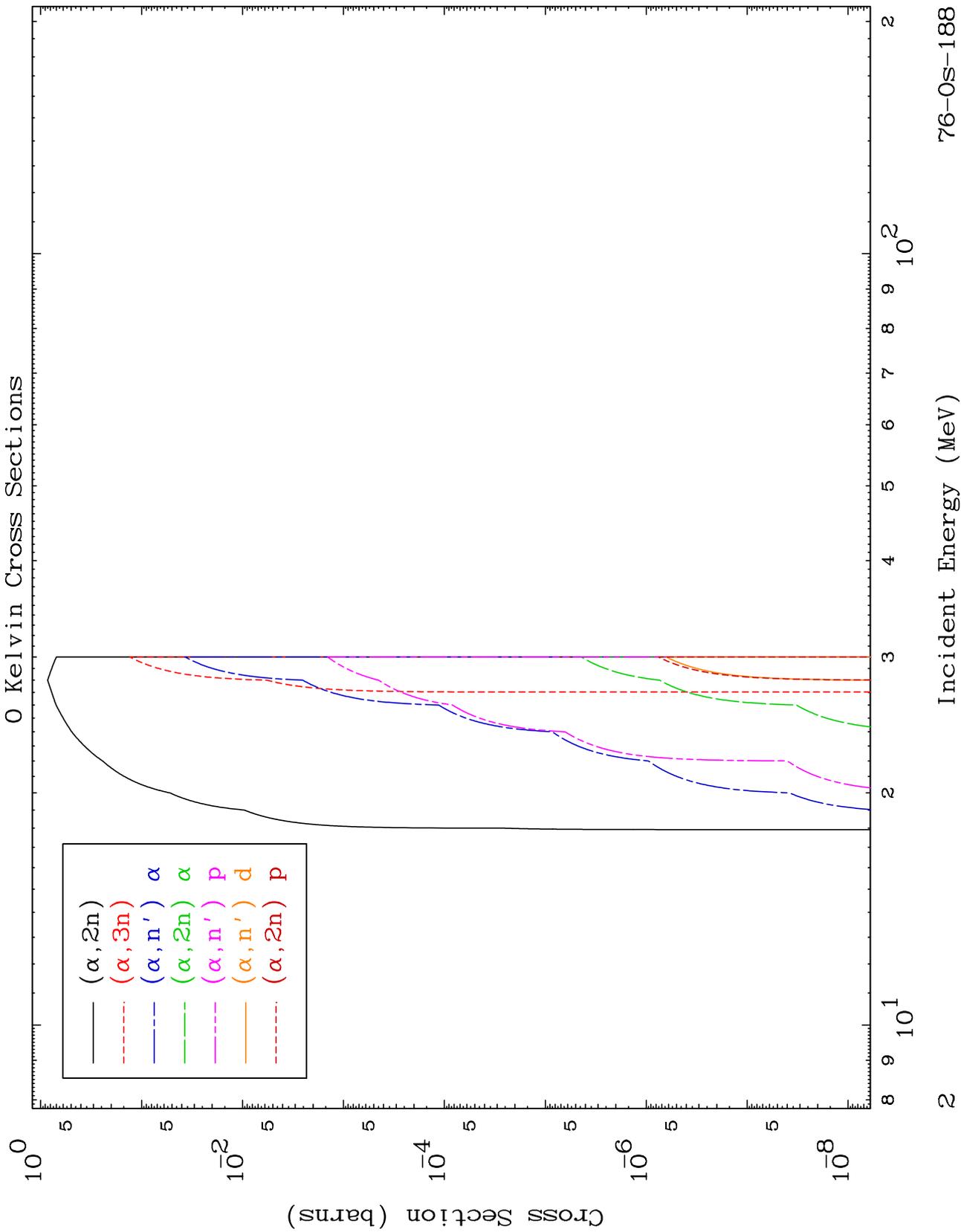
Press Mouse Button to Start



MAT 7637

$\alpha$  Neutron Production

76-Os-188



2

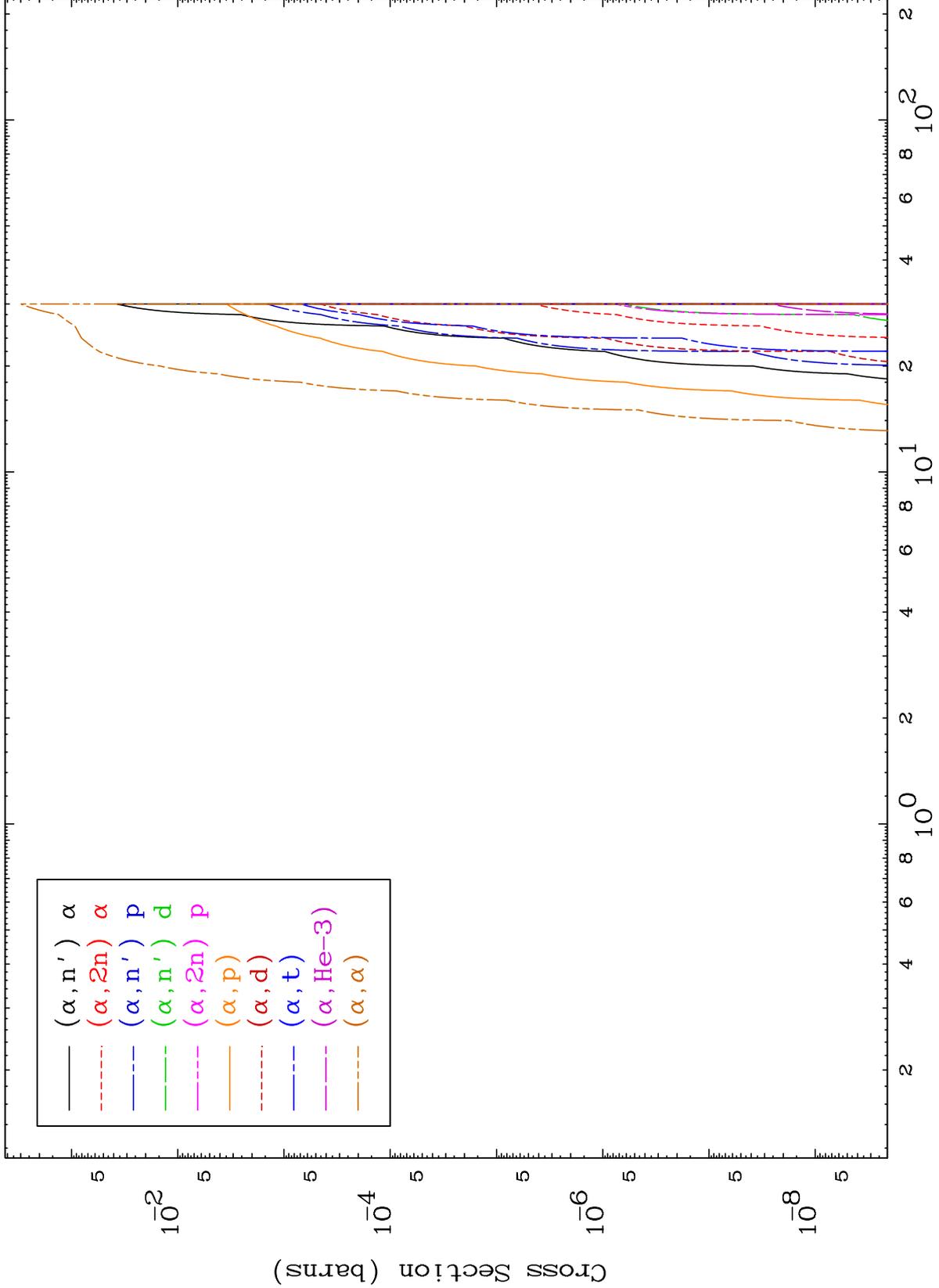
Incident Energy (MeV)

76-Os-188

MAT 7637

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

76-Os-188



76-Os-188

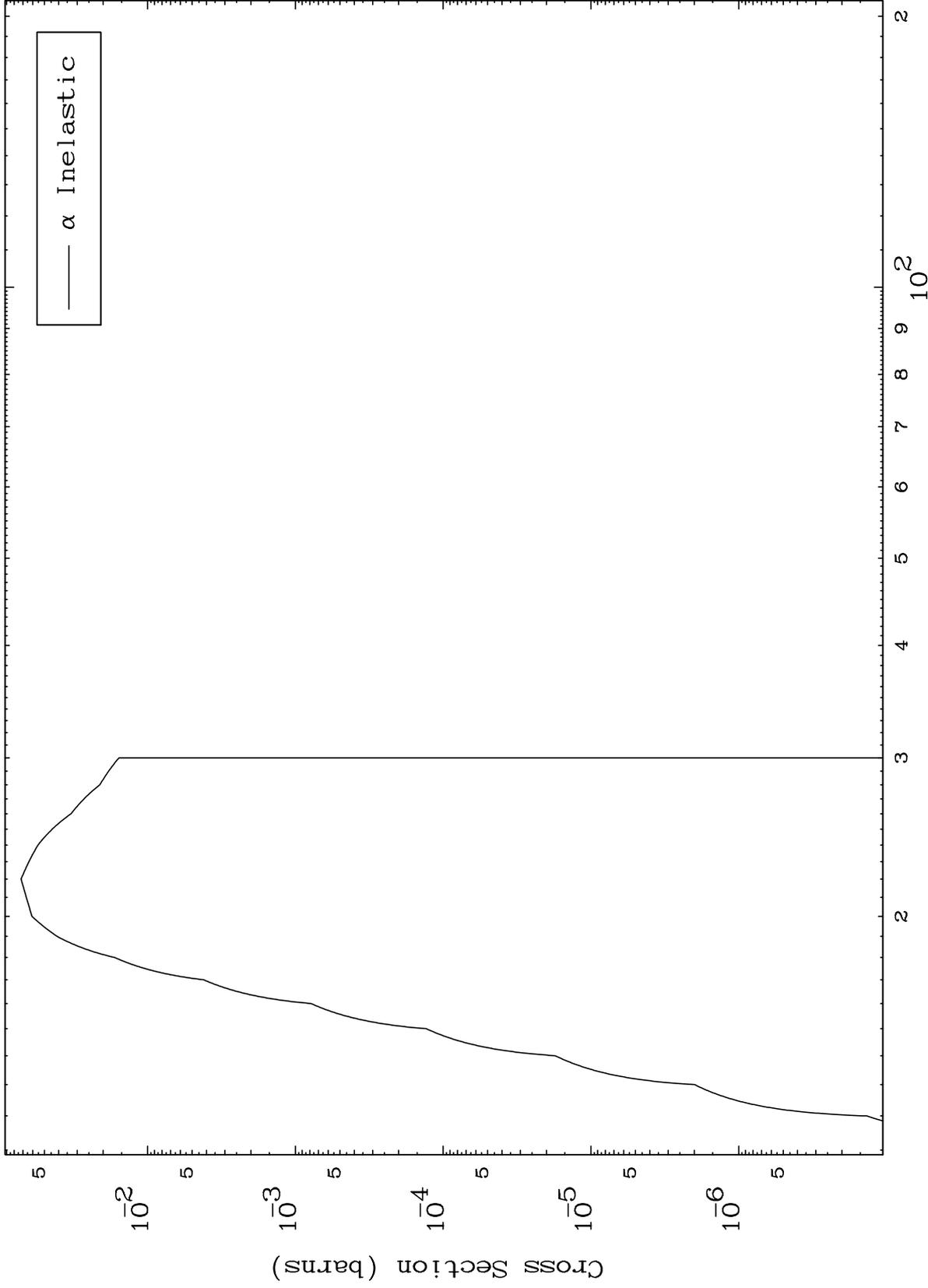
Incident Energy (MeV)

MAT 7637

( $\alpha, n'$ ) Level

76-0s-188

0 Kelvin Cross Sections



4

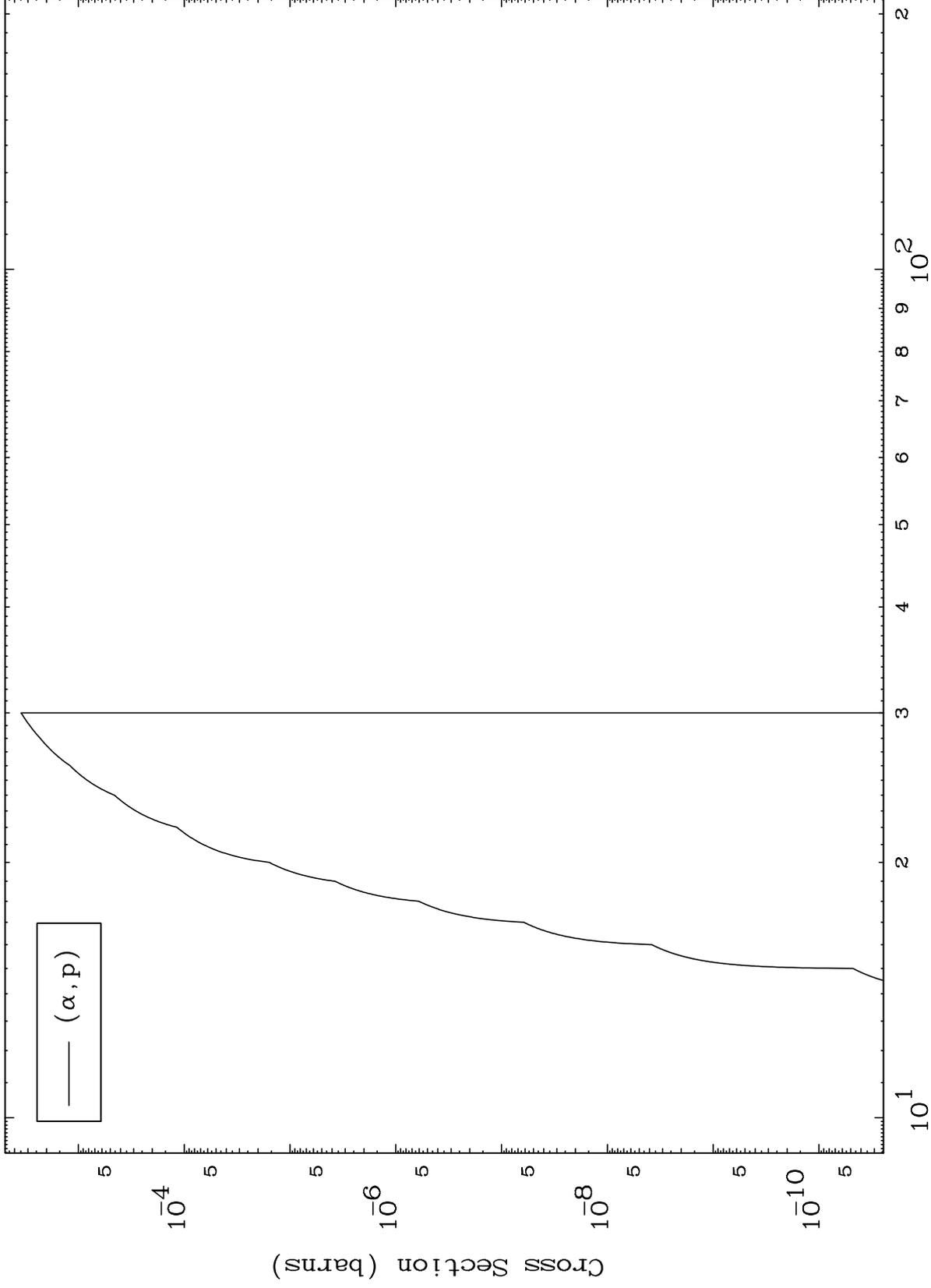
Incident Energy (MeV)

76-0s-188

MAT 7637

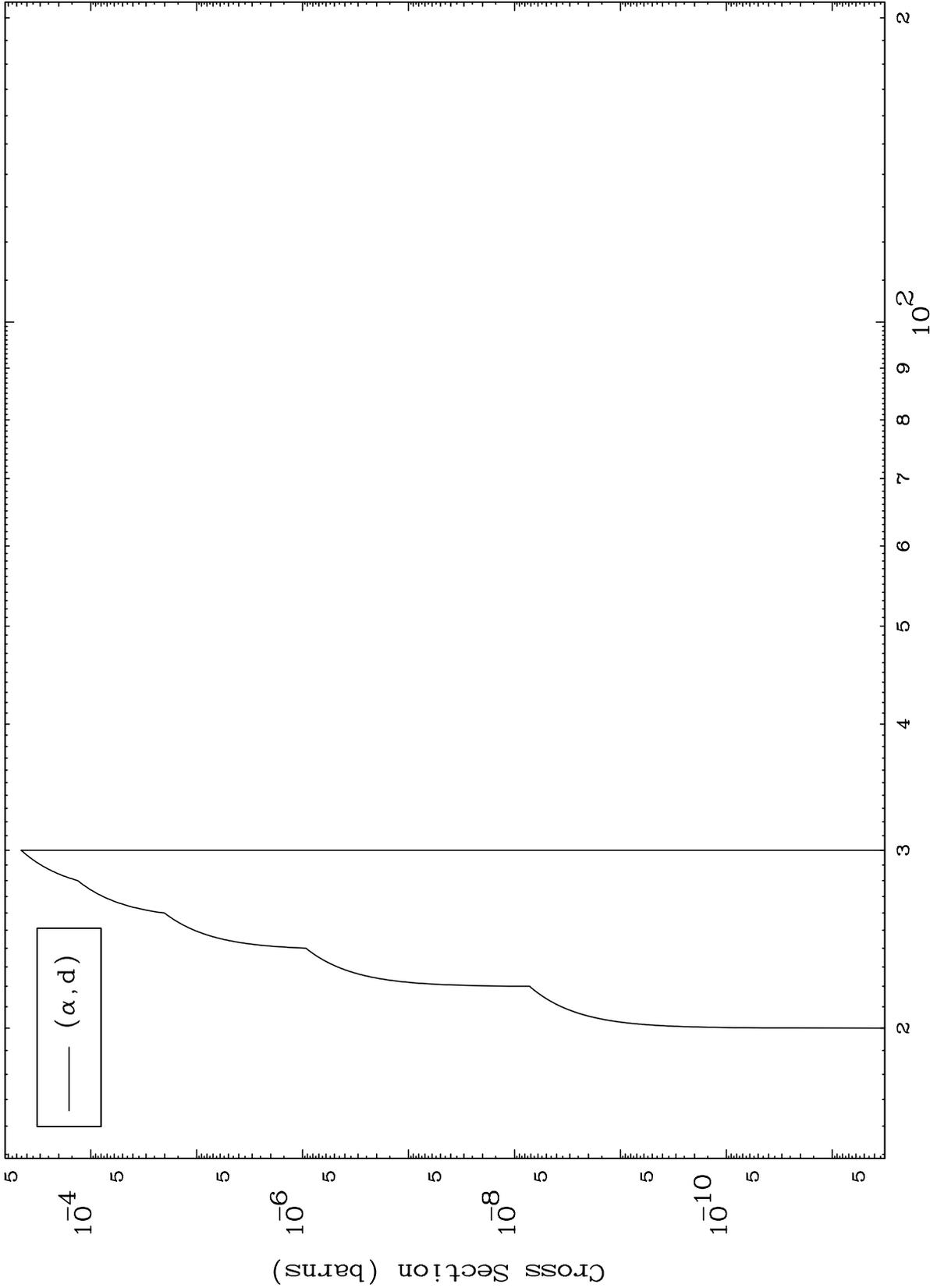
( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

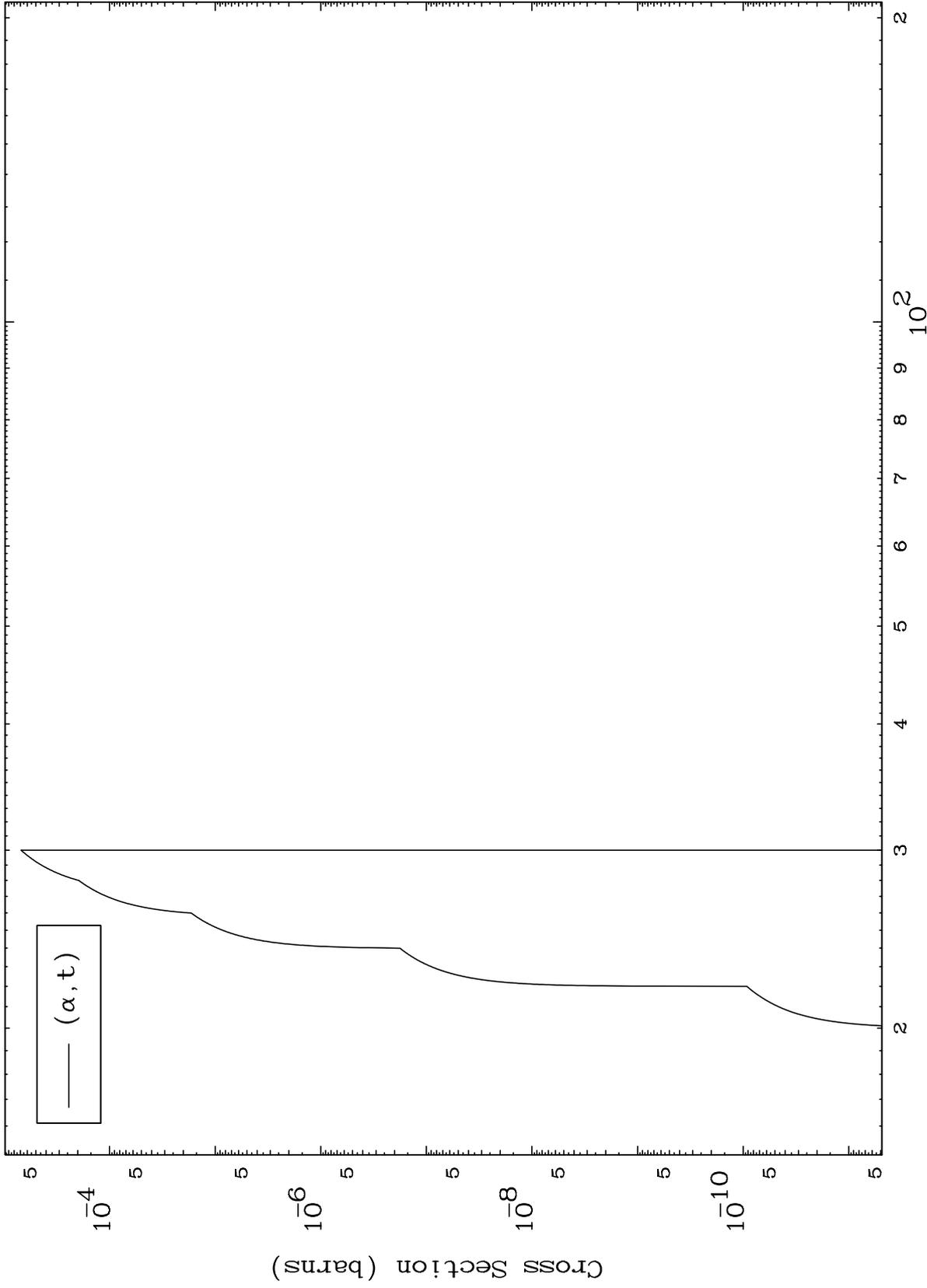
76-Os-188

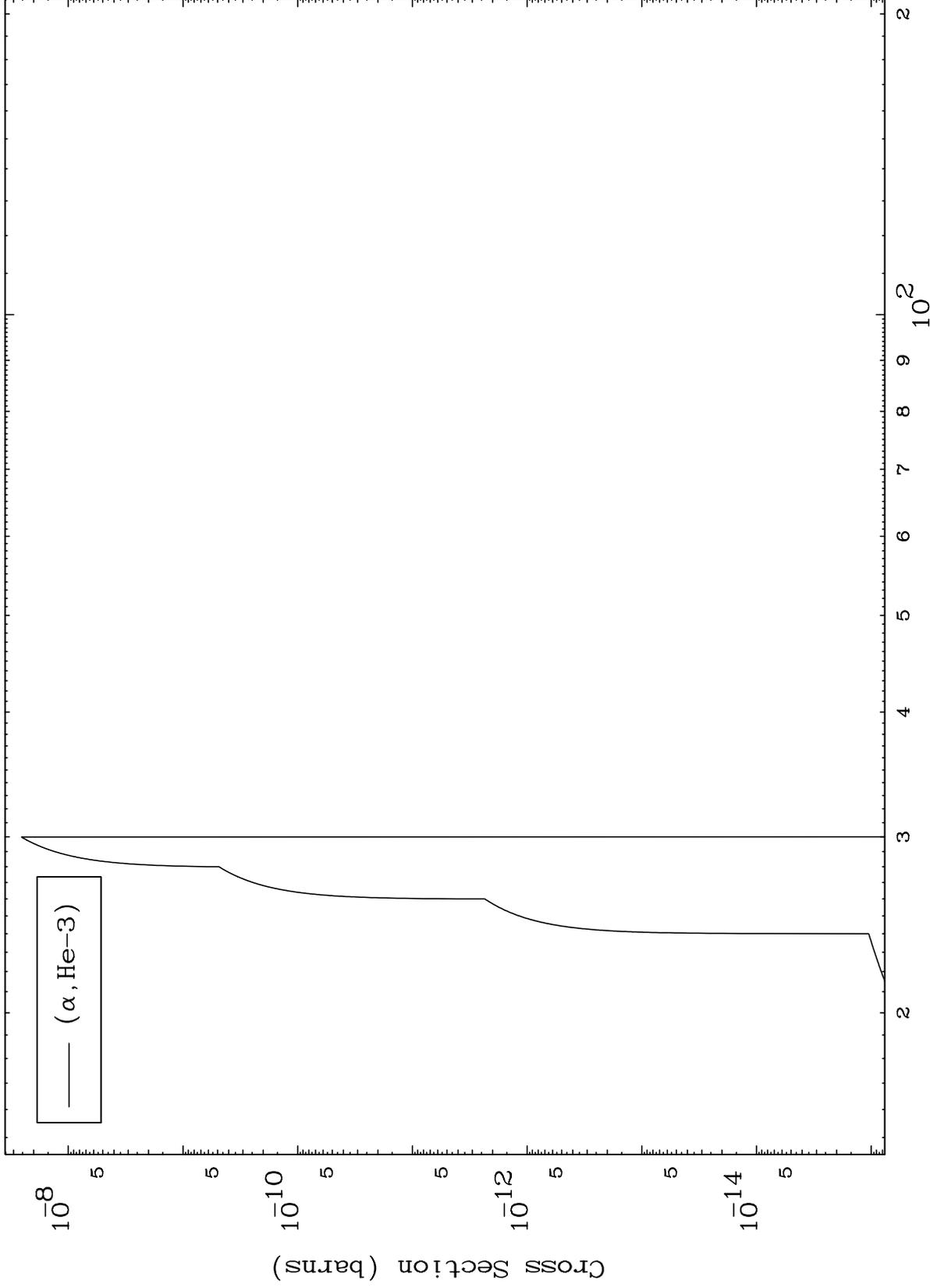


Incident Energy (MeV)

76-Os-188





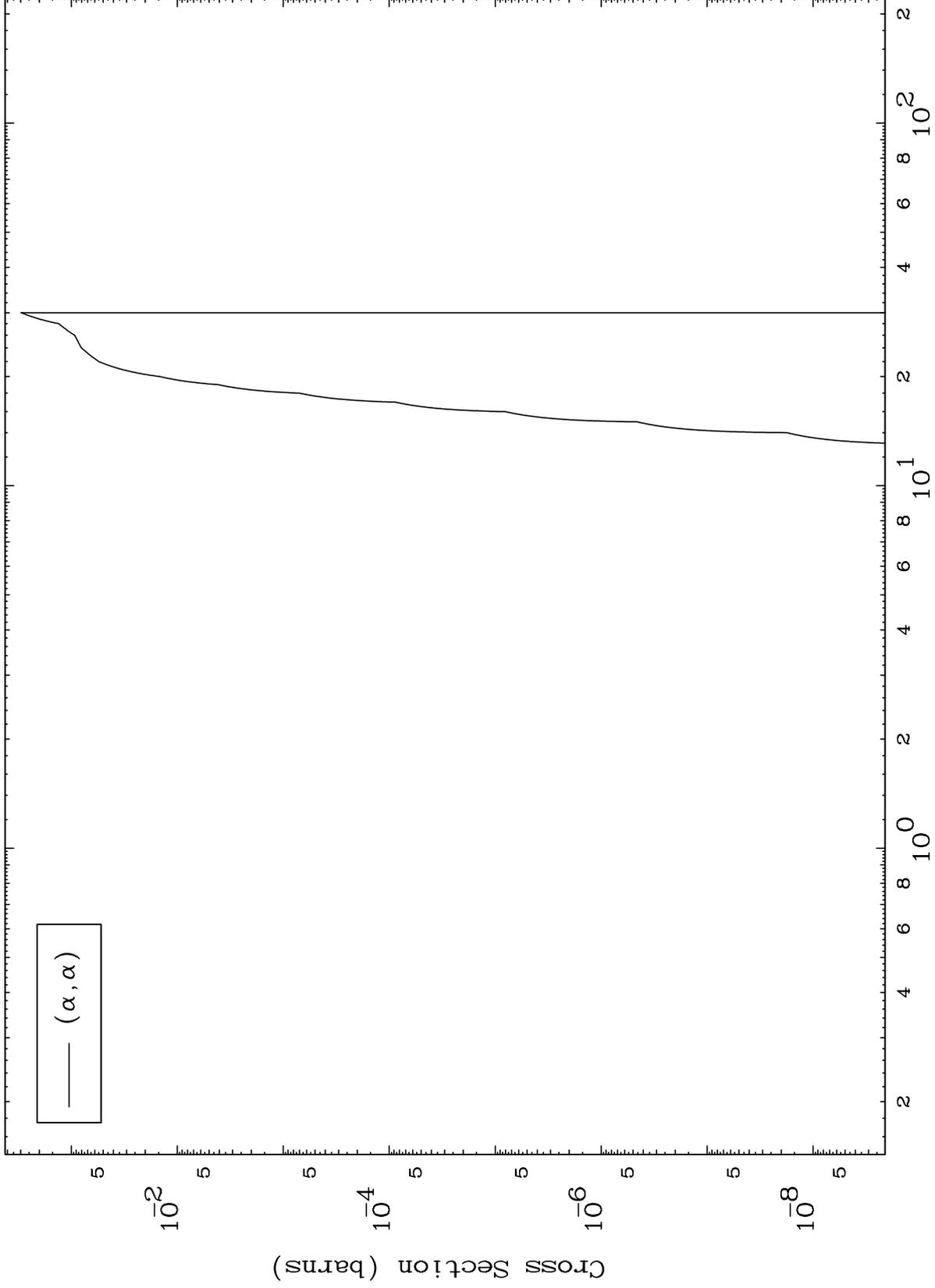


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( $\alpha, \alpha$ ) Levels

76-Os-188

0 Kelvin Cross Sections

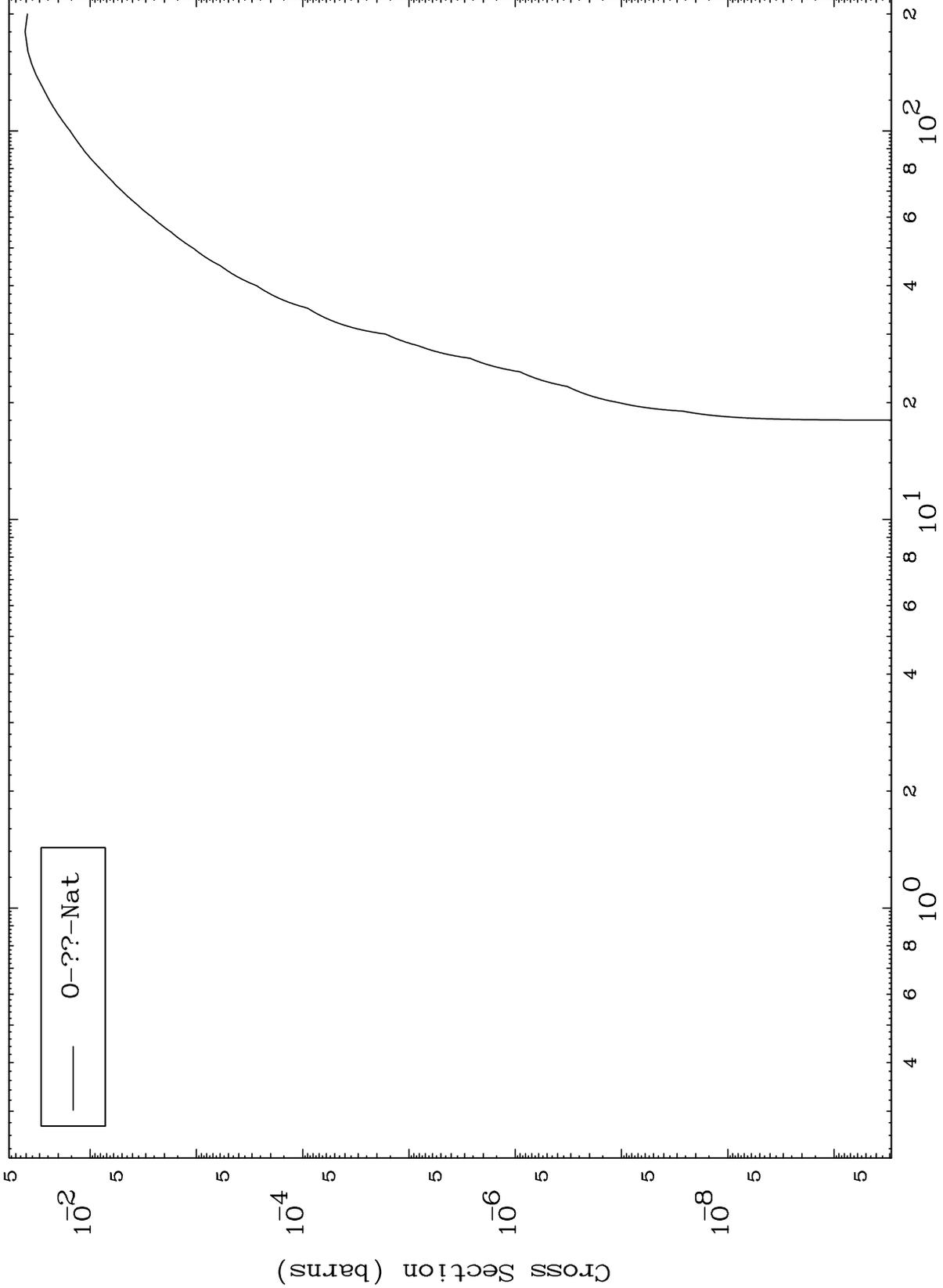


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$\alpha$  Fission

76-Os-188

Radionuclide Production Cross Section

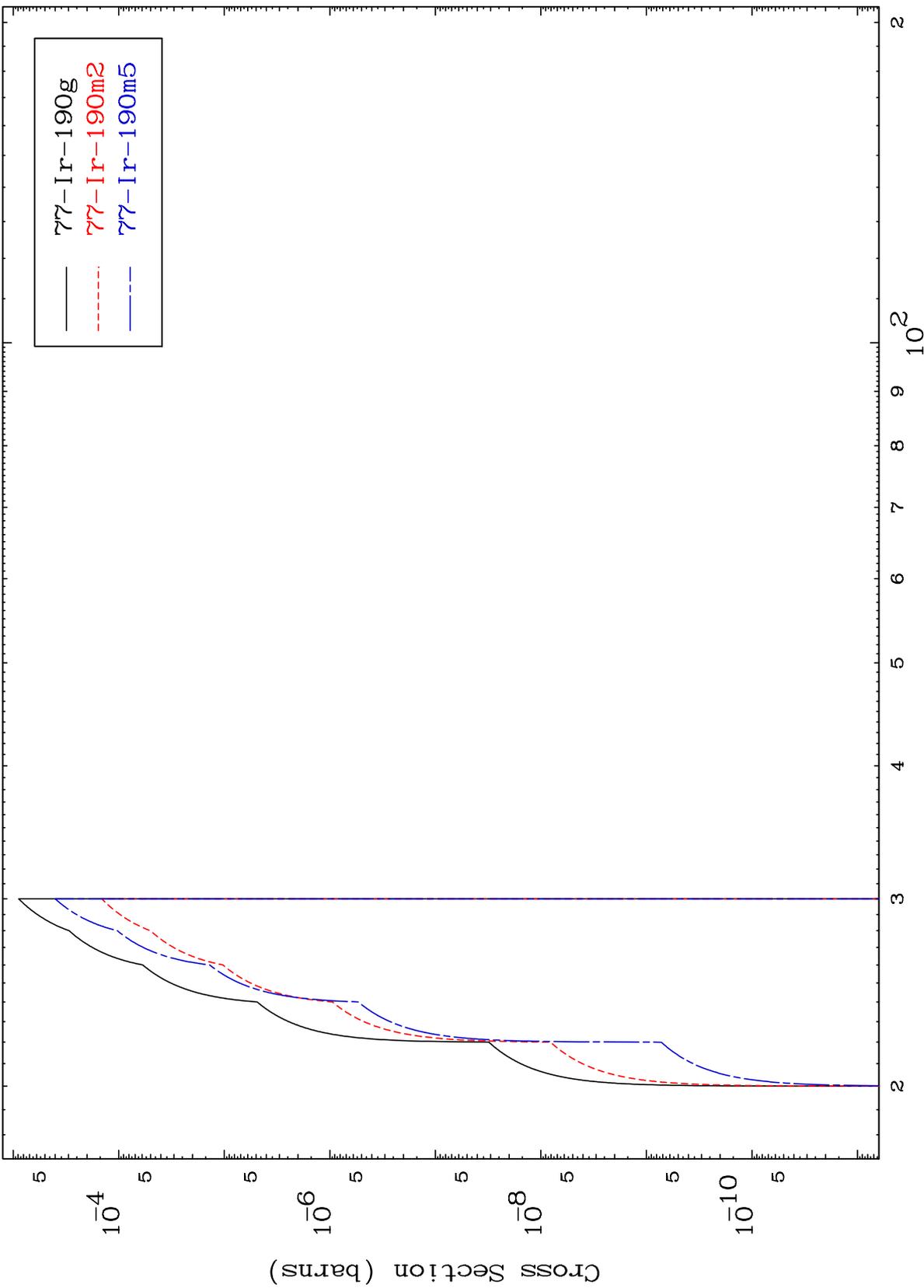


10

Incident Energy (MeV)

76-Os-188

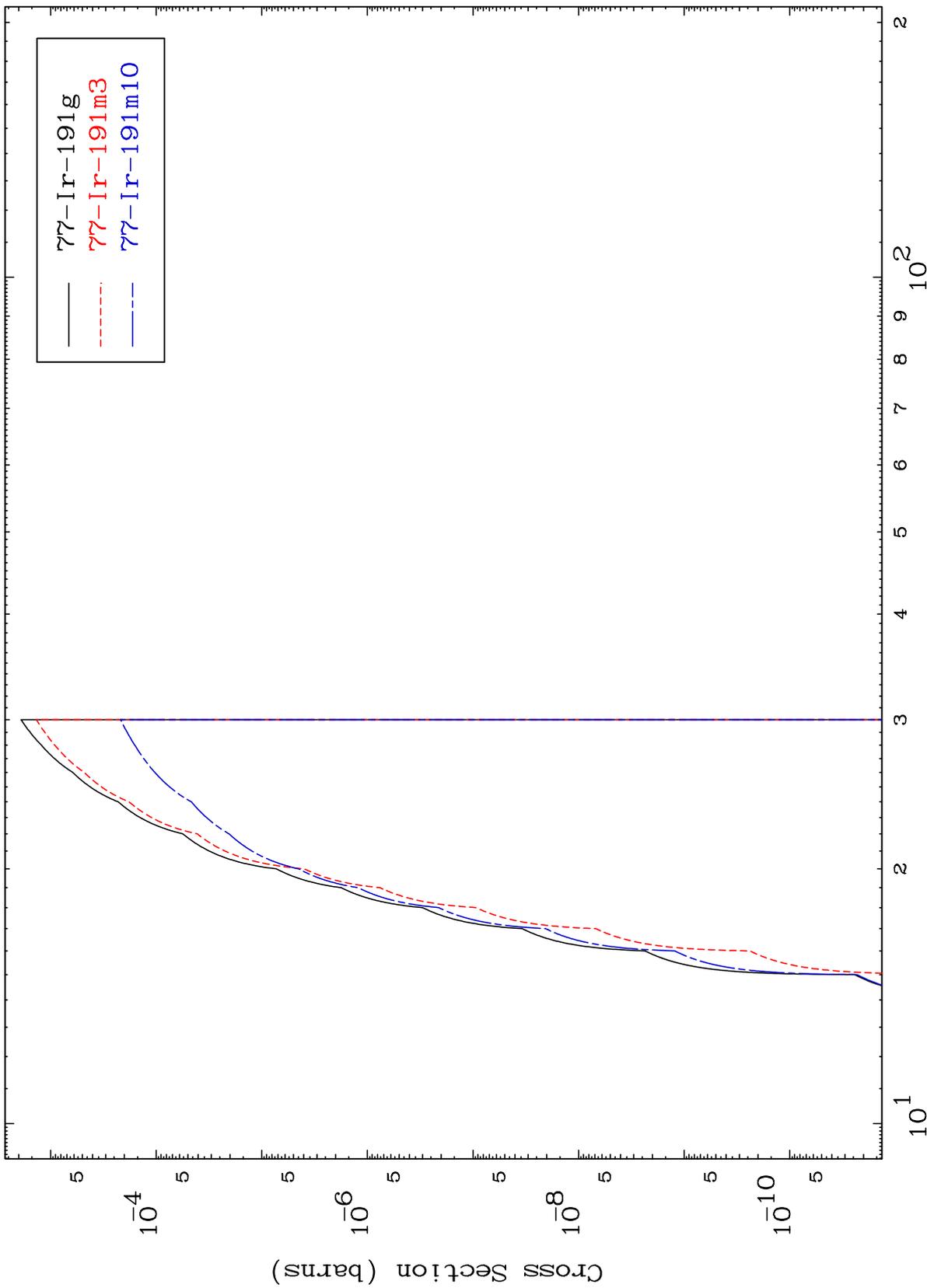
Radionuclide Production Cross Section



MAT 7637

76-Os-188

( $\alpha, p$ )  
Radionuclide Production Cross Section

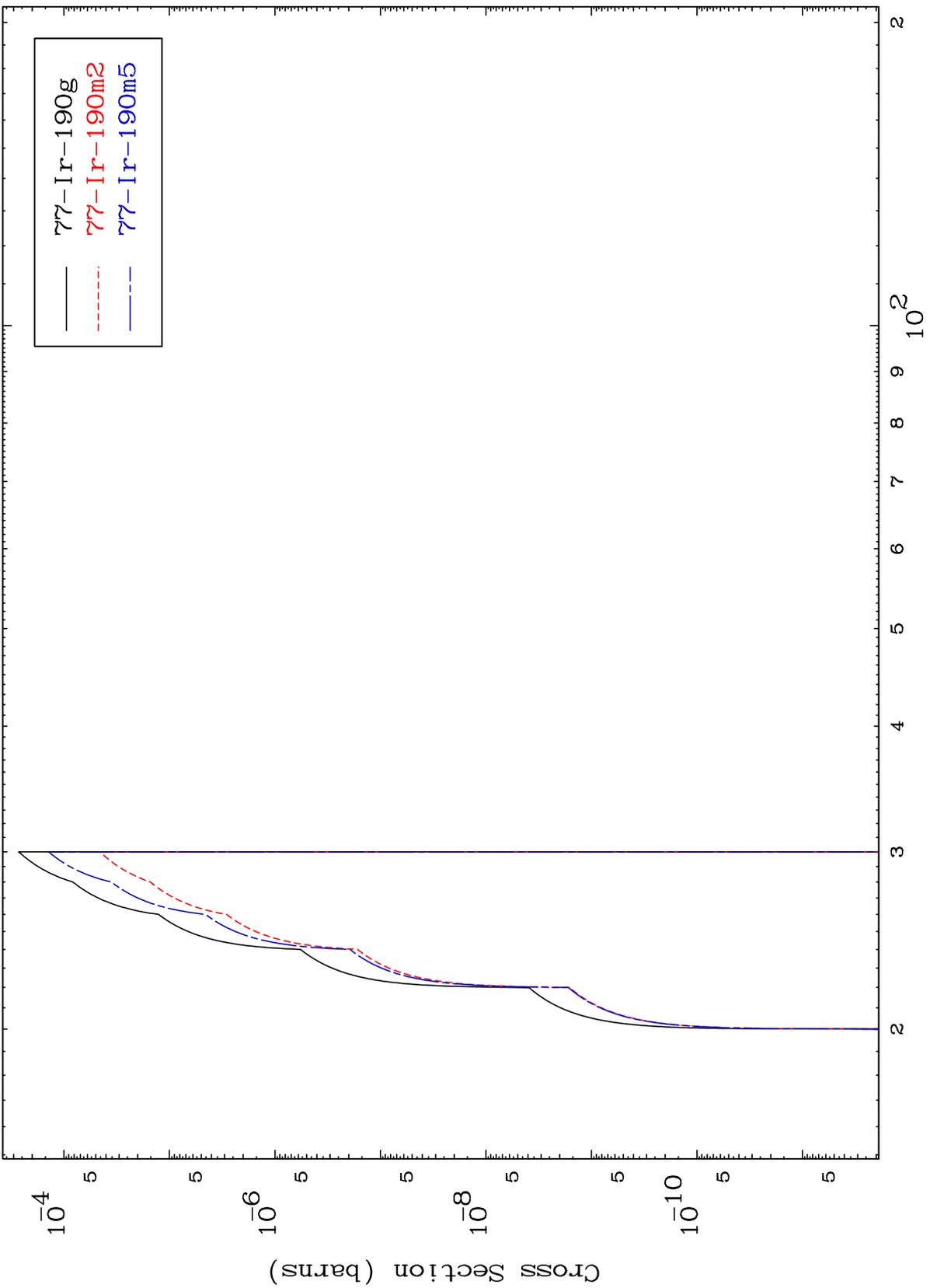


76-Os-188

Incident Energy (MeV)

12

Radionuclide Production Cross Section

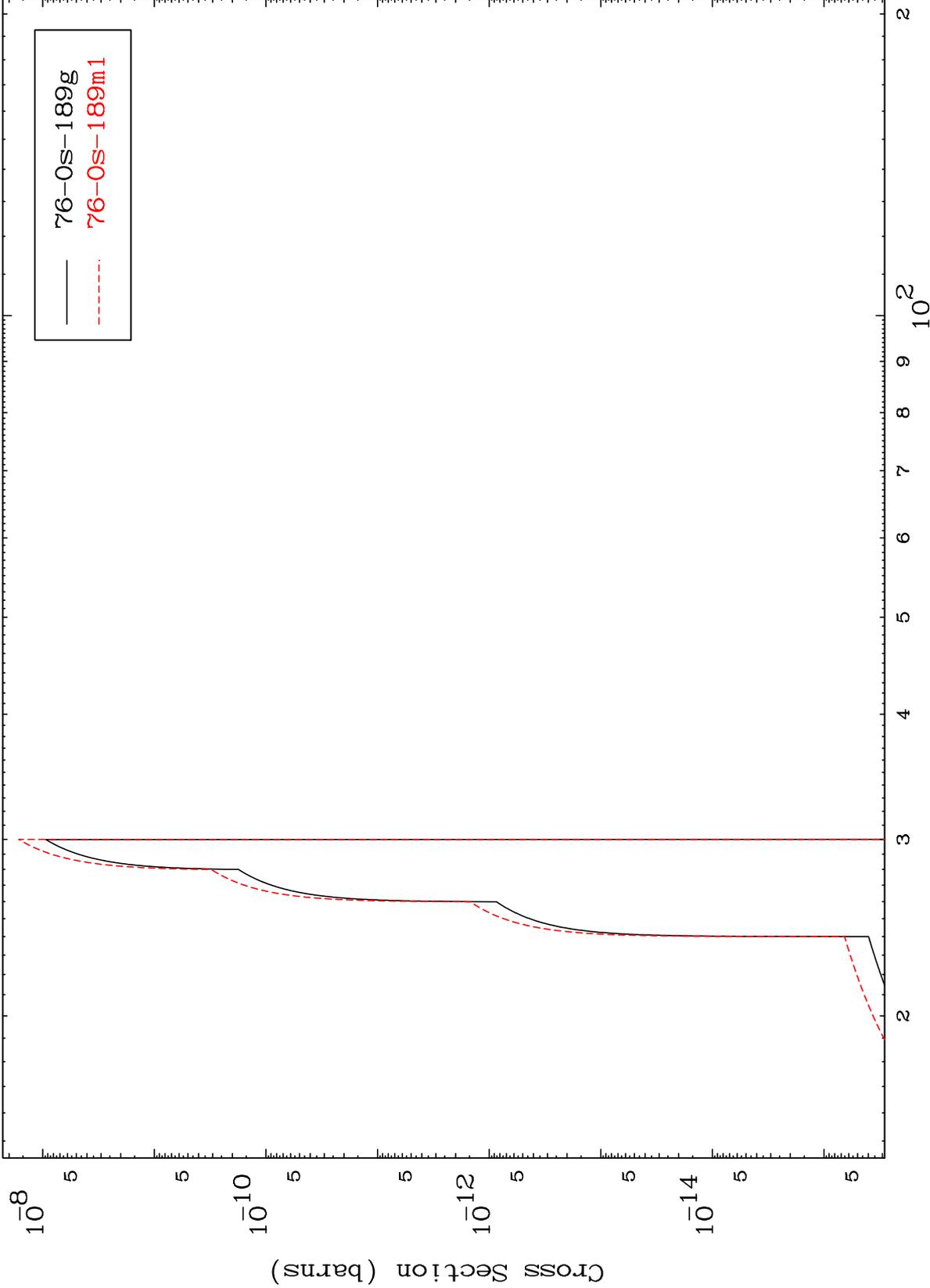


MAT 7637

( $\alpha, \text{He-3}$ )

76-Os-188

Radionuclide Production Cross Section



14

Incident Energy (MeV)

76-Os-188